

PHILADELPHIA ELECTRIC COMPANY

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November 16, 1979

Mr. Boyce H. Grier, Director
Office of Inspection and Enforcement
Region I
United States Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

SUBJECT: Licensee Event Report Narrative Description

The following occurrence was reported to Mr. Greenman, Region I, Office of Inspection and Enforcement on November 2, 1979.

Reference:	Docket Number 50-277
Report No:	LER 2-79-49/1T
Report Date:	November 16, 1979
Occurrence Date:	October 5, 1979
Facility:	Peach Bottom Atomic Power Station R.D. 1, Delta, PA 17314

Technical Specification Reference:

Technical Specification 6.9.a(9) requires reporting "performance of structures, systems, or components that require remedial action or remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analysis in the safety analysis report ..."

Description of the Event:

As part of the inspection program which is being performed in response to IE Bulletin 79-02, an engineering review of vendor design revealed a non-conservative condition existing with four seismic supports associated with one inch Control Rod Drive piping. The affected piping runs from the Control Rod Drive Hydraulic Control Units in the Reactor Building to the

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containment boundary. Four of the six anchor bolts on each support were found to have a design safety factor less than 2.

Consequences of Event:

Postulating the design basis earthquake, failure of these seismic supports and subsequent failure of the supported piping would not prevent the Control Rods from scrambling.

Cause of Event:

The cause of this event is an engineering design deficiency.


Corrective Action:

Redesign of the above seismic supports and corrective actions were implemented within the seven day requirement (reference letter from B. H. Grier, Director, NRC, Region I to S. L. Daltroff, Vice President, Electric Production, PECO. dated October 24, 1979). The anchor bolt safety factors of the above supports are now greater than 2. Future redesign modification work is planned to raise the safety factors above 5.

Previous Similar Occurrences:

2-79-33/1T, 2-79-35/1T, 3-79-19/1T, 3-79-23/1T, 3-79-25/1T, 3-79-26/1T, 3-79-29/1T, 2-79-43/1T, 2-79-44/1T, 3-79-30/1T, 3-79-31/1T, 3-79-32/1T.

Yours truly,


M. J. Cooney
Superintendent
Generation Division-Nuclear

Attachment

cc: Director, NRC - Office of Inspection and Enforcement
Mr. Norman M. Haller, NRC - Office of Management &
Program Analysis

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