

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285

UNIT Fort Calhoun #1

DATE November 8, 1979

COMPLETED BY B. J. Hickie

TELEPHONE 402-536-4413

MONTH October, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>433.0</u>
2	<u>433.4</u>
3	<u>438.1</u>
4	<u>439.8</u>
5	<u>440.3</u>
6	<u>441.3</u>
7	<u>440.9</u>
8	<u>440.5</u>
9	<u>440.0</u>
10	<u>440.6</u>
11	<u>441.2</u>
12	<u>441.0</u>
13	<u>441.5</u>
14	<u>442.3</u>
15	<u>441.9</u>
16	<u>441.2</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>441.0</u>
18	<u>440.5</u>
19	<u>440.1</u>
20	<u>440.4</u>
21	<u>441.0</u>
22	<u>441.6</u>
23	<u>442.6</u>
24	<u>442.5</u>
25	<u>441.9</u>
26	<u>441.1</u>
27	<u>440.9</u>
28	<u>440.7</u>
29	<u>440.1</u>
30	<u>439.4</u>
31	<u>320.9</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE November 8, 1979
 COMPLETED BY B. J. Hickie
 TELEPHONE 402-536-4413

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: October, 1979
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

Notes

N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>745.0</u>	<u>7,296.0</u>	<u>53,473.0</u>
12. Number Of Hours Reactor Was Critical	<u>742.5</u>	<u>7,249.0</u>	<u>43,214.0</u>
13. Reactor Reserve Shutdown Hours	<u>3.1</u>	<u>3.1</u>	<u>1,139.1</u>
14. Hours Generator On-Line	<u>741.9</u>	<u>7,208.4</u>	<u>42,274.5</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,038,441.0</u>	<u>9,974,967.3</u>	<u>51,623,390.6</u>
17. Gross Electrical Energy Generated (MWH)	<u>341,942.0</u>	<u>3,322,001.9</u>	<u>17,126,263.6</u>
18. Net Electrical Energy Generated (MWH)	<u>325,204.5</u>	<u>3,158,584.0</u>	<u>16,179,989.7</u>
19. Unit Service Factor	<u>99.6</u>	<u>98.8</u>	<u>79.1</u>
20. Unit Availability Factor	<u>99.6</u>	<u>98.8</u>	<u>79.1</u>
21. Unit Capacity Factor (Using MDC Net)	<u>95.5</u>	<u>94.7</u>	<u>66.8</u>
22. Unit Capacity Factor (Using DER Net)	<u>95.5</u>	<u>94.7</u>	<u>66.2</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>1.2</u>	<u>4.4</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Refueling Outage, January 18, 8 Week Duration

25. If Shut Down At End Of Report Period, Estimated Date of Startup: November 7, 1979

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1979

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun #1
 DATE November 8, 1979
 COMPLETED BY B. J. Hickie
 TELEPHONE 402-536-4413

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
79-05	791031	S	3.1	B	1				Reactor shutdown to perform non-destructive testing of steam generator nozzle to piping welds pursuant to NRC IE Bulletin 79-14.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

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Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending October 31, 1979

1. Scheduled date for next refueling shutdown.
2. Scheduled date for restart following refueling.
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

January 18, 1980

March 1, 1980

Yes

- a. If answer is yes, what, in general, will these be?

Stretching power to 1500 MWth is planned in conjunction with the change in fuel supplier to Exxon.

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

- c. If no such review has taken place, when is it scheduled?

4. Scheduled date(s) for submitting proposed licensing action and support information.
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Stretch Power Application
- Site Related Information,
July, 1979 - Submitted
- Core Related Analysis
and Tech. Spec. Changes,
November, 1979

First use of Exxon fuel in Fort Calhoun..
Stretching power from 1420 MWth to 1500 MWth

6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>157</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>483</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

1985

Prepared by

JR Lfager

Date

November 1, 1979

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

October 1979
Monthly Operations Report

I. OPERATIONS SUMMARY

During the month of October, 1979, the Fort Calhoun Station operated at a nominal 100% power from October 1 thru October 30. On October 31 the plant started its decension in power in order to be in hot shutdown by midnight of October 31st. This scheduled shutdown is necessary for an inspection and ultrasonic test of the main feedwater line nozzle's weld connections. The inspection of seismic supports (Bulletin 79-02) continued.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
79-019	During normal rotation of component cooling water heat exchangers, inlet valve HCV-489A to C.C.W. heat exchanger AC-1A, would not operate from the remote switch in the control room. The valve was operated locally by use of the manual hand wheel, thus allowing normal operation of component cooling heat exchanger AC-1A as defined in Technical Specifications 2.3 and 2.46.

