

INSTRUCTIONS FOR ADDING REVISION 3 TO THE  
REPORT ON THE REANALYSIS OF SAFETY-RELATED PIPING SYSTEMS

The following listing will serve as a checklist for entering the attached pages into the report. The items listed in the "REMOVE" column should be removed from the report and replaced with the items in the "ENTER" column. Vertical bars (change bars) have been placed in the outside margins of revised text pages and tables to show the location of any technical changes originating with this amendment. Some pages bear a new amendment designation, but no change bars, because revisions on other pages in that section caused a text shift. A few unrevised pages have been reprinted because they fall within a run of closely spaced revised pages. No change bars are used on figures or on new sections, appendices, questions and responses, etc. Change bars from previous amendments have been deleted on pages revised by this amendment. The instruction sheet and cover letter should be filed at the beginning of the report.

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BEAVER VALLEY POWER STATION, UNIT 1

REPORT ON THE  
REANALYSIS OF SAFETY-RELATED PIPING SYSTEMS

FOR

BEAVER VALLEY UNIT 1  
DUQUESNE LIGHT COMPANY

ORIGINAL - JUNE 15, 1979

REVISION 3 - JULY 27, 1979

Problem 270, the discharge from the CC heat exchangers has one support (H-56) out of 13 which has a local overstress in a lug of 78 percent. A new support is being added to the piping system adjacent to this support to correct the condition.

The fuel pool cooling and purification system is presently isolated since there is no spent fuel being stored.

The quench and recirculation spray systems have been completely analyzed for DBE and water hammer loads using NUPIPE and are acceptable. The OBE case will be run in the long term.

3. How have stress intensification factors been applied at branch connections during the reanalysis?

Response

Appropriate stress intensifications from B31.1 have been applied to the run pipe at reduced outlet branch connections. Branches which are uncoupled have been evaluated for the effects of the movements of the run

Field verified piping fabricator isometric drawings provide the basis for program inputs for the pipe stress reanalysis.

Additionally, in some cases, piping is analyzed utilizing amplified response spectra (ARS) that are developed using soil structure interaction techniques (SSI-ARS). The resultant stresses and loads are used to evaluate piping, supports, nozzles, and penetrations. These techniques are discussed in Section 8.7.

Of the 121 SHOCK 2 problems, 116 have been reanalyzed; 115 are within allowable stress values, and one is presently overstressed. Table 4-1 lists the problems including the peak stress values for the SHOCK 3 and NUPIPE pipe stress runs.

Stresses were computed by the SHOCK 3 or NUPIPE program using different mass models and in some cases different ARS than the original calculations. More importantly, the reanalyses were based on field-verified, as-built conditions which in some cases differ significantly from the original design conditions. For these reasons, the originally calculated stresses are not comparable to the new stresses.

The river water discharge line (Problem 122) in the turbine building has been found to have an overstress condition at two unreinforced branch connections

where a stress intensification factor of 10.45 is used. This line has been analyzed to this point because it provides the terminal anchor to end the problem.

The line is seismically supported and analyzed; however, it is located in a non-seismic building and the consequences of a break at these points would not affect any safety-related piping, components, or equipment and would not result in the loss of function of any safety-related system or equipment. There is no possibility that any flooding that might occur in the turbine building can affect any safety-related areas. Therefore, this condition is considered acceptable for the short term and the branch connections will be reinforced during the next refueling outage.

Table 4-2 summarizes the nozzles and penetrations evaluated under the reanalysis program. Of a total of 131 nozzles on problems within the scope of the interim effort, all have been evaluated and found to be acceptable.

The 14 nozzles in the quench and recirculation spray system have been accepted based on the DBE case only. The fuel pool cooling and purification system has 10 nozzles which have not been reviewed at this time.

The SHOCK 2 stress problems contained in the overall effort include 58 penetration connections, all of which have been evaluated and found to be acceptable.

Summary

During the period between the initial issue of this report and this revision, 40 additional problems have been rerun on NUPIPE using the SSI-ARS curve. All of the above 40 problems have been reanalyzed and were found to be within allowable stress limits.

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## BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-1

## PIPE STRESS REEVALUATION SUMMARY

<u>System and Problem No.</u>	<u>Allowable Stress (psi)</u>	<u>Reanalysis Maximum Stress</u>	<u>Reanalysis Method</u>
<u>Reactor Coolant</u>			
653A	<u>12,547</u> 18,820	<u>7,671</u> 8,189	NUPIPE/SSI-ARS
653B	<u>19,200</u> 28,800	<u>10,084</u> 10,575	NUPIPE/SSI-ARS
653C	<u>19,200</u> 28,800	<u>15,060</u> 17,244	NUPIPE/SSI-ARS
833 & 8	<u>17,220/19,200</u> <sup>(1)</sup> 25,830/28,200	<u>12,420</u> <sup>(2)</sup> 17,300	NUPIPE/SSI-ARS
1200	<u>19,200</u> 28,800	<u>12,690</u> 16,424	NUPIPE/SSI-ARS
1201	<u>19,200</u> 28,800	<u>9,711</u> 10,442	NUPIPE/SSI-ARS
<u>Safety Injection</u>			
391A	<u>19,080</u> 28,620	<u>15,425</u> 18,228	SHOCK 3/SSI-ARS
2112	<u>22,500</u> 33,750	<u>20,754</u> 25,002	SHOCK 3
610	<u>18,586</u> 27,878	<u>2,081</u> 2,328	NUPIPE/SSI-ARS
613	<u>21,180</u> 31,770	<u>9,802</u> 14,336	NUPIPE/SSI-ARS
615	<u>19,500/20,280</u> <sup>(1)</sup> 29,250/30,400	<u>17,585</u> 16,069	NUPIPE/SSI-ARS
15	<u>17,340/19,200</u> <sup>(1)</sup> 26,010/28,800	<u>7,214</u> 8,123	SHOCK 3/SSI-ARS

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BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-1 (Cont)

<u>System and Problem No.</u>	<u>Allowable Stress (psi)</u>	<u>Reanalysis Maximum Stress</u>	<u>Reanalysis Method</u>
1011	<u>20,850</u> 31,275	<u>8,245</u> 14,087	SHOCK 3/SSI-ARS
301	<u>19,200</u> 28,800	<u>7,722</u> 8,712	NUPIPE/SSI-ARS
213('')	<u>20,388</u> 30,582	<u>5</u> 7 5 7	NUPIPE/SSI-ARS
2113('')	<u>20,388</u> 30,582	<u>4,078</u> 4,443	NUPIPE/SSI-ARS
<u>Quench Spray</u>			
211	<u>22,500</u> 33,750	<u>1,807</u> 2,653	SHOCK 3/SSI-ARS
212	<u>22,500</u> 33,750	<u>10,445</u> 11,639	SHOCK 3
228	<u>22,500</u> 33,750	<u>12,149</u> 16,589	SHOCK 3
229	<u>22,500</u> 33,750	<u>11,810</u> 15,987	SHOCK 3
23	<u>(?)</u> 33,750	<u>(?)</u> 6,235	NUPIPE
134	<u>(?)</u> 33,750	<u>(?)</u> 17,667	NUPIPE
135	<u>(?)</u> 33,750	<u>(?)</u> 37,452('')	NUPIPE
136	<u>(?)</u> 33,750	<u>(?)</u> 25,435	NUPIPE
210	<u>(?)</u> 33,750	<u>(?)</u> 12,667	NUPIPE
218	<u>(?)</u> 33,750	<u>(?)</u> 6,973	NUPIPE



BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-1 (Cont)

<u>System and Problem No.</u>	<u>Allowable Stress (psi)</u>	<u>Reanalysis Maximum Stress</u>	<u>Reanalysis Method</u>
614	<u>( ? )</u> 33,750	<u>( ? )</u> 11,343	NUPIPE
617	<u>( ? )</u> 33,750	<u>( ? )</u> 14,271	NUPIPE
<u>Recirculation Spray</u>			
17	<u>( ? )</u> 25,560	<u>( ? )</u> 25,158	NUPIPE
18	<u>( ? )</u> 25,560	<u>( ? )</u> 16,316	NUPIPE
19	<u>( ? )</u> 25,560	<u>( ? )</u> 15,741	NUPIPE
20	<u>( ? )</u> 25,560	<u>( ? )</u> 21,895	NUPIPE
22	<u>( ? )</u> 25,560	<u>( ? )</u> 28,739 ( ? )	NUPIPE
24	<u>( ? )</u> 25,560	<u>( ? )</u> 16,298	NUPIPE
25	<u>( ? )</u> 25,560	<u>( ? )</u> 17,260	NUPIPE
27	<u>( ? )</u> 25,560	<u>( ? )</u> 21,415	NUPIPE
28	<u>( ? )</u> 25,560	<u>( ? )</u> 14,696	NUPIPE
29	<u>( ? )</u> 25,560	<u>( ? )</u> 12,991	NUPIPE
130	<u>( ? )</u> 25,560	<u>( ? )</u> 30,728 ( ? )	NUPIPE
131	<u>( ? )</u> 25,560	<u>( ? )</u> 29,952 ( ? )	NUPIPE

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BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-1 (Cont)

<u>System and Problem No.</u>	<u>Allowable Stress (psi)</u>	<u>Reanalysis Maximum Stress</u>	<u>Reanalysis Method</u>
132	(?) 25,560	(?) 17,403	NUPIPE
133	(?) 25,560	(?) 24,337	NUPIPE
611	(?) 28,192	(?) 8,210	NUPIPE
612	18,796 28,193	1,366 1,434	NUPIPE/SSI-ARS
<u>Charging and Volume Control</u>			
100	18,660 27,990	15,220 15,463	SHOCK 3
102	18,560 27,990	6,289 6,621	SHOCK 3/SSI-ARS
<u>Residual Heat Removal</u>			
255A	17,940 26,910	5,075 6,041	NUPIPE/SSI-ARS
256	17,160 25,740	11,843 15,063	NUPIPE/SSI-ARS
14	17,940/19,200(?) 26,910/28,800	8,740 10,376	SHOCK 3/SSI-ARS
3011	17,940 26,910	17,656 18,148	NUPIPE

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-1 (Cont)

<u>System and Problem No.</u>	<u>Allowable Stress (psi)</u>	<u>Reanalysis Maximum Stress</u>	<u>Reanalysis Method</u>
616	<u>18,300</u> 27,450	<u>8,498</u> 11,500	NUPIPE/SSI-ARS
Component Cooling Water			
302	<u>18,000</u> 27,000	<u>6,295</u> 10,271	NUPIPE/SSI-ARS
303	<u>18,000</u> 27,000	<u>6,906</u> 10,377	NUPIPE/SSI-ARS
304	<u>18,000</u> 27,000	<u>7,836</u> 11,108	NUPIPE/SSI-ARS
305	<u>18,000</u> 27,000	<u>5,835</u> 8,330	NUPIPE/SSI-ARS
306	<u>18,000</u> 27,000	<u>4,246</u> 5,077	NUPIPE/SSI-ARS
307	<u>18,000</u> 27,000	<u>6,133</u> 7,780	SHOCK 3/SSI-ARS
180E	<u>18,000</u> 27,000	<u>7,554</u> 7,370	NUPIPE/SSI-ARS
181E	<u>18,000</u> 27,000	<u>7,516</u> 15,168	NUPIPE/SSI-ARS
170C	<u>18,000</u> 27,000	<u>4,539</u> 5,031	NUPIPE/SSI-ARS
171	<u>18,000</u> 27,000	<u>5,472</u> 5,667	NUPIPE/SSI-ARS
172	<u>18,000</u> 27,000	<u>8,319</u> 13,734	NUPIPE/SSI-ARS
173D	<u>18,000</u> 27,000	<u>5,107</u> 5,994	NUPIPE/SSI-ARS
174D	<u>18,000</u> 27,000	<u>3,036</u> 4,115	NUPIPE/SSI-ARS

## BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-1 (Cont)

<u>System and Problem No.</u>	<u>Allowable Stress (psi)</u>	<u>Reanalysis Maximum Stress</u>	<u>Reanalysis Method</u>
175B	<u>18,000</u> 27,000	<u>5,197</u> 5,414	NUPIPE/SSI-ARS
176A	<u>18,000</u> 27,000	<u>3,505</u> 3,777	SHOCK 3/SSI-ARS
177	<u>18,000</u> 27,000	<u>2,223</u> 13,707	SHOCK 3/SSI-ARS
178C	<u>18,000</u> 27,000	<u>15,703</u> 15,797	NUPIPE/SSI-ARS
179	<u>18,000</u> 27,000	<u>2,081</u> 2,995	NUPIPE/SSI-ARS
183	<u>18,000</u> 27,000	<u>9,320</u> 11,336	NUPIPE/SSI-ARS
184	<u>18,000</u> 27,000	<u>10,173</u> 11,734	NUPIPE/SSI-ARS
186A	<u>18,000</u> 27,000	<u>15,703</u> 15,797	NUPIPE/SSI-ARS
270A	<u>18,000</u> 27,000	<u>15,703</u> 15,797	NUPIPE/SSI-ARS
215	<u>18,000</u> 27,000	<u>16,311</u> 26,731	SHOCK 3
217	<u>18,000</u> 27,000	<u>15,751<sup>(2)</sup></u> 23,924	NUPIPE/SSI-ARS
930	<u>18,000</u> 27,000	<u>15,751</u> 23,924	NUPIPE/SSI-ARS
931	<u>18,000</u> 27,000	<u>15,751</u> 23,924	NUPIPE/SSI-ARS
214	<u>18,000</u> 27,000	<u>14,740</u> 25,774	NUPIPE/SSI-ARS

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-1 (Cont)

