

## LICENSEE EVENT REPORT

U. S. NUCLEAR REGULATORY COMMISSION

**POOR ORIGINAL**CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7 8 9 14 15 25 26 30 57 58 59  
[0][1] N C B E P 2 [2] 0 0 - 0 0 0 0 0 - 0 0 [3] 4 1 1 1 1 [4] [5]  
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

CONT

7 8 60 61 68 69 74 75 80  
[0][1] REPORT SOURCE L [6] 0 5 0 - 0 3 2 4 [7] 0 8 3 0 7 9 [8] 0 9 2 5 7 9 [9]  
DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

[0][2] During normal plant operation, the primary containment ambient temperature exceeded  
[0][3] 135°F (135.176°F). The temperatures over the previous several days had been 133-134°F  
[0][4] with all available drywell coolers operating. Drywell leakage had increased to approx-  
[0][5] imately 2 gpm and airborne particulate activity was higher than normal, indicating a  
[0][6] small steam leak. The heat load on the RBCCW system was high due to high service  
[0][7] water injection temperature, RWCU rejecting to the hotwell, and supplying the 20 gpm  
[0][8] concentrator in radwaste. Technical Specification 3.6.1.6, 6.9.1.9b

7 8 9 10 11 12 13 18 19 20 21 22 23 24 26 27 28 29 30 31 32 33 34 35 36 37 40 41 42 43 44 47  
[0][9] SYSTEM CODE [S][A] [11] CAUSE CODE [E] [12] CAUSE SUBCODE [B] [13] COMPONENT CODE [V][A][L][V][E][X] [14] COMP. SUBCODE [C] [15] VALVE SUBCODE [A] [16]  
[17] LER/RO REPORT NUMBER [7][9] [0][8][0] [0][3] [L] [0]  
EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.  
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NRPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER  
[X] [18] [Z] [19] [Z] [20] [Z] [21] [0][0][0][0] [Y] [23] [N] [24] [A] [25] [A][3][9][3] [26]

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

[1][0] RWCU reject to the hotwell was decreased from 130 gpm to 100 gpm and service water flow  
[1][1] to the RBCCW heat exchangers was increased. This lowered the RBCCW outlet temperature  
[1][2] from 99°F to 89°F and drywell ambient temperature was lowered from 135.176°F to 134°F.  
[1][3] During the recent outage, it was discovered that the King pin cover on the feedwater  
[1][4] check valve B21-F010B was leaking and had a steam cut. This leak was repaired and

(CONT)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
FACILITY STATUS [E] [28] % POWER [0][9][4] [29] OTHER STATUS [30] METHOD OF DISCOVERY [A] [31] DISCOVERY DESCRIPTION [32]  
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY [35] LOCATION OF RELEASE [36]  
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION [39] NA  
PERSONNEL INJURIES NUMBER DESCRIPTION [41] NA  
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION [43] NA  
PUBLICITY [Z] [42] NA  
ISSUED DESCRIPTION [45] NA  
[2][0] [N] [44] NA

NAME OF PREPARER A. C. Tollison, Jr.

PHONE

919-457-9521

NRC USE ONLY

7909280 475

1038 335

Facility: BSEP Unit No. 2

Event Date: 8-30-79

during subsequent power operation, drywell leakage had decreased to approximately 0.2 gpm and the containment temperature was averaging 120-125<sup>0</sup>F. The surveillance that detected this event is a daily computer check of drywell temperatures. This surveillance has been increased to a frequency of once per shift to better detect rising temperatures, and to provide for an earlier initiation of methods for temperature reduction before exceeding any limits. This is considered an isolated event and no further action is required.