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which results in Operations Incidents and are not reported elsewhere in the report:

<u>Operations Incident</u>	<u>Deficiency</u>
OI-854 ST-ISI-CVCS-3	Boric Acid Pump showed high discharge pressure.
OI-852 ST-ISI-CC-3	Unable to locate results.
OI-848 ST-ESF-11	C/PIC-902 failed to reset.
OI-838 ST-FD-1	Zone 6 detector 15, 24 and 27 and Zone 7 detector 1 out of tollerance.
OI-847 ST-FD-1	Detector #2 - Zone #13.
OI-856 ST-FD-1	Was not completed as scheduled.
OI-859 ST-ESF-6	Diesel Generator D2 failed to start on secondary air.

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C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS (Continued)

<u>Surveillance Test No.</u>	<u>Description</u>
ST-ESP-11 (F.1)	Steam Generator Pressure Channel Check - Sigma C/PIC-902 failed to reset after trip test. Repaired per Maintenance Order #2035.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
DCR FC-79-9	Addition of loop load resistor to loop C/112-H - completed as designed.
SP-CPTP-12	Variable T _{avg} Test using center CEA/Results documented in test.
SP-RPS-5	Excure Detector Symmetric Offset Recalibration/ results are noted on special procedure.

E. RESULTS OF LEAK RATE TESTS

NONE

F. CHANGES IN PLANT OPERATING STAFF

NONE

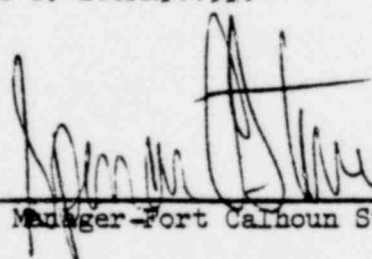
G. TRAINING

Training at Ft. Calhoun Station consisted of radiation protection refresher, waste disposal procedures for operators, general maintenance and H.P. as per OIE-79-19, and procedural compliance for plant staff. Central Maintenance personnel received radiation protection refresher, procedural compliance training, mask fit and physical refresher in lieu of the November outage at the plant.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

NONE

Approved By


Manager - Fort Calhoun Station

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Monthly Operations Report

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II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
1956	10-3-79	SI-2C Discharge Valve (HCV-2918) failed to close during ST-SI/CS-1.	Disassembled operator and air cylinder, cleaned parts, and reassembled. New parts on order. Valve is maintained in its accident (open) position.
2034	9-20-79	Raw Water Hanger RWH - 60 has a loose clevis. Reference IE Bulletin 79-14.	Hanger was tightened up satisfactorily.
2041	9-26-79	Seismic Restraint SIS-84 is missing a cotter pin. Reference IE Bulletin 79-14.	Missing cotter pin replaced. Snubber inspected and found to be in good shape.
2042	9-26-79	Seismic Restraint SIS-24 is missing cotter pins. Reference IE Bulletin 79-14.	Missing cotter pins replaced. Snubber inspected and found to be in good shape.
2043	9-26-79	Seismic Restraint SIS-26 needs to be realigned slightly so that it is horizontal. Reference IE Bulletin 79-14.	Retraint realigned to the correct angle.
2044	9-26-79	Seismic Restraints SIS-45 and SIS-46 need to be realigned slightly so that they are horizontal. Reference IE Bulletin 79-14.	Restraints realigned to the correct angle.
2092	10-2-79	Seismic Retraint SIS-29 needs to be realigned so that pipe hanger SIH-110 can assume its proper position. Reference IE Bulletin 79-14.	Restraint and hanger realigned to correct positions.
2107	11-1-79	Feed water pipe hanger FWH-114 has a loose U-bolt and the support needs to be centered in its guides. Reference IE Bulletin 79-14.	Hanger was centered and the U-bolt was tightened.
2108	9-28-79	Remove U-bolt from Safety Injection Pipe hanger SIH-55. U-bolt was installed per IE Bulletin 79-14.	U-bolt removed. U-bolt was installed when a preliminary analysis indicated that it was needed. Followup computer analysis showed that the U-bolt would restrict thermal expansion and thus was not needed.

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Monthly Operations Report

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II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
2130	10-3-79	Small leak at elbow of demineralized water flushing line on the suction side of "C" charging pump.	Weld repaired as per procedure MP-CH-1-2.
278	3-26-79	FE-532 Waste Gas Release orifice-- change flow orifice.	Completed
2225	10-15-79	Fire Penetration Repair	Temporary Repair Only.
2226	10-15-79	Fire Penetration Repair	Temporary Repair Only.
2227	10-15-79	Fire Penetration Repair	Temporary Repair Only.
2229	10-15-79	Fire Penetration Repair	Temporary Repair Only.
2224	10-15-79	Fire Penetration Repair	Temporary Repair Only.
2166	10-12-79	SI-93 & 94 replace "all thread" and channel iron with 24" certified U-bolts from stores.	Completed
2328	10-24-79	Diesel failed to start on Secondary Air. No air pressure downstream of regulator.	Found loose wire LOC for SAP-2 on TB. Found relay RS-1 and RS-2 loose in base.
2271	10-22-79	WD-708 - Stuck close	Repaired

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