

Nebraska Public Power District

COOPER NUCLEAR STATION
P.O. BOX 8, BROWNVILLE, NEBRASKA 68321
TELEPHONE (402) 825 3811

CNSS790266

May 30, 1979

Mr. K. V. Seyfrit
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region IV
611 Ryan Plaza
Suite 1000
Arlington, Texas 76011

Dear Sir:

This report is submitted in accordance with Section 6.7.2.B.3 of the Technical Specifications for Cooper Nuclear Station and discusses a reportable occurrence that was discovered on May 3, 1979. A licensee event report form is also enclosed.

Report No.: 50-298-79-12
Report Date: May 30, 1979
Occurrence Date: May 3, 1979
Facility: Cooper Nuclear Station
Brownville, Nebraska 68321

Identification of Occurrence:

Observed inadequacies in the implementation of administrative or procedural controls.

Conditions Prior to Occurrence:

Inservice leak test of the reactor pressure boundary.

Description of Occurrence:

During depressurization of reactor vessel following an in-service leak test, vessel water temperature increased to approximately 220°F with reactor vessel head vents closed and primary containment not established.

Designation of Apparent Cause of Occurrence:

The station operating procedure for the reactor vessel in-service leak test did not caution the operator about exceeding 212°F during the hydro operation.

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Analysis of Occurrence:

The Technical Specifications require that primary containment be established when the reactor is critical or when the reactor water temperature is $>212^{\circ}\text{F}$ and fuel is in the reactor vessel. The temperature limit of 212°F was exceeded for a short time when the operators experienced difficulty resetting the Residual Heat Removal (RHR) pressure interlock at 75 psig when attempting to return to the shutdown cooling mode after the hydro. Although the temperature limit was exceeded, this event presented minimal adverse consequences from the standpoint of public health and safety because the reactor was subcritical, secondary containment was established and the Standby Gas Treatment and Low Pressure Core Standby Cooling Systems were operable.

Corrective Action:

The setpoint of the interlock was verified correct. A precaution will be added to the procedure for the Reactor Vessel In-service Leak Test to caution the operator concerning the 212°F limit. The switches are under evaluation. The Technical Specifications are under evaluation to determine whether an exemption to the 212°F limit can be permitted during the in-service leak test.

This event has been discussed with the personnel involved and a copy of the LER will be routed to all licensed operators.

Sincerely,

Paul G. Boser

for L. C. Lessor
Station Superintendent
Cooper Nuclear Station

LCL:cg
Attach.

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