

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-368

UNIT 2

DATE July 11; 1979

COMPLETED BY C. N. Shively

TELEPHONE 501/968-2519

MONTH June

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>1</u>
9	<u>0</u>
10	<u>7</u>
11	<u>73</u>
12	<u>44</u>
13	<u>10</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>6</u>
21	<u>1</u>
22	<u>126</u>
23	<u>80</u>
24	<u>166</u>
25	<u>347</u>
26	<u>175</u>
27	<u>357</u>
28	<u>366</u>
29	<u>371</u>
30	<u>379</u>
31	<u>NA</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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OPERATING DATA REPORT

DOCKET NO 50-368
 DATE July 11, 1979
 COMPLETED BY C. N. Shively
 TELEPHONE 5017968-2519

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: June 1-30, 1979
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): NA
7. Maximum Dependable Capacity (Net MWe): NA

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted, If Any (Net MWe): None

10. Reasons For Restrictions, If Any: NA

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	4343.0	5087.0
12. Number Of Hours Reactor Was Critical	336.7	1017.6	1452.5
13. Reactor Reserve Shutdown Hours	205.3	1910.6	2052.8
14. Hours Generator On-Line	235.4	838.5	891.3
15. Unit Reserve Shutdown Hours	20.6	21.7	21.7
16. Gross Thermal Energy Generated (MWH)	260277.0	582113.0	626691.0
17. Gross Electrical Energy Generated (MWH)	67836.0	137973.0	143540.0
18. Net Electrical Energy Generated (MWH)	60158.0	112515.0	116499.0
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (Using MDC Net)	} NA Until	Commercial Operation	
22. Unit Capacity Factor (Using DER Net)			
23. Unit Forced Outage Rate			
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY

INITIAL ELECTRICITY

COMMERCIAL OPERATION

12-5-78

12-26-78

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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-368
 UNIT NAME ANO-Unit II
 DATE 7/1/79
 COMPLETED BY C.W. Shively
 TELEPHONE 501-968-2519

REPORT MONTH June

No.	Date	Type	Duration (Hours)	Reason	Method of Shutting Down Reactor	Licensee Event Report #	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence
79-8	790528	F	186.9	A	NA	NA	OB	PUMPXX	RCP "C" Seal Failure
79-9	790608	F	43.6	A	1	NA	HB	PIPEXX	Main Steam Leak Repair
79-10	790612	F	20.4	A	2	NA	EB	TRANSF	Auxiliary transformer bus lock out during auxiliary power transfer
79-11	790613	F	163.8	A	3	NA	EA	TRANSF	Loss of offsite power
79-12	790620	S	31.7	B	NA	NA	NA	NA	Turbine Control Valve Tightness Test
79-13	790623	F	27.7	A	3	NA	EG	TRANSF	Turbine Unit Differential Voltage Trip
79-14	790626	F	10.5	B	3	NA	NA	NA	Test equipment hookup error

1. F. Forced
 S. Scheduled
 2. Reason:
 A. Equipment Failure (Explain)
 B. Maintenance or Test
 C. Refueling
 D. Regulatory Restriction
 E. Operator Training & License Examination
 F. Administrative
 G. Operational Error (Explain)
 H. Other (Explain)
 3. Method:
 1. Manual
 2. Manual Scram
 3. Automatic Scram
 4. Other (Explain)
 4. Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NRC 0161)
 5. Exhibit I - Study Source

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REFUELING INFORMATION

DATE: June 1979

1. Name of facility. Arkansas Nuclear One - Unit 2
2. Scheduled date for next refueling shutdown. 09-01-80
3. Scheduled date for restart following refueling. 12-01-80
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
If answer is yes, what, in general, will these be?
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?
Yes. Description of effects of new core loading
5. Scheduled date(s) for submitting proposed licensing action and supporting information. 08/01/80
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
NONE
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 0
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
present 486 increase size by 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: 1988

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NRC MONTHLY OPERATING REPORT
OPERATING SUMMARY - JUNE, 1979
UNIT II

The repair of the "A" Reactor Coolant Pump seal continued from the previous month. After repairs were completed, the Main Steam Relief Valve setpoint and blowdown testing continued. All valves performed satisfactorily.

The reactor was declared critical on 6/6 and the unit was placed on line on 6/8; however, was shutdown five hours later due to a steam leak on one of the main steam lines. The unit was placed back on line on 6/10. During the transfer of auxiliary power supply to the auxiliary transformer, the condenser pumps tripped due to the feeder breaker for electrical bus A-2 being locked out. The main feedwater pump tripped on low suction pressure and the reactor was tripped manually. The unit was returned to service on 6/13, but tripped five hours later when a phase to phase and a phase to ground fault was received on the secondary sides of the unit auxiliary, startup #2 and startup #3 transformers. Faults were caused by water entering the electrical buses through the feeder breakers from a cleaning job in the adjacent work area. Repairs were made and the unit was placed back on line on 6/20. Power Escalation Testing was resumed. As the unit was being escalated from 40% reactor power, the turbine tripped on unit differential voltage. Investigation revealed the A & C phase relay wires were crossed. The unit was returned to power operation, Mode 1, on 6/24. Two days later, a trip was received when test personnel inadvertently switched the ground and positive leads of a test recorder, causing the Feedwater Control System to malfunction. The unit was returned to Mode 1 operation the same day. The 50% power plateau power escalation testing continued throughout the remainder of the month.

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