



Public Service Company of Colorado

16805 Weld County Road 19 1/2, Platteville, Colorado 80651

November 28, 1979
Fort St. Vrain
Unit No. 1
P-79282

Mr. Karl V. Seyfrit, Director
Nuclear Regulatory Commission
Region IV
Office of Inspection and Enforcement
611 Ryan Plaza Drive
Suite 1000
Arlington, Texas 76012

REF: Facility Operating License
No. DPR-34

Docket No. 50-267

Dear Mr. Seyfrit:

Enclosed please find a copy of Reportable Occurrence Report No. 50-267/79-54, Final, submitted per the requirements of Technical Specification AC 7.5.2(b)3.

Also, please find enclosed one copy of the Licensee Event Report for Reportable Occurrence Report No. 50-267/79-54.

Very truly yours,

Don Warembourg/jm

Don Warembourg
Manager, Nuclear Production

DW/alk

cc: Director, MIPC

A002
S
1475 129 1/1

7912040429

REPORT DATE: November 28, 1979

REPORTABLE OCCURRENCE 79-54

ISSUE 0

OCCURRENCE DATE: October 29, 1979

Page 1 of 4

FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
16805 WELD COUNTY ROAD 19 1/2
PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/79-54/03-L-0

Final

IDENTIFICATION OF
OCCURRENCE:

Standby diesel engine 1B tripped on high cooling water temperature during surveillance test because cooling water valves were lined up incorrectly.

This is reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)3.

EVENT
DESCRIPTION:

On October 24, 1979, while operating at 26% thermal power and 50 MW electrical power, a mechanical clearance was issued at 0845 hours to repair a cooling water tube leak on the standby diesel engine 1B cooler, S-7539. See Figure 1. The cooler, S-7539, has two cooling coils, one for the 1A engine and one for the 1B engine. The standard clearance point form specified that the inlet valve, V-42385, and the outlet valve, V-42384, both be shut and the bypass valve, V-42389 be opened. This isolated the cooler for repair of the leaking tube but still should have allowed the normal temperature control of the 1B diesel engine cooler via TCV-4268 if the diesel generator unit had been required.

However, the bypass valve, V-42389 was not open as specified, thus the cooling water to the 1B diesel engine was isolated.

On October 29, 1979, while the reactor was shutdown for scheduled maintenance, operations personnel performed the standby diesel generator 50% load test.

At 0820 hours, the 1A and 1B diesel engines were started and generator 1A was synchronized and loaded to 50% per the surveillance test. The test is performed on a weekly basis and continues for at least two hours to enable the engines and generators to attain normal operating temperatures.

At 1000 hours, the 1B diesel engine automatically tripped on high cooling water temperature of 500°F and declutched the engine.

CAUSE
DESCRIPTION:

An incorrect valve lineup caused the 1B diesel engine to trip on high cooling water temperature.

1475 130

CORRECTIVE
ACTION:

Cooling water was restored to the 1B diesel engine cooler.

Personnel involved have been reinstructed in the proper methods to be used for clearing out equipment.

Standby diesel generator surveillance test completed.

No further corrective action is anticipated or required.

1475 131

POOR ORIGINAL

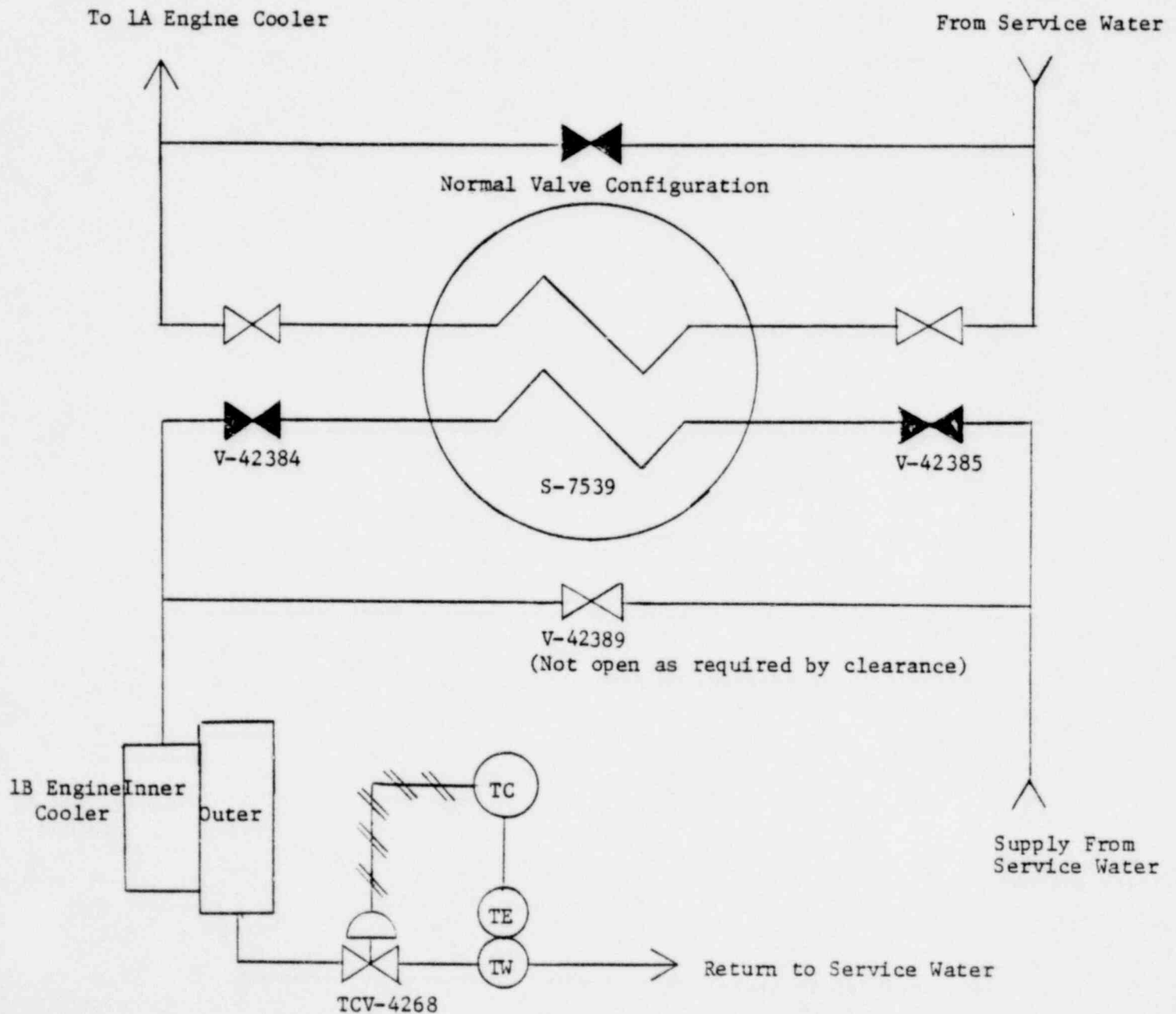


FIGURE 1

1475 132

Prepared by: Michael J. Ferris
Michael J. Ferris
Technical Services Engineer

Reviewed by: J. W. Gahm
J. W. Gahm
Technical Services Supervisor

Reviewed by: Frank M. Mathie
Frank M. Mathie
Operations Manager

Approved by: Don Warembourg
Don Warembourg
Manager, Nuclear Production

1475 133