Figure 15.0-1 Core Limits and Overpower-Overtemperature Delta T Setpoints (T_{ref} = 576.0°F)

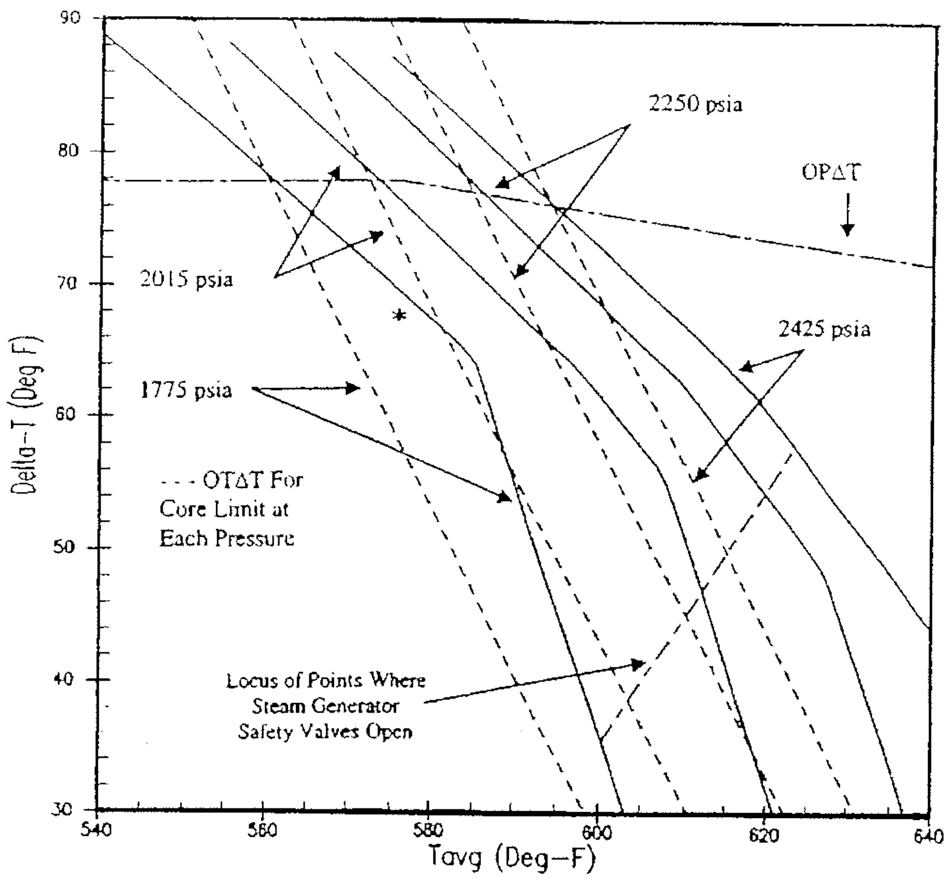

Figure 15.0-2 Reactivity Coefficients Used in Non-LOCA Safety Analysis

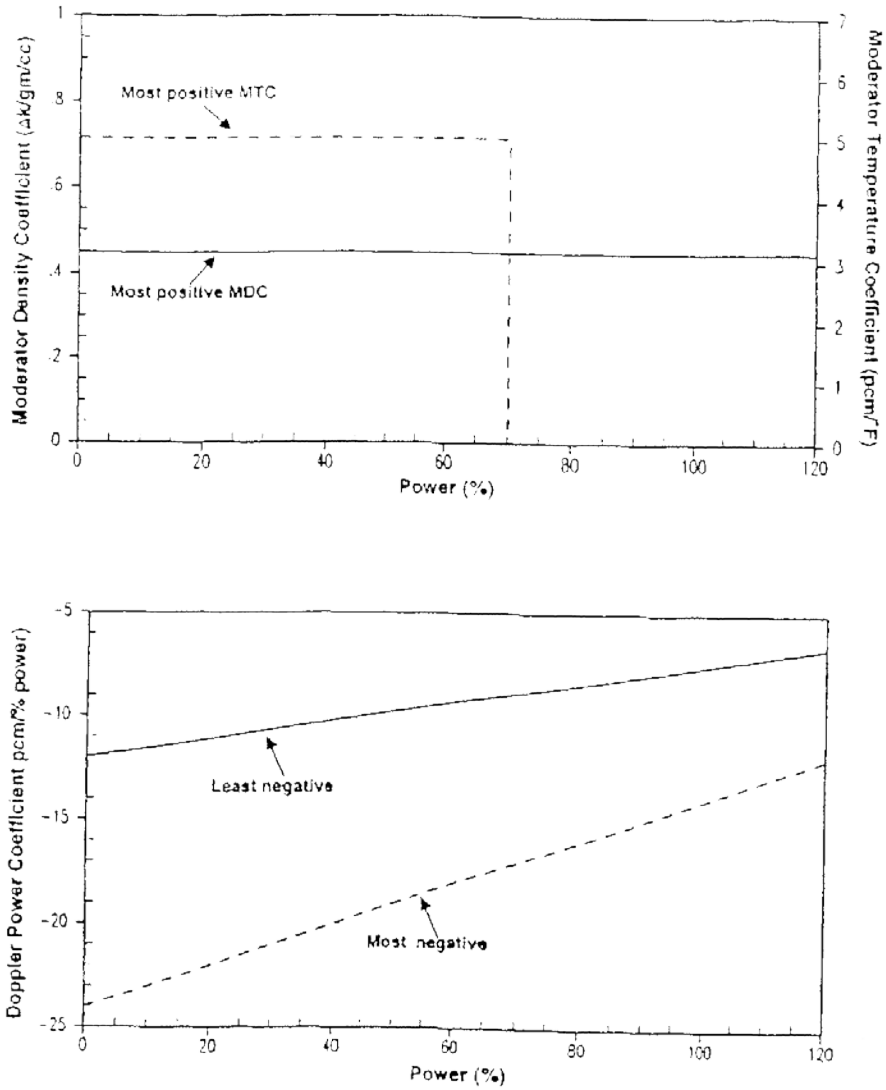

Figure 15.0-3   Reactivity Insertion Scram Curves

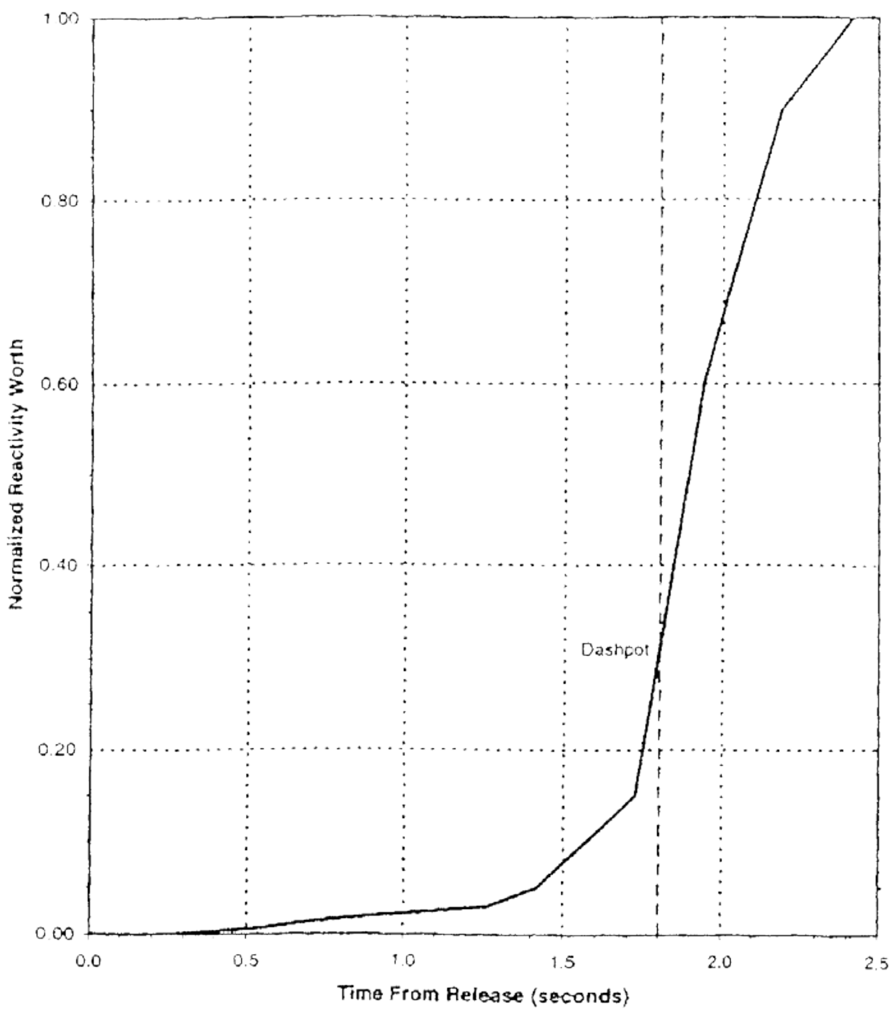

Figure 15.1-1 Feedwater Flow Increase at Full Power, Nuclear Power and Loop Average Temperature Versus Time

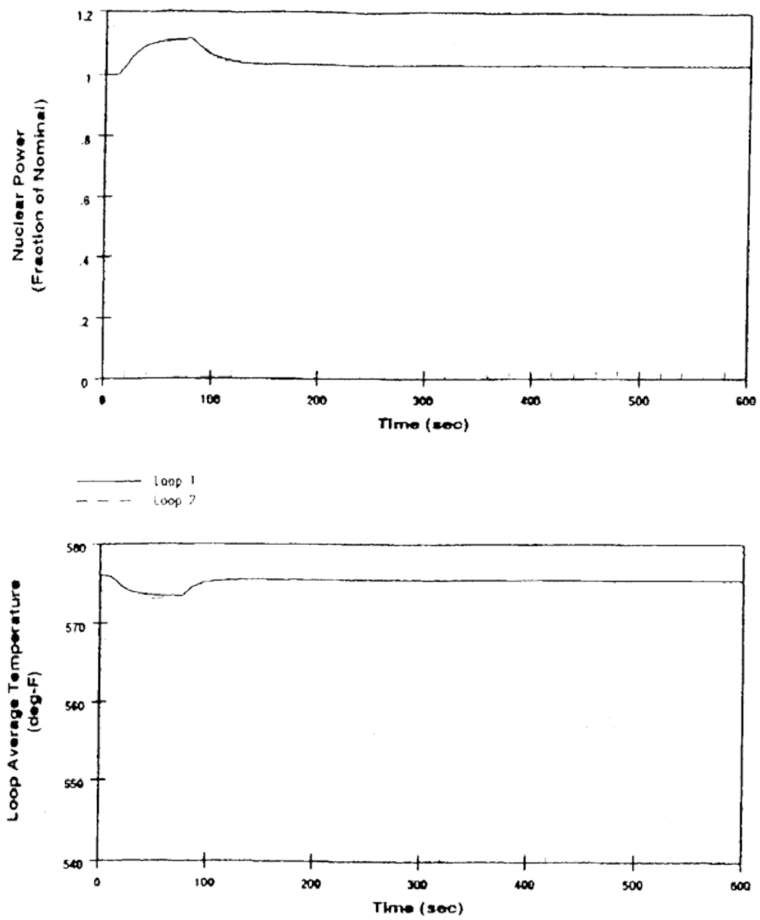

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Figure 15.1-1

Feedwater Flow Increase at Full Power, Nuclear Power and Loop Average Temperature Versus Time

Figure 15.1-2 Feedwater Flow Increase at Full Power, Pressurizer Pressure and Steam Generator Pressure Versus Time

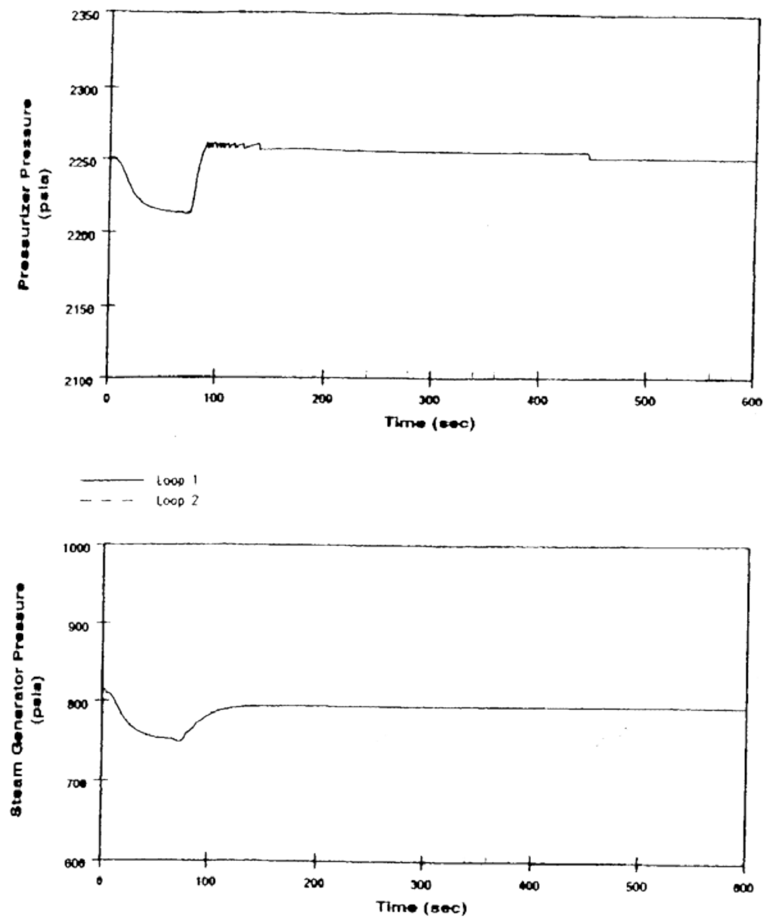

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Figure 15.1-2

Feedwater Flow Increase at Full Power, Pressurizer Pressure and Steam Generator Pressure Versus Time

Figure 15.1-3 Feedwater Flow Increase at Full Power, Steam Generator Mass Versus Time

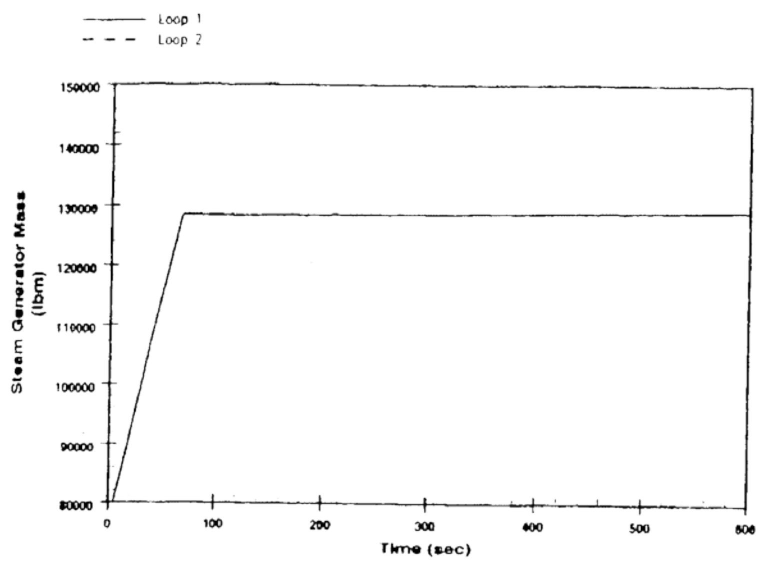

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Figure 15.1-3
Feedwater Flow Increase at Full Power, Steam Generator Mass Versus Time

Figure 15.1-4 Steam Line Rupture, Multiplication Factor Versus Core Average Temperature (Calculated at 1050 psia)

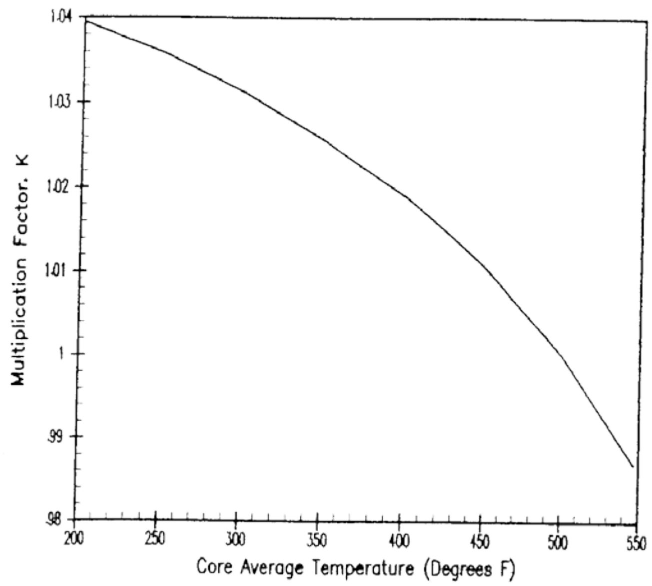

Figure 15.1-5 Steam Line Rupture, Integrated Doppler Defect Versus Fraction of Power

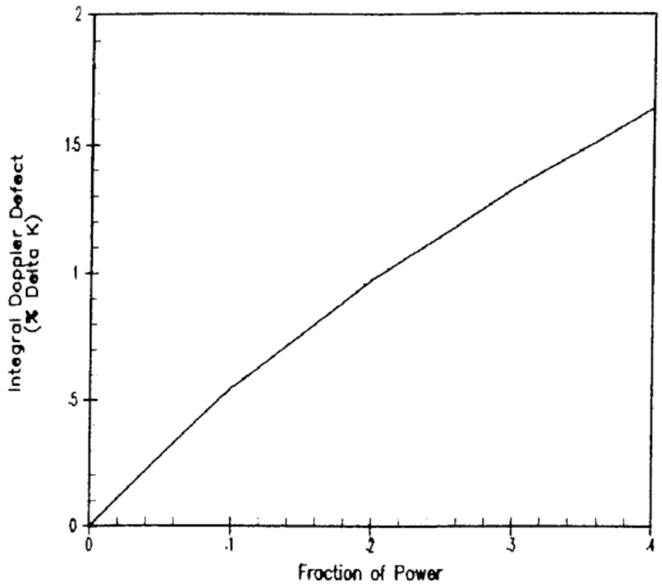

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Figure 15.1-5 *Steam Line Rupture, Integral Doppler Defect Versus Fraction of Power*

Figure 15.1-6 Steam Line Rupture, 1.4ft² Break with Power, Two Loops in Service, Core Heat Flux and Nuclear Power Versus Time

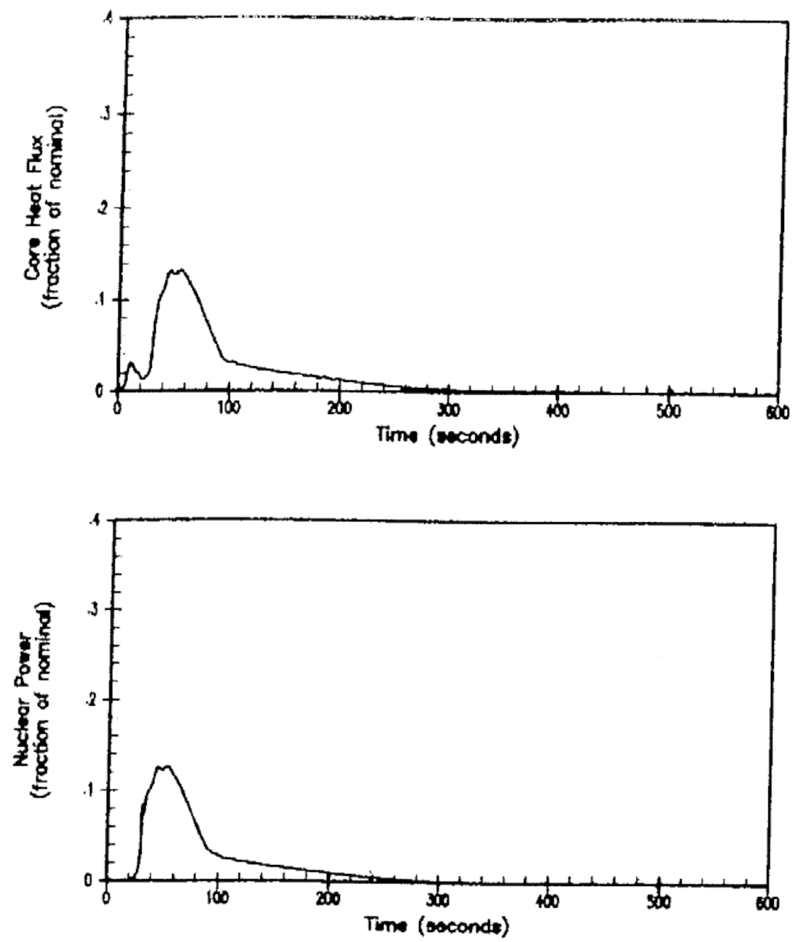

Figure 15.1-7 Steam Line Rupture, 1.4ft^2 Break with Power, Two Loops in Service, Pressurizer Water Volume and Pressurizer Pressure Versus Time

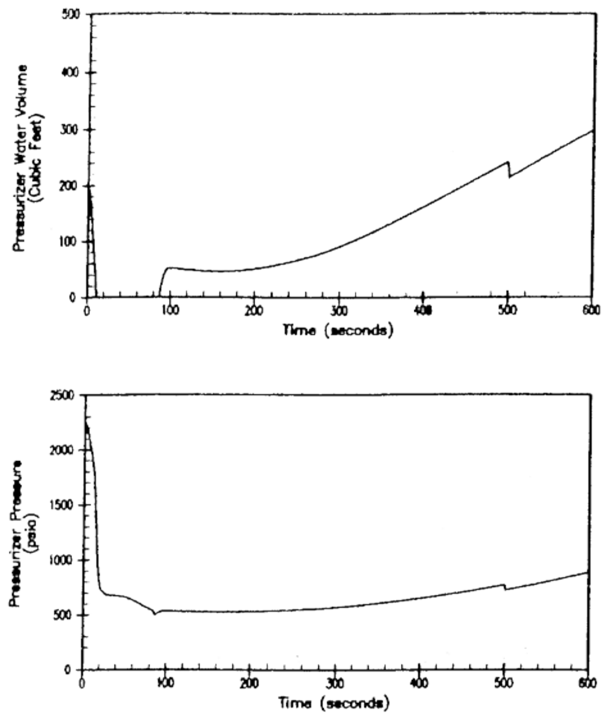

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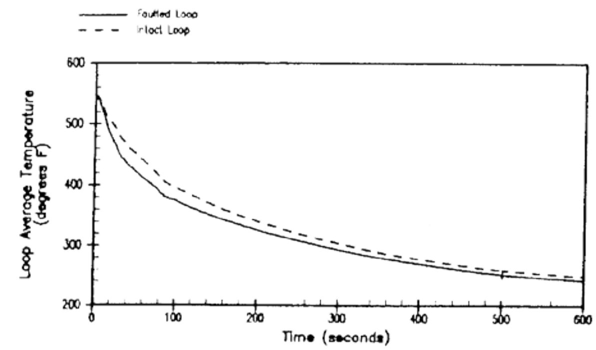
Figure 15.1-7 *Steam Line Rupture, 1.4ft^2 Break with Power, Two Loops in Service, Pressurizer Water Volume and Pressurizer Pressure Versus Time*

Figure 15.1-8 Steam Line Rupture, 1.4ft^2 Break with Power, Two Loops in Service, Loop T_{AVG} and Cold Leg Loop Temperature Versus Time



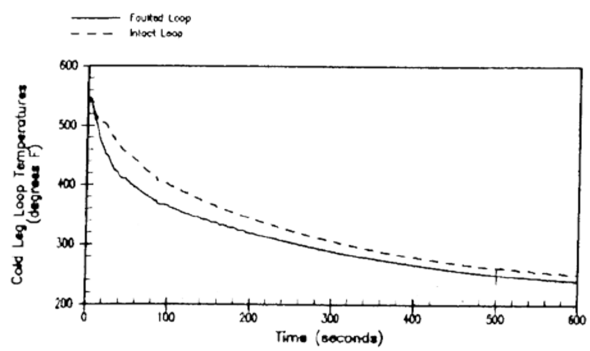


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Figure 15.1-8 *Steam Line Rupture, 1.4ft^2 Break with Power, Two Loops in Service, Loop T_{avg} and Cold Leg Loop Temperature Versus Time*

