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TO: R.W. Reid

FROM: Metropolitan Edison Co.
Reading, Pa.
R.C. Arnold

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DESCRIPTION

ENCLOSURE

Ltr. re. our 3-9-76 ltr.....
Review of Reactor Coolant System Pressure temperature
limits contained in the Tech. Spec.....

(1 Sigend Cy. Received)

POOR ORIGINAL

PLANT NAME: Three Mile Island # 1

SAFETY

FOR ACTION INFORMATION

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5016

7910290 641



METROPOLITAN EDISON COMPANY

POST OFFICE BOX 542 READING, PENNSYLVANIA 19603

TELEPHONE 215-929-8601

May 14, 1976
GQL 0714



Director of Nuclear Reactor Regulation
Attn: Mr. R. W. Reid, Chief
Operating Reactors Branch No. 4
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Reid:

Three Mile Island Nuclear Station Unit 1 (TMI-1)
Docket No. 50-289
Operating License No. DPR-50

Your letter of March 9, 1976 requested Met-Ed to review reactor coolant system pressure-temperature limits contained in technical specifications for TMI-1 to determine if they are in full compliance with Appendix G, 10CFR part 50.

The present pressurization limits and heatup and cooldown rates contained in Technical Specification 3.1.1 are adequate and within the requirements of Appendix G to 10CFR50 for heatup, cooldown and inservice leak and hydrostatic tests up to two effective full power years of power operation. The justification for the present limitations is a comparison of technical specification figures 3.1-1 and 3.1-2 to the typical curves contained in B&W Topical Report BAW-10046P, "Methods of Compliance with Fracture Toughness and Operational Requirements of 10CFR50, Appendix G". Figures 5-8, 5-9, and 5-10 of BAW-10046P indicate that the technical specification figures are adequately conservative. These figures in BAW-10046P are based on conservative material properties, heatup and cooldown rates of 100°F/hr and are applicable for up to five effective full power years (EFPY). The limitations in the technical specification figures are based on heatup and cooldown rates of 50°F/hr or less at the lower temperatures of interest, and are applicable for only two EFPY. The temperature limitation on reactor criticality contained in technical specification 3.1.3.1(525°F) provides a substantial margin from the requirements of Appendix G as shown by Figure 5-7 of BAW-10046P.

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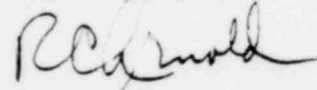
Director of Nuclear Reactor Regulation
Attn: Mr. R. W. Reid, Chief

GQL 0714
May 14, 1976

- 2 -

One surveillance capsule from TMI-1 is to be tested and analyzed as part of the Reactor Vessel Surveillance Program. Following evaluation of the results of the surveillance specimen tests, the limitations of technical specification 3.1.1 will be revised in accordance with the requirements of Appendix G, and TMI Technical Specification 3.1.2.4.

Very truly yours,



R. C. Arnold
Vice President

RCA:JJM:rk

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02.0016.0001.0001.02

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