

Vepco

USNRC REGION II
ATLANTA, GEORGIA

VIRGINIA ELECTRIC AND POWER COMPANY, RICHMOND, VIRGINIA 23261

73 JUN 13 1979
June 11, 1979

Mr. James P. O'Reilly, Director
Office of Inspection & Enforcement
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Serial No. 344A
PSE&C/JMM:bam:wang

Docket Nos. 50-339
50-404
50-405

Dear Mr. O'Reilly:

NRC letter of March 12, 1979 forwarded IE Bulletin 79-03
"Longitudinal Weld Defects in ASME SA-312 Type 304 stainless steel pipe spools
manufactured by Youngstown Welding and Engineering Company."

Vepco letter Serial No. 344, docket Nos. 50-339, 50-404, and 50-405
dated May 10, 1979 indicated that due to the extensive records review required
for compliance with IE Bulletin No. 79-03 an additional 30 days minimum would
be required for a reply.

At this time Vepco has obtained information on the use of Youngstown
Welding and Engineering Company, SA/A 312 type 304/316 material welded without
filler metal (noted hereafter as "Youngstown Material") for most of the items
and components in safety-related systems. We feel, however, that an
additional 30 days will be required to complete the review and obtain written
confirmation of the results of all reviews.

The following interim report of the progress of our review is
provided:

I. Docket No. 50-339: North Anna Unit 2:

A. Review of items purchased by the Architect-Engineer, Stone
Webster Engineering Corporation:

1. Pipe and Fittings: A complete review of pipe and
fittings has revealed only one item, a single fitting,
which was made of Youngstown material. This item is
an ASTM A 312, Type 304 welded without filler metal,
10" schedule 40S 90 degree short radius elbow in the
Recirculation Spray System. The Recirculation Spray
System is a Q-2 (equivalent to ASME III, Subsection
NC) System with design pressure of 154 psig and design
temperature of 219 F in the region of the elbow. The
elbow is located in the reactor containment. Vepco
will perform 100% radiographic inspection of the
longitudinal weld seam of this fitting to the
requirements and acceptance criteria of ANSI B 31.7.

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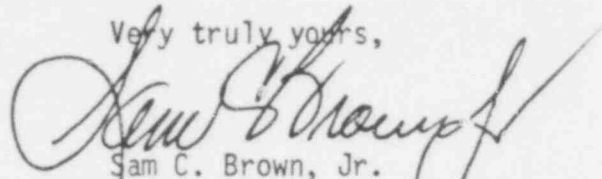
2. Other equipment: The review of all other equipment where Youngstown material could have been used has been completed except for two items. No Youngstown material has been identified in any of the equipment reviewed.
- B. Review of items manufactured/purchased by the Nuclear Steam Supply System Vendor, Westinghouse:
1. Primary system components: A review of all pipe, fittings, nozzles etc., purchased by Westinghouse has been completed and reveals that no Youngstown material was used.
 2. Auxiliary components supplied by Westinghouse vendors: This review has not been completed by Westinghouse. For items which have been reviewed, it has been determined that no Youngstown material was used.

II. Docket Nos. 50-404 and 50-405: North Anna 3 and 4:

- A. Review of items purchased by the Architect-Engineer, Stone and Webster Engineering Corporation:
1. Pipe and Fittings: A review of fabricated subassemblies, pipe and fittings is presently underway. No Youngstown material has been identified in items reviewed. Also, no Youngstown pipe, or fittings made from Youngstown pipe, are planned for use in Units 3 and 4. If Youngstown material is identified in our inventory or purchased later, all such material will be 100% radiographically inspected in accordance with ASME Section III, paragraph NB 2553. Material not meeting this criterion will not be used.
 2. Equipment: The review of equipment has not been completed. No Youngstown material has been identified in items reviewed.
- B. Review of items manufactured/purchased by the Nuclear Steam Supply System Vendor, Babcock & Wilcox (B&W):
1. Primary system components: A review of all pipe, fittings, nozzles etc. purchased by B&W has been completed and reveals that no Youngstown material was used.
 2. Auxiliary components supplied by B&W vendors: This review has been completed and reveals that no Youngstown material was used in any components.

It is our intent to provide a final report in accordance with IE Bulletin 79-03 to the NRC by July 11, 1979, which will finalize the incomplete items noted in I.A.2., I.B.2, II.A.1, and II.A.2 above.

Very truly yours,



Sam C. Brown, Jr.
Senior Vice President - Power Station
Engineering and Construction

cc: Mr. John G. Davis, Acting Director
Office of Inspection & Enforcement

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation