

LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

0	1
7	8

REPORT SOURCE

L	6	0	5	0	0	0	3	3	3	7	1	0	3	1	7	9	8	1	1	0	9	7	9	9
60	61									68	69						74	75						80
DOCKET NUMBER											EVENT DATE						REPORT DATE							

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

7 8

SYSTEM CODE		CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE					COMP. SUBCODE	VALVE SUBCODE		
0	9	X X	B	A	S	U	P	O	R	T	B	Z
9	10	11	12	13	14	15	16	17	18	19	20	

LER RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.	
17		7 9		0 8 8		0 1		T		0	
21 22		23		24 25 26		27		28 29		30 31	
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED	
F 18		Z 19		Z 20		Z 21		0 0 0 22		Y 23	
33 34		35		36		37 38 39 40		41		42 43	
NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER							
N 24		A 25		S 4 2 0 26							
44		45		46		47					

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION

1 5 E 28 0 8 0 29 NA D 31 ARCHITECT ENGINEER 32

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)	Z	(38)	NA	(39)

1 9 Z 42 NA 109

PUBICITY		ISSUED		DESCRIPTION		NRC USE ONLY	
2	0	N	44	NA			

PHONE: (315) 342-3840

NRC USE ONLY

7911140376

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-088/01T-0

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The Architect-Engineer informed the plant staff that as the result of the Pipe Stress Reanalysis Program, two supports, H19-29 and H19-219, in the Fuel Pool Cooling System, were inoperable. Further analysis was performed which also showed that without these pipe supports, the associated piping became inoperable following a Design Basis Earthquake. These pipe supports were modified to accept the increased loading within the seven day time period specified in the 14 August 1979 letter lifting the Show Cause Order of 13 March 1979. These supports are now fully acceptable.

The plant staff was similarly notified that pipe support H66-42 in the Containment Vent and Purge System was inoperable and caused system inoperability when removed. The necessary corrective action was taken prior to final notification based upon preliminary worst case load information. Again this support is now fully acceptable.

Since fuel pool cooling requirements are currently low and since these supports were expeditiously upgraded, this event did not represent a significant hazard to the public health and safety.

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