Figure 15.1-9 Steam Line Rupture, 1.4ft^2 Break with Power, Two Loops in Service, Faulted Loop Steam Flow and Total Feedwater Flow Versus Time

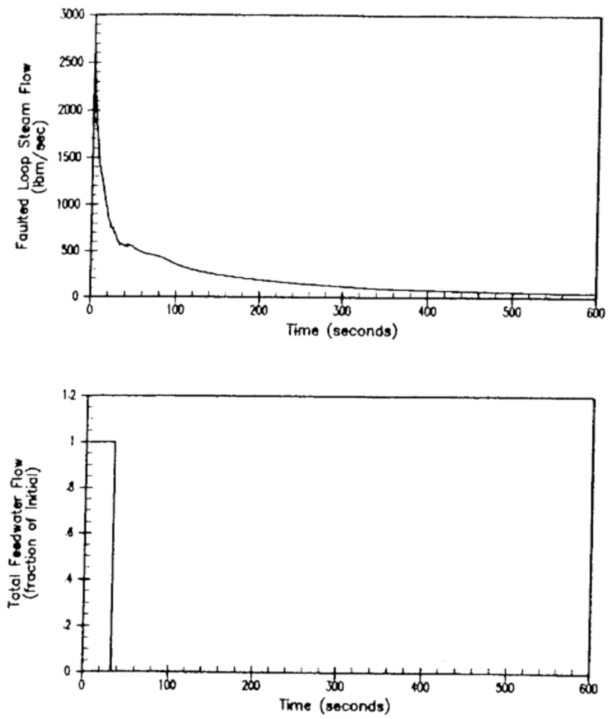

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Figure 15.1-9 *Steam Line Rupture, 1.4ft^2 Break with Power, Two Loops in Service, Faulted Loop Steam Flow and Total Feedwater Flow Versus Time*

Figure 15.1-10 Steam Line Rupture, 1.4ft² Break with Power, Two Loops in Service, Core Averaged Boron and Reactivity Versus Time

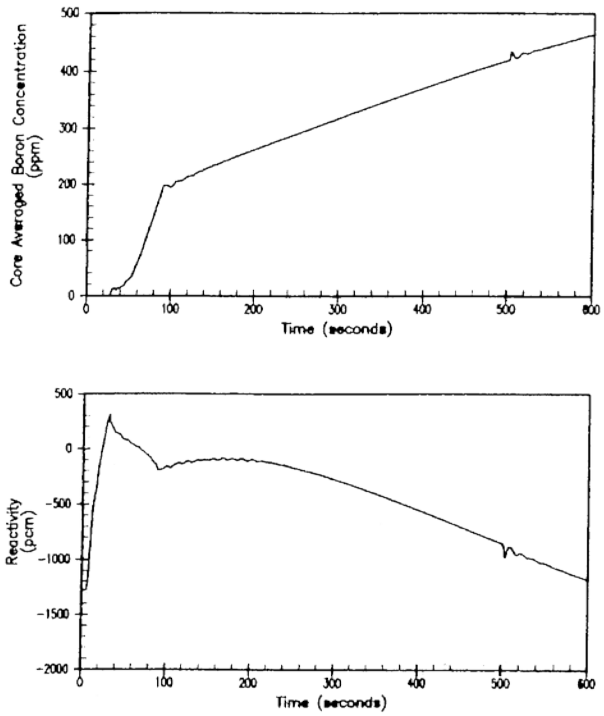

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Figure 15.1-10 *Steam Line Rupture, 1.4ft² Break with Power, Two Loops in Service, Core Averaged Boron and Reactivity Versus Time*

Figure 15.1-11 Steam Line Rupture, 1.4ft² Break Without Power, Two Loops in Service, Core Heat Flux and Nuclear Power Versus Time

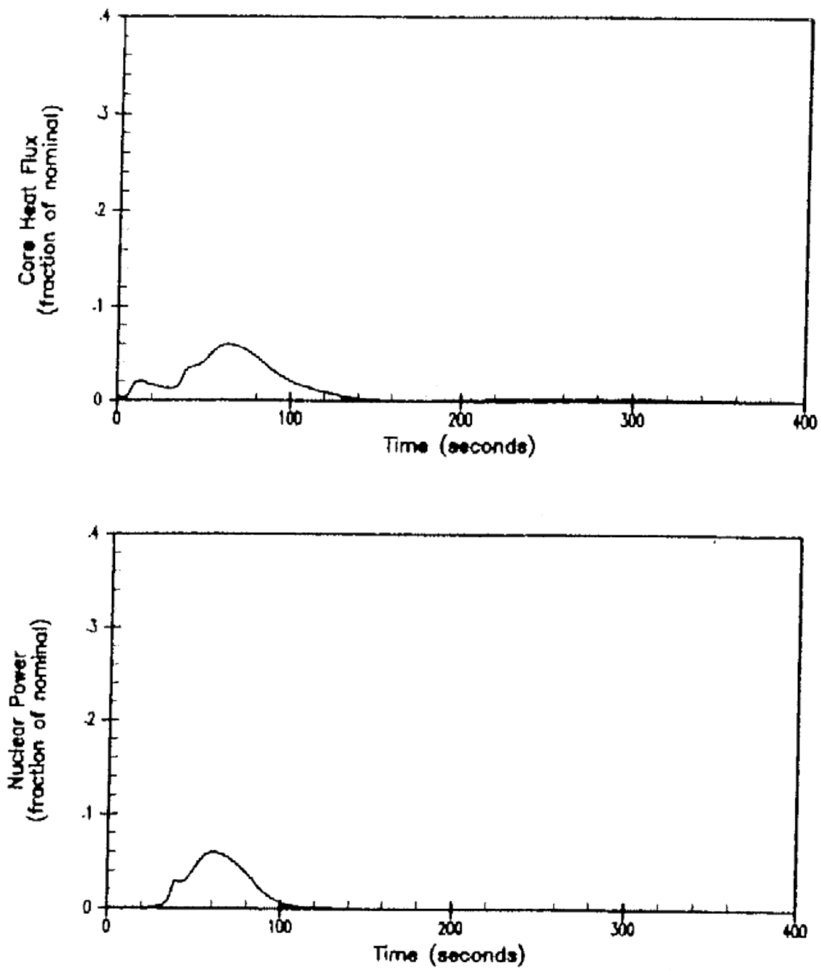

Figure 15.1-12 Steam Line Rupture, 1.4ft^2 Break Without Power, Two Loops in Service, Pressurizer Water Volume and Pressurizer Pressure Versus Time

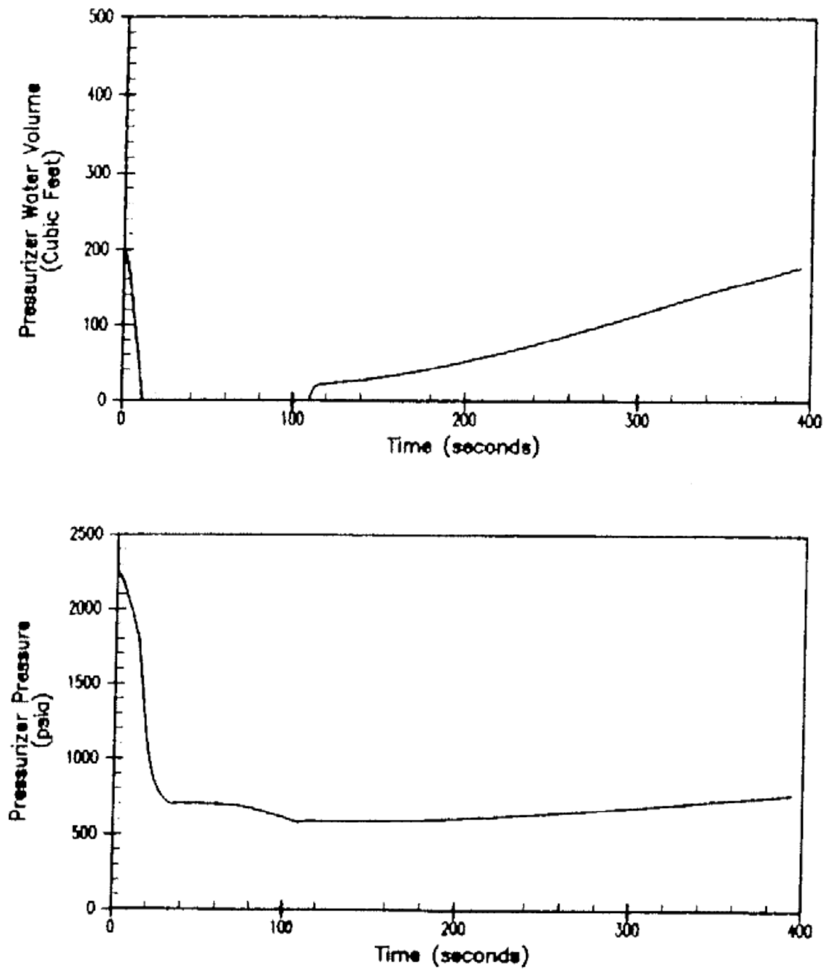

Figure 15.1-13 Steam Line Rupture, 1.4ft^2 Break Without Power, Two Loops in Service, Loop T_{AVG} and Cold Leg Loop Temperatures Versus Time

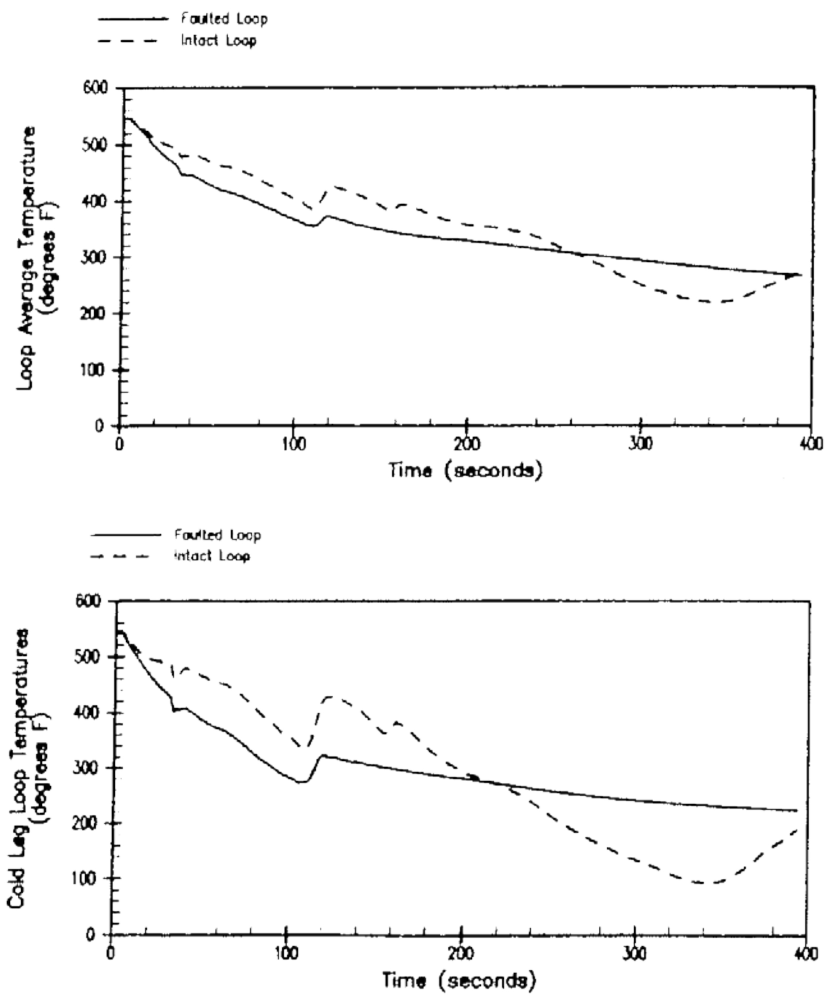

Figure 15.1-14 Steam Line Rupture, 1.4ft² Break Without Power, Two Loops in Service, Faulted Loop Steam Flow and Total Feedwater Flow Versus Time

