

1. Newmark
Illinois 61891

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U.S. ATOMIC ENERGY COMMISSION

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Trip Report:

Site: 3-Mile Island Nuclear Station Unit 1
Metropolitan-Edison Company
Harrisburg, Pennsylvania

Date of Visit: 7 October 1970

Attendees: AEC

D. F. Ross -- DRL
D. M. Hunnicutt -- Compliance
R. Brown -- Compliance

AEC Consultants

N. M. Newmark -- Newmark & Associates
W. J. Hall -- Newmark & Associates

Gilbert Associates, Inc.

H. Lorenz
D. Croneberger
C. Chen
L. Patterson

Metropolitan-Edison Company

R. M. Klingaman
J. L. C. Bachofer, Jr.
D. H. Reppert



The morning was devoted to an inspection tour of the facility. Those principal items examined included the containment structure, auxiliary and turbine building, and the pump house. Also examined were such items as piping, equipment mounting, tendon gallery, cable trays, etc.

In the afternoon a technical meeting was held in which the following items were discussed:

(a) Clarification was provided to Gilbert Associates personnel regarding several statements made in a report to AEC Regulatory Staff, "Adequacy of the Structural Criteria for Three Mile Island Nuclear Station Unit 1" by N. M. Newmark and W. J. Hall, dated December 1967.

(b) Most of the time was spent in reviewing in detail the questions of seismic concern contained in the list of questions dated 16 September 1970, as transmitted from Dr. Peter A. Morris, Director of Division of Reactor Licens-

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ing, U. S. Atomic Energy Commission, to the Metropolitan-Edison Company, Attention: Mr. J. G. Miller. As a result of this review of the questions and status of the analyses and design, it was believed that the final answers to these questions will be simplified, yet at the same time contain considerably more information of engineering significance than would otherwise have been the case.

Drs. Newmark and Hall will undertake review of the Gilbert Associates, Inc. topical report on piping in conjunction with the procedures of Biggs and Roesset, as applicable, by the time the answers to the questions are submitted in order to provide a basis for evaluation of the piping design. We shall also review requests BAW-10000 Part 1 and Part 2 dealing with Reactor Internals and Fuel Assembly Analyses. Comments on these reviews will be transmitted to DRS/DRL AEC in the near future if it appears necessary.

In studying the "aircraft hardening" that has gone into the design, and as a result of the inspection of the facility, it would be helpful if the project staff could obtain additional information about the properties of the packing materials placed between the edges of the Control Room floor and the walls, and its expected behavior under seismic response in terms of shielding the floor from high-intensity seismic motions.

The air coolers were noted to be items of significance from a seismic design standpoint and it is assumed that additional information on this aspect of the design will be transmitted to us in the near future by DRS.

N. M. Newmark

N. M. Newmark

clc

cc: W. J. Hall

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