

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285

UNIT Fort Calhoun #1

DATE September 10, 1979

COMPLETED BY B. J. Hickie

TELEPHONE 402-536-4413

MONTH August, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>437.7</u>
2	<u>436.8</u>
3	<u>435.9</u>
4	<u>434.2</u>
5	<u>432.6</u>
6	<u>431.6</u>
7	<u>431.8</u>
8	<u>431.1</u>
9	<u>432.3</u>
10	<u>434.5</u>
11	<u>435.9</u>
12	<u>436.9</u>
13	<u>439.0</u>
14	<u>390.8</u>
15	<u>387.8</u>
16	<u>437.6</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>438.6</u>
18	<u>437.6</u>
19	<u>435.0</u>
20	<u>434.9</u>
21	<u>279.4</u>
22	<u>0.0</u>
23	<u>27.0</u>
24	<u>310.3</u>
25	<u>404.6</u>
26	<u>404.8</u>
27	<u>406.6</u>
28	<u>407.1</u>
29	<u>410.5</u>
30	<u>429.1</u>
31	<u>427.8</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE September 10, 1979
 COMPLETED BY B. J. Hickie
 TELEPHONE 402-536-4413

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: August, 1979
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A

10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.0</u>	<u>5,831.0</u>	<u>52,008.0</u>
12. Number Of Hours Reactor Was Critical	<u>721.1</u>	<u>5,786.5</u>	<u>41,751.5</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1,136.0</u>
14. Hours Generator On-Line	<u>695.9</u>	<u>5,746.5</u>	<u>40,812.6</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>948,080.5</u>	<u>7,922,647.8</u>	<u>49,571,071.1</u>
17. Gross Electrical Energy Generated (MWH)	<u>306,984.0</u>	<u>2,652,205.9</u>	<u>16,456,467.6</u>
18. Net Electrical Energy Generated (MWH)	<u>290,873.2</u>	<u>2,521,767.4</u>	<u>15,543,173.1</u>
19. Unit Service Factor	<u>93.5</u>	<u>98.6</u>	<u>78.5</u>
20. Unit Availability Factor	<u>93.5</u>	<u>98.6</u>	<u>78.5</u>
21. Unit Capacity Factor (Using MDC Net)	<u>85.5</u>	<u>94.6</u>	<u>66.0</u>
22. Unit Capacity Factor (Using DER Net)	<u>85.5</u>	<u>94.6</u>	<u>65.4</u>
23. Unit Forced Outage Rate	<u>6.5</u>	<u>1.4</u>	<u>4.5</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
January 1, 1980 - Refueling Outage - Two Month Duration

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH August, 1979

DOCKET NO. 50-285
 UNIT NAME Port Calhoun #1
 DATE September 10, 1979
 COMPLETED BY B. J. Hickie
 TELEPHONE 402-536-4413

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
79-03	79821	F	48.1	H	3	N/A	-	-	Electrical noise spike caused turbine trip/reactor trip. Timer to be installed into circuit to prevent spurious trip.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

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Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending August 31, 1979.

1. Scheduled date for next refueling shutdown. January 1, 1980
 2. Scheduled date for restart following refueling. March 1, 1980
 3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be?
Stretching power to 1500 MWth is planned in conjunction with the change in fuel supplier to Exxon.
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

 - c. If no such review has taken place, when is it scheduled?

 4. Scheduled date(s) for submitting proposed licensing action and support information. Stretch Power Application
Site Related Information,
July, 1979 - Submitted
 5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. Non-Core Related Information,
October, 1979
Core Related Analysis
and Tech. Spec. Changes,
November, 1979
- First use of Exxon fuel in Fort Calhoun.
Stretching power from 1420 MWth to 1500 MWth.
6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>157</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>483</u>	"
 7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by

J K Gasper

Date September 5, 1979

956202

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

August 1979
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun maintained essentially 100% power during the month of August, 1979, with the exception of one unit outage on August 21. This outage resulted from a momentary DC power interrupt to the electro-hydraulic control unit of the turbine. During the unit outage several maintenance items were completed and certain required surveillance tests completed as scheduled.

The full compliment of operations staff has been achieved and a complete change to six section shift work is expected by late September.

A Nuclear Regulatory Commission license examiner administered reactor operator license exams to four (4) candidates during August.

