

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346  
UNIT Davis-Besse Unit 1  
DATE July 6, 1979  
COMPLETED BY Erdal C. Caba  
TELEPHONE 419-259-5000, Ext. 236

MONTH June, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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# OPERATING DATA REPORT

DOCKET NO. 50-346  
 DATE July 6, 1979  
 COMPLETED BY Erdal C. Caba  
 TELEPHONE 419-259-5000, Ext. 236

## OPERATING STATUS

1. Unit Name: Davis-Besse Unit 1
2. Reporting Period: June, 1979
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): to be determined
7. Maximum Dependable Capacity (Net MWe): to be determined
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): Zero
10. Reasons For Restrictions, If Any: NRC OIE Bulletins and Shutdown Orders

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720	4,343	16,108
12. Number Of Hours Reactor Was Critical	0	1,747.4	8,379.2
13. Reactor Reserve Shutdown Hours	720	1,612.2	2,402.5
14. Hours Generator On-Line	0	1,675.1	7,408.3
15. Unit Reserve Shutdown Hours	720	1,464	1,464
16. Gross Thermal Energy Generated (MWH)	0	3,879,097	14,066,667
17. Gross Electrical Energy Generated (MWH)	0	1,293,268	4,677,023
18. Net Electrical Energy Generated (MWH)	0	1,212,558	4,254,018
19. Unit Service Factor	0	38.6	47.5
20. Unit Availability Factor	100	72.3	57.9
21. Unit Capacity Factor (Using MDC Net)	to be determined		
22. Unit Capacity Factor (Using DER Net)	0	30.8	32.5
23. Unit Forced Outage Rate	0	4.8	22.4
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: July 10, 1979
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

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## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June, 1979

DOCKET NO. 50-346  
 UNIT NAME Davis-Besse Unit 1  
 DATE July 6, 1979  
 COMPLETED BY Erdal C. Caba  
 TELEPHONE 419-259-5000, Ext. 236

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
9	79 03 30	S	720	D	1	N/A	N/A	N/A	The unit remained in an outage the entire month. Refer to the attached outage summary of June, 1979, for outage activities this month.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

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OUTAGE SUMMARY  
June, 1979

The unit outage which began at 2142 hours on March 1979, was still in progress through the end of June, 1979. The outage was extended longer than anticipated due to additional NRC startup restraints which were imposed as a result of an ongoing analysis of the Three Mile Island incident.

The following are the more significant outage activities performed during the month of June:

1. Procedure modification to comply with NRC requests. In addition, modifications of procedures were made due to the re-evaluation of the Babcock and Wilcox small break analysis and natural circulation.
2. The performance of hydraulic snubber surveillance testing was completed.
3. Testing of the anticipatory reactor trip system which was installed last month was completed.
4. Testing of the auxiliary feed pump flow indication and dynamic braking was completed.
5. Personnel instruction on what the procedure major modifications involved.
6. The containment air cooler fans had to be purged of the old grease and replaced with new grease.
7. Radiography on the first six elbow welds on the number 1 auxiliary feedwater line per NRC concerns.
8. Work was done on the auxiliary boiler due to various steam leaks.
9. The performance of eighteen month surveillance tests which were required to be completed prior to the reactor internal vent valve test. This was done to lengthen the available operation time after this outage.
10. Oil leaks were found on the 01 Startup Transformer and repaired.
11. The implementation of various Facility Change Requests which were restraints to startup.

## REFUELING INFORMATION

DATE: June, 1979

1. Name of facility: Davis-Besse Nuclear Power Station Unit 1
2. Scheduled date for next refueling shutdown: March, 1980
3. Scheduled date for restart following refueling: May, 1980
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

5. Scheduled date(s) for submitting proposed licensing action and supporting information. December, 1979
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

The spent fuel pool capacity expansion program is awaiting a final NRC Safety Evaluation Report to proceed in time for completion prior to refueling. All licensee submittals are complete including environmental assessment questions.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
- (a) 177 (b) 0 (zero)
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
- Present 260 Increase size by 475 (735 total)
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date March, 1980 - May, 1980 (assuming ability to unload the entire core into the spent fuel pool is maintained.)

REFUELING INFORMATION (Continued)

June, 1979

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4. The following Technical Specifications (Part A) will require revision:

- 2.1.1 & 2.1.2 - Reactor Core Safety Limits (and Bases)
- 2.2.1 - Reactor Protection System Instrumentation Setpoints (and Bases)
- 3.1.3.6 - Regulating Rod Insertion Limits
- 3.1.3.7 - Rod Program
- 3.2.1 - Axial Power Imbalance (and Bases)

The following Technical Specifications (Part A) may also require revision:

- 3.1.2.8 & 3.1.2.9 - Borated Water Sources (and Bases)
- 3.2.4 - Quadrant Power Tilt (and Bases)
- 3.2.5 - DNB Parameters (and Bases)

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