<u>System and Problem No.</u>	<u>Allowable Stress (psi)</u>	<u>Reanalysis Maximum Stress</u>	<u>Reanalysis Method</u>
<u>River Water</u>			
1	<u>18,000</u> 27,000	<u>10,801</u> 13,759	SHOCK 3
30	<u>18,000</u> 27,000	<u>4,830</u> 7,576	NUPIPE/SSI-ARS
31	<u>18,000</u> 27,000	<u>4,830</u> 7,576	NUPIPE/SSI-ARS
32	<u>18,000</u> 27,000	<u>5,363</u> 8,390	NUPIPE/SSI-ARS
33	<u>18,000</u> 27,000	<u>5,169</u> 8,241	NUPIPE/SSI-ARS
140	<u>18,000</u> 27,000	<u>13,758</u> 16,349	SHOCK 3/SSI-ARS
384	<u>18,000</u> 27,000	<u>6,156</u> 8,512	SHOCK 3(')
157	<u>18,000</u> 27,000	<u>1,884</u> 2,011	NUPIPE/SSI-ARS
158	<u>18,000</u> 27,000	<u>1,976</u> 2,090	NUPIPE/SSI-ARS
159(')	<u>18,000</u> 27,000	<u>10,443</u> 17,277	NUPIPE/SSI-ARS
128	<u>18,000</u> 27,000	<u>10,760</u> 12,562	NUPIPE/SSI-ARS
127	<u>18,000</u> 27,000	<u>13,384</u> 15,970	NUPIPE/SSI-ARS
125	<u>18,000</u> 27,000	<u>10,760</u> 12,562	NUPIPE/SSI-ARS
124	<u>18,000</u> 27,000	<u>13,384</u> 15,970	NUPIPE/SSI-ARS

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-1 (Cont)

<u>System and Problem No.</u>	<u>Allowable Stress (psi)</u>	<u>Reanalysis Maximum Stress</u>	<u>Reanalysis Method</u>
123	<u>18,000</u> 27,000	<u>10,861</u> 17,797	NUPIPE/SSI-ARS
120	<u>18,000</u> 27,000	<u>(?)</u> 16,471	NUPIPE/SSI-ARS
126	<u>18,000</u> 27,000	<u>13,384</u> 15,970	NUPIPE/SSI-ARS
216	<u>18,000</u> 27,000	<u>6,047</u> 9,989	NUPIPE/SSI-ARS
203	<u>18,000</u> 27,000	<u>2,444</u> 4,189	NUPIPE/SSI-ARS
2031	<u>18,000</u> 27,000	<u>8,260</u> 9,699	NUPIPE/SSI-ARS
152	<u>18,000</u> 27,000	<u>4,950</u> 6,032	NUPIPE/SSI-ARS
121	<u>18,000</u> 27,000	<u>6,068</u> 8,354	NUPIPE/SSI-ARS
122 (•)	<u>18,000</u> 27,000	<u>43,256</u> 62,608	NUPIPE/SSI-ARS
165	<u>18,000</u> 27,000	<u>4,950</u> 6,032	NUPIPE/SSI-ARS
652	<u>18,000</u> 27,000	<u>1,231</u> 1,394	NUPIPE/SSI-ARS
653	<u>18,000</u> 27,000	<u>1,495</u> 1,624	NUPIPE/SSI-ARS
<u>Main Steam</u>			
658	<u>22,500</u> 33,750	<u>10,248</u> 12,025	SHOCK 3/SSI-ARS
6590	<u>18,000</u> 27,000	<u>9,977</u> 11,108	SHOCK 3/SSI-ARS

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-1 (Cont)

<u>System and Problem No.</u>	<u>Allowable Stress (psi)</u>	<u>Reanalysis Maximum Stress</u>	<u>Reanalysis Method</u>
101	<u>18,000</u> 27,000	<u>16,917</u> 18,277	SHOCK 3
659	<u>22,500</u> 33,750	<u>10,544</u> 12,570	SHOCK 3/SSI-ARS
660	<u>22,500</u> 33,750	<u>11,121</u> 13,304	SHOCK 3/SSI-ARS
3063	<u>22,500</u> 33,750	<u>12,289</u> 16,481	SHOCK 3
<u>Feed Water</u>			
204	<u>18,000</u> 27,000	<u>2,952</u> 3,761	SHOCK 3
783	<u>18,000</u> 27,000	<u>9,361</u> 11,624	SHOCK 3/SSI-ARS
784	<u>18,000</u> 27,000	<u>10,853</u> 13,726	SHOCK 3/SSI-ARS
785	<u>18,000</u> 27,000	<u>14,397</u> 20,232	SHOCK 3
261	<u>18,000</u> 27,000	<u>10,479</u> 13,585	SHOCK 3/SSI-ARS
<u>Diesel Generator Exhaust</u>			
651	<u>12,960</u> 19,440	<u>1,201</u> 1,717	NUPIPE/SSI-ARS

Notes: SSI-ARS = Amplified response spectra developed using soils  
structure interaction techniques

Stresses shown are Operational Basis Earthquake (OBE) Stresses  
Design Basis Earthquake (DBE) Stresses

(( TP304/TP316 allowables

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-1 (Cont)

<u>System and Problem No.</u>	<u>Allowable Stress (psi)</u>	<u>Reanalysis Maximum Stress</u>	<u>Reanalysis Method</u>
(2) After modification			
(3) Are acceptable based on water hammer and OBE analysis using 2.4 . For the long term, this will be acceptable based on 1.8 .			
			SH
(4) Problems 213 and 2113 include S + S + S and S + S + S only.			
		DL LP OBEI	DL LP DBEI
(5) Being rerun with SSI-ARS.			
(6) Problem 159 includes Problems 160 and 161.			
(7) Evaluated for the DBE case only (S + S + S ).			
		DL LP DBEI	
(8) Overstress condition occurs in turbine building; considered acceptable for short term.			



BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-2

NOZZLE AND PENETRATION SUMMARY

<u>System/ Problem No.</u>	<u>Total No. of Nozzles/ Penetrations</u>	<u>No. Acceptable After Pipe Stress Re- analysis</u>	<u>No. Requiring Further Re- Analysis</u>
<u>Reactor Coolant</u>			-
653A	6/0	6/0	0/0
653B	8/0	8/0	0/0
653C	8/0	8/0	0/0
833 & 8	4/0	4/0	0/0
1200	1/0	1/0	0/0
1201	0/0	0/0	0/0
<u>Safety Injection</u>			
391A	1/0	1/0	0/0
2112	0/0	0/0	0/0
610	2/2	2/2	0/0
613	0/0	0/0	0/0
615	2/3	2/3	0/0
15	1/0	1/0	0/0
1011	0/0	0/0	0/0
301	0/2	0/2	0/0
213	0/0	0/0	0/0
2113	0/0	0/0	0/0

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BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-2 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Nozzles/ Penetrations</u>	<u>No. Acceptable After Pipe Stress Re- analysis</u>	<u>No. Requiring Further Re- Analysis</u>
<u>Quench Spray</u>			
211	1/0	1/0	0/0
212	1/0	1/0	0/0
228	1/0	1/0	0/0
229	1/0	1/	0/0
23	0/0	0/0	0/0
134	0/0	0/0	0/0
135	0/1	0/1	0/0
136	0/1	0/1	0/0
210	1/0	1/0	0/0
218	1/0	1/0	0/0
614	0/1	0/1	0/0
617	0/1	0/1	0/0
<u>Recirculation Spray</u>			
17	2/0	2/0	0/0
18	2/0	2/0	0/0
19	2/0	2/0	0/0
20	2/0	2/0	0/0
22	0/0	0/0	0/0
24	0/0	0/0	0/0
25	0/0	0/0	0/0
27	1/0	1/0	0/0

