

LICENSEE EVENT REPORT

CONTROL BLOCK: [] [] [] [] [] [] (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

[0][1] [F][I][L][T][P][I][S][3] [2][0][0]-[0][0][0][0][0]-[0][0] [4][1][1][1][1] [4] [5]
7 3 3 14 15 25 26 28 29 30 31 32 33

CON'T
[0][1] REPORT SOURCE [X][5][0][5][0][0][0][2][5][0] [7][0][6][2][6][7][9] [3][0][7][1][0][7][9] [9]
7 3 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

[0][2] Our NSSS vendor notified us of a potential level bias which could be
[0][3] imposed on the steam generator water level instrumentation signal as a
[0][4] result of high containment temperature influencing steam generator level
[0][5] transmitter reference leg temperature. This bias could slightly delay
[0][6] reactor trip and auxiliary feedwater initiation on low-low steam generator
[0][7] level, or cause indicated level to be greater than actual steam generator
[0][8] level in a post-accident environment.
7 3 9

[0][9] SYSTEM CODE [I][E] [11] CAUSE CODE [B] [12] CAUSE SUBCODE [A] [13] COMPONENT CODE [I][N][S][T][R][U] [14] COMP SUBCODE [X] [15] VALVE SUBCODE [Z] [16]
7 3 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

[17] LER/RO REPORT NUMBER [7][9] [21] EVENT YEAR [7][9] [22] SEQUENTIAL REPORT NO. [0][2][1] [23] OCCURRENCE CODE [0][1] [24] REPORT TYPE [T] [25] REVISION NO. [0]
20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

ACTION TAKEN [X] [18] FUTURE ACTION [X] [19] EFFECT ON PLANT [Z] [20] SHUTDOWN METHOD [Z] [21] HOURS [0][0][0][0] [22] ATTACHMENT SUBMITTED [Y] [23] NPRO-4 FORM SUB. [N] [24] PRIME COMP SUPPLIER [N] [25] COMPONENT MANUFACTURER [W][1][2][0]
32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

[1][0] There is no adverse safety significance because of the conservatism of our
[1][1] actual low-low steam generator level trip setpoint, however, a license
[1][2] amendment will be proposed to increase the limiting safety system setpoint.
[1][3] The significance of post-accident steam generator level bias is under
[1][4] evaluation, and an update will follow if changes are deemed necessary.
7 3 9

[1][5] FACILITY STATUS [G] [26] % POWER [0][0][0] [27] OTHER STATUS [NA] [28] METHOD OF DISCOVERY [D] [29] DISCOVERY DESCRIPTION [NSSS notification.] [30]
7 3 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

[1][5] ACTIVITY CONTENT RELEASED OR RELEASE [Z] [31] AMOUNT OF ACTIVITY [NA] [32] LOCATION OF RELEASE [NA] [33]
7 3 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

[1][7] PERSONNEL EXPOSURES NUMBER [0][0][0] [34] TYPE [Z] [35] DESCRIPTION [NA] [36]
7 3 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

[1][3] PERSONNEL INJURIES NUMBER [0][0][0] [37] DESCRIPTION [NA] [38]
7 3 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

[1][9] LOSS OF OR DAMAGE TO FACILITY TYPE [Z] [39] DESCRIPTION [NA] [40]
7 3 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

[2][0] PUBLICITY DESCRIPTION [N] [41] [42]
7 3 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

NAME OF PREPARER M. A. Schoppman

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Additional Event Description and Probable Consequences:

Notification was received from our NSSS vendor regarding a potential level bias which could be imposed on the steam generator water level instrumentation signal. This bias could occur as a result of high containment temperature influencing steam generator level transmitter reference leg temperature and consequently fluid density. In the event of a high energy line break inside containment, the increase in reference leg temperature could theoretically cause a bias such that the indicated steam generator level could be 10% greater than the actual level. In the case of a feedline break, this condition could slightly delay reactor trip and auxiliary feedwater initiation based on low-low steam generator level. This phenomenon could also exist in the post-accident containment environment and cause a similar discrepancy between indicated and actual steam generator level.

Additional Cause Description and Corrective Action:

Review of this information indicates there is no adverse safety significance because of the conservatism of our low-low steam generator level trip setpoint, i.e., 15% versus 0% used for accident analysis basis. However, our Technical Specification limiting safety system setting requirement allows a setpoint of 5% for low-low steam generator level. Therefore, a license amendment will be proposed to increase the limiting safety system setpoint. Our review of the significance of post-accident steam generator level bias is continuing. At present, we believe the effect of the postulated difference between indicated and actual steam generator levels would be inconsequential. We are considering the use of correction factors for indicated steam generator level versus containment temperature should this be necessary.