

## LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N Y J A F 1 2 0 0 - 0 0 0 0 - 0 0 0 3 4 1 1 1 1 4 5  
7 8 9 14 15 25 26 30 37 CAT 58

CON'T

0 1 REPORT SOURCE L 6 0 5 0 0 0 3 3 3 7 1 0 1 1 1 7 9 8 1 1 0 7 7 9 9  
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 SEE ATTACHMENT

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0 9 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE  
7 8 9 10 11 12 13 14 15 16  
S F X Z Z Z Z Z Z Z Z  
17 LER RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
7 9 -- 0 9 0 / 0 3 L -- 0  
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NRPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER  
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47  
X Z Z Z 0 0 0 0 Y N Z Z 9 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 SEE ATTACHMENT

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1 5 FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
E 0 9 9 NA Z NA

1 6 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
Z Z NA NA

1 7 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
0 0 0 Z NA

1 8 PERSONNEL INJURIES NUMBER DESCRIPTION (41)  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
0 0 0 NA

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
Z NA

2 0 PUBLICITY ISSUED DESCRIPTION (45)  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32  
N NA

NAME OF PREPARER W. Verne Childs

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7911130246

1319 354

NRC USE ONLY

POWER AUTHORITY OF THE STATE OF NEW YORK  
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-090/03L-0

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During normal operation as a result of hanger modifications being performed under the Pipe Stress Reanalysis Program, a cable associated with the HPCI system condensate storage tank suction valve (23-MOV-17) required relocation. This cable relocation disabled the referenced valve. Prior to the actual cable relocation 23-MOV-17 was verified in its open position and a temporary procedure was written instructing operations personnel to manually close the valve in the unlikely event the HPCI system should be required to operate and condensate storage tank level decreased to its minimum or the torus water level increased to 5" above normal. Either of these conditions (low storage tank level or high torus level) would ordinarily cause an automatic shifting of the HPCI suction from the condensate storage tanks to the torus. In addition to providing a written procedure to accomplish this transfer by manual action, an operator was assigned to this task with no concurrent duties. It should further be noted that the HPCI system suction transfer would only be required following operation at rated flow conditions for approximately 50 minutes. This time period would be more than adequate to allow manual transfer. While operation under the conditions stated above is considered to be a degraded mode in accordance with Technical Specifications, Appendix A, Paragraph 3.5.C, the HPCI system was continuously available to perform its intended safety function. Therefore, the event did not represent a significant hazard to the public health and safety.

Following rerouting of the cable, 23-MOV-17 was verified fully operable approximately 50 hours later, and returned to service.

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