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July 16, 1979

United States Nuclear Regulatory Commission  
Attention: Boyce H. Grier, Director  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Reference: Beaver Valley Power Station, Unit No. 1  
Docket No. 50-334  
License No. DPR-66  
Response To IE Bulletin 79-13

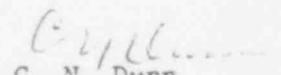
Dear Mr. Grier:

We have reviewed IE Bulletin 79-13 which reports cracking in feedwater system piping.

Attached is a report addressing the information requested in Item 5 of the subject Bulletin. We will continue to keep you informed of the actions being taken regarding this problem.

If you have any questions regarding this response, please contact my office.

Very truly yours,

  
C. N. Dunn  
Vice President, Operations

Attachment

cc: United States Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Division of Reactor Operations Inspection  
Washington, D. C. 20555

Mr. D. L. Wigginton, BVPS Licensing Project Manager  
United States Nuclear Regulatory Commission  
Operating Reactors Branch #4  
Washington, D. C. 20555

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DUQUESNE LIGHT COMPANY  
Beaver Valley Power Station  
Unit No. 1

RESPONSE TO IE BULLETIN NO. 79-13  
Letter Dated June 25, 1979

Item 5 Response

- a. The inspection program as required in Item 1 has been performed. The examination of all feedwater pipe weld areas inside containment and of welds connecting auxiliary feedwater piping to the main feedwater line outside containment required in Item 2 have also been performed. The inspection program as required at this time was completed July 8, 1979.

During the next refueling outage, a volumetric examination of the feedwater nozzle-to-pipe weld area for the "C" Steam Generator will again be performed. The re-examination of the feedwater nozzle-to-pipe weld area for all three steam generators will be performed at the second refueling outage as previously discussed in a meeting with the NRR on July 9, 1979.

- b. We have reviewed the BVPS operating and emergency procedures and determined that procedures that address a feedwater line break accident are adequate. If such a break were to occur, the operator would place the plant in a safe condition with residual heat of the core being dissipated in a controlled manner by means of other intact lines and the line with the leak being isolated. The procedures accomplish this in the following manner:
- 1) Verification of reactor trip and steam dump actuation for temperature control of the Reactor Coolant System.
  - 2) Verification of the delivery of flow from the auxiliary feedwater system to provide a heat sink for the Reactor Coolant System.
  - 3) Isolation of the main feedwater lines by means of the containment isolation valves and tripping of the main feedwater pumps to terminate the leak if it is upstream of the containment isolation valves.
  - 4) Monitoring auxiliary feedwater flows, steam generator levels, and steam generator pressures to determine if the leak is downstream of the containment isolation valves. If that is the case, auxiliary feedwater flow will be isolated to the faulted steam generator so that the leak itself is totally isolated.
  - 5) Commencing a plant cooldown using the intact lines if required to repair the break. A faulted line will not restrict the capability to cool the plant down in a controlled manner.

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- c. The containment at BVPS is the subatmospheric type. The subatmospheric containment is very sensitive to changes in moisture and vacuum. A leak from a feedwater line in containment will readily be detected by increasing moisture and pressure in containment. As a backup, containment temperatures and sump level indications will detect the addition of large quantities of high energy fluids into the containment itself.

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