

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun #1
DATE July 9, 1979
COMPLETED BY B. J. Hickie
TELEPHONE (402) 536-4413

MONTH June, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>452.1</u>	17	<u>443.7</u>
2	<u>451.4</u>	18	<u>445.4</u>
3	<u>450.1</u>	19	<u>443.8</u>
4	<u>449.7</u>	20	<u>444.9</u>
5	<u>448.7</u>	21	<u>444.5</u>
6	<u>447.7</u>	22	<u>444.0</u>
7	<u>337.2</u>	23	<u>443.8</u>
8	<u>147.1</u>	24	<u>443.7</u>
9	<u>416.5</u>	25	<u>444.1</u>
10	<u>421.3</u>	26	<u>442.9</u>
11	<u>434.1</u>	27	<u>442.0</u>
12	<u>442.8</u>	28	<u>439.8</u>
13	<u>441.7</u>	29	<u>438.7</u>
14	<u>442.8</u>	30	<u>439.9</u>
15	<u>442.8</u>	31	<u> </u>
16	<u>442.8</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE July 9, 1979
 COMPLETED BY B. J. Hickie
 TELEPHONE (402) 536-4413

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No.1
2. Reporting Period: June 1979
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	4,343.0	50,520.0
12. Number Of Hours Reactor Was Critical	707.1	4,321.4	40,286.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,130.0
14. Hours Generator On-Line	704.3	4,306.6	39,372.7
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	973,249.5	5,929,473.3	47,577,896.6
17. Gross Electrical Energy Generated (MWH)	325,050.0	2,002,079.9	15,806,341.6
18. Net Electrical Energy Generated (MWH)	308,875.9	1,904,659.1	14,926,064.8
19. Unit Service Factor	97.8	99.2	77.9
20. Unit Availability Factor	97.8	99.2	77.9
21. Unit Capacity Factor (Using MDC Net)	93.9	96.0	65.2
22. Unit Capacity Factor (Using DER Net)	93.9	96.0	64.4
23. Unit Forced Outage Rate	2.2	0.8	4.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

January 1, 1980 - Refueling Outage - Two-month Duration

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY

INITIAL ELECTRICITY

COMMERCIAL OPERATION

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun #1
 DATE July 9, 1979
 COMPLETED BY B. J. Hickie
 TELEPHONE (402) 536-4413

REPORT MONTH June 1979

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
79-01	790607	F	15.7	A	3	N/A	N/A	N/A	Fire Protection System deluge valve failure causing wetting of turbine instrumentation resulting in turbine/reactor trip.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

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 Exhibit I - Same Source

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

June 1979
Monthly Operations Report

I. OPERATIONS SUMMARY

The Fort Calhoun Station operated at a nominal 100 per cent power for the month except for a short duration overnight outage on June 7 caused by the inadvertent actuation of the main turbine lube oil reservoir fire protection deluge valve when the system was repressurized following fire protection modification work. Licensed operator training at the Combustion Engineering simulator was completed this month.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
79-016	On May 17, 1979, the Omaha Public Power District was informed by its NSSS supplier, Combustion Engineering, that, as a result of testing the first removed irradiated reactor vessel surveillance capsule, the fluence, and thus the Nil Ductility Transition Temperature, were higher than predicted. As a result of the NDTT shift, the existing Technical Specification curves, Figures 2-1A and B and 2-2A and B, for Reactor Coolant System Pressure-Temperature Limitations were not sufficiently restrictive.

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
DCR 76-78	Storage bracket for long tool, FH-6, completed as designed.
EEAR 78-72	Auxiliary steam to grid backwash lines and strainer CW-7/completed as designed.
DCR 75A-64	Plastic relief domes in Room 81/completed as designed.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

<u>Procedure</u>	<u>Description</u>
SP-FAUD-1	Fuel assembly uplift condition detection/all loops greater than 99 plus %.
SP-RPS-5	Excore detector symmetric offset recalibration/all conditions normal.

E. RESULTS OF LEAK RATE TESTS

None

F. CHANGES IN PLANT OPERATING STAFF

None

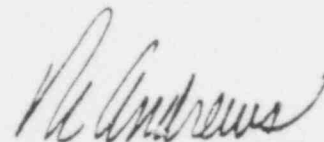
G. TRAINING

Training for the month of June consisted of the completion of all monitor team training, hot license and operator requalification training, and systems training for the crafts. Particular emphasis was shown towards natural circulation and loss of coolant accidents in light of Three Mile Island.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

Modifications and administrative changes designated in the NRC Safety Evaluation report for the fire protection program at the Fort Calhoun Station, designated to be completed in June 1979 were completed, except where specific relief was requested of the Commission.

Approved By



Manager-Fort Calhoun Station

Monthly Operations Report

Page

II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
317	6-12-79	Seismic Support FWS-83	Ordinary U-bolts replaced with U-bolts meeting requirements of ASME B & PV Code Section III, Subsection NG.
1118	6-26-79	Replace RPS "BC" matrix relay.	Relay replaced per PRC approved procedure.
1083	6-27-79	RPS Channel "A" NI outputs from auctioneer circuit are fluctuating.	Replaced amplitude selector and ran applicable sections of ST-RPS-11.
1052	6-26-79	Remove ASCO Solenoid Valve from containment, disassemble, and inspect as per NRC requirement.	PCV-742C was removed and inspected - appeared to be in good shape.
1109	6-26-79	C/T-122H hot leg temperature indicator failed low.	Replaced power supply.

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

June 1979
Monthly Operations Report

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None

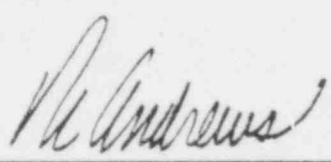
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Monthly Operations Report

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 COMPLETED BY B. J. Hickie
 TELEPHONE (402) 536-4413

MONTH June, 1979

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(9/77)

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Notes

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INITIAL CRITICALITY

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COMMERCIAL OPERATION

(9/77)

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June 1979

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun #1
 DATE July 9, 1979
 COMPLETED BY B. J. Hickie
 TELEPHONE (402) 536-4413

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
79-01	790607	F	15.7	A	3	N/A	N/A	N/A	Fire Protection System deluge valve failure causing wetting of turbine instrumentation resulting in turbine/reactor trip.

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² Reason:
 A-Equipment Failure (Explain)
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³ Method:
 1-Manual
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 3-Automatic Scram.
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⁴ Exhibit G - Instructions
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 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵ Exhibit I - Same Source

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Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending June 30, 1979.

1. Scheduled date for next refueling shutdown. January 1, 1980
2. Scheduled date for restart following refueling. March 1, 1980
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes

- a. If answer is yes, what, in general, will these be?

Stretching power to 1500 MWth is planned in conjunction with the change in fuel supplier to Exxon.

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

- c. If no such review has taken place, when is it scheduled?

4. Scheduled date(s) for submitting proposed licensing action and support information.
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Stretch Power Application
• Site Related Information,
July, 1979
• Non-Core Related Information,
October, 1979
• Core Related Analysis and
Tech. Spec. Changes,
November, 1979

First use of Exxon fuel in Fort Calhoun.
Stretching power from 1420 MWth to 1500 MWth.

6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>157</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>483</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by R-L Jaworski Date July 2, 1979