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-2 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Nozzles/ Penetrations</u>	<u>No. Acceptable After Pipe Stress Re- analysis</u>	<u>No. Requiring Further Re- Analysis</u>
28	1/0	1/0	0/0
29	0/0	0/0	0/0
130	0/1	0/1	0/0
131	0/1	0/1	0/0
132	0/0	0/0	0/0
133	0/0	0/0	0/0
611	2/2	2/2	0/0
612	2/2	2/2	0/0
<u>Charging &amp; Volume Control</u>			
100	2/0	2/0	0/0
102	1/0	1/0	0/0
<u>Residual Heat Removal</u>			
255A	6/0	6/0	0/0
256	0/0	0/0	0/0
14	1/0	1/0	0/0
3011	0/0	0/0	0/0
616	0/1	0/1	0/0
<u>Component Cooling Water</u>			
302	1/1	1/1	0/0
303	1/1	1/1	0/0

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## BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-2 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Nozzles/ Penetrations</u>	<u>No. Acceptable After Pipe Stress Re- analysis</u>	<u>No. Requiring Further Re- Analysis</u>
304	1/1	1/1	0/0
305	1/1	1/1	0/0
306	0/1	0/1	0/0
307	0/1	0/1	0/0
180E	2/0	2/0	0/0
181E	2/0	2/0	0/0
170C	3/0	3/0	0/0
171	6/0	6/0	0/0
172	0/0	0/0	0/0
173D	1/0	1/0	0/0
174D	1/0	1/0	0/0
175B	0/0	0/0	0/0
176A	0/0	0/0	0/0
177	1/0	1/0	0/0
178C	1/0	1/0	0/0
179	1/0	1/0	0/0
183	3/0	3/0	0/0
184	2/0	2/0	0/0
186A	0/0	0/0	0/0
270A	3/0	3/0	0/0
215	0/4	0/4	0/0
217	0/4	0/4	0/0

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-2 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Nozzles/ Penetrations</u>	<u>No. Acceptable After Pipe Stress Re- analysis</u>	<u>No. Requiring Further Re- Analysis</u>
930	0/1	0/1	0/0
931	0/1	0/1	0/0
214	0/1	0/1	0/0
<u>River Water</u>			
1	4/4	4/4	0/0
30	1/1	1/1	0/0
31	1/1	1/1	0/0
32	1/1	1/1	0/0
33	1/1	1/1	0/0
140	1/0	1/0	0/0
384	1/0	1/0	0/0
157	0/0	0/0	0/0
158	0/0	0/0	0/0
159	3/0	3/0	0/0
128	0/0	0/0	0/0
127	0/0	0/0	0/0
125	0/0	0/0	0/0
124	0/0	0/0	0/0
123	0/4	0/4	0/0
120	0/4	0/4	0/0
126	3/0	3/0	0/0
216	1/1	1/1	0/0

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## BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-2 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Nozzles/ Penetrations</u>	<u>No. Acceptable After Pipe Stress Re- analysis</u>	<u>No. Requiring Further Re- Analysis</u>
203	3/0	3/0	0/0
2031	0/0	0/0	0/0
152	2/0	2/0	0/0
121	3/0	3/0	0/0
122	0/0	0/0	0/0
165	0/0	0/0	0/0
652	2/0	2/0	0/0
653	2/0	2/0	0/0
<u>Main Steam</u>			
658	1/1	1/1	0/0
6590	0/0	0/0	0/0
101	0/0	0/0	0/0
659	1/1	1/1	0/0
660	1/1	1/1	0/0
3063	0/0	0/0	0/0
<u>Feed- water</u>			
204	3/0	3/0	0/0
783	1/1	1/1	0/0
784	1/1	1/1	0/0
785	1/1	1/1	0/0
261	0/0	0/0	0/0

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 4-2 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Nozzles/ Penetrations</u>	<u>No. Acceptable After Pipe Stress Re- analysis</u>	<u>No. Requiring Further Re- Analysis</u>
Diesel Generator <u>Exhaust</u>			
651	0/0	0/0	0/0

## SECTION 5

## PIPE SUPPORT RESULTS

Table 5-1 summarizes the pipe supports evaluated in the reanalysis program. There are 1060 pipe supports on lines within the interim reanalysis effort; of these, 667 have been evaluated and found acceptable and 7 have been modified to be acceptable. A support is considered acceptable if all the load components are lower in magnitude than those for which the support was originally designed. If some load components are greater than the original design load components, the support is reanalyzed using the new loads. Of the total 386 supports requiring reanalysis, 383 have been found to be acceptable based on DBEI+DL, 3 have not been accepted at this time. There is sufficient analytical information available for the remaining 3 supports to exercise engineering judgment in determining whether the unacceptable condition will become acceptable.

1. The use of one time load for snubbers
2. Use of DBEI plus dead load

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If a support is unacceptable using any of the above approaches, a modification is required. Table 5-2 identifies those supports where acceptance is based on the future use of the options listed above. Hardware modifications and additions are discussed in Section 6.

Support designs which are not in accordance with either of these criteria will be suitably modified against the acceptance design criteria of Table 3-1 prior to interim plant operation.

Base plate design criteria and anchor bolt pullout and shear allowable loads are addressed in Section 3. The seismic support loadings which will be utilized for the NF evaluation will be the result of either SHOCK3 or NUPIPE evaluations using SSI-ARS.

#### Summary

The pipe support reanalysis effort which took place between the original issue of this report and this revision includes accepting 377 supports; 190 based on DBEI+DL and 187 based on long-term criteria. Also, one additional modification was necessary for the 14" RHR line off the reactor coolant loop and one additional support is required on the 24 inch header from the component cooling heat exchangers.

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 5-1  
PIPE SUPPORTS SUMMARY

<u>System/ Problem No.</u>	<u>Total No. of Supports</u>	<u>No. Presently Acceptable Based on Reanalysis</u>	<u>No. Acceptable for Interim Operation</u>	<u>Modifications or Additions Required</u>
<u>Reactor Coolant</u>				
653A	2	2	0	0
653B	16	12	0	4
653C	8	8	0	0
833&8	15	15	0	1
1200	18	15	3	0
1201	19	19	0	0
<u>Safety Injection</u>				
391A	11	11	0	0
2112	8	8	0	0
610	2	2	0	0
613	5	5	0	0
615(2)	11	6	5	0
15	11	8	3	0
1011	19	19	0	0
301(8)	56	0	56	0
213	16	0	16	0
2113	16	0	16	0