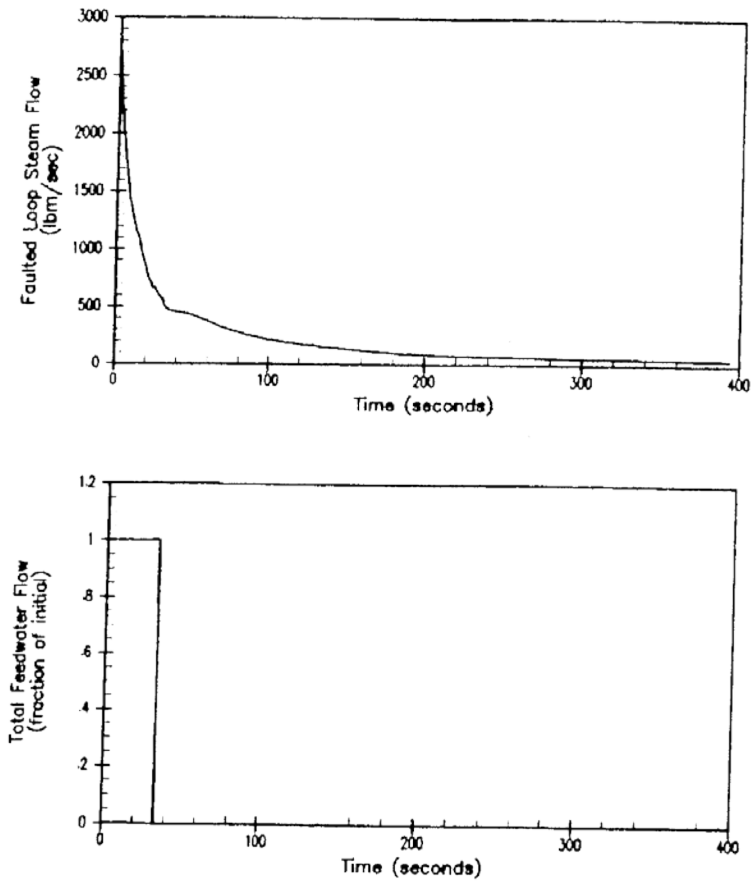

Figure 15.1-15 Steam Line Rupture, 1.4ft² Break without Power, Two Loops in Service, Core Averaged Boron and Reactivity Versus Time

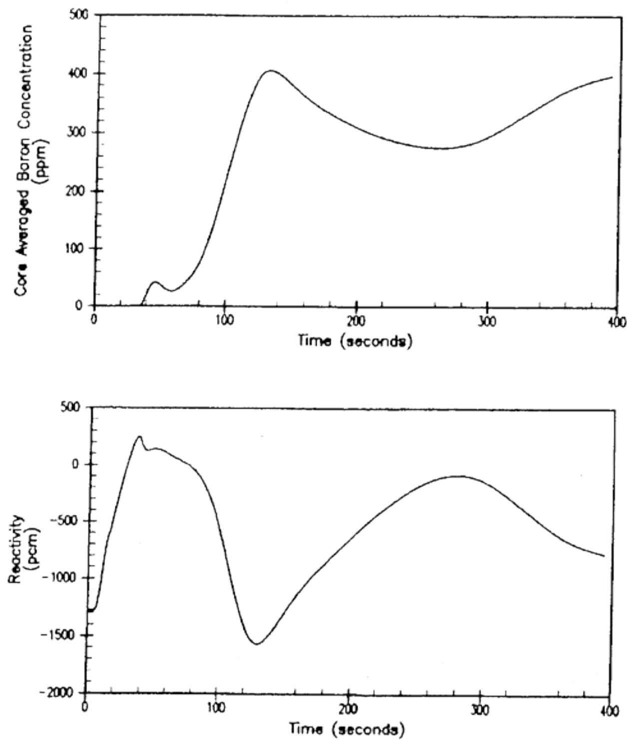

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Figure 15.1-15 *Steam Line Rupture, 1.4ft² Break without Power, Two Loops in Service, Core Averaged Boron and Reactivity Versus Time*

Figure 15.1-16 Steam Line Rupture, 1.4ft^2 Break with Power, One Loop in Service, Core Heat Flux and Nuclear Power Versus Time

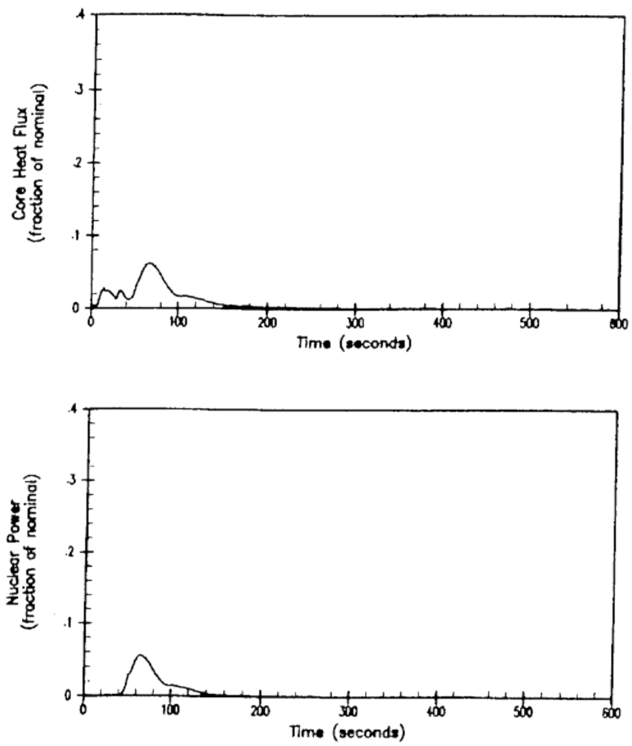

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Figure 15.1-16 *Steam Line Rupture, 1.4ft^2 Break with Power, One Loop in Service, Core heat Flux and Nuclear Power Versus Time*

Figure 15.1-17 Steam Line Rupture, 1.4ft^2 Break with Power, One Loop in Service, Pressurizer Water Volume and Pressurizer Pressure Versus Time

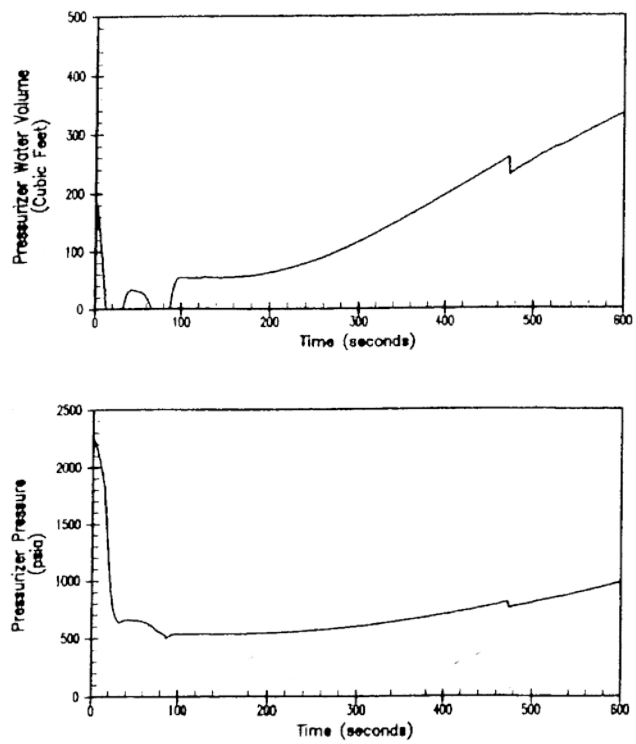

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Figure 15.1-17
Steam Line Rupture, 1.4ft^2 Break with Power, One Loop in Service, Pressurizer Water Volume and Pressurizer Pressure Versus Time

Figure 15.1-18 Steam Line Rupture, 1.4ft^2 Break with Power, One Loop in Service, Loop T_{AVG} and Cold Leg Loop Temperatures Versus Time

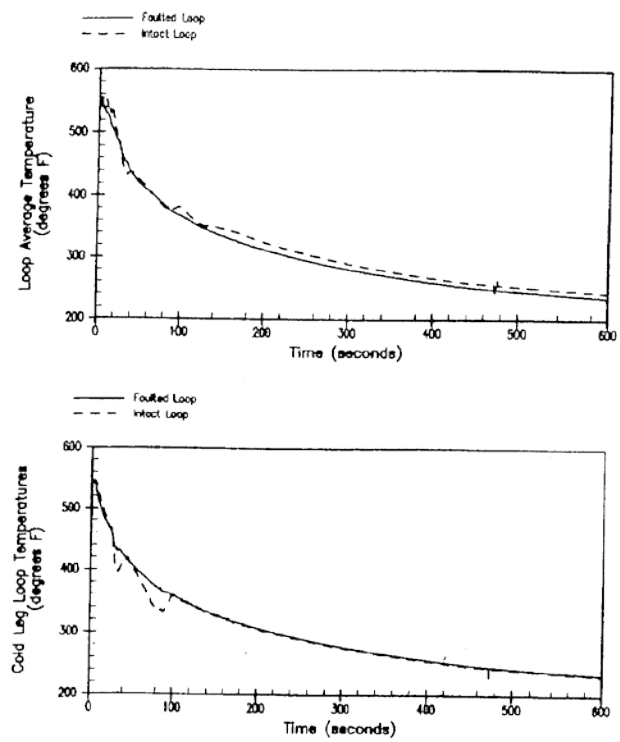

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Figure 15.1-18

Steam Line Rupture, 1.4ft^2 Break with Power, One Loop in Service, Loop T_{avg} and Cold Leg Loop Temperatures Versus Time

Figure 15.1-19 Steam Line Rupture, 1.4ft^2 Break with Power, One Loop in Service, Faulted Loop Steam Flow and Total Feedwater Flow Versus Time

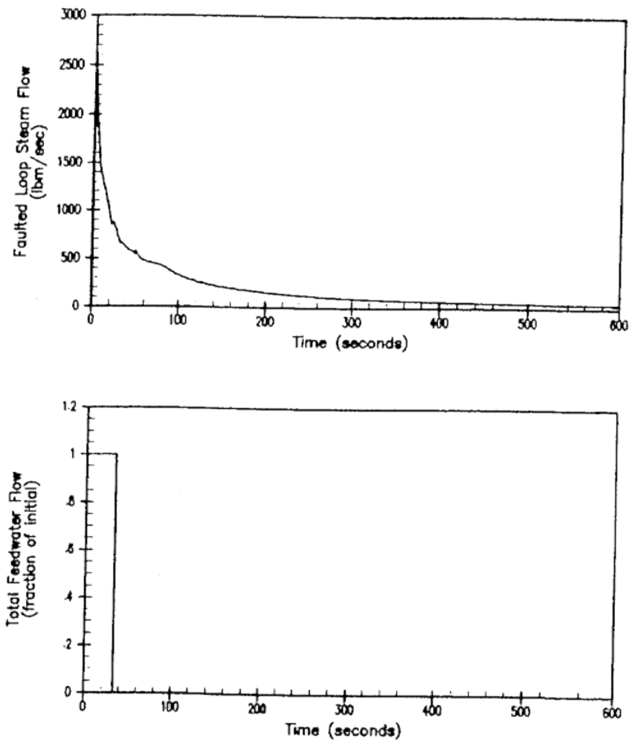

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Figure 15.1-19 *Steam Line Rupture, 1.4ft^2 Break with Power, One Loop in Service, Faulted Loop Steam Flow and Total Feedwater FlowVersus Time*

Figure 15.1-20 Steam Line Rupture, 1.4ft^2 Break with Power, One Loop in Service, Core Averaged Boron and Reactivity Versus Time

