

# OPERATING DATA REPORT

DOCKET NO. 50-317  
 DATE 9/15/79  
 COMPLETED BY S. D. Merson  
 TELEPHONE 301-234-5240

## OPERATING STATUS

1. Unit Name: Calvert Cliffs No. 1
2. Reporting Period: August, 1979
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): 790 MWe
10. Reasons For Restrictions, If Any: Blade problems in the high-pressure turbine.

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5,831	37,836
12. Number Of Hours Reactor Was Critical	720.7	3,445.8	29,534.4
13. Reactor Reserve Shutdown Hours	9.8	143.1	1,036.7
14. Hours Generator On-Line	712.0	3,270.2	28,815.2
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,769,457.6	8,082,144.4	69,352,348.8
17. Gross Electrical Energy Generated (MWH)	538,384	2,577,437	22,963,772
18. Net Electrical Energy Generated (MWH)	511,519	2,447,605	21,902,593
19. Unit Service Factor	95.7	56.1	76.2
20. Unit Availability Factor	95.7	56.1	76.2
21. Unit Capacity Factor (Using MDC Net)	84.9	51.8	71.5
22. Unit Capacity Factor (Using DER Net)	81.4	49.7	68.5
23. Unit Forced Outage Rate	4.3	22.9	9.7
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

\_\_\_\_\_  
 \_\_\_\_\_  
 \_\_\_\_\_

975 210  
 7909200 314: (9/77)

# OPERATING DATA REPORT

DOCKET NO. 50-318  
 DATE 9/15/79  
 COMPLETED BY S. D. Merson  
 TELEPHONE 301-234-5240

## OPERATING STATUS

1. Unit Name: Calvert Cliffs No. 2
2. Reporting Period: August, 1979
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings (Items Number 3 Thru 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reasons For Restrictions, If Any: \_\_\_\_\_

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744</u>	<u>5,831</u>	<u>21,191</u>
12. Number Of Hours Reactor Was Critical	<u>690.3</u>	<u>5,454.2</u>	<u>18,446.7</u>
13. Reactor Reserve Shutdown Hours	<u>53.7</u>	<u>174.9</u>	<u>394.3</u>
14. Hours Generator On-Line	<u>676.6</u>	<u>5,346.6</u>	<u>18,158.4</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,704,472.8</u>	<u>13,825,036.8</u>	<u>44,709,053.4</u>
17. Gross Electrical Energy Generated (MWH)	<u>544,591</u>	<u>4,581,600</u>	<u>14,835,672</u>
18. Net Electrical Energy Generated (MWH)	<u>518,395</u>	<u>4,380,501</u>	<u>14,148,530</u>
19. Unit Service Factor	<u>90.9</u>	<u>91.7</u>	<u>85.7</u>
20. Unit Availability Factor	<u>90.9</u>	<u>91.7</u>	<u>85.7</u>
21. Unit Capacity Factor (Using MDC Net)	<u>86.0</u>	<u>92.7</u>	<u>82.4</u>
22. Unit Capacity Factor (Using DER Net)	<u>82.5</u>	<u>88.9</u>	<u>79.0</u>
23. Unit Forced Outage Rate	<u>9.1</u>	<u>4.4</u>	<u>5.5</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):  
Calvert Cliffs No. 2 is scheduled for a planned outage starting October 14, 1979,  
and will be six weeks in duration for general inspection and refueling.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: \_\_\_\_\_
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-317  
UNIT Calvert Cliffs #1  
DATE 9/15/79  
COMPLETED BY S. D. Merson  
TELEPHONE 301-234-5240

MONTH August, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>701</u>
2	<u>709</u>
3	<u>792</u>
4	<u>787</u>
5	<u>685</u>
6	<u>686</u>
7	<u>700</u>
8	<u>675</u>
9	<u>666</u>
10	<u>414</u>
11	<u>548</u>
12	<u>328</u>
13	<u>588</u>
14	<u>577</u>
15	<u>745</u>
16	<u>804</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>769</u>
18	<u>743</u>
19	<u>792</u>
20	<u>735</u>
21	<u>774</u>
22	<u>756</u>
23	<u>807</u>
24	<u>808</u>
25	<u>807</u>
26	<u>721</u>
27	<u>517</u>
28	<u>324</u>
29	<u>765</u>
30	<u>793</u>
31	<u>796</u>