## BEAVER VALLEY POWER STATION, UNIT 1

TABLE 5-1 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Supports</u>	<u>No. Presently Acceptable Based on Reanalysis</u>	<u>No. Acceptable for Interim Operation</u>	<u>Modifications or Additions Required</u>
<u>Quench Spray</u>				
211	5	5	0	0
212	3	3	0	0
228	0	0	0	0
229	0	0	0	0
23	12	0	12	0
134	5	0	5	0
135	9	0	9	0
136	7	0	7	0
210	12	0	12	0
218	9	0	9	0
614	3	0	3	0
617	9	0	9	0
<u>Recirculation Spray</u>				
17	3	0	3	0
18	2	0	2	0
19	4	0	4	0
20	4	0	4	0
22	9	0	9	0
24	9	0	9	0
25	9	0	9	0
27	12	0	12	0

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 5-1 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Supports</u>	<u>No. Presently Acceptable Based on Reanalysis</u>	<u>No. Acceptable for Interim Operation</u>	<u>Modifications or Additions Required</u>
28	6	0	6	0
29	22	0	22	0
130	2	0	2	0
131	2	0	2	0
132	2	0	2	0
133	35	0	35	0
611	6	0	6	0
612	0	0	0	0
<u>Charging Volume Control</u>				
100	9	9	0	0
102	8	8	0	0
<u>Residual Heat Removal</u>				
255A	8	4	4	0
256	6	6	0	0
14	15	15	0	0
3011	11	10	1	0
616	7	0	7	0
<u>Component Cooling Water</u>				
302	23	23	0	0
303	23	23	0	0
304	33	33	0	0

## BEAVER VALLEY POWER STATION, UNIT 1

TABLE 5-1 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Supports</u>	<u>No. Presently Acceptable Based on Reanalysis</u>	<u>No. Acceptable for Interim Operation</u>	<u>Modifications or Additions Required</u>
305	30	30	0	0
306	11	10	1	0
307	10	10	0	0
180E	5	5	0	0
181E	5	4	1	0
170C	17	16	1	0
171	15	11	4	0
172	13	12	1	0
173D	15	14	1	0
174D	20	16	4	0
175B	6	5	1	0
176A	5	5	0	0
177	9	9	0	0
178C	14	10	4	0
179	8	8	0	0
183	9	8	1	0
184	14	11	3	0
186A	6	3	3	0
270A	10	6	4	1
215	8	8	0	0
217	10	10	0	1
930	3	3	0	0

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 5-1 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Supports</u>	<u>No. Presently Acceptable Based on Reanalysis</u>	<u>No. Acceptable for Interim Operation</u>	<u>Modifications or Additions Required</u>
931	2	2	0	0
214	5	5	0	0
<u>River Water</u>				
1	0	0	0	0
30	2	2	0	0
31	2	2	0	0
32	2	1	1	0
33	2	2	0	0
140	2	2	0	0
384	5	5	0	0
157	3	3	0	0
158	2	2	0	0
159(*)	8	8	0	0
128	1	0	1	0
127	10	6	4	0
125	12	12	0	0
124	13	13	0	0
123	15	12	0	3
120	11	5	6	0
126	7	7	0	0
216	2	2	0	0
203	16	15	1	0

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BEAVER VALLEY POWER STATION, UNIT 1

TABLE 5-1 (Cont)

<u>System/ Problem No.</u>	<u>Total No. of Supports</u>	<u>No. Presently Acceptable Based on Reanalysis</u>	<u>No. Acceptable for Interim Operation</u>	<u>Modifications or Additions Required</u>
2031	9	9	0	0
121	15	0	15	0
122	19	0	19	0
165	1	0	1	0
152	8	8	0	0
652(')	1	0	1	0
653(')	2	0	2	0
<u>Main Steam</u>				
658	6	6	0	0
6590	3	3	0	0
101	4	4	0	0
659	2	2	0	0
660	7	7	0	0
3063	0	0	0	0
<u>Feedwater</u>				
204	15	15	0	0
783	9	9	0	0
784	6	6	0	0
785	3	3	0	0
261	6	6	0	0
<u>Diesel Generator Exhaust</u>				
651	2	0	2	0

BEAVER VALLEY POWER STATION, UNIT 1

TABLE 5-1 (Cont)

NOTES:

' ' Deleted.

' ' Problem 615 contains modifications (NPSH) scheduled for installation during the first refueling outage. Therefore, it will only be analyzed for interim operation.

' ' Problem 159 includes Problems 160 and 161.

' ' Deleted.

' ' Analyzed based on DBEI+DL only.



BEAVER VALLEY POWER STATION, UNIT 1

TABLE 5-2

ENGINEERING EVALUATION OF REMAINING SUPPORTS

<u>Problem</u> <u>No.</u>	<u>Support</u> <u>No.</u>	<u>Overstress Condition</u>	<u>Resolution</u>
<u>RIVER WATER SYSTEM</u>			
120	H307	Snubber Overloaded	Will be Acceptable Based on DBEI+DL
	R6	Vertical Support Installed/ Required Double Acting	Will be Acceptable Based on DBEI+DL
	H306	Member/Weld Overstress/ Bolt Pullout	Will be Acceptable Based on DBEI+DL

Following reanalysis of Problem No. 833, an additional snubber was designed and will be installed to alleviate a pipe overstress occurring under upset (OBE) and faulted (DBE) conditions.

Similarly, an additional snubber was designed and will be installed in Problem No. 217 to alleviate a pipe overstress occurring under the same conditions.

Three supports in Problem No. 123 will be modified, one to make the as-built condition agree with the original design, one to strengthen a marginal original design, and one to alleviate an overstressed weld in the support resulting from seismic uplift forces.

Similarly, four supports in Problem No. 653B will be modified, three to make the as-built condition agree with the original design, and one to alleviate an overstressed member in the support resulting from seismic forces.

A new support is being added adjacent to an existing support in Problem No. 270 in order to relieve a local overstress in a lug.

BEAVER VALLEY POWER STATION, UNIT 1

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>	<u>FSAR Fig. No.</u>
Quench Spray (QS)	6"-SI-29-1502-Q1		
	6"-SI-20-1502-Q1		
	6"-SI-19-1502-Q1		
	6"-SI-32-1502-Q1		
	6"-SI-33-1502-Q1		
	6"-SI-40-153W-Q2	2113	6.3-1
	6"-SI-44-153W-Q2	213	6.3-1
	12"-QS-2-153B-Q3	211	6.4-1
	12"-QS-1-153B-Q3	212	6.4-1
	12"-QS-1-153B-Q3	228	6.4-1
	12"-QS-2-153B-Q3	229	6.4-1
	10"-QS-4-153B-Q3	614	6.4-1
	10"-QS-3-153B-Q3	617	6.4-1
	4"-QS-6-153B-Q3	210	6.4-1
	10"-QS-4-153B-Q3		
	4"-QS-5-153B-Q3	218	6.4-1
	10"-QS-3-153B-Q3		
	10"-QS-3-153B-Q3	134	6.4-1
	10"-QS-3-153B-Q3	135	6.4-1
	10"-QS-4-153B-Q3	31	6.4-1
	10"-QS-3-153B-Q3	23	6.4-1
	10"-QS-4-153B-Q3		
	8"-QS-22-153B-Q3		
	8"-QS-23-153B-Q3		
Recirculation Spray (RS)	12"-RS-7-153A-Q2	612	6.4-1
	12"-RS-8-153A-Q2		
	12"-RS-5-153A-Q2		
	10"-RS-9-153B-Q2	611	6.4-1
	4"-RS-14-153B-Q2		
	10"-RS-10-153B-Q2		
	4"-RS-15-153B-Q2		