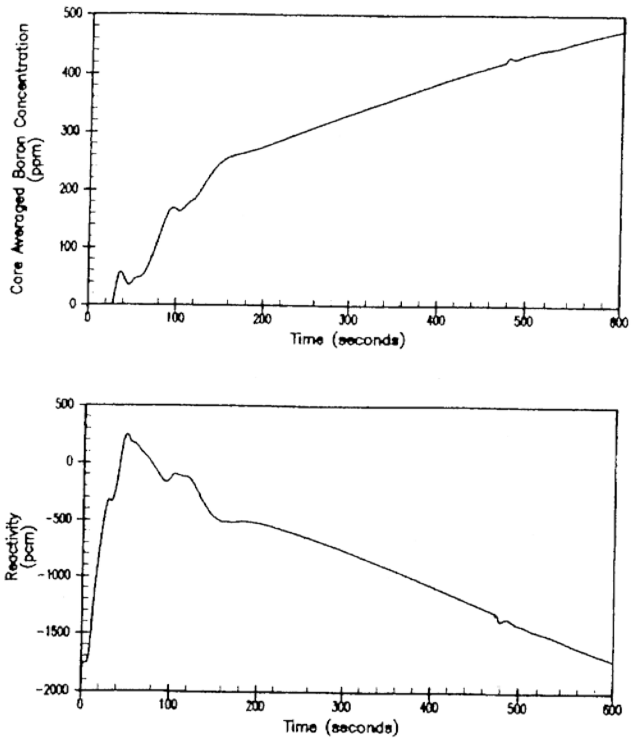

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Figure 15.1-20

Steam Line Rupture, 1.4ft^2 Break with Power, One Loop in Service, Core Averaged Boron and Reactivity Versus Time

Figure 15.1-21 Combined Atmospheric Relief Valve and Main Feedwater Regulating Valve Failure, Nuclear Power and Core Heat Flux Versus Time

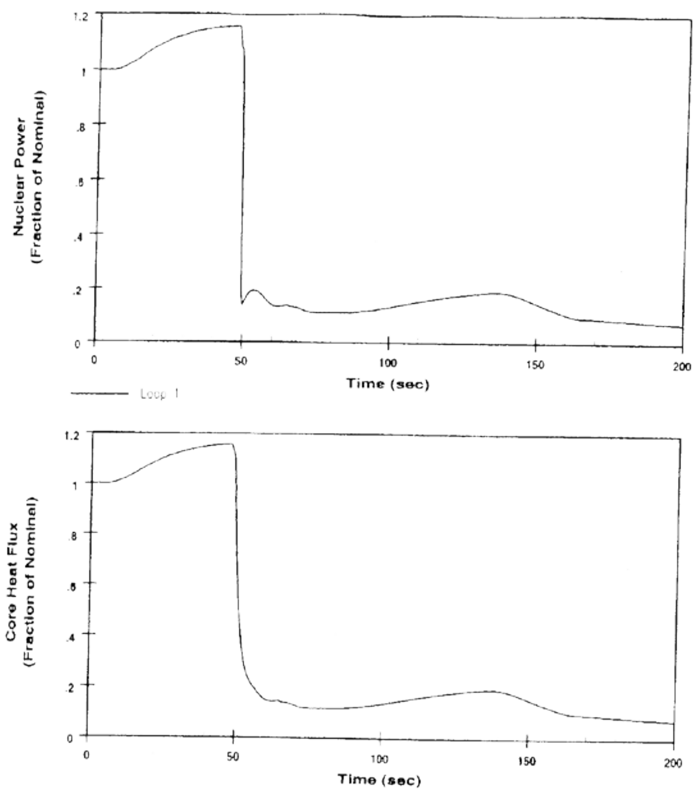

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Figure 15.1-21 *Combined Atmospheric Relief Valve and Main Feedwater Regulating Valve Failure, Nuclear Power and Core Heat Flux Versus Time*

Figure 15.1-22 Combined Atmospheric Relief Valve and Main Feedwater Regulating Valve Failure, Loop Average Temperature and Pressurizer Pressure Versus Time

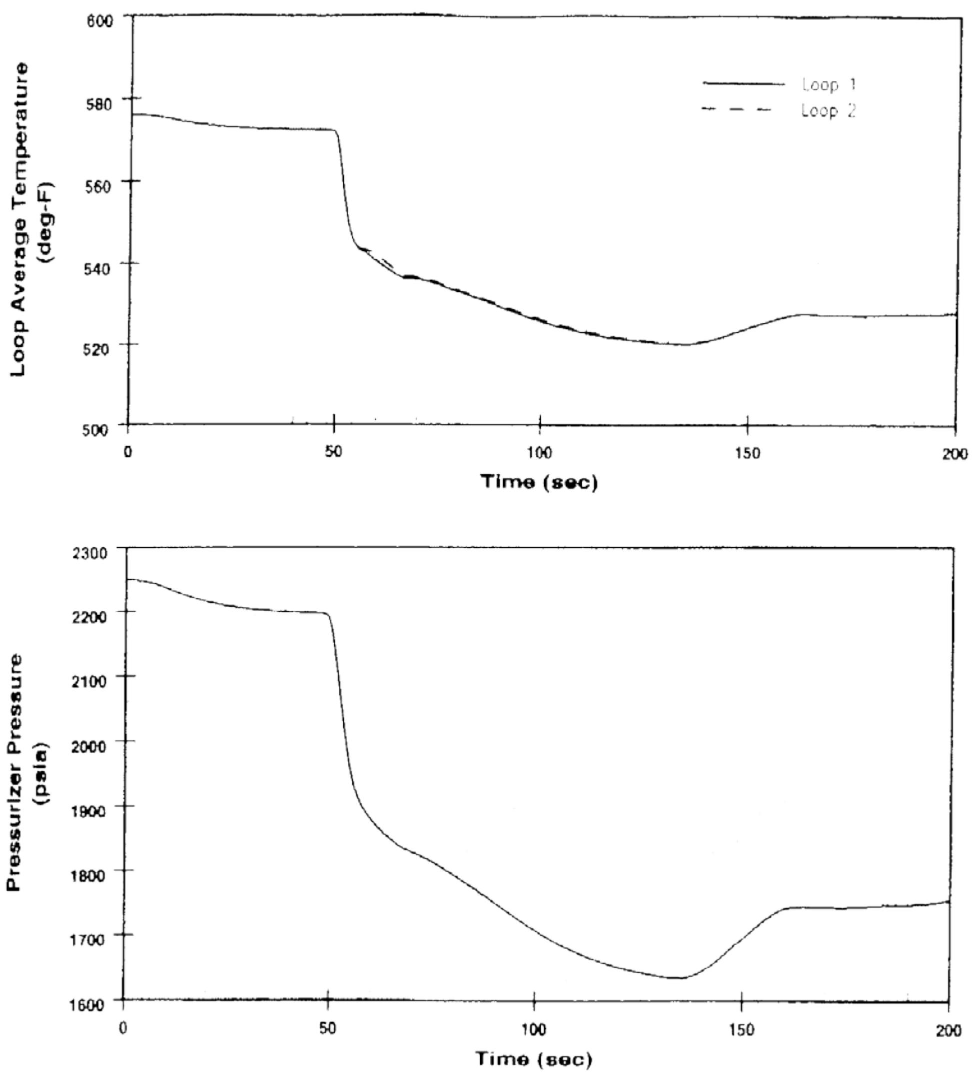

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Figure 15.1-22
Combined Atmospheric Relief Valve and Main Feedwater Regulating Valve Failure, Loop Average Temperature and Pressurizer Pressure Versus Time

Figure 15.1-23 Combined Atmospheric Relief Valve and Main Feedwater Regulating Valve Failure, DNBR Versus Time

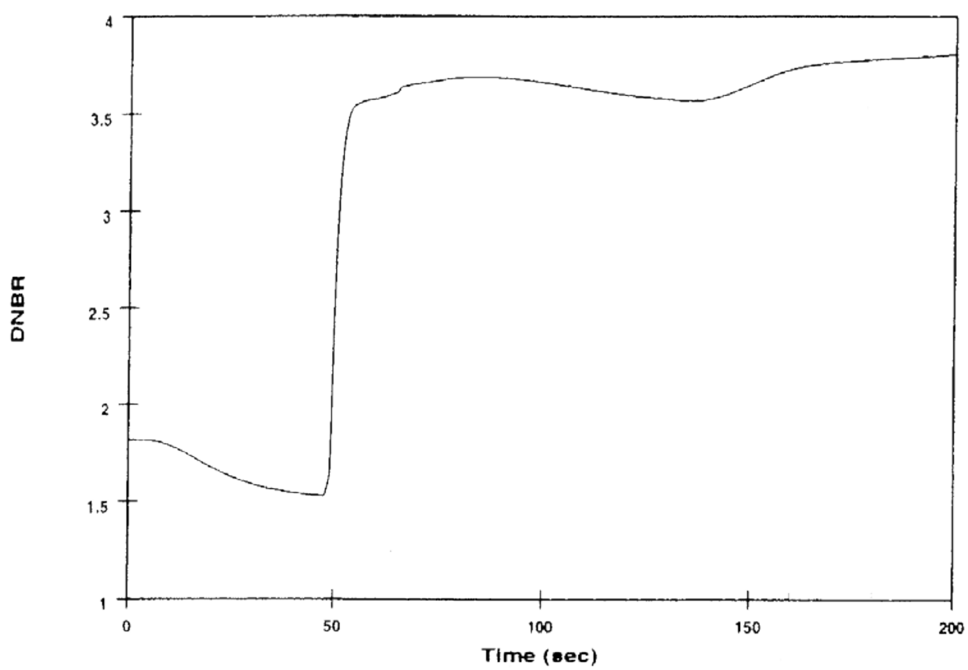

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Figure 15.1-23

Combined Atmospheric Relief Valve and Main Feedwater Regulating Valve Failure, DNBR Versus Time

Figure 15.1-24 Combined Atmospheric Relief Valve and Main Feedwater Regulating Valve Failure, Steam Generator Level and Steam Generator Mass Versus Time

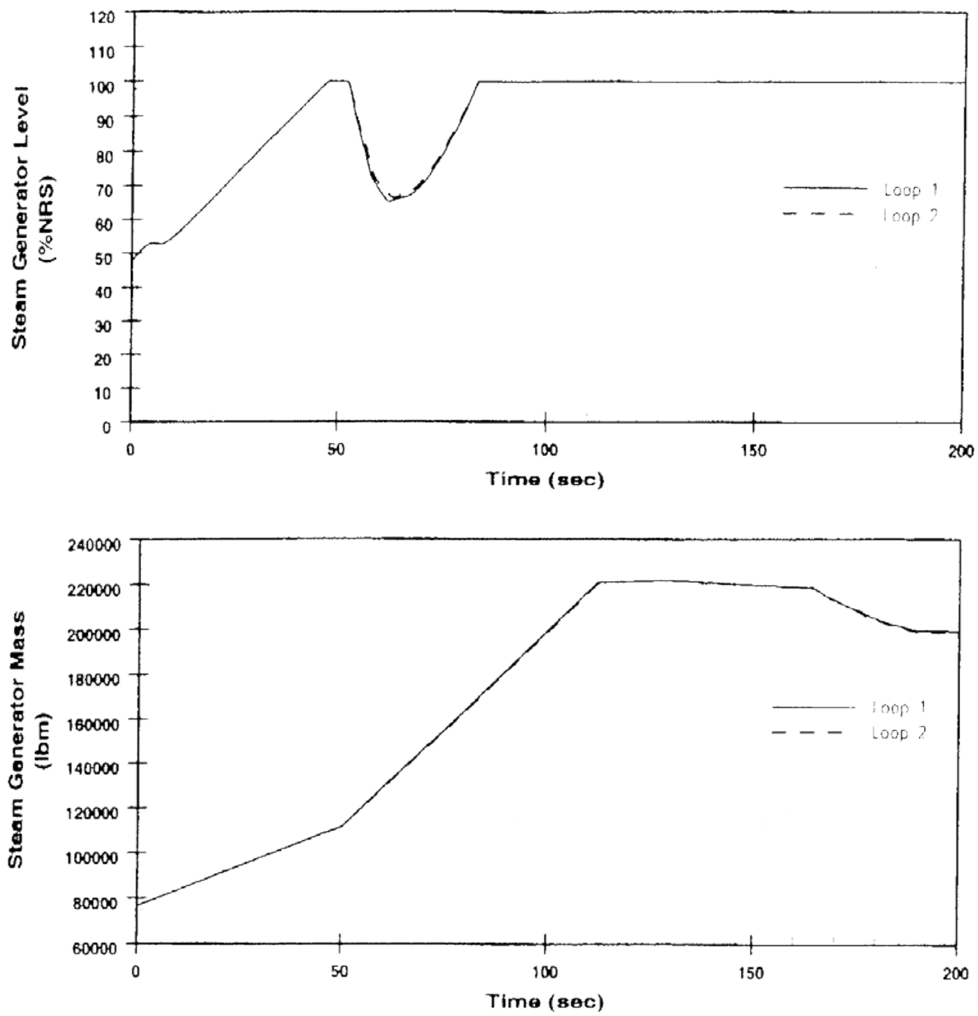

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Figure 15.1-24
Combined Atmospheric Relief Valve and Main Feedwater Regulating Valve Failure, Steam Generator Level and Steam Generator Mass Versus Time

