

AEC

CONTRIBUTION FOR PART 50 DOCKET MATERIALS
(TEMPORARY FORM)

AL

CONTROL NO: 7085

FILE:

| | | | | | | | | |
|---|---------------|-----------|------------------------|-----------------------|-----------------------------|--|-----|-------|
| FROM: Metropolitan Edison Company Reading Pa. 19603 J. G. Miller | | | DATE OF DOC 9-10-73 | DATE REC'D 9-21-73 | LTR X | MEMO | RPT | OTHER |
| TO: A. Giambusso | | | ORIG 1 signed | CC | OTHER | SENT AEC PDR <u>X</u> SENT LOCAL PDR <u>X</u> | | |
| CLASS | UNCLASS XX | PROP INFO | INPUT | NO CYS REC'D 1 | DOCKET NO: <u>50-289</u> | | | |

DESCRIPTION:

Ltr re ACRS 8-14-73 ltr..... Trans the following:

ENCLOSURES:

Post Accident Instrumentation- instrumentation for determining the course of potentially serious accident

ACKNOWLEDGED
DO NOT REMOVE

(1 cys rec'd)

PLANT NAME: Three Mile Island Unit-1

FOR ACTION/INFORMATION

9-21-73 GC

| | | | |
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| CASE | KNIGHT | BALLARD | LEE (L) | MCDONALD |
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| BOYD | SHAO | | SERVICE (L) | |
| ✓MOORE (L)(EWR) | ✓STELLO | ENVIRO | SHEPPARD (E) | INFO |
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| 16 - CYS ACRS HOLDING | 1-GERALD ULRIKSON...ORNL | |

7911070 700



METROPOLITAN EDISON COMPANY

POST OFFICE BOX 542 READING, PENNSYLVANIA 19603

TELEPHONE 215 - 929-3601

September 10, 1973

Mr. A. Giambusso
Deputy Director for Reactor Projects
United States Atomic Energy Commission
Washington, DC 20545

SUBJECT: THREE MILE ISLAND NUCLEAR STATION
UNIT 1
DOCKET NO. 50-289

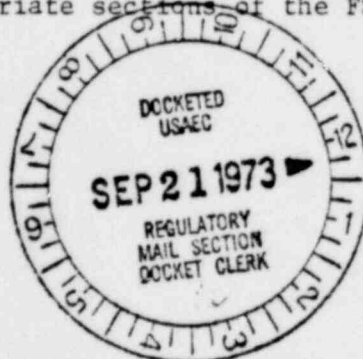
Dear Mr. Giambusso:

The August 14, 1973 letter written by Mr. H. G. Mangelsdorf of the ACRS Committee to the Honorable Dixy Lee Ray regarding the subject reactor, suggested that the Applicant assure himself that instrumentation for determining the course of potentially serious accidents, on a time scale that will permit appropriate emergency action, is provided at the station. In addition, Applicant was asked to assure that appropriate calibration methods and calculated bases for interpreting instrument responses are available.

We have reviewed the situation and we are satisfied that:

- (1) Adequate instrumentation has been provided;
- (2) Adequate calibration methods and procedures are available; and,
- (3) Calculated bases for interpreting instrument responses are available.

Enclosed is a listing of the instrumentation available to follow the course of an accident. The list includes references to the appropriate sections of the FSAR and the Technical Specifications.



Very truly yours,

J. G. Miller
Vice President

as
Enclosure

CC: Mr. W. A. Verrochi

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POST ACCIDENT INSTRUMENTATION

Received with dated 9-10-77

1. Following an accident, the nuclear instrumentation will be available to confirm that the reactor is tripped. (see FSAR Section 7)

2. The balance of the reactor protection system instrumentation located outside of the primary shield will also be available, this includes the measurement of reactor coolant pressure and reactor coolant temperature. (see FSAR Section 7)

3. The qualification testing of the reactor protection system is discussed in Babcock & Wilcox Topical Report BAW 10003. Rev. 2.

4. We have additional post accident monitoring instrumentation which display, record and alarm in the control room. This instrumentation includes the following:

A. The reactor building radiation level, monitored by a high range gamma detector (channel RM-G8) which has a range of 1r/hr. to 1×10^6 r/hr. (see FSAR Section 11.4)

B. The reactor building pressure with a range of 0-100 PSIG. (see FSAR Section 7.4.7)

C. The reactor building sump level.

These three measurements are qualified to operate following an accident.

5. There are 12 resistance temperature detectors located in different areas of the reactor building. These resistance temperature detectors are expected to provide information on reactor building temperature after an accident.

6. In addition, we have outside the reactor building the following instruments which indicate in the control room:

A. High Pressure Injection flow (see FSAR Figure 6.2)

B. Low Pressure Injection flow (see FSAR Figure 6.2)

C. Low Pressure Injection pump suction temperature (see FSAR Figure 6.2)

D. Decay heat closed cycle temperature

Table 3.5-1 (Instrument Operating Conditions) of Technical Specification 3.5.1 lists the requirements for reactor startup or critical operation. Table 4.1-1 (Instrument Surveillance Requirements) of Tech. Spec. 4.1 indicates the calibration schedule for the instrumentation mentioned above.