

POWER AUTHORITY OF THE STATE OF NEW YORK

JAMES A. FITZPATRICK NUCLEAR POWER PLANT



JOHN D. LEONARD, JR.  
Resident Manager

June 8, 1979  
JAFP-79-291

P.O. BOX 41  
Lycoming, New York 13093

315-342-384C

Mr. Boyce H. Grier, Director  
United States Nuclear Regulatory Commission  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

SUBJECT: NRC I & E BULLETIN 79-02 - INTERIM REPORT

Dear Mr. Grier:

Pursuant to a telephone conversation between Mr. Stetka of your staff and myself, I am forwarding an Interim Report on the status of our action regarding I&E Bulletin 79-02 entitled "Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts."

The bulletin was received at the James A. FitzPatrick Nuclear Power Plant in mid-March and reviewed by the staff at the plant. It was determined by the plant staff that this bulletin significantly impacted the piping stress analysis being conducted on the FitzPatrick Plant due to the Show Cause Order issued by the Regulatory Commission on March 13, 1979 and therefore it was requested that our headquarters office take this bulletin for action and include it as part of the overall piping engineering survey and analysis.

The justification for this action was affirmed when, on April 2, 1979, Mr. Victor Stello, Director, Division of Operating Reactor Office of Nuclear Reactor Regulations, issued a letter to the Authority which stated that the schedule for compliance with this bulletin would be an element considered in the Regulatory Commission review of the piping stress analysis program for the FitzPatrick Plant.

Since that time, an extensive amount of work has been conducted at the plant in the verification of the adequacy of hangers and in the physical inspection of the anchor bolts. This inspection has formed the basis for the program for physically correcting any discrepancies noted at the plant. The initial program for correcting anchor bolt discrepancies if found, was to assure correct installation in accordance with the manufacturer's instructions and take credit for the guaranteed anchor strength of the support. Concrete expansion anchors were then to be torqued to the proper value to insure precast in accordance with the bulletin.

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Boyce H. Grier, Director  
United States Nuclear Regulatory Commission  
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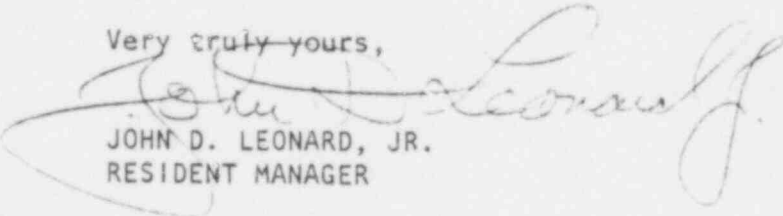
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We are expanding this program to include a pull test on the anchor itself for those anchors which are not fully embedded or have other discrepancies determined by inspection. The procedure for this test is expected to be completed in the immediate future. Experimental results on non-safety related system anchor bolts indicate that these anchors may hold a considerable load in excess of the manufacturer's guaranteed holding strength and that in spite of installation deficiencies such as non-perpendicularity, reduced length embedment, and partially set plugs in "redheads," the anchor is more than capable of performing its intended function.

Based on conversation with Mr. Orr of your staff, we will attempt to inspect and repair any noted deficiencies in as many as possible of the anchors in locations inaccessible during power operation. Should the Show Cause Order be lifted and plant startup be permitted, we will continue the program in accessible areas on safety related systems until completion.

The results of this program will insure that anchors and base plates will perform their intended function. In addition, the headquarters office of the Authority is proceeding, through a consultant, with an engineering program to determine the effect of flexibility on presently installed base plates. The headquarters staff has also committed to the Office of Nuclear Reactor Regulation that any re-design of piping supports which might occur due to the piping stress analysis which is presently being carried out for the plant, will be re-designed using the criteria of Bulletin 79-02.

Very truly yours,

  
JOHN D. LEONARD, JR.  
RESIDENT MANAGER

JDL:brp

Attachment

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