BEAVER VALLEY POWER STATION, UNIT 1

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>	<u>FSAR Fig. No.</u>
	10"-RS-1-153B-Q2	27	6.4-1
	4"-RS-28-153B		
	4"-RS-27-153B		
	10"-RS-1-153B-Q2	20	6.4-1
	12"-RS-3-153B-Q2		
	10"-RS-2-153B-Q2	28	6.4-1
	10"-RS-2-153B-Q2	18	6.4-1
	12"-RS-4-153B-Q2		
	10"-RS-25-153B-Q2	131	6.4-1
	10"-RS-26-153B-Q2	130	6.4-1
	10"-RS-25-153B-Q2	19	6.4-1
	12"-RS-11-153B-Q2		
	10"-RS-26-153B-Q2	17	6.4-1
	12"-RS-12-153B-Q2		
	12"-RS-3-153B-Q2	133	6.4-1
	12"-RS-4-153B-Q2	25	6.4-1
	12"-RS-11-153B-Q2	24	6.4-1
	12"-RS-12-153B-Q2	29	6.4-1
	12"-RS-12-153B-Q2	132	6.4-1
	8"-RS-21-153B-Q2	22	6.4-1
	8"-RS-22-153B-Q2		
	8"-RS-23-153B-Q2		
	8"-RS-24-153B-Q2		
Charging and Volume Control (CH)	6"-CH-63-153W-Q2	100	9.1-1
	6"-CH-67-153W-Q2		
	8"-CH-15-153W-Q2		
	8"-CH-15-153W-Q2	102	
	6"-CH-68-153W-Q2		
Residual Heat Removal (RH)	12"-RH-6-602-Q2	255A	9.3-1
	12"-RH-9-602-Q2		
	12"-RH-12-602-Q2		
	10"-RH-4-602-Q2		
	10"-RH-5-602-Q2		

BEAVER VALLEY POWER STATION, UNIT 1

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>	<u>FSAR Fig. No.</u>
	10"-RH-7-602-Q2		
	10"-RH-8-602-Q2		
	10"-RH-10-602-Q2		
	10"-RH-19-602-Q2		
	12"-RH-9-602-Q2	256	9.3-1
	12"-RH-12-602-Q2		
	10"-RH-16-602-Q2		
	10"-RH-17-602-Q2		
	6"-RH-20-602-Q2		
	3"-RH-13-602-Q2		
	10"-RH-16-602-Q2	3011	9.3-1
	14"-RH-1-1502-Q1	653B	4-1, 9.3-1
	14"-RH-2-602-Q2		
	14"-RH-18-602-Q2		
	10"-RH-24-1502-Q1	653C	6.3-2
	10"-RH-23-1502-Q2	14	9.3-1, 6.3-2
	6"-RH-14-152-Q2	616	9.3-1
Component Cooling	18"-CC-118-151-Q3	302	9.4-4
	18"-CC-116-151-Q3	303	9.4-4
	18"-CC-114-151-Q3	304	9.4-4
	18"-CC-130-151-Q3	305	9.4-4
	8"-CC-255-151-Q3	306	9.4-3
	8"-CC-256-151-Q3		
	8"-CC-257-151-Q3		
	6"-CC-261-151-Q3		
	8"-CC-476-151-Q3		
	6"-CC-258-151-Q3	307	9.4-3
	6"-CC-265-151-Q3		
	8"-CC-259-151-Q3		
	8"-CC-260-151-Q3		
	8"-CC-517-151-Q3		
	6"-CC-519-151-Q3	215	9.4-4
	24"-CC-125-151-Q3		
	18"-CC-489-151-Q2		
	18"-CC-490-151-Q2		

BEAVER VALLEY POWER STATION, UNIT 1

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>	<u>FSAR Fig. No.</u>
	18"-CC-529-151-Q3		
	18"-CC-530-151-Q3		
	6"-CC-488-151-Q2		
	6"-CC-526-151-Q3		
	4"-CC-487-151-Q3		
	4"-CC-525-151-Q2		
	3"-CC-486-151-Q3		
	3"-CC-523-151-Q2		
	2"-CC-485-151-Q2		
	2"-CC-524-151-Q3		
	24"-CC-266-151-Q3		
	6"-CC-518-151-Q3		
	24"-CC-112-151-Q3	217	9.4-3,
	24"-CC-113-151-Q3		9.4-4
	6"-CC-510-151-Q3		9.4-3,
	6"-CC-511-151-Q3		9.4-4
	6"-CC-512-151-Q3		
	6"-CC-482-151-Q3		
	18"-CC-483-151-Q3		
	18"-CC-484-151-Q3		
	18"-CC-527		
	18"-CC-528-151-Q3		
	8"-CC-517-151-Q2	214	9.4-3
	6"-CC-481-151-Q2	930	9.4-4
	6"-CC-511-151-Q3		
	6"-CC-480-151-Q2	931	9.4-4
	6"-CC-510-151-Q3		
	10"-CC-20-151-Q3	172	9.4-1, 9.4-2
	8"-CC-81-151	173D	
	8"-CC-469-151-Q3		
	6"-CC-82-151		
	8"-CC-65-151	174D	
	8"-CC-468-151-Q3		
	6"-CC-55-151		
	8"-CC-58-151-Q3	175B	9.4-1, 9.4-2

BEAVER VALLEY POWER STATION, UNIT 1

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>	<u>FSAR Fig. No.</u>
	10"-CC-320-151-Q3 8"-CC-323-151-Q3 6"-CC-470-151-Q3 6"-CC-83-151	176A	9.4-1
	12"-CC-46-151-Q3 8"-CC-61-151 8"-CC-465-151-Q3 6"-CC-65-151 6"-CC-92-151-Q3	177	9.4-3
	14"-CC-45-151-Q3 12"-CC-46-151-Q3 10"-CC-48-151-Q3 8"-CC-8-151-Q3 8"-CC-51-151-Q3 8"-CC-466-151-Q3 8"-CC-80-151 6"-CC-84-151	178C	9.4-1, 9.4-2
	6"-CC-23-151 4"-CC-26-151	179	9.4-2
	8"-CC-51-151-Q3 6"-CC-53-151-Q3 6"-CC-56-151-Q3	180E	9.4-2
	8"-CC-58-151-Q3 6"-CC-54-151-Q3 6"-CC-60-151-Q3	181E	9.4-2
	8"-CC-19-151-Q3 6"-CC-34-151 6"-CC-42-141-Q3 6"-CC-515-151-Q3 3"-CC-38-151-Q3 4"-CC-27-151	183	9.4-2
	8"-CC-35-151-Q3 6"-CC-41-151-Q3 3"-CC-37-151-Q3	184	9.4-2
	8"-CC-6-151-Q3 8"-CC-8-151-Q3 6"-CC-513-151-Q3	186A	9.4-1, 9.4-2

