

LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	M	A	P	P	S	1	2	0	0	0	0	0	0	0	0	0	0	3	4	1	1	1	1	1	4	5				
7	8	LICENSEE CODE						14	LICENSE NUMBER										25	LICENSE TYPE						30	CAT				58

CON'T

REPORT SOURCE: 01 60 61 05 01 02 93 68 69 07 11 07 94 75 76 77 78 79 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | On July 10, 1979, water was observed dripping from lagging on the CRD return
0 3 | line in the steam tunnel area. The lagging was removed and a pin hole leak dis-
0 4 | covered adjacent to the inboard weld on the 3" SS inboard manual isolation valve
0 5 | (301-99). This leak was within the reactor coolant pressure boundary. The unit
0 6 | was shut down on July 11, 1979, to examine and repair the leak.
0 7 |
0 8 |
7 8 9

09		SYSTEM CODE R B 11		CAUSE CODE B 12		CAUSE SUBCODE C 13		COMPONENT CODE P I P E X X 14				COMP. SUBCODE A 15		VALVE SUBCODE Z 16			
7 8		EVENT YEAR 7 9 21 22		SEQUENTIAL REPORT NO. 0 2 1 24 26		OCCURRENCE CODE 0 1 28 29		REPORT TYPE T 30		REVISION NO. 0 32							
ACTION TAKEN F 18		FUTURE ACTION C 19		EFFECT ON PLANT A 20		SHUTDOWN METHOD B 21		HOURS 0 1 6 6 22 40		ATTACHMENT SUBMITTED Y 23		NPRD-4 FORM SUB. N 24		PRIME COMP. SUPPLIER X 25		COMPONENT MANUFACTURER X 9 9 9 9 26 47	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Investigation revealed a crack in the inboard weld. The valve was replaced with

1 1 a straight run section of pipe eliminating the bi-metal junction. Non-destructive

1 2 testing was successfully performed and the system returned to service.

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FACILITY STATUS				% POWER				OTHER STATUS				METHOD OF DISCOVERY				DISCOVERY DESCRIPTION			
1	5	E	28	0	8	0	29	N/A				B	31	Observation					
ACTIVITY CONTENT				AMOUNT OF ACTIVITY				LOCATION OF RELEASE											
1	6	Z	33	Z	34	N/A				N/A									
PERSONNEL EXPOSURES				DESCRIPTION															
1	7	0	37	0	38	N/A													
PERSONNEL INJURIES				DESCRIPTION															
1	8	0	40	N/A															
LOSS OF OR DAMAGE TO FACILITY				DESCRIPTION															
1	9	Z	42	N/A															
PUBLICITY				DESCRIPTION															
2	0	N	44	N/A															

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NRC USE ONLY

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BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION

DOCKET NO.: 50-293
ATTACHMENT TO LER - 79-027/01T-0

It has been determined that the pinhole leak in the CRD return line originated in the root weld between the 3" SS manual isolation valve (301-99) and the 3" CS-160 pipe. This crack migrated circumferentially through the crown weld and longitudinally into the pipe. The valve and piping were removed for radiographing and repair. During the examination it was decided to postpone replacement of the valve until the 1980 refueling outage and to install a section of 3" CS-160 pipe in its place.

A Plant Design Change Request (PDCR) was initiated, a Safety Evaluation performed by the Engineering Department and the section of pipe installed. The purpose of the manual isolation valve was to both provide a convenient means of isolating the CRD return line for Local Leak Rate testing and operational purposes. Replacing this valve with a straight run of pipe will not affect system operation and performance.

Non-destructive testing was successfully performed and the system returned to service.

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