

VIRGINIA ELECTRIC AND POWER COMPANY

RICHMOND, VIRGINIA 23261

December 17, 1979

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
Attn: Mr. A. Schwencer, Chief  
Operating Reactors Branch No. 1  
Division of Operating Reactors  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

Serial No. 973  
FR/RWC: mvc  
Docket No.: 50-338  
50-339  
License No. NPF-4

Dear Mr. Denton:

PROCEDURAL CHANGES  
REQUIRED TO ADDRESS CONTROL  
ROD DROP ANALYSIS CONCERNS

Westinghouse Electric Corporation notified Vepco of an unreviewed safety question on November 15, 1979 concerning the potential for predicting more limiting results for single dropped rod events. Discussions with Westinghouse confirmed that the only Vepco plants affected were North Anna Units 1 and 2. Consequently, the appropriate Nuclear Regulatory Commission (NRC) notifications were made.

The event was investigated further by Vepco through detailed discussions with Westinghouse and the NRC Staff at a meeting in Bethesda, Maryland on November 19, 1979. The interim solution agreed upon at the meeting is provided in Attachment 1 and will be incorporated in the appropriate Operating Procedures applicable to both North Anna Units 1 and 2 prior to operation. Since the current Technical Specifications limit on control rod insertion is nonconservative, we are immediately implementing an administrative reduction in the control rod insertion limits consistent with Attachment 1. Operation in compliance with this solution will eliminate any safety concerns associated with potential dropped rod events. Westinghouse is continuing to address the issue, and we will evaluate their proposed permanent solutions for applicability to North Anna Units 1 and 2 when the solutions are finalized.

In conjunction with this interim solution, we are withdrawing our proposed Technical Specifications Change No. 21 transmitted by my letter to you dated July 6, 1979, Serial No. 504. As discussed at the meeting on November 19, 1979, these changes are no longer appropriate.

The proposed solution has been reviewed and approved by the Station Nuclear Safety and Operating Committee and the System Nuclear Safety and Operating Committee. If you require additional information, please contact us.

Very truly yours,

*C. M. Stallings*

C. M. Stallings  
Vice-President-Power Supply  
and Production Operations

Attachment

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cc: Mr. James P. O'Reilly, Director  
Office of Inspection and Enforcement  
Region II

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ATTACHMENT 1

INTERIM PROCEDURAL CHANGES  
REQUIRED FOR NORTH ANNA UNITS 1 AND 2 AS A RESULT OF  
CONTROL ROD DROP CONCERNS

- MANUAL ROD CONTROL

- 0 TO 100% POWER: NO CHANGE FROM CURRENT  
ROD INSERTION LIMITS (RIL)

- AUTOMATIC ROD CONTROL

- 0 to 90% POWER: NO CHANGE FROM CURRENT RIL
- 90 TO 100% POWER: D BANK GREATER THAN 215 STEPS

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