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Duquesne Light

435 Sixth Avenue
Pittsburgh, Pennsylvania
15219

(412) 456-6000

July 30, 1979

United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Attention: Mr. Boyce H. Grier, Director

Subject: Beaver Valley Power Station, Unit No. 1
Docket No. 50-334
Response to IE Bulletin 79-14 (30 Day Report)

Dear Mr. Grier:

In response to IE Bulletin 79-14 (30 Day Report) we are transmitting herewith the applicable list of pipe lines required to comply with your item one of Enclosure one. These lists identify each seismically analyzed line number segregated by group and method of analysis. The group definitions are as follows:

Group 1

- Includes lines 2" and under, which are Q1, Q2, and Q3, for which adequacy was determined by the hand-calculation method. These lines are listed for system accountability only and will not be included in the verification program.

Group 2

- Includes Q1, Q2, and Q3 lines, which are 2 1/2" through and including 6" in size, and were analyzed by the hand-calculation or chart method.

Group 3

- Includes Q1, Q2, and Q3 lines over 6" in size and selected lines which are 2 1/2" through and including 6" in size. These lines have been seismically analyzed with the computer programs.

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The seismic analysis input sources are identified for each of the inspection attributes by the following letter designation:

- A - Pipe Geometry
- B - Support and Restraint Design, including pipe attachments
- C - Penetrations
- D - Valve and Valve Operator

Embedments are not considered to be attributes that should be listed since they are not part of the seismic analysis input.

The verification method and assignments are identified as follows:

- "A" - Verification was performed by physical inspection of the system, or portion of the system, to determine that the pipe geometry, support and restraint design, locations, penetration clearances, valve, valve operator, and valve orientation agree with the referenced drawings.
- "B" - Designated systems require no physical inspection, because we are confident that the drawings represent the as-built conditions since inspections performed in "A" confirm that plant conditions agree with the drawings.

As-built drawings require comparison with the seismic analysis input data for the hand-calculation methods (Group 2). The computer reanalysis resulting from the Show Cause Order of March 13, 1979, is in complete agreement with the as-built isometrics and thus meets the intent of IE Bulletin 79-14.

The investigation into the baseplate problem, required by IE Bulletin 79-02, indicates that a high degree of agreement now exists between the drawings and the plant conditions for the baseplates.

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It should be noted that Appendix A of IE Bulletin 79-14 refers to seismic analyses being affected by differences between the as-built conditions and the original design. These conditions are attributed to the seismic analyses having been based on out-of-date and inaccurate MSK drawings or sketches. This situation has been corrected by the seismic analysis input for Group 3, being based on the as-built isometric drawings.

Approximately 20% of the seismic analysis problems are in process and we are unable at this time to provide the source data for the valve and valve operators.

This information will be available within the next month and will be included in the report for action item 2.

We trust the above satisfies the requirements of Section 1 to Enclosure 1 of IE Bulletin 79-14. Verification results will be included in the upcoming 60-day report.

Very truly yours,



C. N. Dunn
Vice President, Operations

cc: United States Nuclear Regulatory Commission
Office of Inspection and Enforcement
Division of Reactor Operations Inspection
Washington, D. C. 20555

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