NRC FORM 366
(7-77)

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE: P A B V S 1 (2)	LICENSE NUMBER: 0 0 - ? 0 0 0 0 - 0 0 (3)	LICENSE TYPE: 4 1 1 1 1 (4)	CAT: (5)				
CON'T 0 1	REPORT SOURCE: L (6)	DOCKET NUMBER: 0 5 0 0 0 3 3 4 (7)	EVENT DATE: 0 8 1 9 7 9 (8)	REPORT DATE: 0 8 3 1 7 9 (9)				

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 At 1446 hours, the reactor was manually tripped following the trip of the 1B Steam
0 3 Generator Feed Pump due to low NPSH. The operators performed the immediate manual
0 4 actions following a reactor trip and loss of normal feedwater. An operator, thinking
0 5 that the generator output breakers were taking too long to trip, manually tripped them
0 6 prior to the automatic trip which normally occurs 30 seconds after the trip. Frequency
0 7 dropped on the 1A and 1B 4KV Buses and all three RCPs tripped on underfrequency.
0 8

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	H A (11)	A (12)	A (13)	C K T B R K (14)	A (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	7 9	—	0 2 8	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
H (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	Z (25)	Z 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The incident resulted from operator error in tripping the main generator output breaker
1 1 prior to the automatic trip. The 1C RCP was started four minutes after the trip and
1 2 forced flow was returned to the RCS. The involved operator was reprimanded for his
1 3 action and all Operations personnel will review the incident circumstances.
1 4

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	E (28)	0 5 0 (29)	N/A	A (31)	Operator observation

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	N/A	N/A

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	N/A

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	N/A

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	N/A

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PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)	N/A	

NAME OF PREPARER J. A. Werling PHONE: 412-643-1258

GPO 917-926

Attachment To LER 79-28/01T
Beaver Valley Power Station
Duquesne Light Company
Docket No. 50-334

At 1446 hours on August 19, 1979, the reactor was manually tripped from approximately 50% rated power following the trip of the 1B Main Feed Pump due to low net positive suction head. The 1A Condensate Pump was shutdown for cleaning of its strainer. The 1B Main Feed Pump had just been started and the 1A Pump secured to investigate a high bearing temperature on the 1A Pump.

Upon initiation of the reactor trip, the operators performed the immediate manual actions following a reactor trip and loss of normal feedwater. An operator, thinking that the generator output breakers were taking too long to trip, manually tripped them prior to the automatic trip which normally occurs 30 seconds after the reactor/turbine trip. Since the main generator was no longer tied to the system, frequency dropped on the 1A and 1B 4KV Buses and all three reactor coolant pumps tripped on underfrequency (2 of 3 buses). The 1C and 1D 4KV Buses were tied into the system and did not experience the underfrequency condition. Thirty seconds after the turbine trip, the 1A and 1B Buses transferred to system power and the underfrequency condition cleared. Four minutes after the reactor trip, the 1C Reactor Coolant Pump was started and forced flow returned to the reactor coolant system. All systems operated as designed and at no time was the health and safety of the public jeopardized. The involved operator was reprimanded for his action. All Operations personnel will review the incident.