BEAVER VALLEY POWER STATION, UNIT 1

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>	<u>FSAR Fig. No.</u>
	24"-CC-5-151-Q3	270	9.4-1
	24"-CC-112-151-Q3		
	18"-CC-4-151-Q3		
	18"-CC-105-151-Q3		
	18"-CC-109-151-Q3		
	24"-CC-22-151-Q3	170C	9.4-1
	24"-CC-266-151-Q3		
	18"-CC-101-151-Q3		
	18"-CC-102-151-Q3		
	18"-CC-106-151-Q3		
	8"-CC-469-151-Q3		
	24"-CC-2-151-Q3	171	9.4-1
	18"-CC-1-151-Q3		
	18"-CC-3-151-Q3		
	18"-CC-103-151-Q3		
	18"-CC-104-151-Q3		
	18"-CC-107-151-Q3		
	18"-CC-108-151-Q3		
	8"-CC-321-151-Q3		
	10"-CC-321-151-Q3		
Chilled Water (CW)	8"-CW-8-151	216	9.4-3
	8"-CW-9-151	214	9.4-3
River Water (WR)	6"-WR-117-151-Q3	203	10.3-5
	14"-WR-64-151-Q2	30	9.9-1A
	14"-WR-82-151-Q2	31	9.9-1A
	14"-WR-89-151-Q2	32	9.9-1A
	14"-WR-87-151-Q2	33	9.9-1A
	8"-WR-228-151-Q3	140	9.9-1A
	8"-WR-229-151-Q3		
	8"-WR-230-151		
	8"-WR-231-151		
	8"-WR-234-151-Q3	214	9.4-3



BEAVER VALLEY POWER STATION, UNIT 1

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>	<u>FSAR Fig. No.</u>
	14"-WR-63-151-Q2 14"-WR-65-151-Q2 14"-WR-86-151-Q2 14"-WR-88-151-Q2	1	9.9-1A
	14"-WR-25-151-Q3 14"-WR-26-151-Q3 14"-WR-27-151-Q3 14"-WR-28-151-Q3 24"-WR-29-151-Q3	120	9.9-1A
	14"-WR-21-151-Q3 14"-WR-22-151-Q3 14"-WR-23-151-Q3 14"-WR-24-151-Q3 24"-WR-19-151-Q3 24"-WR-20-151-Q3	123	9.9-1A
	24"-WR-19-151-Q3 24"-WR-187-151-Q3	124	9.9-1A
	24"-WR-20-151-Q3 24"-WR-186-151-Q3	125	9.9-1A
	24"-WR-7-151-Q3 24"-WR-8-151-Q3 24"-WR-9-151-Q3 18"-WR-11-151-Q3 18"-WR-12-151-Q3 18"-WR-13-151-Q3	126	9.9-1A
	24"-WR-19-151-Q3	127	9.9-1A
	24"-WR-20-151-Q3	128	9.9-1A
	24"-WR-99-151-Q3	157	9.9-1A
	24"-WR-100-151-Q3	158	9.9-1A,B
	20"-WR-1-151-Q3 20"-WR-2-151-Q3 20"-WR-3-151-Q3 20"-WR-4-151-Q3 20"-WR-5-151-Q3 20"-WR-6-151-Q3 24"-WR-99-151-Q3 24"-WR-100-151-Q3 18"-WR-154-151-Q3 12"-WR-177-151-Q3	159	9.9-1A,B

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<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>	<u>FSAR Fig. No.</u>
	8"-WR-227-151	216	9.4-3, 9.9-1A
	30"-WR-171-151-Q3	152	9.9-1B
	30"-WR-172-151-Q3		
	30"-WR-175-151-Q3		
	10"-SWW-14-151-Q3	165(2)	9.9-1B
	10"-SWW-1-121		
	18"-WR-14-151-Q3	121	9.9-1A
	18"-WR-15-151-Q3		
	18"-WR-16-151-Q3		
	30"-WR-17-151-Q3	122	9.9-1A
	6"-WR-155-151-Q3	384	9.9-1B
	6"-WR-214-151-Q3	652	RM-53A
	6"-WR-215-151-Q3	653	RM-53A
Main Steam (MS)	3"-SDHV-1-601-Q2	101	10.3-1
	3"-SDHV-2-601-Q2		
	3"-SDHV-3-601-Q2		
	4"-SDHV-4-601-Q2		
	32"-SHP-56-601-Q2	658	10.3-1
	32"-SHP-57-601-Q2	659	10.3-1
	32"-SHP-58-601-Q2	660	10.3-1
	4"-SHP-19-601-Q2	6590	10.3-1
	4"-SHP-20-601-Q2		
	4"-SHP-21-601-Q2		
	6"-SAE-1-601		
	6"-SAE-2-601		
	6"-SAE-3-601		
	32"-SHP-56-601-Q2	3063	10.3-1
	32"-SHP-57-601-Q2		
	32"-SHP-58-601-Q2		
	32"-SHP-22-601-Q2		
	32"-SHP-23-601-Q2		
	32"-SHP-24-601-Q2		
	10"-SSVD-1-601		
	10"-SSVD-2-601		
	10"-SSVD-3-601		
	10"-SSVD-4-601		

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<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>	<u>FSAR Fig. No.</u>
	10"-SSVD-5-601		
	10"-SSVD-6-601		
	10"-SSVD-7-601		
	10"-SSVD-8-601		
	10"-SSVD-9-601		
	10"-SSVD-10-601		
	10"-SSVD-11-601		
	10"-SSVD-12-601		
	10"-SSVD-13-601		
	10"-SSVD-14-601		
	10"-SSVD-15-601		
Main and Auxiliary Feedwater (FW)	4"-WAPD-3-601-Q3	204	10.3-5
	4"-WAPD-4-601-Q3		
	4"-WAPD-5-601-Q3		
	4"-WAPD-6-601-Q3		
	6"-WAPD-1-601-Q3		
	6"-WAPD-2-601-Q3		
	16"-WFPD-22-601-Q2	783	10.3-5
	16"-WFPD-24-601-Q2	784	10.3-5
	16"-WFPD-23-601-Q2	785	10.3-5
	16"-WFPD-9-601-Q2	0261	10.3-5
	16"-WFPD-13-601-Q2		
	16"-WFPD-17-601-Q2		
	6"-WD-23-151-Q3	203	10.3-5
	6"-WD-24-151-Q3		
	6"-WD-25-151-Q3		
	6"-WD-26-151-Q3		
	4"-WD-27-151-Q3		
	4"-WD-41-151-Q3		
	8"-WD-22-151-Q3	2031	10.3-5
	6"-WD-25-151-Q3		
	6"-WD-26-151-Q3		
Diesel Generator Exhaust (OL)	22"-OL-55-151-Q3	651	RM-53A

NOTES:

(( Problems 160 and 161 are included within the scope of the reanalysis effort for problem 159.

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(2) Problem 165 has been analyzed on NUPIPE as part of the Beaver Valley Unit 2 stress analysis effort.

These lines are identified on the flow diagrams included in Appendix C.

In addition to the problems referenced above, a number of other computer analyses were also performed for Beaver Valley - Unit 1, using the SHOCK2 code. These have been excluded from the scope of the reanalysis for interim startup and are discussed in Appendix B.

