



LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION

P.O. BOX 618, NORTH COUNTRY ROAD • WADING RIVER, N.Y. 11792

November 19, 1979

SNRC-448

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

SHOREHAM NUCLEAR POWER STATION - UNIT 1
DOCKET NO. 50-322

Dear Mr. Denton:

Mr. S. A. Varga's letter of October 9, 1979, requested that LILCO, along with other Mark II Owners, embrace the generic positions on the Mark II Suppression Pool Dynamic Loads Program wherever feasible in order to minimize both NRC and utility manpower requirements, and to expedite the completion of the NRC review of the Mark II program. Mr. Varga's letter also requested specific definitions of any areas which might require "plant specific" positions by LILCO.

The Mark II program has divided the Mark II containment plants into a "Lead Plant" group and a "Follow-On" plant group. The Shoreham station, a member of the Lead Plant group along with the Zimmer and LaSalle Stations, has employed a largely generic approach to the resolution of the Mark II suppression pool loads. In fact, Shoreham has accepted what we believe to be overly conservative load specifications in some areas in order to expedite resolution of the overall program. In a great many cases, the NRC Acceptance Criteria, as defined in NUREG-0487, has been applied to our plant "as is". In other cases the Lead Plants have presented generic alternative positions which we believe are acceptable to the NRC Staff on the basis of technical meetings held on these subjects. In a few instances, plant-unique load specifications have been required because of physical differences between the units and/or particular calculational techniques being employed. Such information has been provided to the NRC Staff for evaluation as required.

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In summary, the Mark II lead plants have been employing, and will continue to employ, a largely generic approach to the resolution of the Mark II Suppression Pool Dynamic Loads Program. In addition, we have adopted the NRC Staff Acceptance Criteria verbatim wherever feasible. To illustrate this point, Attachment 1 shows a listing of the major Mark II Lead Plant open items which were discussed in our letter SNRC-362 dated March 5, 1979 and updated here to reflect their current status. As can be seen from this list, the generic approach has been extensively employed.

We believe that we are approaching a final resolution of the Mark II lead plant program with the NRC Staff and that we have already submitted all information required. If there is any additional information required by the Staff in order to complete their review of the Mark II issue on the Shoreham docket, we request that you inform us of these as soon as possible.

Very truly yours,

J. Musler for J. P. Novarro

J. P. Novarro,
Project Manager
Shoreham Nuclear Power Station

WJM/cc

Attachment

cc: Messrs. S. A. Varga
J. Higgins

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3/5/79 (SNRC-362) SHOREHAM POSITION UPDATED TO 11/01/79 FOR MK II NRC ACCEPTANCE CRITERIA

<u>NRC OPEN ITEM 2/9/79</u>	<u>SHOREHAM POSITION</u>	<u>LEAD PLANT GENERIC</u>	<u>SHOREHAM UNIQUE</u>	<u>COMMENTS</u>
1. Pool Swell Elevation	Question Response 020.68.		X	Submitted 2/16/79 via SNRC-360 for Shoreham-Acceptable to NRC per 9/13/79 ACRS Meeting.
2. Small Structure Impact Loads	Comply with NRC criteria.	X		
3. Asymmetric Pool Swell Loads	Comply with current NRC position.	X		Current NRC position uses twice 10% of maximum bubble pressure applied to 1/2 of submerged boundary per G.E. letter of 3/16/79.
4. Submerged Boundary Loads During Vent Clearing	Alternate position.	X		Agree to 24 psi below vent exit plane per G.E. 3/20/79 letter - Acceptable to NRC 9/13/79.
5. LOCA/SRV Submerged Drag Loads	Comply with NRC criteria.	X		Ring vortex model relegated to MK II follow-on plant program.
a,b LOCA Water Jet c. Lift Coefficient) d. Sensitivity Studies) e. Interference Effect)	Alternate position	X		Documentation submitted on LaSalle Docket DAR Appendix C, Rev. 6 of 10/79.
f. Zone of Influence Quencher Water Jet	Alternate position	X		Using 5 ft radius cylinder; Susquehanna DAR Section 3.0 shows no water jet effect at a distance less than this.
6. SRV Bubble Phasing 7. and Frequency	Alternate position	X		Documentation provided by Shoreham letter submitted SNRC- 374 of 3/30/79. S&L 12/13 presentation showed equivalence of lead plant approach to NRC request. Acceptable to NRC.

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<u>NRC OPEN ITEM 2/9/79</u>	<u>SHOREHAM POSITION</u>	<u>LEAD PLANT GENERIC</u>	<u>SHOREHAM UNIQUE</u>	<u>COMMENTS</u>
8. Chugging FSI Effects	NRC request acceptable.	X		Ongoing task in MK II Program as part of improved chugging load definition A.16. Shoreham evaluation for structures showed FSI insignificant.
9. Load Case 10	Alternate position	X		Based on 7/26/79 G.E. presentation, letter report 10/19/79 - acceptable to NRC if documentation accepted.

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