

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-289

Unit TMI-1

Date 5-3-78

Completed By D. G. Mitchell

Telephone (215)929-3601 Ext. 169

MONTH April

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>-5</u>
2	<u>-5</u>
3	<u>-6</u>
4	<u>-6</u>
5	<u>-5</u>
6	<u>-6</u>
7	<u>-5</u>
8	<u>-5</u>
9	<u>-6</u>
10	<u>-6</u>
11	<u>-6</u>
12	<u>-5</u>
13	<u>-5</u>
14	<u>-5</u>
15	<u>-5</u>
16	<u>-5</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>-6</u>
18	<u>-9</u>
19	<u>-9</u>
20	<u>-9</u>
21	<u>-10</u>
22	<u>-9</u>
23	<u>-9</u>
24	<u>-10</u>
25	<u>-13</u>
26	<u>-32</u>
27	<u>-35</u>
28	<u>-38</u>
29	<u>-39</u>
30	<u>-39</u>
31	<u>-</u>

1587 030

7910310 688

OPERATING DATA REPORT

Docket No. 50-289

Date 5-3-78

Completed By D. G. Mitchell

Telephone 215-929-3601 Ext.

OPERATING STATUS

1. Unit Name: Three Mile Island Unit 1
2. Reporting Period: April, 1978
3. Licensed Thermal Power (MWt): 535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Max. Dependable Capacity (Gross MWe): 840
7. Max. Dependable Capacity (Net MWe): 792
8. If Changes Occur in Capacity Ratings (Items No. 3 through 7) Since Last Report, Give Reasons:

9. Power Level to which Restricted. If Any (Net MWe): 2311 Mwt
10. Reasons for Restrictions, If Any: NRC must review letter from B&W on Small Break LOCA Analysis.

	This Month	Yr.-to-Date	Cumulative
11. Hours in Reporting Period	719	2879	32,088
12. No. of Hours Reactor was Critical	47	1,871	24,929.3
13. Reactor Reserve Shutdown Hours	0	0	838.5
14. Hours Generator On-Line	0	1824	24,421.5
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	4,469,915	59,863,450
17. Gross Elect. Energy Generated (MWH)	0	1,491,977	19,985,779
18. Net Electrical Energy Generated (MWH)	0	1,392,271	18,710,670
19. Unit Service Factor	0	63.4	76.1
20. Unit Availability Factor	0	63.4	76.1
21. Unit Capacity Factor (Using MDC Net)	-	61.1	73.6
22. Unit Capacity Factor (Using DER Net)	-	59.0	71.2
23. Unit Forced Outage Rate	0	0	5.2
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down at End of Report Period, Estimated Date of Startup: May 2, 1978

26. Units In Test Status (Prior to Commercial Operation):	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

1589 031

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April 1978Docket No. 50-289Unit Name TMI-1Date 5-3-78Completed By D. G. MitchellTelephone 215-929-3601 Ext. 169

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report Number	System Code ⁴	Component Code ⁵	Cause and Corrective Action to Prevent Recurrence
4	4-1-78	S	719	C	1				

¹F: Forced
S: Scheduled

²Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensee Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File
(NUREG-0161)

⁵Exhibit 1 - Same Source

1589 032

OPERATING SUMMARY

The Unit remained shutdown for the entire month for the 1978 Refueling Outage.
The Refueling Outage Milestones which were completed this month are as follows:

04/04/78	Replaced Reactor Vessel Head
04/10/78	Completed Local Leak Rate Testing
04/16/78	Completed Integrated Leak Rate Testing
04/24/78	Vacuum on Main Condenser

1589 033

MAINTENANCE SUMMARY

The refueling outage continued throughout the month with breakers being closed on May 2, 1978. The refueling outage work performed included:

- (1) Fuel transfer canal drained and decontaminated
- (2) Upper plenum installed
- (3) Reactor vessel head installed
- (4) New incores installed
- (5) CRDM's and APSR's coupled
- (6) Reactor vessel head tensioned
- (7) Reactor vessel head cables connected
- (8) Reactor vessel head fans installed

Other maintenance activities performed during the refueling outage are outlined below:

Local leak rate testing of containment isolation valves was performed satisfactorily with the total leakage below the acceptance level. Integrated leak rate test of the reactor building was performed with satisfactory results.

Eddy current inspection of OTSG "A" and "B" resulted in the tube plugging of one (1) tube in OTSG "A" and one (1) tube in OTSG "B" as preventive measures only.

RC-P-1B #2 and #3 seal inspection was completed with the installation of these seals and the coupling of motor to pump.

RC-P-1C #1, #2, and #3 seals were installed and the motor and pump coupled. Attempts to rotate the pump and motor by hand, after the satisfactory completion of the Integrated Leak Rate test of the reactor building, failed. The RC-P-1C was again disassembled. A bolt (5/8" diameter x 4" long) was found on top of the pump radial bearing. The following precautions were taken: (1) pump shaft was lifted approximately 1" off of backseat to verify that no other foreign material was lodged in the pump cavity, (2) the shaft alley was flushed with demineralized water. The pump was assembled with no problems noted.

The Decay Heat pumps 1A and 1B were disassembled, one at a time, to replace the pump shafts. Disassembly of pumps included:

- (1) Uncoupling of motor from pump
- (2) Disassembly of pump parts
- (3) Inspection and cleanup of pump parts
- (4) Replacement of pump shaft and worn parts
- (5) Reassembly of pump parts
- (6) Modification to pump stuffing box cooling water lines and drain water lines
- (7) Alignment of motor to pump
- (8) Installation of coupling
- (9) Satisfactory testing of pump

The pump shaft replacement completed an NRC commitment.

