

OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE August 9, 1979
 COMPLETED BY B. J. Hickie
 TELEPHONE (402) 536-4413

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: July, 1979
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5,087.0	51,264.0
12. Number Of Hours Reactor Was Critical	744	5,065.4	41,030.4
13. Reactor Reserve Shutdown Hours	0	0	1136.0
14. Hours Generator On-Line	744	5,050.6	40,116.7
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,045,094.0	6,974,567.3	48,622,990.6
17. Gross Electrical Energy Generated (MWH)	343,142.0	2,345,221.9	15,149,483.6
18. Net Electrical Energy Generated (MWH)	326,235.1	2,230,894.2	15,252,299.9
19. Unit Service Factor	100.0	99.3	78.3
20. Unit Availability Factor	100.0	99.3	78.3
21. Unit Capacity Factor (Using MDC Net)	95.9	96.0	65.7
22. Unit Capacity Factor (Using DER Net)	95.9	96.0	65.1
23. Unit Forced Outage Rate	0.0	0.7	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
January 1, 1980 - Refueling Outage - Two-month Duration

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

634 324

(9/77)

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Ft. Calhoun #1
DATE July 9, 1979
COMPLETED BY B. J. Hickie
TELEPHONE (402) 536-4413

MONTH July, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>439.5</u>
2	<u>439.1</u>
3	<u>437.2</u>
4	<u>436.0</u>
5	<u>438.7</u>
6	<u>441.0</u>
7	<u>444.8</u>
8	<u>444.7</u>
9	<u>443.9</u>
10	<u>441.6</u>
11	<u>438.8</u>
12	<u>437.2</u>
13	<u>436.8</u>
14	<u>437.3</u>
15	<u>436.8</u>
16	<u>437.1</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>437.3</u>
18	<u>438.7</u>
19	<u>438.6</u>
20	<u>438.4</u>
21	<u>438.5</u>
22	<u>438.5</u>
23	<u>438.0</u>
24	<u>437.5</u>
25	<u>436.4</u>
26	<u>436.8</u>
27	<u>434.6</u>
28	<u>435.9</u>
29	<u>437.2</u>
30	<u>437.4</u>
31	<u>438.3</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1979

DOCKET NO. 50-285
 UNIT NAME Ft. Calhoun #1
 DATE August 9, 1979
 COMPLETED BY B. J. HICKLE
 TELEPHONE (402) 536-4413

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
	None								

1 F: Forced
S: Scheduled

2 Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3 Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4 Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

5 Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending July 31, 1979.

1. Scheduled date for next refueling shutdown.
2. Scheduled date for restart following refueling.
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
 - a. If answer is yes, what, in general, will these be?

January 1, 1980

March 1, 1980

Yes

Stretching power to 1500 MWth is planned in conjunction with the change in fuel supplier to Exxon.

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.
 - c. If no such review has taken place, when is it scheduled?
4. Scheduled date(s) for submitting proposed licensing action and support information.
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Stretch Power Applicatio
•Site Related Information,
July, 1979 - Submitted
•Non-Core Related Informa-
tion, October, 1979
•Core Related Analysis and
Tech. Spec. Changes,
November, 1979

First use of Exxon fuel in Fort Calhoun.
Stretching power from 1420 MWth to 1500 MWth.

6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>157</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>483</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

<u>1985</u>

Prepared by R L Jaworski Date August 1, 1979

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

July 1979
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun operated near a nominal 100% power during the month of July. The fire protection projects continued on site.

Post Three Mile Island training continued as the shifts rotated through the training week.

A mutual fire drill was held with the Blair Fire Department. This drill demonstrated the ability of the Blair Fire Department pump truck to take suction from the Missouri River and pressurize the station fire header. This capability provides a defense in depth for providing fire suppression water at Fort Calhoun Station.

The annual nuclear emergency drill was held during July. The operations group at the plant responded to the alert and no significant deficiencies were noted.

Normal in service, surveillance and operational testing were performed.

The training of a hot license candidate group continued in preparation for the examination in late August.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
NONE	

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

The following is a surveillance test where an unusual result occurred:

<u>Surveillance Test No.</u>	<u>Description</u>
ST-RM-3	Not completed within specified period of time.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT
COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
ST-RPS-5	Excure Detector Symmetric Offset Calibration: Parameters within limits, no recalibration required.
PM-RM-1	Radioactive Source Surveillance of Licensed Sources not at the Fort Calhoun Station.
DCR 79-84	Install audible alarms on portal monitor in Security Building.
SP-FAUD-1	Fuel Assembly Uplift Condition Detection/all loops with available instrumentation show greater than 99% assurance.
M.O. 10699	Install PR's in New Fuel Storage Area RM-25A/ completed as designed.
SP-CONT-2	Containment Purge Supply and Exhaust Penetration (M-88 and M-87) Type C Leak Rate Test/completed per procedure.
SP-DI-2	Water Plant Recirculating Test/completed per procedure. Plant Engin-ers evaluating results.
SP-FAUD-1	Fuel Assembly Uplift Condition Detection/all loops with available instrumentation show greater than 99% assurance.
M.O. 983	VA-71A & B/completed as designed
M.O. 10200	Additional PR's in Room 71/completed as designed.
EEAR 79-115	Control Switch-Breaker 1A22/Completed as designed.
M.O. 8625	Additional PR's in Room 69/completed per design.
DCR 77-26	Auxiliary Building Fire Protection System - Diesel Generator Sprinkler Modification/completed as designed.
SP-RPS-5	Excure Detector Symmetric Off Set Recalibration/ completed per procedure.
DCR 78-58	Fire damper modification/completed as designed.
DCR 77-45	Fire Protection Water Line Tie-in for Security Building/completed as designed.

D. Continued

<u>Procedure</u>	<u>Description</u>
DCR 76A-13	Modification of Feedwater Heater Pipe Supports/ completed as designed.
EEAR 79-22	Security Diesel Battery Rack/completed per design.
EEAR 73-45	Bullet Nose Pressurization/completed per design.

E. RESULTS OF LEAK RATE TESTS

The six month 60 psig leak rate test was performed on the Personnel Air Lock. Results were within the .6 L_A limit.

F. CHANGES IN PLANT OPERATING STAFF

NONE

G. TRAINING

The training at Fort Calhoun Station consisted of requalification training and hot license training for the operators, Radiation Protection Training, First Aid Training and general system training for the crafts.

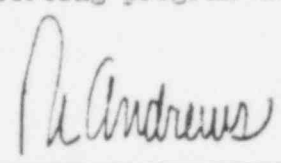
H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10 CFR 50.59.

Changes to Operating License DPR-40

Amendment No. 46 dated July 2, 1979

The amendment revised the Technical Specifications to incorporate surveillance requirements for in-service inspection of steam generator tubes and replaced existing in-service inspection and testing Technical Specifications with an in-service inspection and testing program that meets the requirements of 10CFR50.55A.

Approved by


Manager-Fort Calhoun Station

Monthly Operations Report

July 1979

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II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
1202	7-7-79	SIS-40 Anchor Bolts	Completed per procedure
1155	7-9-79	Stud Tensioning per Bulletin 79-02	Completed per procedure
1334	7-19-79	RPS-Channel "A" VOPT meter operating improperly	Completed per procedure
1421	7-25-79	MSS-10/Concrete expansion anchor located in south baseplate	Completed per M.O.
1426	7-25-79	SIS-27 Pipe support cotter pin missing on upper support	Completed per M.O.
1425	7-25-79	SIS-27A pipe support nut missing from bolt on support SIS-27A	Completed per M.O.