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-318

UNIT Calvert Cliffs #2

DATE 9/15/79

COMPLETED BY S. D. Merson

TELEPHONE 301-234-5240

MONTH August, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	-
2	-
3	-
4	719
5	809
6	809
7	809
8	782
9	759
10	741
11	719
12	652
13	798
14	745
15	826
16	794

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	795
18	617
19	731
20	770
21	816
22	794
23	812
24	793
25	772
26	810
27	740
28	781
29	786
30	832
31	816

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

975 213

# UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH August, 1979

DOCKET NO. 50-317  
 UNIT NAME Calvert Cliffs No. 1  
 DATE 9/15/79  
 COMPLETED BY S.D. Merson  
 TELEPHONE 301-234-5240

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
79-8	790810	F	8.3	A	3	N/A	ZZ	ZZZZZZ	Forced outage due to the loss of No. 15 circulating water pump which resulted in a loss of vacuum trip.
79-9	790812	F	7.3	H	3	N/A	ZZ	ZZZZZZ	Forced outage due to low steam generator level which initiated a reactor trip.
79-10	790827	F	16.4	A	1	79-046	PC	PIPEXX	Forced outage due to a leak in the chemical volume control system.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

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975  
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ORIGINAL  
POOR

# UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH August, 1979

DOCKET NO. 50-318  
UNIT NAME Calvert Cliffs No. 2  
DATE 9/15/79  
COMPLETED BY S.D. Merson  
TELEPHONE 301-234-5240

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
79-12	790728	F	67.4	A	1	N/A	ZZ	ZZZZZZ	Forced outage due to condenser tube failures.
79-13	790818	F	0.0	A	4				Forced reduction due to condenser tube leaks.

<sup>1</sup>  
F: Forced  
S: Scheduled

<sup>2</sup>  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup>  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Other (Explain)

<sup>4</sup>  
Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File (NUREG-  
0161)

<sup>5</sup>  
Exhibit I - Same Source

975 215

(9/77)



9/10/79

REFUELING INFORMATION REQUEST

1. Name of Facility: Calvert Cliffs Nuclear Power Plant, Unit No. 1
2. Scheduled date for next Refueling Shutdown: April 19, 1980
3. Scheduled date for restart following refueling: May 29, 1980
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Resumption of operation after refueling will require changes to Technical Specifications. The changes will be such as to allow operation of the plant with a fresh reload batch and reshuffled core.

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

February 22, 1980

6. Important licensing considerations associated with refueling .

Reload fuel will be similar to that reload fuel inserted into the previous cycle.

Selected fuel assemblies will be modified by installation of sleeves in the guide tubes.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 217  
(b) 300

Spent Fuel Pools are common to Units 1 and 2.

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

1056 Licensed  
728 Currently Installed  
650 Addition is Planned

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

October, 1983

975 216

REFUELING INFORMATION REQUEST

1. Name of Facility: Calvert Cliffs Nuclear Power Plant, Unit 2
2. Scheduled date for next Refueling Shutdown: October 14, 1979
3. Scheduled date for restart following refueling: November 21, 1979
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

A preliminary review of the design and safety analysis indicate that no changes to the Technical Specification or other amendments are required and that there will be no unreviewed safety questions as defined by 10 CFR 50.59 involved with this reload core design.

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

August 22, 1979 (if required)

6. Important licensing considerations associated with refueling.

None, reload fuel will be identical to that reload fuel inserted in the previous cycle.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 217  
(b) 300

Spent Fuel Pools are common to Units 1 and 2.

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been required or is planned, in number of fuel assemblies.

