

NRR-DRMAPEm Resource

From: Kuntz, Robert
Sent: Wednesday, August 14, 2019 9:45 AM
To: Kivi, Jeffrey L.
Cc: Scott, Sara
Subject: Request for Additional Information RE: Prairie Island Relief Requests 1-RR-10 and 2-RR-10

By letter dated June 13, 2019 (Agencywide Document Access and Management System (ADAMS) Accession No. ML19164A166), Northern States Power Company (the licensee) requested relief from the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI, Table IWB-2500-1 for Category B-A and B-D examinations for Prairie Island Nuclear Generating Plant (PINGP), Units 1 and 2 reactor pressure vessel (RPV) welds and nozzle welds.

The NRC staff has determined that additional information is required to complete its review. The following is a draft request for additional information (RAI). If clarification on the following draft RAI is required contact me to set up a teleconference. If no clarification is required this draft RAI will be considered an RAI and the NRC will expect a response within 30 days of this e-mail.

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DRAFT REQUEST FOR ADDITIONAL INFORMATION
BY THE OFFICE OF NUCLEAR REACTOR REGULATION
ALTERNATIVE REQUESTS 1-RR-5-10 AND 2-RR-5-10 REGARDING
FIFTH AND SIXTH TEN-YEAR INSERVICE INSPECTION PROGRAM INTERVALS
NORTHERN STATES POWER COMPANY
PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2
DOCKET NUMBERS 50-282 AND 50-306
EPID: L-2019-LLR-0055

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Specifically, pursuant to Title 10 of the Code of Federal Regulations (10 CFR) 50.55a(z)(1), the licensee requested to use the proposed alternative to extend the fifth inservice inspection (ISI) interval at PINGP for Category B-A and B-D examinations so that the fifth ASME Code required examination can be performed in the current sixth ISI interval in 2033 and 2034 respectively on the basis that the alternative provides an acceptable level of quality and safety. The NRC staff reviewed the submittal and needs additional information to complete the review.

RAI-1

Tables 2 and 3 in Title 10 of Code of Federal Regulations Part 50.61a, "Alternative fracture toughness requirements for protection against pressurized thermal shock events" list maximum number of flaws depthwise for welds and plates (or forgings) of a generic RPV. However, Tables 2a and 2b of Alternative Requests 1-RR-5-10 and 2-RR-5-10 show different maximum number of weld and forging flaws for the PINGP, Units 1 and 2 RPV beltline, considering plant-specific RPV geometries. Please explain the differences and the manner in which the values in Tables 2a and 2b were derived (scaled).

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Tracking Status: None
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