1589 034

CA-V-13 was inspected, after an initial leak rate test of the valve passed satisfactorily, to fulfill a NRC commitment. The valve was disassembled, valve parts inspected/cleaned, worn parts replaced, and reassembled. After the valve was reassembled, another leak rate test was performed satisfactorily.

The snubber test program was performed satisfactorily during the month. Snubber work included:

Removal, testing, repair/adjustment of bleed rates, installation, and Tech. Spec. surveillance

MU-V-74A&B repacks were performed to complete an NRC commitment. Valves were repacked and tested satisfactorily.

1589 035

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-289

Unit TMI-1

Date 5-3-78

Completed By D. G. Mitchell

Telephone (215)929-3601 Ext. 169

MONTH April

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1	<u>-5</u>
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4	<u>-6</u>
5	<u>-5</u>
6	<u>-6</u>
7	<u>-5</u>
8	<u>-5</u>
9	<u>-6</u>
10	<u>-6</u>
11	<u>-6</u>
12	<u>-5</u>
13	<u>-5</u>
14	<u>-5</u>
15	<u>-5</u>
16	<u>-5</u>

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

17	<u>-6</u>
18	<u>-9</u>
19	<u>-9</u>
20	<u>-9</u>
21	<u>-10</u>
22	<u>-9</u>
23	<u>-9</u>
24	<u>-10</u>
25	<u>-13</u>
26	<u>-32</u>
27	<u>-35</u>
28	<u>-38</u>
29	<u>-39</u>
30	<u>-39</u>
31	<u>-</u>

1587 036

OPERATING DATA REPORT

Docket No. 50-289

Date 5-3-78

Completed By. D. G. Mitchell

Telephone 215-929-3601 Ext. 1

OPERATING STATUS

1. Unit Name: Three Mile Island Unit 1
2. Reporting Period: April, 1978
3. Licensed Thermal Power (MWt): 535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Max. Dependable Capacity (Gross MWe): 840
7. Max. Dependable Capacity (Net MWe): 792
8. If Changes Occur in Capacity Ratings (Items No. 3 through 7) Since Last Report, Give Reasons:

9. Power Level to which Restricted. If Any (Net MWe): 2311 Mwt
10. Reasons for Restrictions, If Any: NRC must review letter from B&W on Small Break LOCA Analysis.

	<u>This Month</u>	<u>Yr.-to-Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>719</u>	<u>2879</u>	<u>32,088</u>
12. No. of Hours Reactor was Critical	<u>47</u>	<u>1,871</u>	<u>24,929.3</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>838.5</u>
14. Hours Generator On-Line	<u>0</u>	<u>1824</u>	<u>24,421.5</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>4,469,915</u>	<u>59,863,450</u>
17. Gross Elect. Energy Generated (MWH)	<u>0</u>	<u>1,491,977</u>	<u>19,985,779</u>
18. Net Electrical Energy Generated (MWH)	<u>0</u>	<u>1,392,271</u>	<u>18,710,670</u>
19. Unit Service Factor	<u>0</u>	<u>63.4</u>	<u>76.1</u>
20. Unit Availability Factor	<u>0</u>	<u>63.4</u>	<u>76.1</u>
21. Unit Capacity Factor (Using MDC Net)	<u>-</u>	<u>61.1</u>	<u>73.6</u>
22. Unit Capacity Factor (Using DER Net)	<u>-</u>	<u>59.0</u>	<u>71.2</u>
23. Unit Forced Outage Rate	<u>0</u>	<u>0</u>	<u>5.2</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down at End of Report Period, Estimated Date of Startup: May 2, 1978

26. Units In Test Status (Prior to Commercial Operation):	FORECAST	ACHIEVED
INITIAL CRITICALITY	<u> </u>	<u> </u>
INITIAL ELECTRICITY	<u> </u>	<u> </u>
COMMERCIAL OPERATION	<u> </u>	<u> </u>

1589 037

OPERATING SUMMARY

The Unit remained shutdown for the entire month for the 1978 Refueling Outage.
The Refueling Outage Milestones which were completed this month are as follows:

04/04	Replaced Reactor Vessel Head
04/10/78	Completed Local Leak Rate Testing
04/16/78	Completed Integrated Leak Rate Testing
04/24/78	Vacuum on Main Condenser

1589 038

UNIT SHUTDOWNS AND POWER REDUCTIONS

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1589 039

MAINTENANCE SUMMARY

The refueling outage continued throughout the month with breakers being closed on May 2, 1978. The refueling outage work performed included:

- (1) Fuel transfer canal drained and decontaminated
- (2) Upper plenum installed
- (3) Reactor vessel head installed
- (4) New incores installed
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Other maintenance activities performed during the refueling outage are outlined below:

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- (4) Replacement of pump shaft and worn parts
- (5) Reassembly of pump parts
- (6) Modification to pump stuffing box cooling water lines and drain water lines
- (7) Alignment of motor to pump
- (8) Installation of coupling
- (9) Satisfactory testing of pump

The pump shaft replacement completed an NRC commitment.

1589 040

CA-V-13 was inspected, after an initial leak rate test of the valve passed satisfactorily, to fulfill a NRC commitment. The valve was disassembled, valve parts inspected/cleaned, worn parts replaced, and reassembled. After the valve was reassembled, another leak rate test was performed satisfactorily.

The snubber test program was performed satisfactorily during the month. Snubber work included:

Removal, testing, repair/adjustment of bleed rates, installation, and Tech. Spec. surveillance

MU-V-74A&B repacks were performed to complete an NRC commitment. Valves were repacked and tested satisfactorily.

1589 041