Figure 15.2-1 Loss of Load, with Automatic Pressure Control, Nuclear Power and DNBR Versus Time

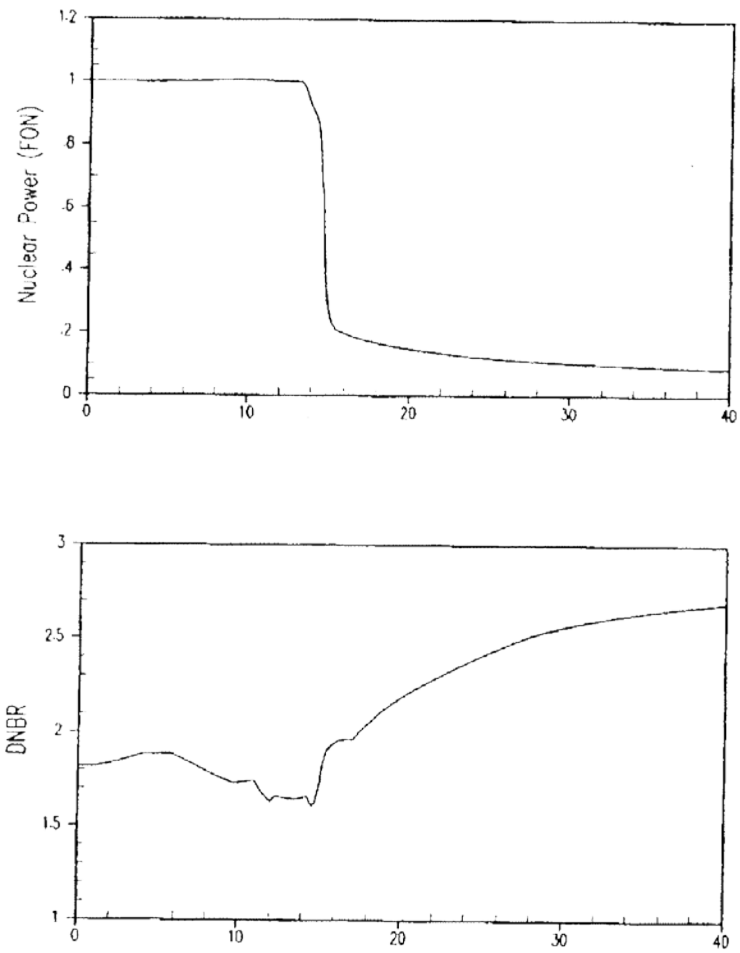

Figure 15.2-2 Loss of Load, with Automatic Pressure Control, RCS Average Temperature and Pressurizer Water Volume Versus Time

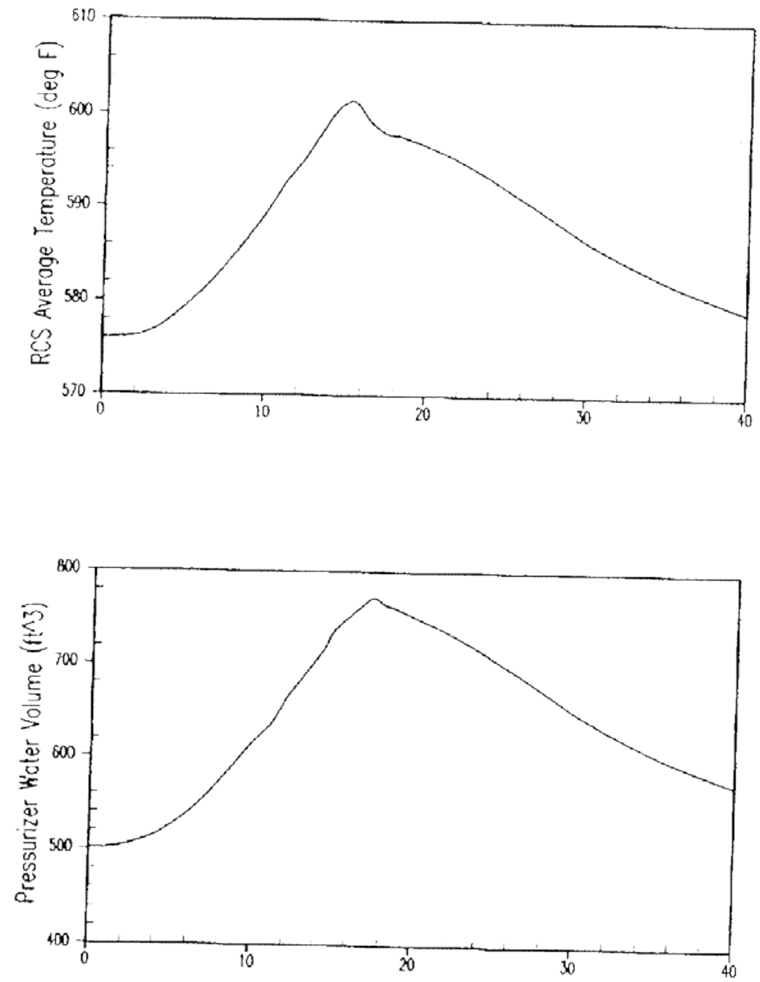

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Figure 15.2-2
Loss of Load, with Automatic Pressure Control, RCS Average Temperature and Pressurizer Water Volume Versus Time

Figure 15.2-3 Loss of Load, with Automatic Pressure Control, Steam Generator Pressure and Pressurizer Pressure Versus Time

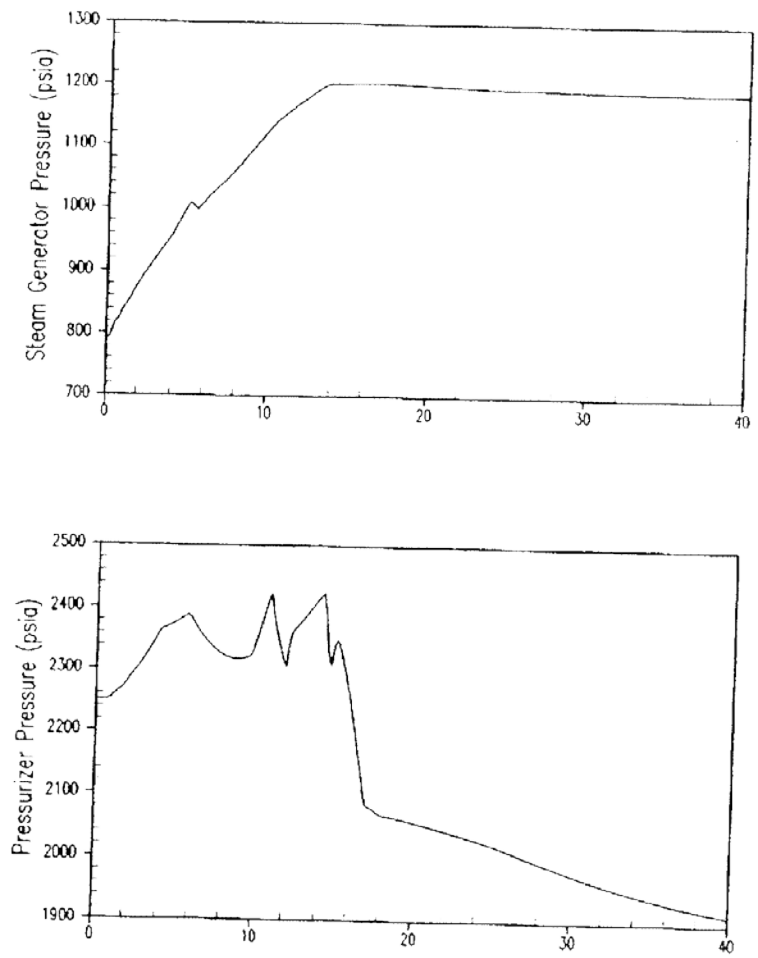

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Figure 15.2-3
Loss of Load, with Automatic Pressure Control, Steam Generator Pressure and Pressurizer Pressure Versus Time

Figure 15.2-4 Loss of Load, Without Pressure Control, Nuclear Power Versus Time

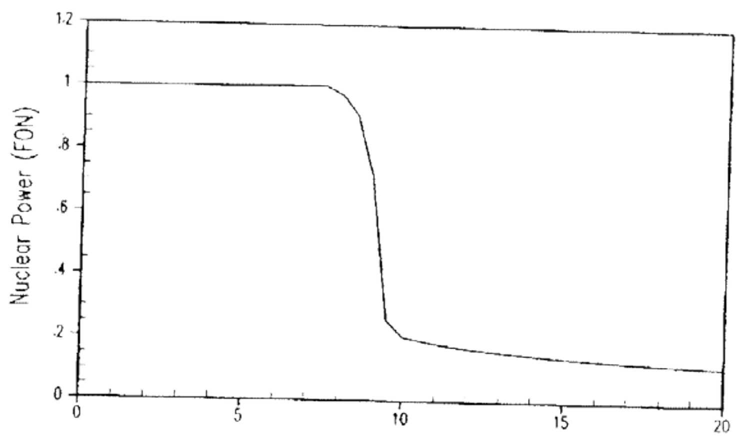

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Figure 15.2-4
Loss of Load, Without Pressure Control, Nuclear Power Versus Time

Figure 15.2-5 Loss of Load, Without Pressure Control, RCS Average Temperature and Pressurizer Water Volume Versus Time

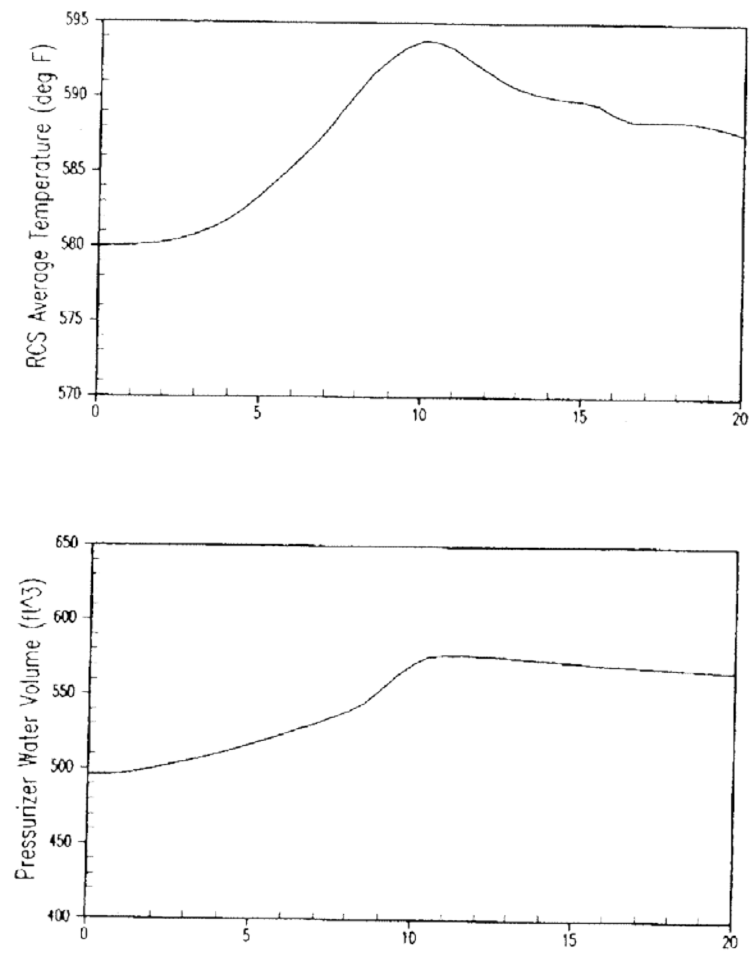

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Figure 15.2-5
Loss of Load, Without Pressure Control, RCS Average Temperature and Pressurizer Water Volume Versus Time

