



Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

November 2, 1979

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. NUCLEAR REGULATORY COMMISSION
Washington, D. C. 20555

Dear Mr. Denton:

DOCKET NOS. 50-266 AND 50-301
REQUEST FOR TECHNICAL SPECIFICATION CHANGE NO. 60
OPERATION AT 2000 PSIA PRIMARY SYSTEM PRESSURE
POINT BEACH NUCLEAR PLANT UNITS 1 AND 2

In accordance with Section 50.59 of 10 CFR 50, Wisconsin Electric Power Company (Licensee) hereby requests an amendment to Facility Operating Licenses DPR-24 and DPR-27 to incorporate changes in the Technical Specifications for the Point Beach Nuclear Plant, Units 1 and 2. The proposed changes are concerned with those revisions necessary for operation of Point Beach Nuclear Plant Unit 1 and Unit 2 at reduced primary system operating pressure.

Point Beach Units 1 and 2 were originally designed and operated at 2250 psia. Primary system operating pressure was reduced to 2000 psia during Unit 1 Cycle 3 and Unit 2 Cycle 1 in order to extend the time to clad collapse associated with fuel densification problems. Justification for operation at 2000 psia was contained in the Westinghouse report (1), WCAP-8151 "Fuel Densification Point Beach Nuclear Plant Unit 2 Low Pressure Analysis, June 1973". (2) Operation of Unit 1 at 2000 psia was the subject of the NRC Safety Evaluation for Licensing Amendment No. 3 to DPR-24 (Change No. 8 to Technical Specifications). For Unit 2 operation at 2000 psia primary system pressure was covered by the NRC Safety Evaluation (3) supporting Amendment 5 to License No. DPR-27 (Change No. 11 to the Technical Specifications). Return to a primary system operating pressure of 2250 psia for Unit 1 Cycle 4 and Unit 2 Cycle 3 was covered by NRC Safety Evaluations (4,5) supporting Amendment Nos. 14 and 18 to Licenses DPR-24/27 (Change Nos. 19 and 24 to the Technical Specifications) and Amendment No. 21 to License No. DPR-27, respectively. Return to 2250 psia resulted from concerns with the possibility of experiencing excessive internal fuel rod pressures, given the fuel rod prepressurization utilized at that time and the gas release model and internal pressure criteria then in effect.

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Operation at 2000 psia primary system pressure provides benefits unrelated to fuel rod clad collapse concerns. Stresses on important plant components are significantly reduced, e.g., on steam generator tubes, valve packing and gaskets, and charging pump discharge piping.

Wisconsin Electric is proceeding with plans to return to reduced primary system pressure operation. Fuel rod designs currently utilized in the Unit 1 and Unit 2 cores are based on the latest Westinghouse models (6) and criteria (7) and can accommodate operation at either 2250 psia or 2000 psia primary system pressure.

Westinghouse has reviewed the safety implications of operation of Unit 1 and Unit 2 at 2000 psia during the Unit 2 Cycle 6 and the Unit 1 Cycle 7 and 8 reload safety evaluations. Incidents analyzed and reported in the Point Beach Final Facility Description and Safety Analysis Report (FFDSAR) which could potentially be affected by reduced primary system pressure operation were reviewed. Results of later analyses were included and the applicability of previous results was verified. The methodology (8) described in WCAP-9273 "Westinghouse Reload Safety Evaluation Methodology" was employed in the evaluation. Assuming cycle burnups as discussed in the respective cycle Reload Safety Evaluations, and adherence to the Technical Specifications, the Point Beach Units can be operated at rated power at either 2250 psia or 2000 psia primary system operating pressure.

The recommended changes in the Technical Specifications attached cover operation at either 2250 psia or 2000 psia and include:

1. Page 15.2.3-2 to change the overtemperature ΔT setpoint, K_1 value, consistent with the previous value, which contemplated operation at 2250 psia.
2. Pages 15.3.1-1 through 15.3.1-3 are changed in format and Section 15.3.1.A.4.A and its basis are located on a new page, page 15.3.1-19, to eliminate confusion.
3. Page 15.3.1-2, Item 4 Operational Limitations, (page 15.3.1-19 of this revision) is changed to reflect operation at 2000 psia as well as to eliminate certain problems with interpretation and to provide consistent operating flexibility.
4. Page 15.4.3-1, Item a) and Basis are changed to reflect lower system leak testing requirements while operating at 2000 psia primary system pressure.

