

## Nebraska Public Power District

COOPER NUCLEAR STATION  
P.O. BOX 98, BROWNVILLE, NEBRASKA 68021  
TELEPHONE (402) 825-3811

CNSS790271

May 30, 1979

Mr. K. V. Seyffert  
U.S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region IV  
611 Ryan Plaza  
Suite 1000  
Arlington, Texas 76011

Dear Sir:

This report is submitted in accordance with Section 6.7.2.B.2 of the Technical Specifications for Cooper Nuclear Station and discusses a reportable occurrence that was discovered on May 8, 1979. A licensee event report form is also enclosed.

Report No.: 50-298-79-13  
Report Date: May 30, 1979  
Occurrence Date: May 8, 1979  
Facility: Cooper Nuclear Station  
Brownville, Nebraska 68021

### Identification of Occurrence:

A condition leading to operation in a degraded mode permitted by a limiting condition for operation established in Section 3.7.E.1.a of the Technical Specifications.

### Conditions Prior to Occurrence:

The reactor was in the "Run" mode at 672 MWt.

### Description of Occurrence:

Technical Specification Section 3.7.E.1.a was not met, in that differential pressure between the drywell and suppression chamber of  $\geq 1.47$  psid was established in 26 hours 52 minutes and not within the required 26 hours of achieving operating temperature and pressure.

### Designation of Apparent Cause of Occurrence:

The cause of the failure to establish drywell to suppression chamber  $\Delta P \geq 1.47$  psid has been attributed to valve PC-236 AV, drywell inlet purge shutoff, leaking excessively.

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Analysis of Occurrence:

The maintenance of the drywell to suppression chamber  $\Delta P$  of 1.47 psid with a suppression chamber water level corresponding to a downcomer submergence range of four to five feet will assure the integrity of the suppression chamber when subjected to post-LOCA suppression pool hydrodynamic forces.

The drywell to suppression chamber  $\Delta P$  was  $\geq 1.15$  psid within the 26 hour period following achieving operating temperature and pressure, and increasing. The leakage of nitrogen through PC-236 AV to the inlet purge line prevented the nitrogen purge from pressurizing the drywell prior to establishing the  $\Delta P$ .

This occurrence presented minimal potential consequences from the standpoint of public health and safety.

Corrective Action:

The valve lineup for purging the drywell and suppression chamber with nitrogen and establishing the required drywell to suppression chamber  $\Delta P$  was temporarily changed to use the ACAD System.

PC-236 AV was removed, repaired, leak tested and reinstalled. The nitrogen purging and drywell to torus  $\Delta P$  systems were returned to normal lineup.

This occurrence has been discussed with the personnel involved and a copy of the nonconformance report has been routed to all licensed personnel.

Sincerely,

*Gaul G Bower*

*for*

L. C. Lessor  
Station Superintendent  
Croper Nuclear Station

LCL:cg  
Attach.