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(TEMPORARY FORM)

CONTROL NO: 7019
FILE: INCIDENT REPORT FILE

FROM: Metropolitan Edison Co. Reading, Penna. R.C. Arnold		DATE OF DOC 6-27-75	DATE REC'D 7-1-75	LTR XXX	TWX	RPT	OTHER
TO: NRC		ORIG 1 Signed	CC	OTHER	SENT AEC PDR <u>XXXXX</u> SENT LOCAL PDR <u>XXX</u>		
CLASS	UNCLASS XXXX	PROP INFO	INPUT	NO CYS REC'D 1	DOCKET NO: 50-289		

DESCRIPTION:

A/O # 75-19, on 6-18-75, concerning Out of Spec.
ification Reactor Coolant Pressure Trip
Setpoint.....

(1 cy. ltr. Rec'd)

ENCLOSURES:

POOR ORIGINAL

PLANT NAME: Three Mile Island # 1

FOR ACTION/INFORMATION

VCR 7-3-75

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METROPOLITAN EDISON COMPANY SUBSIDIARY OF GENERAL PUBLIC UTILITIES CORPORATION

POST OFFICE BOX 542 READING, PENNSYLVANIA 19603

TELEPHONE 215 - 929-3601

June 27, 1975
GQL 1213

Director
Directorate of Reactor Licensing
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

Operating License DPR-50
Docket #50-289

In accordance with Technical Specifications for the Three Mile Island Nuclear Station Unit I, we are reporting the following abnormal occurrence.

- (1) Report Number: AO 50-289/75-19
- (2a) Report Date: June 27, 1975
- (2b) Occurrence Date: June 18, 1975
- (3) Facility: Three Mile Island Nuclear Station Unit I
- (4) Identification of Occurrence:

Title: Out of Specification Reactor Coolant Pressure Trip
Setpoint

Type: An abnormal occurrence as defined by Technical Specifications, paragraph 1.8a, in that the out of specification setpoint for the Reactor Protection System (RPS) was less conservative than the limiting setting established in the Technical Specifications.

- (5) Conditions Prior to Occurrence:

Power: Core: 99%
Elect: 850 MWe Gross
RC Flow: 140.68×10^6 lb/hr
RC Pressure: 2155 psig
RC Temp: 579°F
PRZR level: 240 inches
PRZR temp: 655°F

- (6) Description of Occurrence:

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On June 18, 1975, the variable low reactor coolant system pressure trip setpoints for all four RPS channels were being tested to determine instrument drift as specified in Abnormal Occurrence Report AO 50-289/75-16. Test results showed that the trip setpoint for one RPS channel was less conservative than required by the Technical Specifications.

The required and actual trip point for the channel out of specification was as follows:

<u>Channel</u>	Table 2.3-1 Trip Setpoint at 590°F (16.25T out -7756)	Actual Trip Point <u>Measured at the Bistable</u>
C	1831.5	1831.2

The out-of-specification channel was recalibrated and retested, and the resulting trip setpoint was verified as being within Technical Specification limits.

The variable low pressure trip setpoints for the remaining 3 RPS channels were tested and found to be within Technical Specification limits.

(7) Designation of Apparent Cause of Occurrence:

The apparent cause of the occurrence has been determined to be material, in that the RPS channel C trip setpoint was out of specification because the temperature-to-pressure signal converter was out of calibration due to minor instrument drift.

(8) Analysis of Occurrence:

It has been determined that the out of specification trip setpoint did not constitute a threat to the health and safety of the public in that:

- a. The setpoints for the other 3 RPS channels were within Technical Specification limits and these channels were fully capable of performing their protective function, and
- b. The setpoint error for channel C (0.3 psig level low) was more conservative than the 30 psig measurement error assumed in the accident safety analysis, so that channel C was fully capable of performing its protective function.

(9) Corrective Actions:

In addition to the corrective actions outlined above the variable low pressure trip feature for all four RPS channels will be tested each week until the cause of the instrument drift has been determined and corrected.

The Plant Operations Review Committee and the Station Superintendent have reviewed and approved the above listed corrective actions and have taken steps to ensure their implementation.

(10) Failure Data:

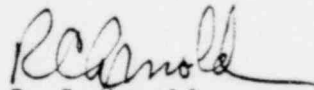
a. Previous Failures:

see Abnormal Occurrence reports AO 50-289/74-24, AO 50-289/75-15 and AO 50-289/75-16.

b. Equipment Identification:

Bailey System 880 Reactor Protection System

Sincerely,


R. C. Arnold
Vice President

RCA:CWS:lw

cc: Office of Inspection and Enforcement,
Region I

File 7.7.3.5.1
20.1.1

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