The Operations Department is maintaining an active participation in the Three Mile Island owners group. Several meetings were held during the month of August to develop operator guidelines for the emergency procedure following a loss of coolant accident. In addition, the District has developed a Three Mile Island task force to start an evaluation of NUREG-0578's impact on Fort Calhoun Station.

Surveillance tests and normal operational tests were completed with no outstanding deficiencies noted.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
79-017	During normal power operation, "D" channel wide range nuclear instrumentation failed with complete loss of power. The system was placed in a 2 out of 3 trip logic by bypassing "D" channel while the power supply was being troubleshot. No previous event of this nature has occurred at FCS1. Reference Tech. Spec. 2.15, Table 2-2, Item 10. No Technical Specification requirements were violated.
79-018	While performing ST-ESF-11, F.1, it was noted that C/PIC-902, "A" steam generator pressure indicator/controller for "C" Channel would not reset. The "C" channel steam generator "A" pressure was bypassed with the three remaining channels operable. Reference Tech. Spec. 2.15. No Technical Specification violation occurred.

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

The following are surveillance tests where an unusual result occurred:

<u>Surveillance Test No.</u>	<u>Description</u>
ST-CEA-1, F.6	During performance of ST-CEA-1 it was found that Group 2 Rod Motions was not prohibited after PDIL & Rod Block alarms were in alarm.
ST-ENV-4	OI-805 - Fish Impingement
ST-RM-3	OI-821 - Failed to perform test on time
ST-RM-3	OI-821 - Test completed one day late

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
DCR 78-67	SI-186 Pipe Support/Completed as designed.
SP-FAUD-1	All loops were functional instruments indicate greater than 99% assurance that greater than 150 pounds downward force exists on fuel bundles.
EEAR-FC-79-96	Exhaust fan in Room 12 of Environmental Lab. Completed as designed.
DCR 75A-70	Spent Fuel Rack Modification (Part 2) completed as designed.
EEAR FC-79-127	Telephone System Modification
EEAR 79-105	Service Air to Intake Structure Isolation/ completed as designed.
MR FC-78-51	Fuel Tank for CP-1B modification/completed as designed.
SP-MEAS-2	Measurements for oil collection system on RC pump motors. Critical measurements taken.
Annual Emerg. Drill, Conducted as planned 7-31-79.	
SP-CPTP-12	Measurement of isothermal temperature coefficient using Center CEA.

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E. RESULTS OF LEAK RATE TESTS

Surveillance Test ST-CONT-2, F.2, was performed on PAL Door and leakage was 12 cc per minute.

F. CHANGES IN PLANT OPERATING STAFF

The following management changes have been made at O.P.P.D. and are effective September 1, 1979:

W. C. Jones, Division Manager-Production Operations

R. L. Andrews, Section Manager-Operations

R. L. Jaworski, Section Manager-Technical Services

J. K. Gasper, Manager-Reactor and Computer Technical Services

S. C. Stevens, Manager-Fort Calhoun Station

J. L. Connolley, Supervisor-Instrument and Control and Electric
Field Maintenance, Fort Calhoun Station

D. W. Jones, Supervisor-Maintenance, Central Maintenance

G. R. Peterson, Supervisor-Maintenance, Fort Calhoun Station

L. G. Sealock, Supervisor-Nuclear Physics and Computer Applications,
Technical Services

A. W. Richard, Plant Engineer, Fort Calhoun Station

G. TRAINING

Training for August included hot license and requalification sessions. Four Hot License candidates were examined on August 21, 22 and 23 by the NRC and preliminary indications were favorable towards the candidates. Areas covered by training also included crane operations, detector theory, EDO, and radiation protection in addition to systems training for plant maintenance.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

None

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Approved by _____

Manager-Fort Calhoun Station

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August 1979
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II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
997	6-16-79	Elec. Fire Pump Full Flow Test	Satisfied Tech. Spec. requirements for full flow.
1015	7-17-79	Diesel Fire Pump Full Flow Test	Satisfied Tech. Spec. requirements for full flow.
1453	7-27-79	Valves HCV-401B and HCV-401D will not shut.	Found dirt in orifice. Accumulated the clean out ports on the relays. Valves checked satisfactorily.
			956206