APPENDIX B

PROBLEMS TO BE REANALYZED IN THE LONG TERM

The problems described in Tables B-1 and B-2 are the remaining problems run on SHOCK 2, five of which are within the scope of the long term effort. These problems are identified on the flow diagrams included in Appendix D.

1. Other Safety Systems

Table B-1 identifies the SHOCK 2 problems that are within the scope of the long-term reanalysis effort; these lines are not required for safe shutdown.

2. Hand Calculations

Table B-2 identifies SHOCK 2 problems that are not within the scope of the long-term reanalysis effort; these SHOCK 2 runs are only check calculations of manual hand calculations. They are identified here only to show the scope of the original SHOCK 2 effort.

3. Superseded Calculations

The following SHOCK 2 runs have been superseded by a problem presently within the interim and long term reanalysis effort.

Superseded <u>SHOCK 2 Run</u>	New Problem <u>Number</u>
122A	122
312	840
657	785
916	217
1012	391
2110	341B
6230	310

4. Seismically Supported Non-Q Lines

The following lines are not safety related but have been seismically supported as designated by an "E" in the line designation table.

2"-CV-1-154

2"-SHPD-5-601

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2"-SHPL-6-601

2"-SHPD-7-601

2"-SHPD-8-601

1/4-SS-163-N9

1/4-SS-173-N9

1/4-SS-174-N9

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TABLE B-1

SAFETY SYSTEMS TO BE ANALYZED IN THE LONG TERM

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>	<u>FSAR Fig. No.</u>
Fuel Pool Cooling & Purification System (FC)	6"-FC-4-152-Q3	104	9.5-1
	6"-FC-5-152-Q3		
	6"-FC-6-152-Q3		
	6"-FC-8-152-Q3	105E	9.5-1
	6"-FC-9-152-Q3		
	10"-FC-1-152-Q3	198B	9.5-1
	6"-FC-2-152-Q3		
	6"-FC-31-152-Q3		
	4"-FC-10-152	107	9.5-1
	4"-FC-11-152		
River Water (WR)	6"-FC-14-152		
	6"-FC-17-152		
	6"-FC-32-152		
	30"-WR-175-151-Q3	153	9.9-1B



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TABLE B-2

HAND CALCULATIONS CHECKED BY SHOCK2

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>
High Pressure Steam (SHP)	3"-SHP-26-601-Q2	3043
	3"-SHP-31-601-Q2	
Steam Generator Auxiliary Feedwater Pump Discharge (WAPD)	3"-WAPD-13-601-Q3	207
	3"-WAPD-11-601-Q3	208
Generator Water Blowdown (WGCB)	3"-WGCB-8-601-Q2	309,
	3"-WGCB-12-601-Q2	3017,
		6220,
		3002,
		3018,
		6216
	3"-WGCB-4-601-Q2	310
	3"-WGCB-4-601-Q2	3100
Fuel Pool Cooling and Purification System (FC)	6"-FC-12-152-Q2	301
	6"-FC-17-152-Q2	655C
Charging and Volume Control System (CH)	3"-CH-125-1503-Q2	911,
		260,
		3001
	2"-CH-97-1502-Q1	200
	2"-CH-141-1503-Q1	220
	2"-CH-100-1502-Q2	230
	2"-CH-186-152-Q2	
	2"-CH-1-1502-Q1	240
	2"-CH-96-1502-Q1	250
	2"-CH-23-1502-Q1	300
	2"-CH-143-1502-Q1	350
	2"-CH-149-1502-Q1	
	2"-CH-145-602-Q1	
	2"-CH-2-602-Q1	
	2"-CH-3-602-Q1	

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TABLE B-2 (Cont)

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>
	2"-CH-4-602-Q1	
	2"-CH-146-152-Q3	
	3/4"-CH-115-1502-Q2	380
	2"-CH-2-602-Q2	702
	2"-CH-148-602-Q2	703
	3"-CH-106-153W-Q2	901, 3135
	3"-CH-107-153W-Q2	3135
	3"-CH-108-153W-Q2	704, 3135
	3"-CH-110-153W-Q2	704,
	3"-CH-111-153W-Q2	3057
	3"-CH-114-152W-Q2	3129, 3044
	4"-CH-14-153W-Q2	3057
	3"-CH-6-153W-Q2	3122
	3"-CH-226-153W-Q2	
	3"-CH-13-153W-Q2	3125
	4"-CH-72-1503-Q2	3131
	4"-CH-76-1503-Q2	
	3"-CH-71-1503-Q2	
	3"-CH-75-1503-Q2	
	3"-CH-80-1503-Q2	
	3"-CH-69-1503-Q2	3031
	3"-CH-70-1503-Q2	
	3"-CH-73-1503-Q2	
	3"-CH-74-1503-Q2	
	4"-CH-72-1503-Q2	
	4"-CH-76-1503-Q2	
	3"-CH-126-1503-Q1	3035
Safety Injection (SI)	3"-SI-81-1503-Q1&Q2	900, 3004
	3"-SI-140-1503-Q1	902, 3004

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TABLE B-2 (Cont)

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>
	6"-SI-34-1502-Q1	3006
	6"-SI-74-1502-Q1	
	3"-SI-60-1503-Q2	3124
	3"-SI-57-1503-Q1&Q2	900
	3"-SI-130-1503-Q1&Q2	313, 902
	3"-SI-134-1503-Q2	922
	3"-SI-81-1503-Q2	3120
	3"-SI-56-1503-Q3	3052
	3"-SI-60-1503-Q3	
	3"-SI-133-1503-Q3	
	4"-SI-75-1503-Q3	
	3"-SI-134-1503-Q1	
	3"-SI-31-153W-Q2	3127
	3"-SI-145-153W-Q2	
	3"-SI-35-152-Q3	965
Residual Heat Removal System (RH)	6"-RH-14-152-Q2	3012
Reactor Coolant (RC)	3"-RC-13-1502-Q1	6530
	3"-RC-23-1502-Q1	
	3"-RC-33-1502-Q1	
	3"-RC-160-153W-Q2	
	2"-RC-54-1502-Q1	220
	3"-RC-160-153W	917
	4"-RC-112-152-Q3	360
	3"-RC-160-153W	917
	3"-RC-160-153W-Q2	3021
Component Cooling (CC)	6"-CC-512-151-Q3	914
	4"-CC-487-151-Q2	913

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TABLE B-2 (Cont)

<u>System</u>	<u>Line Number</u>	<u>Problem No.</u>
	4"-CC-525-151-Q3	
	3"-CC-235-151-Q3	921
	3"-CC-486-151-Q2	
	3"-CC-523-151-Q3	
Diesel Generator Oil Line (OL)	3"-OL-46-151-Q3	650
Primary Grade Water (PG)	3"-PG-5-152	917
Quench Spray (QS)	2"-QS-29-152	315X
	6"-QS-30-153B-Q3	840
	6"-QS-31-153B-Q3	
	6"-QS-16-152	139
	4"-QS-8-152	341B
Neutron Shield Tank Cooling (NSL)	6"-NSL-2-152-Q3	312