1056 Licensed  
728 Currently Installed  
650 Addition is Planned

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

October, 1983



SUMMARY OF UNIT 1 OPERATING EXPERIENCE - AUGUST 1979

- 8/1 At the beginning of this reporting period, load was being decreased on Unit 1 to 635 MWe to clean condenser water boxes. Began increasing load and at 1930 reduced load to 745 MWe to investigate saltwater leakage into the main condenser.
- 8/2 Load was increased to capacity (815 MWe) at 0405.
- 8/5 Reduced load to 655 MWe at 1030 to investigate saltwater leakage into the main condenser.
- 8/6 Resumed full load operation (815 MWe) at 1800. Reduced load to 655 MWe at 2241 to investigate saltwater leakage into the main condenser.
- 8/7 At 0430 started increasing load and at 1525 reduced load to 685 MWe to investigate saltwater leakage into the main condenser. Began increasing load at 1852.
- 8/8 Load reduced to 685 MWe at 0430 to investigate saltwater leakage into the main condenser.
- 8/9 At 2000 started increasing load.
- 8/10 Reduced load to 655 MWe at 1025 to investigate saltwater leakage into the main condenser. The reactor tripped on loss of vacuum due to loss of 15 circulating water pump at 1040. The reactor was brought critical at 1531 and the unit paralleled at 1825. Load was limited to 615 MWe at 2300 to investigate saltwater leakage into the main condenser.
- 8/11 At 2140 started reducing power to repair an EHC leak on #2 turbine intercept valve. At 2325 the reactor tripped due to low steam generator level.
- 8/12 The reactor was brought critical at 0420 and the unit paralleled at 0717. Resumed full load operation (815 MWe) at 1900. Load was reduced to 725 MWe due to the ETS 120°F limit on main condenser circulating water  $\Delta T$ .
- 8/13 Decreased load to 535 MWe at 1500 to investigate saltwater leakage into the main condenser.
- 8/14 Started increasing load at 1345 and at 2230 commenced reducing power to 605 MWe to clean main condenser water boxes. Resumed full load operation (825 MWe) at 1200.
- 8/17 At 0345 decreased load to 745 MWe to investigate saltwater leakage into the main condenser. Full load operation (835 MWe) was resumed at 1600.

- 8/18 Load was reduced to 745 MWe at 0345 to investigate saltwater leakage into the main condenser.
- 8/19 Increased load to capacity (845 MWe) at 0700.
- 8/20 At 0415 load was reduced to 755 MWe to investigate saltwater leakage into the main condenser.
- 8/21 Full load operation (835 MWe) was resumed at 1000. Reduced load to 735 MWe at 2300 to investigate saltwater leakage into the main condenser.
- 8/22 Increased load to capacity (835 MWe) at 1800.
- 8/26 A load reduction was necessitated at 0430 to 735 MWe to investigate saltwater leakage into the main condenser.
- 8/27 Started reducing power at 1500 to repair a chemical volume control system leak. The unit was taken off the line at 1830 and the reactor shutdown.
- 8/28 The reactor was brought critical and the unit paralleled at 1055. Started increasing load and at 1600 load was limited to 755 MWe to investigate saltwater leakage into the main condenser.
- 8/29 Full load operation (835 MWe) was resumed at 1700.
- 8/30 Load was reduced to 815 MWe due to in-core power distribution problems.
- 8/31 At the end of this reporting period Unit 1 was operating at 815 MWe. Reactor power was limited to 97% due to in-core power distribution problems.

SUMMARY OF UNIT 2 OPERATING EXPERIENCE - AUGUST 1979

- 8/1 At the beginning of this reporting period, Unit 2 was shutdown to cleanup secondary chemistry due to a major condenser tube rupture.
- 8/2 The reactor was brought critical at 2304.
- 8/3 The reactor was shutdown at 0810 to comply with Technical Specifications on containment radiation sampling. The reactor was brought critical at 0910 and the unit paralleled at 1054. At 1220 the reactor tripped on loss of load. The reactor was brought critical at 1615 and the unit paralleled at 1815. The reactor tripped at 1900 due to loss of EHC pressure in the main turbine valve control system. At 2045 the reactor was brought critical. The unit was paralleled at 2135.
- 8/5 Load was increased to capacity (845 MWe) at 0100.
- 8/8 At 1100 load was reduced to 765 MWe to investigate salt-water leakage into the main condenser. Full load operation (845 MWe) was resumed at 2040.
- 8/9 Load was reduced to 760 MWe at 1100 to investigate salt-water leakage into the main condenser.
- 8/12 At 0500 load was decreased to 705 MWe due to the ETS 12<sup>0</sup>F limit on main condenser circulating water  $\Delta T$ .
- 8/13 Resumed full load operation (845 MWe) at 1930.
- 8/14 At 2100 load was reduced to 765 MWe to investigate salt-water leakage into the main condenser. Increased load to capacity (845 MWe) at 2300.
- 8/16 Decreased load to 755 MWe at 1500 to investigate saltwater leakage into the main condenser. Started to increase load at 1830.
- 8/17 Reduced load at 0300 to 735 MWe to investigate saltwater leakage into the main condenser. Full load operation (845 MWe) was resumed at 0900.
- 8/18 Decreased load to 635 MWe at 0530 for maintenance on 22 Heater Drain Pump.
- 8/19 Increased load to capacity (845 MWe). Reduced load at 1400 to 735 MWe to investigate saltwater leakage into main condenser.
- 8/20 Resumed full load operation (865 MWe) at 1730.
- 8/21 At 1800 reduced load to 765 MWe to investigate saltwater leakage into the main condenser.