Figure 15.2-6 Loss of Load, Without Pressure Control, Steam Generator Pressure and Reactor Coolant System Pressures Versus Time

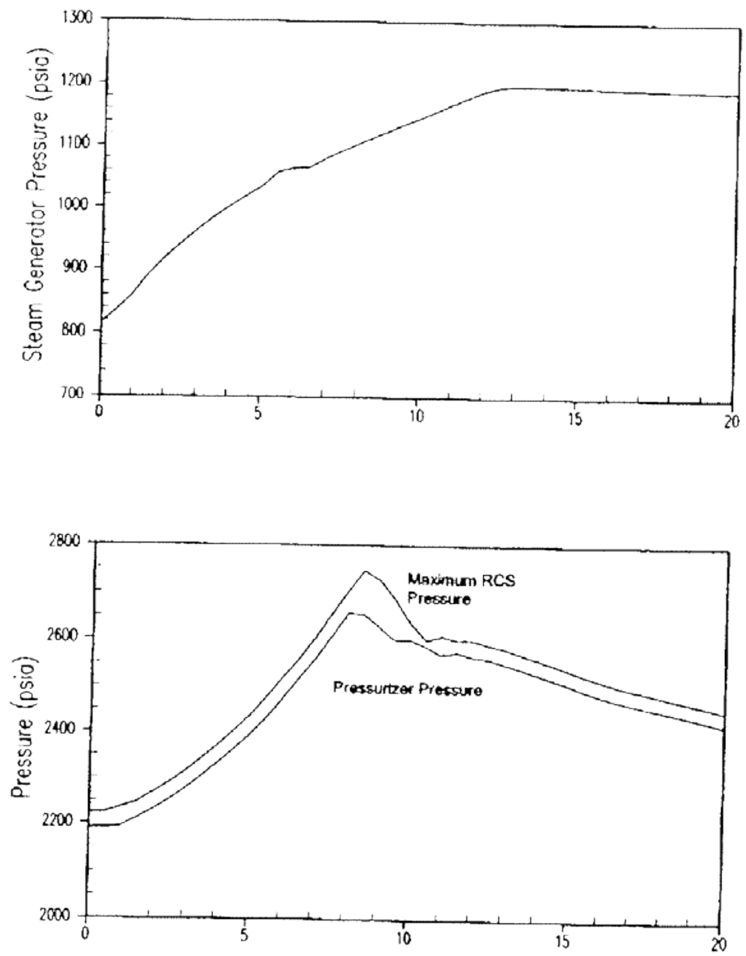

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Figure 15.2-6
Loss of Load, Without Pressure Control, Steam Generator Pressure and Reactor Coolant System Pressures Versus Time

Figure 15.2-7 Loss of Load, Peak MSS Pressure Case, Nuclear Power Versus Time

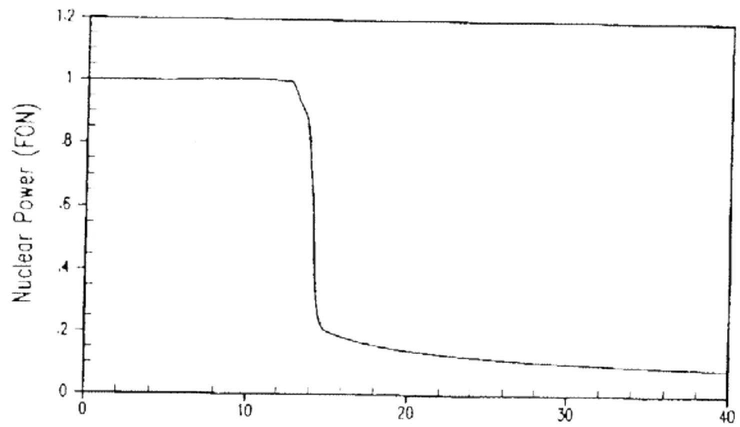

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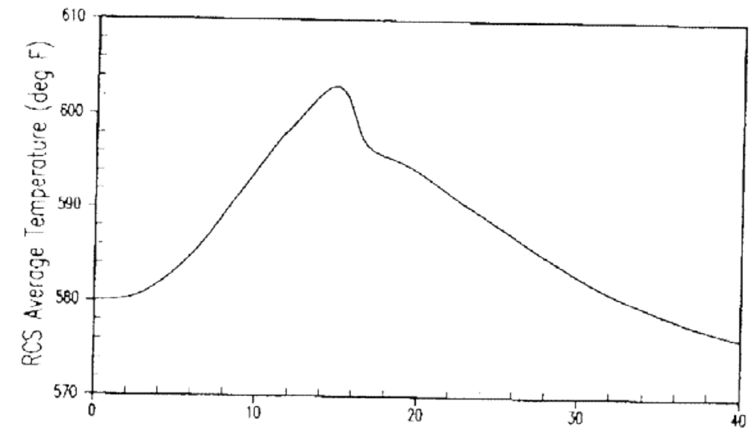
Figure 15.2-7
Loss of Load, Peak MSS Pressure Case, Nuclear Power Versus Time

Figure 15.2-8 Loss of Load, Peak MSS Pressure Case, RCS Average Temperature and Pressurizer Water Volume Versus Time



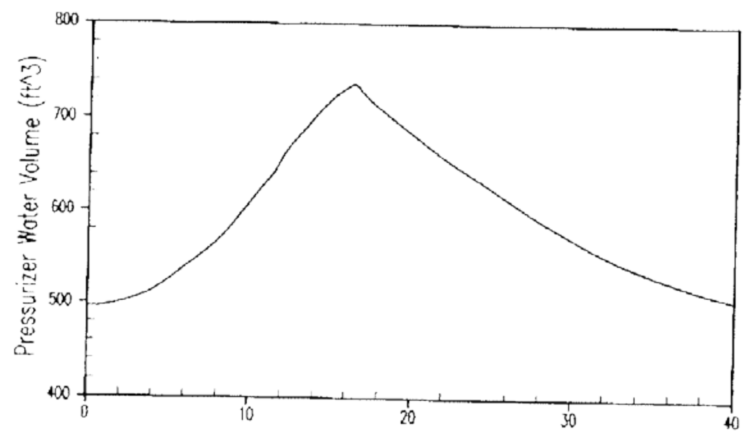


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Figure 15.2-8
Loss of Load, Peak MSS Pressure Case, RCS Average Temperature and Pressurizer Water Volume Versus Time

Figure 15.2-9 Loss of Load, Peak MSS Pressure Case, Steam Generator Pressure and Pressurizer Pressure Versus Time

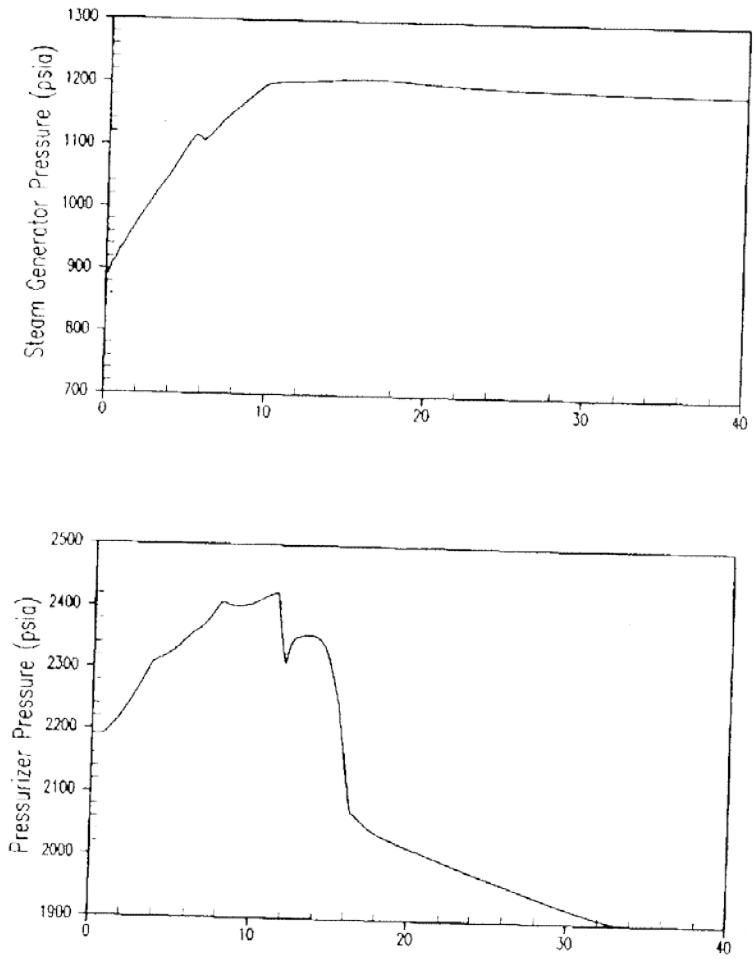

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Figure 15.2-9
Loss of Load, Peak MSS Pressure Case, Steam Generator Pressure and Pressurizer Pressure Versus Time

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Figure 15.2-11 Figure Deleted

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Figure 15.2-12 Figure Deleted

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Figure 15.2-13 Loss of Offsite Alternating Current Power to the Station Auxiliaries, Nuclear Power and Pressurizer Pressure Versus Time

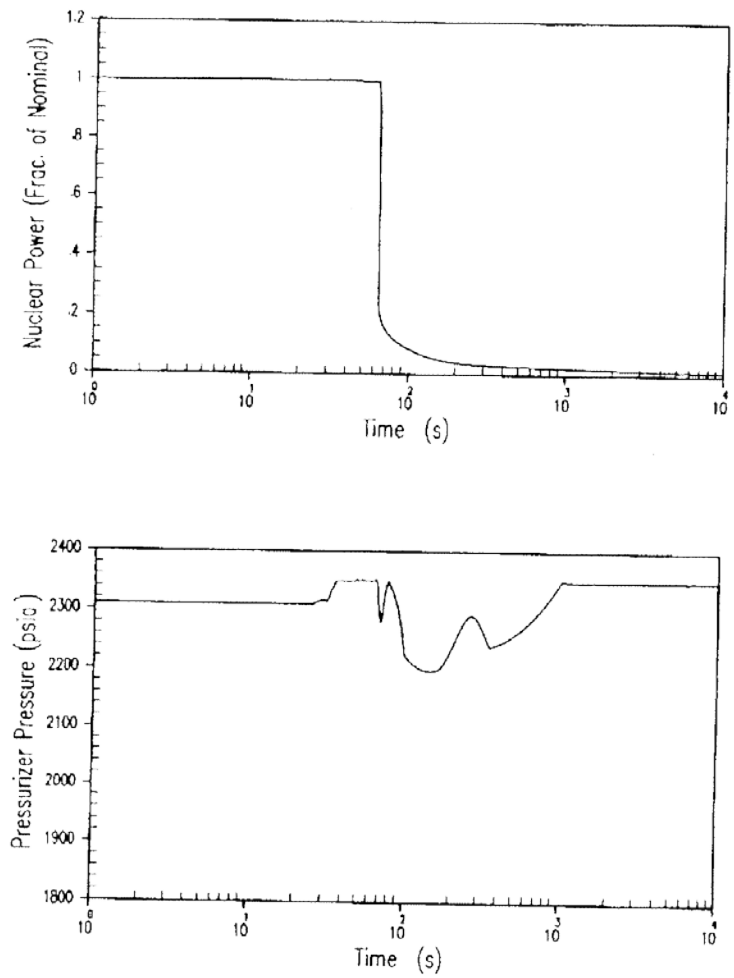

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Figure 15.2-13

Loss of Offsite Alternating Current Power to the Station Auxiliaries, Nuclear Power and Pressurizer Pressure Versus Time

Figure 15.2-14 Loss of Offsite Alternating Current Power to the Station Auxiliaries, Pressurizer Water Volume and Pressurizer Steam Relief Rate Versus Time

