

CONTROL BLOCK:

							(1)
--	--	--	--	--	--	--	-----

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N	Y	J	A	F	1	2	0	0	-	0	0	0	0	-	0	0	0	3	4	1	1	1	1	4			5
7	8	9					14	15											25	26							57	CAT	58
LICENSEE CODE		LICENSE NUMBER																LICENSE TYPE											

CON'T

REPORT SOURCE L 6 0 5 0 0 0 3 3 3 7 0 9 2 2 7 9 8 1 0 0 5 7 9 9

60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | Please See Attachment

03 | Page

0	4
---	---

05 | Page

0	6	
---	---	--

0	7	
---	---	--

0	8	
---	---	--

7 8

SYSTEM CODE
W B (11)

CAUSE CODE
B (12)

CAUSE SUBCODE
C (13)

COMPONENT CODE
S U P P O R T (14)

COMP. SUBCODE
B (15)

VALVE SUBCODE
Z (16)

LER RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION	
17		7 9		0 7 4		0 1		T		0	
21 22		23		24 25 26		27		28 29		30 31 32	
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED	
33		34		35		36		37 38 39 40		41	
F 18		Z 19		Z 20		Z 21		0 0 0 0 22		Y 23	
24		25		26		27		28		29	
NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER							
42		43		44		45		46		47	
N 24		A 25		S 4		Z 20				26	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0		Please See Attachment
---	---	--	-----------------------

[illegible]

1	2	
---	---	--

1	3	
---	---	--

1	4	
---	---	--

FACILITY STATUS (1) 5 (E) (28) % POWER (0) 9 (2) (29) OTHER STATUS (30) NA METHOD OF DISCOVERY (D) (31) A/E DISCOVERY DESCRIPTION (32)

ACTIVITY CONTENT
RELEASED OF RELEASE

1 6 Z 33 Z 34

AMOUNT OF ACTIVITY (35)

NA

LOCATION OF RELEASE (36)

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	37	Z	39	NA	

PERSONNEL INJURIES		7910010 310		80
NUMBER	DESCRIPTION	(41)		
1 2	0 0 0	(40)	NA	

8		9		10		11		12		
LOSS OF OR DAMAGE TO FACILITY						(43)				
TYPE		DESCRIPTION								
1	9	Z	(42)	NA						1138 146

20		N		44	DESCRIPTION	45	NA	NRC USE ONLY									
----	--	---	--	----	-------------	----	----	--------------	--	--	--	--	--	--	--	--	--

NAME OF PREPARER W. Verne Childs

PHONE:

68 69
315-342-3840

NRC USE ONLY

000 017.936

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-074/OIT-0

Page 1 of 1

During normal operation as a result of the Pipe Stress Reanalysis being conducted by the Architect Engineer, the plant staff was notified that a pipe support associated with the Service Water System return line from the Reactor Building Cooling System was considered inoperable in accordance with the requirements of the August 14, 1979 letter lifting the Show Cause Order of March 13, 1979.

As noted above the pipe support was considered inoperable as set forth in Paragraph IV 4 of the August 14, 1979 letter. However, Technical Specifications do not include the Service Water System therefore, repair or modification of the pipe support was required within seven (7) days. Since the hanger was repaired within the allowable time frame and the affected system would have remained operable during and following a seismic event of the magnitude presented in the letter lifting the Show Cause Order, the event did not represent a significant safety hazard to the public health and safety.

As noted above, repair and modification of the pipe support was completed within the prescribed time frame and the pipe support is now considered fully operable.

1138 147