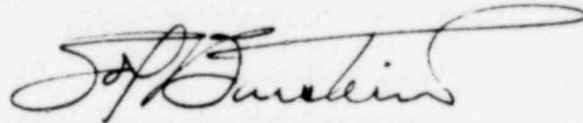
Licensee has reviewed the requirements of 10 CFR Part 170.22 regarding the schedule of fees for facility license amendments. It is our determination that the license amendment for DPR-24 for Point Beach Unit 1 should be classified as a Class III amendment in that a single issue having no significant hazards considerations is involved. The license amendment for DPR-27 for Point Beach Nuclear Plant Unit 2 is a duplicate of the Unit 1 application, deals with the identical subject and concerns, and is therefore classified as a Class I amendment. Accordingly, we have enclosed herewith check number 461322 for \$4,400 which is the full amount of the amendment fees.

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We have enclosed herewith three signed originals of the license amendment request. We shall provide under separate cover forty copies of the request. Attached to each copy of the request are proposed revised Technical Specification pages which reflect the changes discussed herein. Any questions you may have on this license amendment request should be directed to me. It is requested that your consideration and approval of this license amendment application be as prompt as possible. We believe that reducing the Unit 1 operating pressure coincident with or as soon as possible after Unit 1's return to power upon completion of the present refueling shutdown will minimize the hoop stress in the steam generator tubes.

Very truly yours,



Executive Vice President

Sol Burstein

Enclosures

Subscribed and sworn to before me

on November 2, 1979 Karolyn S. Schumann
Notary Public, State of Wisconsin

My Commission expires July 6, 1980.

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REFERENCES

1. WCAP-8151, "Fuel Densification Point Beach Nuclear Plant Unit No. 2 Low Pressure Analysis", (June, 1973).
2. "Safety Evaluation by the Directorate of Licensing, Amendment No. 3 to Facility Operating License No. DPR-24, (Change No. 8 to Appendix A of Technical Specifications), Wisconsin Michigan and Wisconsin Electric Power Company, Point Beach Nuclear Plant Unit No. 1, Docket No. 50-266", May 23, 1974, transmitted by letter; Dennis L. Zieman for Karl R. Goller to Mr. Sol Burstein, May 23, 1974.
3. "Safety Evaluation by the Directorate of Licensing, Supporting Amendment No. 5 to License No. DPR-27, Change No. 11 to the Technical Specifications, Wisconsin Electric Power Company and Wisconsin Michigan Power Company", September 30, 1974, transmitted by letter; Karl R. Goller to Mr. Sol Burstein, September 30, 1974.
4. "Safety Evaluation by the Office of Nuclear Reactor Regulation, Supporting Amendment Nos. 14 and 18 to Licenses DPR-24 and DPR-27 (Change Nos. 19 and 24 to the Technical Specifications, Wisconsin Electric Power Company, Wisconsin Michigan Power Company, Point Beach Nuclear Plant Units 1 and 2, Docket Nos. 50-266 and 50-301", December 24, 1975, transmitted by letter; Donald M. Elliot for George Lear to Mr. Sol Burstein, December 24, 1975.
5. "Safety Evaluation by the Office of Nuclear Reactor Regulation, Supporting Amendment No. 21 to License No. DPR-27, Wisconsin Electric Power Company Wisconsin Michigan Power Company, Point Beach Nuclear Plant Unit 2, Docket No. 50-301", March 22, 1976, transmitted by letter; George Lear to Mr. Sol Burstein, March 22, 1976.
6. WCAP-8785, "Improved Analytical Models used in Westinghouse Fuel Rod Design Computations", October 1976.
7. Westinghouse "Safety Analysis for the Revised Fuel Rod Internal Pressure Design Basis"; enclosure to Westinghouse letter NS-CE-1290, C. Eicheldinger to Dr. Denwood F. Ross, Jr., November 24, 1976.
8. WCAP-9273, "Westinghouse Reload Safety Evaluation Methodology", March 1978.