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June 27, 1979

Mr. Boyce H. Grier, Director
Office of Inspection and Enforcement
Region I
United States Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

SUBJECT: Licensee Event Report Narrative Description

The following occurrence was reported to Mr. Plumley, Region I, Office of Inspection and Enforcement on June 13, 1979.

Reference:	Docket Number 50-278
Report No:	LER 3-79-13/1T
Report Date:	June 27, 1979
Occurrence Date:	June 13, 1979
Facility:	Peach Bottom Atomic Power Station R. D. 1, Delta, PA 17314

Technical Specification Reference:

Technical Specification 3.6.A.1 requires that "the average rate of reactor coolant temperature change during normal heatup or cooldown shall not exceed 100°F increase (or decrease) in any one-hour period".

Technical Specification 3.7.A.1.b requires that "at any time that the nuclear system is pressurized above atmospheric pressure or work is being done which has the potential to drain the vessel, the pressure suppression pool water volume and temperature shall be maintained within the following limits except as specified in 3.7.A.2.

a. Minimum Water Volume - 122,900 ft.^{3}

b. Maximum Water Volume - 127,300 ft.^{3}

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Description of the Event:

With Unit 3 operating at 95% power, main steam relief valve RV 3-2-71L lifted spontaneously causing torus water volume to reach 130,675 cu. ft., 2.6% above Technical Specification limit of 127,300 cu. ft. After the relief valve opened and attempts to reclose it were unsuccessful, the reactor was manually scrammed. The reactor cool-down rate reached a maximum of 114°F in one hour, 14°F above Technical Specification limit of 100°F per hour. The relief valve was open for about 95 minutes, being reseated when reactor pressure was reduced to 135 psig.

Consequences of Event:

After the relief valve opened and attempts to reclose it by reducing the main steam pressure were unsuccessful, the reactor was manually scrammed, per procedure, before the torus temperature reached 110°F. During the reactor depressurization and cool-down the main condenser was available as a heat sink and maximum reactor cool-down rate was below the design fatigue evaluation limit for a relief valve blowdown. For these reasons the consequences of the event were minimal.

Cause of Event:

The cause of the event was the spontaneous lifting of main steam relief valve RV 3-2-71L while Unit 3 was at approximately 95% power.

Corrective Action:

The relief valve was replaced with another qualified valve. The valve that lifted was sent to Wiley Lab for inspection to determine the reason for failure. A visual inspection of the drywell and torus interior and exterior was performed prior to return to power, and was found to be in satisfactory condition.

Yours truly,

M. J. Cooney
Superintendent
Generation Division-Nuclear

Attachment

cc: Director, NRC - Office of Inspection and Enforcement
Mr. Norman M. Haller, NRC - Office of Management & Program Analysis

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