- 8/22 Started increasing load and at 2000 reduced load to 740 MWe to investigate saltwater leakage into the main condenser.
- 8/23 Increased load to capacity (865 MWe) at 0640.
- 8/24 Reduced load at 2030 to 735 MWe to investigate saltwater leakage into the main condenser.
- 8/25 Resumed full load operation (845 MWe) at 1700.
- 8/26 Load was decreased to 765 MWe at 0150 for main turbine governor valve testing. Load was returned to capacity (865 MWe) at 0900.
- 8/27 At 0130 load was decreased to 765 MWe to investigate saltwater leakage into the main condenser.
- 8/28 Began increasing load and at 2230 reduced load to 810 MWe to investigate saltwater leakage into the main condenser.
- 8/29 Increased load to capacity (855 MWe) at 1700.
- 8/31 At 2150 reduced load to 745 MWe to investigate saltwater leakage into the main condenser. At the end of this reporting period, Unit 2 was operating at 745 MWe. Reactor power was limited to 93% while investigating for saltwater leakage into the main condenser.

## SAFETY-RELATED MAINTENANCE

UNIT IGROUP ELECTRIC SHOPMONTH AUGUST YEAR 1979

SYSTEM OR COMPONENT	MR NO. - DATE	MALFUNCTION		CORRECTIVE ACTION
		CAUSE	RESULT	
#11 Refueling Water Tank Outlet, Motor Operated Valves	0-79-1690 6/7/79	Close limit switches out of adjustment	The valves stay open 4 to 6 turns when electrically shut	Adjusted the close limit switch
#11 Switchgear Air Conditioner	0-79-2014 7/5/79	Condenser fan motor defective	Condenser fan breaker kept tripping	Replaced fan motor
Reactor Trip Switchgear/ Breaker TCB-9	0-79-2067 7/8/79	Wire on trip pushbutton was disconnected	Breaker would not close	Reconnected wire
#13 Inverter/Vital AC Power	0-79-2211 7/20/79	Defective fuse holder solder connection	Low output voltage alarm with output voltage at normal value	Re-soldered fuse-holder
#11 Control Room Air Conditioner	0-79-1708 6/9/79	Defective compressor motor	A/C unit would not run	Replaced A/C compressor motor
075 222				

## SAFETY-RELATED MAINTENANCE

UNIT IGROUP I&CMONTH AUGUST YEAR 1979

SYSTEM OR COMPONENT	MR NO. - DATE	MALFUNCTION		CORRECTIVE ACTION
		CAUSE	RESULT	
Pressurizer Relief Valve Outlet Temperature 1-TE-108	0-79-1306 5/21/79	Loose field lead connection on temperature element	Erroneous temperature reading	Tightened loose connection on temperature element
#11A Safety Injection Tank Vent 1-SI-613-CV	0-79-1645 6/12/79	Limit switch on valve was out of adjustment	Incorrect valve position indication	Adjusted limit switch
Reactor Coolant System/ 12A Loop Cold Leg Temperature 1-TE-125	0-79-1297 5/2/79	Defective temperature element	Temperature indication failed high	Replaced temperature element
Steam Generator Outlet Temperature 1-TE-4008	0-79-1311 5/16/79	Defective temperature element	Erroneous indication	Replaced temperature element
975 223				



