

LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N	Y	J	A	F	1	2	0	0	-	0	0	0	0	-	0	0	0	3	4	1	1	1	1	4			5
7	8	LICENSE CODE						14	LICENSE NUMBER										25	LICENSE TYPE					30	CAT		58	

0 1
7 8

REPORT SOURCE L 6 0 5 0 0 0 3 3 3 7 1 0 2 0 7 9 8 1 1 1 5 7 9 9
60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	2	SEE ATTACHMENT
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0	5	
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7 8 9

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SYSTEM CODE

S F (11)

CAUSE CODE

A (12)

CAUSE SUBCODE

C (13)

COMPONENT CODE

Z Z Z Z Z Z (14)

COMP. SUBCODE

Z (15)

VALVE SUBCODE

Z (16)

(17) LER RO REPORT NUMBER [EVENT YEAR] [21] [7] [22] [9] [23] [—] [24] [0] [25] [9] [26] [1] [27] [—] [28] [0] [29] [3] [30] [L] [31] [—] [32] [0]

ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS				ATTACHMENT SUBMITTED		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER										
[]	X	[]	Z	[]	Z	[]	Z	[]	0	[]	0	[]	Y	[]	[]	Z	[]	Z	[]	9	[]	9	[]	9	[]	9
33	(18)	34	(19)	35	(20)	36	(21)	37		40		41	(23)	42	(24)	43	(25)	44								

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0	SEE ATTACHMENT
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1	1	
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1	2	
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1	3	
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1	4	
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FACILITY STATUS (28) E % POWER (29) 100 OTHER STATUS (30) NA METHOD OF DISCOVERY (31) B DISCOVERY DESCRIPTION (32) SURVEILLANCE TEST

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35)

1 6 2 33 34 NA

7 8 9 10 11 44

LOCATION OF RELEASE (36)

NA

45 8

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)	Z	(38)	NA	(39)
7	8	9	11	12	13				

PERSONNEL INJURIES										
NUMBER				DESCRIPTION						
1	R	0	0	0	40	NA				

7 8 9 11 12

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LOSS OF OR DAMAGE TO FACILITY						(43)	7911200	508
		TYPE	DESCRIPTION					
1	9	Z	(42)	NA				
7	8	9	10					

							PUBLICITY		NRC USE ONLY
ISSUED		(44)	DESCRIPTION (45) NA						
2	0	N							
7	8	9	10						68 69 8

NAME OF PREPARER W. Verne Childs

PHONE: (315) 342-3840

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-091/03L-0

Page 1 of 1

During normal operation while conducting Operations Surveillance Test, F-ST-2B, titled "RHR Pump Operability Test (ISI)", RHR pump C failed to start properly. The testing was being conducted to verify operability of components in the A LPCI loop following work on pipe supports which had required that the loop be administratively declared inoperable on October 18, 1979 and reported in LER 79-081. Following the completion of the work described in LER 79-081, the components of the A LPCI loop were being tested to satisfy the requirements of Technical Specifications, Appendix A Paragraph 3.5.A. The other LPCI loop, the core spray systems and the emergency diesel generators were operable in accordance with the requirements of the Technical Specifications. Therefore, the event did not represent a significant hazard to the public health and safety.

When RHR pump C was started, the pump would trip following release of the control switch. Investigation revealed that during the time period in which the A LPCI loop had been declared inoperable, corrective and preventive maintenance had been performed on the C RHR pump suction valve (10-MOV-13C). During this maintenance, the valve operator open limit switches had been adjusted to assure that valve travel in the open direction was terminated prior to the valve reaching its backseat. While this open limit switch was properly adjusted, another set of limit switches associated with the pump and valve interlock was not adjusted. Failure to adjust this second set of limit switches, which are designated as spares in most cases, resulted in a pump trip as soon as the control switch was released since the interlock indicated that the valve was not fully open.

Following proper adjustment of the limit switches pump operability was demonstrated by satisfactory completion of F-ST-2B. In addition, maintenance personnel attended a training meeting which discussed the event and the necessity to check proper operation of all valve operator position rotors (limit switches) when adjusting any of the valve position rotors.

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