

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

0	1
7	8

REPORT SOURCE

L	6	0	5	0	-	0	3	4	6	7	1	1	0	9	7	9	8	1	1	2	1	7	9	9
60	61	DOCKET NUMBER					68	69	EVENT DATE					74	75	REPORT DATE					80			

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

09		SYSTEM CODE C H		11	CAUSE CODE B		12	CAUSE SUBCODE A		13	COMPONENT CODE I N S T R U					14	COMP. SUBCODE E		15	VALVE SUBCODE Z		16				
7	8	9	10		11		12		13					18		19		20								
17		LER/RO REPORT NUMBER		EVENT YEAR 7 9		21	22	SEQUENTIAL REPORT NO. 1 0 5		24	26	OCCURRENCE CODE /		27	REPORT TYPE T		30	REVISION NO. 0		32						
ACTION TAKEN F		18	FUTURE ACTION Z		19	EFFECT ON PLANT C		20	SHUTDOWN METHOD Z		21	HOURS 0 1 0		22	ATTACHMENT SUBMITTED Y		23	NPRD-4 FORM SUB. N		24	PRIME COMP. SUPPLIER N		25	COMPONENT MANUFACTURER C 6 5 0		26
33	34					25			36			37			40	41		42		43		44			47	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

7	8	9											80																							
FACILITY STATUS			% POWER			OTHER STATUS (30)										METHOD OF DISCOVERY										DISCOVERY DESCRIPTION (32)										80
1	5	G	(28)	10	11	12	(29)	13	NA										44	45	(31)	46	NRC Inspection Report 79-21										80			
7	8	9		10	11	12		13											44	45		46											80			

PERSONNEL EXPOSURES											
NUMBER			TYPE	DESCRIPTION							
1	7	0	0	0	37	Z	38	NA			39

7		8	9	11	12	80
		LOSS OF OR DAMAGE TO FACILITY				
TYPE		DESCRIPTION				
1	9	Z	(42)	NA	(43)	1398 277

7 8 9 10 80  
PUBLICITY  
ISSUED DESCRIPTION (45)  
2 0 N (44) NA 7911270 418 NRC USE ONLY

NRC USE ONLY

7911270 418

419-259-5000, Ext. 235<sup>8</sup>

DVR 79-163

NAME OF PREPARER

Dan Trautman

PHONE:

TOLEDO EDISON COMPANY  
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE  
SUPPLEMENTAL INFORMATION FOR LER NP-32-79-12

DATE OF EVENT: November 9, 1979

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Steam Generator low level trip setpoints found to be less conservative than assumed.

Conditions Prior to Occurrence: The unit was in Mode 5, with Power (MWT) = 0, and Load (Gross MWE) = 0.

Description of Occurrence: On November 9, 1979 at 1400 hours, during the review of NRC I&E Bulletin 79-21 response, it was discovered that our steam generator low level trip setpoints to Steam and Feedwater Rupture Control System (SFRCS) Channels 1 and 2 were less conservative than the limits of Technical Specification 3.3.2.2. The steam generator low level bistable trip setpoint was 23 inches + 2 inches. However, the re-evaluation of the method used to determine the setpoints based on actual versus indicated level found that the actual level would be less conservative than the minimum 20 inches of water above the lower tube sheet as required by Technical Specifications.

As a result, a Mode 3 restraint was placed on the unit until new values could be determined and adjustments to the bistable trip setpoint made. This occurrence is being reported in accordance with Technical Specification 6.9.1.8.h which requires prompt notification with a followup report within two weeks for an error in analysis, which would have permitted reactor operation in a manner less conservative than assumed.

Designation of Apparent Cause of Occurrence: The apparent cause of the occurrence was found to be an error in the analysis used in determining the steam generator low level bistable setpoints. The analysis failed to take into account the actual pressures and temperatures as seen by the steam generator during operation and in transients.

Analysis of Occurrence: A safety analysis is being performed and will be forwarded as a revision.

Corrective Action: On November 17, 1979, new steam generator low level bistable trip setpoints were established and adjustments were made. All four channels were proven operable by performance of ST 5031.14, "SFRCS Monthly Surveillance Test". The mode restraint relating this event was removed as of 1755 hours on November 19, 1979.

Failure Data: There have been no previous similar occurrences.