OPERATING DATA REPORT

DOCKET NO. 50-267
DATE 790901
COMPLETED BY J. W. Gahm
TELEPHONE (303) 785-2253

OPERATING STATUS

NOTES



1. Unit Name: Fort St. Vrain, Unit No. 1
2. Reporting Period: 790801 thru 790831
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   None
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any: Nuclear Regulatory Commission restriction (70%) pending resolution of temperature fluctuations.

| | | This Month | Year to Date | Cumulative |
|---|---|---|---|---|
| 11. | Hours in Reporting Period | 744 | 5,831 | 1,488 |
| 12. | Number of Hours Reactor Was Critical | 712.7 | 2,556.0 | 16,029.7 |
| 13. | Reactor Reserve Shutdown Hours | 0.0 | 0.0 | 0.0 |
| 14. | Hours Generator On-Line | 542.5 | 1,293.4 | 9,136.0 |
| 15. | Unit Reserve Shutdown Hours | 0.0 | 0.0 | 0.0 |
| 16. | Gross Thermal Energy Generated (MWH) | 253,407 | 628,291 | 3,831,289 |
| 17. | Gross Electrical Energy Generated (MWH) | 88,079 | 202,453 | 1,150,723 |
| 18. | Net Electrical Energy Generated (MWH) | 81,868 | 183,045 | 1,034,496 |
| 19. | Unit Service Factor | 72.9% | 42.2% | 42.2% |
| 20. | Unit Availability Factor | 72.9% | 42.2% | 42.2% |
| 21. | Unit Capacity Factor (Using MDC Net) | 33.3% | 16.7% | 16.7% |
| 22. | Unit Capacity Factor (Using DER Net) | 33.3% | 16.7% | 16.7% |
| 23. | Unit Forced Outage Rate | 27.1% | 57.7% | 57.7% |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Maintenance Shutdown, October 20, 1979, 30 Days
25. If Shut Down at End of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

| | Forecast | Achieved |
|---|---|---|
| INITIAL CRITICALITY | 740201 | 740131 |
| INITIAL ELECTRICITY | 7612 | 761211 |
| COMMERCIAL OPERATION | 790701 | 790701 |

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UNIT SHUTDOWNS AND POWER REDUCTONS

DOCKET NO. 50-267
UNIT NAME Fort St. Vrain, Unit No. 1
DATE 790901
COMPLETED BY J. W. Gahm
TELEPHONE (303) 785-2253

REPORT MONTH August, 1979

| NO. | DATE | TYPE | DURATION | REASON | METHOD OF SHUTTING DOWN REACTOR | LER # | SYSTEM CODE | COMPONENT CODE | CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE |
|---|---|---|---|---|---|---|---|---|---|
| 79-08 | 790731 | F | 63.1 | H | 3 | N/A | N/A | N/A | While transferring to partial arc on the main turbine generator, throttle pressure dropped and load decreased 20 MW. During recovery, three circulators tripped, Loop 2 shutdown, reactor scrammed, and turbine tripped. |
| 79-09 | 790811 | F | 24.1 | H | 3 | N/A | N/A | N/A | While reducing power to recover a tripped circulator, hot reheat reactor scram and turbine trip occurred. |
| 79-10 | 790817 | F | 68.4 | H | 3 | 79-028/01-T | ED | ZZZZZ | An instrument panel was shorted to ground and tripped, resulting in reactor scram and turbine trip. |
| 79-11 | 790824 | F | 45.9 | H | N/A | N/A | N/A | N/A | Turbine generator taken off line as reactor power was reduced in an attempt to isolate cause of high primary coolant moisture. Reactor not shutdown. |
| | | | | | | | | | |

SUMMARY: Plan to continue operation for electric production and fluctuation testing until release to continue power ascension testing above 70% received from Nuclear Regulatory Commission.

# AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-267

Unit Fort St. Vrain, Unit No. 1

Date 790901

Completed By J. W. Gahm

Telephone (303) 785-2253

Month August, 1979

| DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|---|---|---|---|
| 1 | -0- | 17 | 93 |
| 2 | -0- | 18 | -0- |
| 3 | 21 | 19 | -0- |
| 4 | 116 | 20 | 37 |
| 5 | 129 | 21 | 158 |
| 6 | 153 | 22 | 211 |
| 7 | 197 | 23 | 205 |
| 8 | 200 | 24 | 120 |
| 9 | 206 | 25 | -0- |
| 10 | 208 | 26 | 39 |
| 11 | 156 | 27 | 120 |
| 12 | 5 | 28 | 110 |
| 13 | 136 | 29 | 109 |
| 14 | 188 | 30 | 105 |
| 15 | 196 | 31 | 98 |
| 16 | 128 | | |

*Generator on line but no net generation.

# REFUELING INFORMATION

| | |
|---|---|
| 1. Name of Facility. | Fort St. Vrain, Unit No. 1 |
| 2. Scheduled date for next refueling shutdown. | September 1, 1980 |
| 3. Scheduled date for restart following refueling. | November 1, 1980 |
| 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? | No |
| If answer is yes, what, in general, will these be? | |
| If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10CFR Section 50.59)? | The Plant Operations Review Committee will review any questions associated with the core reload. |
| If no such review has taken place, when is it scheduled? | March 1, 1980 |
| 5. Scheduled date(s) for submitting proposed licensing action and supporting information. | ---------- |
| 6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. | ---------- |
| 7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. | a) 1482 HTGR fuel elements.<br>b) 244 spent HTGR fuel elements. |
| 8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies. | Capacity is limited in size to about one third of core (approximately 500 HTGR elements). No change is planned. |

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REFUELING INFORMATION (CONTINUED)

| | |
|---|---|
| 9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. | 1986 under the Three Party Agreement (Contract AT (04-3)-633) between DOE, Public Service Company of Colorado (PSCo), and General Atomic Company.* |

*The 1986 date is based on the understanding that spent fuel discharged during the term of the Three Party Agreement will be shipped to the Idaho National Engineering Laboratory for storage by DOE at the Idaho Chemical Processing Plant (ICPP). The storage capacity has evidently been sized to accomodate fuel which is expected to be discharged during the eight year period covered by the Three Party Agreement.