## SAFETY-RELATED MAINTENANCE

UNIT IGROUP I&CMONTH AUGUST YEAR 1979

SYSTEM OR COMPONENT	MR NO. - DATE	MALFUNCTION		CORRECTIVE ACTION
		CAUSE	RESULT	
Pressurizer Level Indication LT-103	0-79-2055 7/11/79	Defective oscillator in transmitter	Erroneous level indication	Replaced defective oscillator
Nuclear Instrumentation Channel "A" Wide Range	0-79-1955 7/11/79	Dirty signal connector on drawer	Increase in counts for no apparent reason	Cleaned signal connector on drawer
Nuclear Instrumentation Wide Range/Audible Count Rate Instrument	0-78-1292 7/2/79	Defective audio amplifier	Channels A, B, & C Inoperable	Replaced audio Amplifier
Engineered Safety Features Actuation System/A2 Logic Module	0-79-1947 6/28/79	Defective logic module	#11 High pressure safety injection pump did not start on SIAS signal	Replaced defective logic module
Reactor Protection System/Matrix Relay AC-3	0-79-2076 7/9/79	Defective matrix relay	Associated trip circuit breakers would not trip while performing surveillance test procedure	Replaced defective matrix relay
Steam Flow Indication on 1-FR-1011/1111	0-79-208 5/28/79	Defective transmitter	Steam flow pegged low on recorder	Replaced defective transmitter 1-FT-1011

## SAFETY-RELATED MAINTENANCE

UNIT II

GROUP MACHINE SHOP

MONTH      AUGUST                                      YEAR      1979

SYSTEM OR COMPONENT	MR NO. - DATE	MALFUNCTION		CORRECTIVE ACTION
		CAUSE	RESULT	
Number 22 Main Steam Isolation Valve Hydraulic Package, Number 2 High Pressure Pump	0-79-2339 8/2/79	Air in pump	Pump had no discharge pressure	Vented pump

075 225

## SAFETY-RELATED MAINTENANCE

UNIT II

GROUP ELECTRIC SHOP

MONTH AUGUST YEAR 1979

SYSTEM OR COMPONENT	MR NO. - DATE	MALFUNCTION		CORRECTIVE ACTION
		CAUSE	RESULT	
#21 Diesel Generator/ Exciter Cabinet 2C61C	E-79-45 6/25/79	Field flash circuit energized after diesel shutdown	Over heated and damaged resistors in exciter cabinet	Replaced resistors and connecting wire
Switchgear Air Conditioner/Con- denser Fan Motor	M-79-3129 7/31/79	#21 Condenser fan motor was grounded	The breaker for the condenser fans tripped when the A/C unit was started	Replaced #21 fan motor

## SAFETY-RELATED MAINTENANCE

UNIT II

GROUP I&C

MONTH AUGUST YEAR 1979

SYSTEM OR COMPONENT	MR NO. - DATE	MALFUNCTION		CORRECTIVE ACTION
		CAUSE	RESULT	
Engineered Safety Features Actuation System/Containment Pressure	0-79-1602 5/25/79	B-8 Safety Injection actuation signal module was faulty	SIAS B-8 would not actuate on containment pressure from any of the sensor channels	Replaced faulty module
Service Water System/Turbine Bldg. Isolation Valve 2-CV-1600	0-79-1782 6/13/79	Valve position microswitch was out of adjustment	Valve was open but the close position indication would not go out	Adjusted the valve position microswitch
Engineered Safety Features Actuation System/CVCS Ch. ZE	0-79-1649 6/5/79	Defective bistable	Spurious trips	Replaced bistable
Safety Injection Tank Level Indication/2-LIA-331	0-79-1164 4/9/79	Broken glass on indication instrument in Control Room	Incorrect level indication	Replaced broken glass
Reactor Protection System/Channel "A" Variable Over Power Trip	IC-79-2039 4/11/79	Faulty relay in G-2 module	Reset demand light on channel "A" would not come on	Replaced G-2 module
Engineered Safety Features Actuation System/Channel "ZD" 5VDC Power Supply Voltmeter	IC-79-2068 7/17/79	Faulty voltmeter	Power supply voltmeter pegged low	Replaced voltmeter