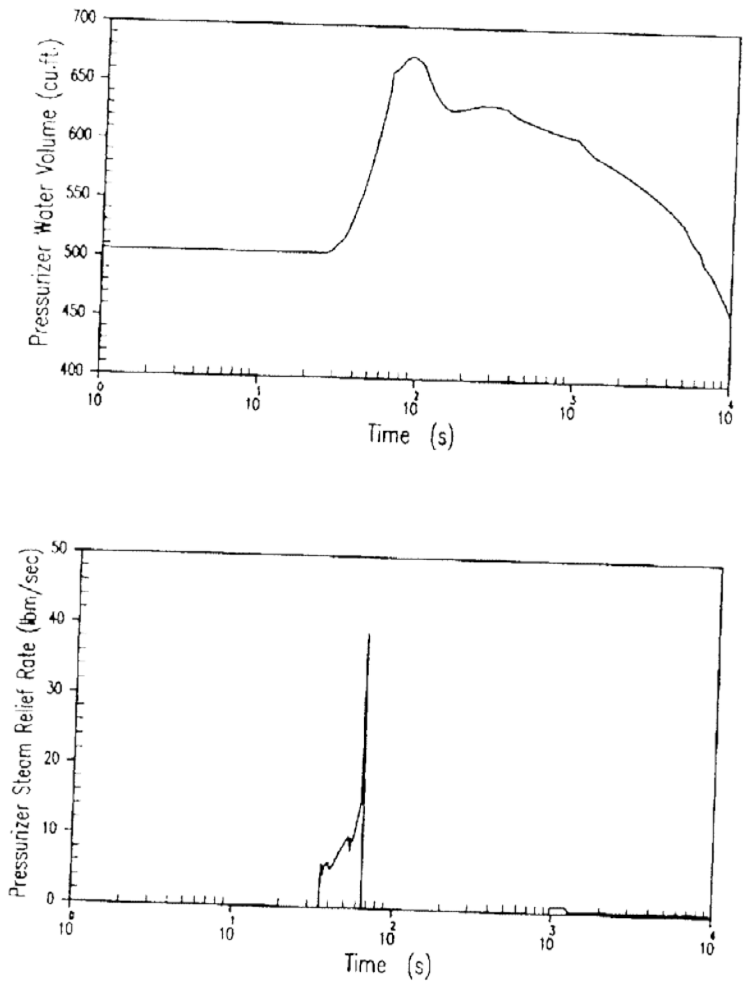

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Figure 15.2-14

Loss of Offsite Alternating Current Power to the Station Auxiliaries, Pressurizer Water Volume and Pressurizer Steam Relief Rate Versus Time

Figure 15.2-15 Loss of Offsite Alternating Current Power to the Station Auxiliaries, Reactor Coolant Flow and Core Inlet/Outlet Temperatures Versus Time

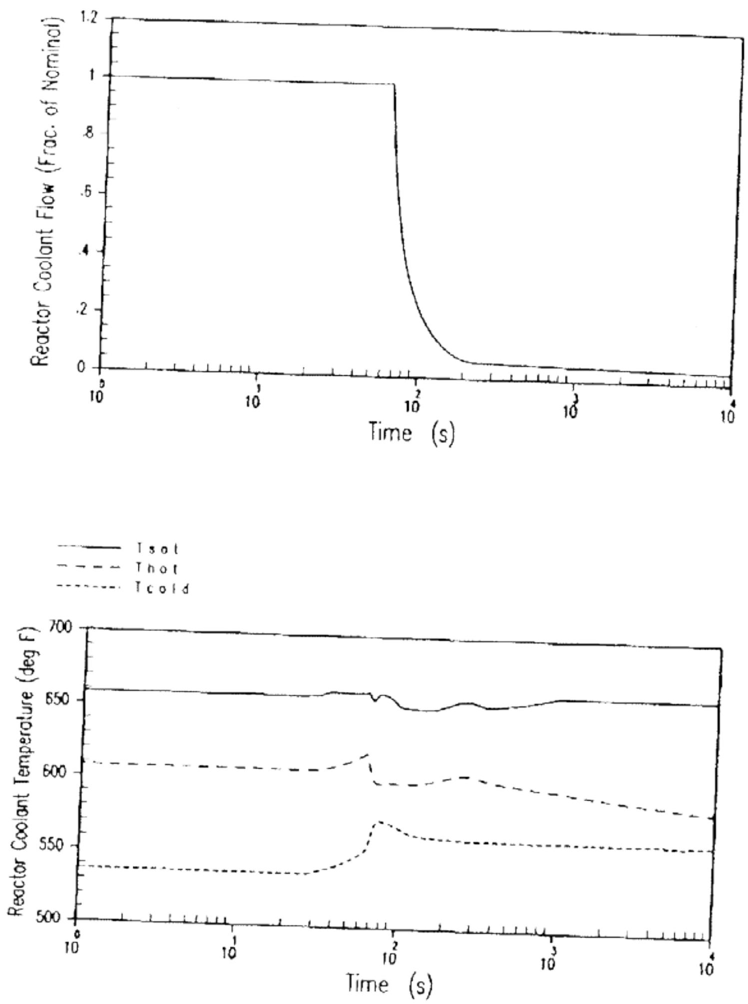

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Figure 15.2-15

Loss of Offsite Alternating Current Power to the Station Auxiliaries, Reactor Coolant Flow and Core Inlet/Outlet Temperatures Versus Time

Figure 15.2-16 Loss of Offsite Alternating Current Power to the Station Auxiliaries, Steam Generator Mass and Steam Generator Pressure Versus Time

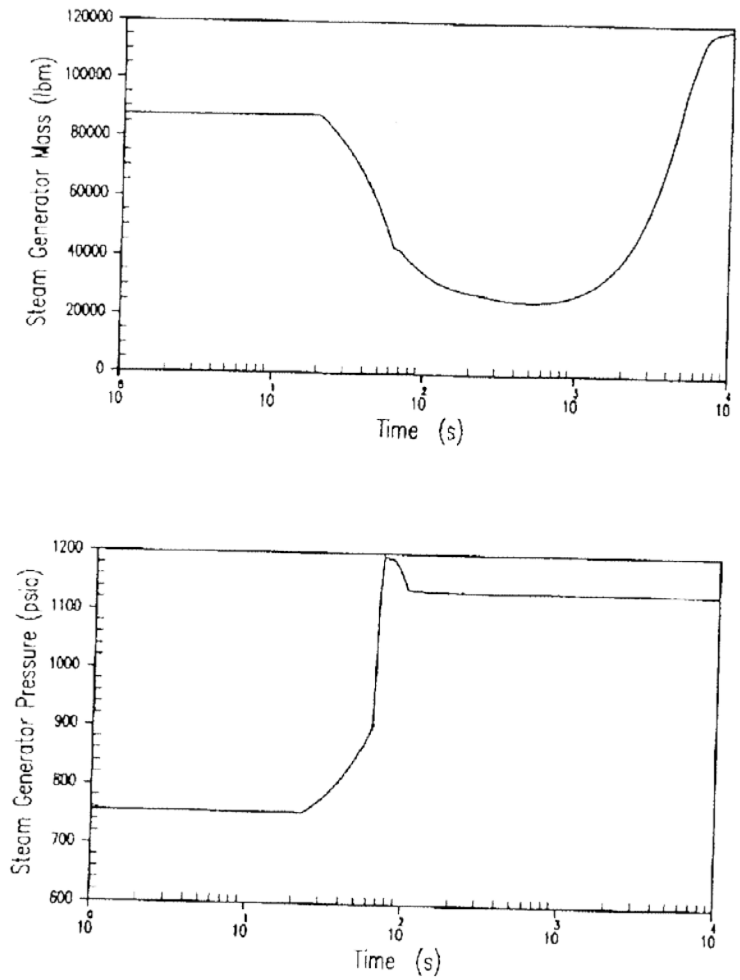

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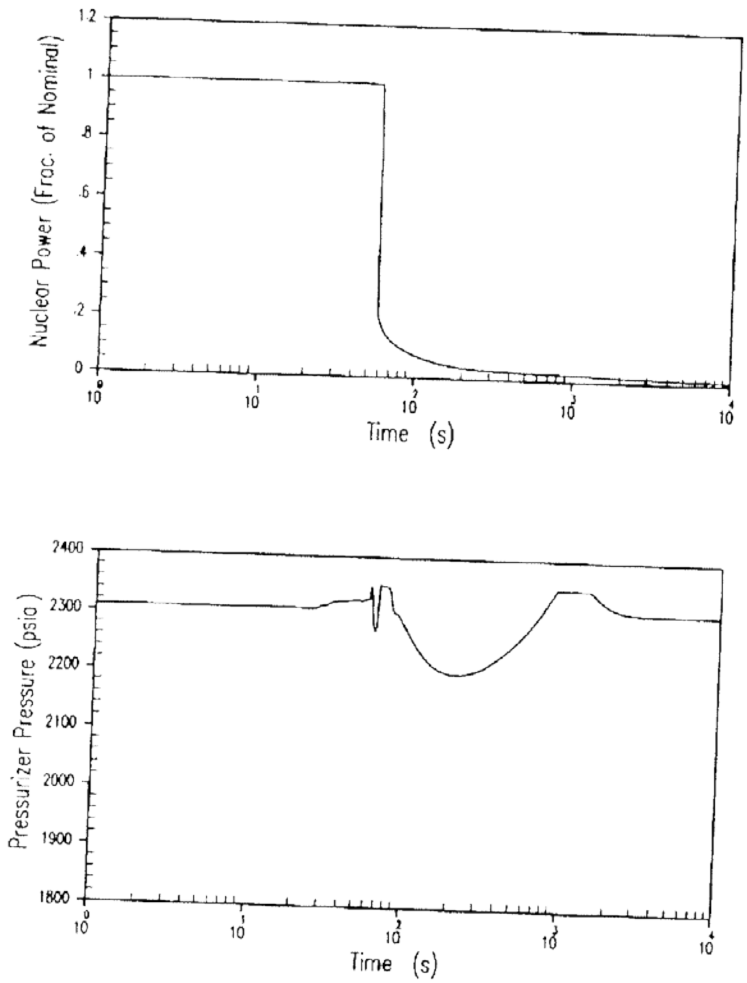
Figure 15.2-16

Loss of Offsite Alternating Current Power to the Station Auxiliaries, Steam Generator Mass and Steam Generator Pressure Versus Time

Figure 15.2-17 Loss of Normal Feedwater With Power, Nuclear Power and Pressurizer Pressure Versus Time



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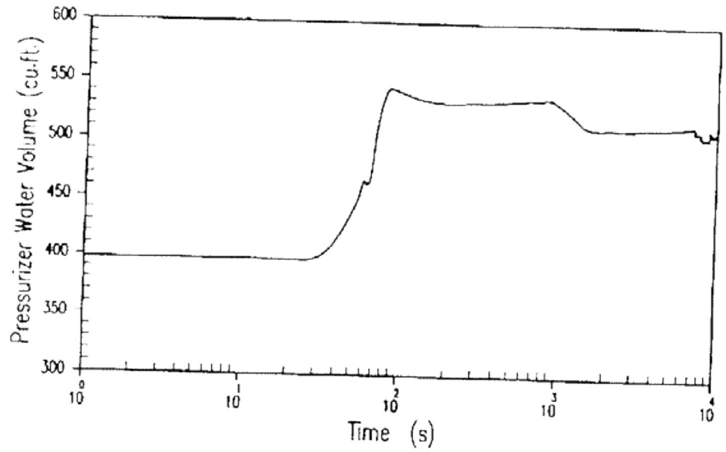
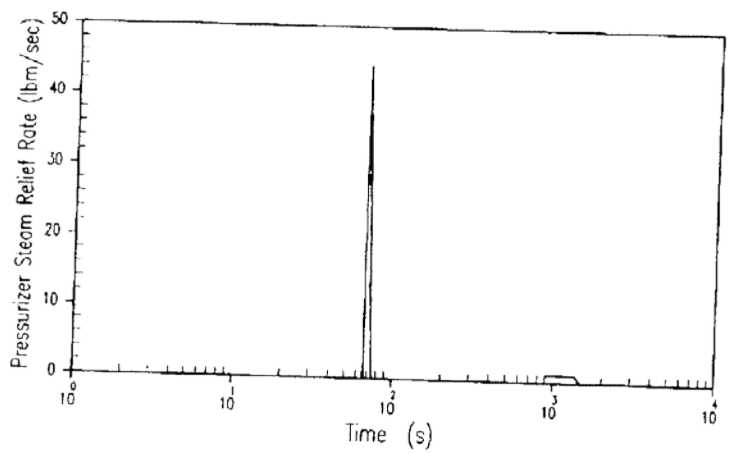
R.E. GINNA NUCLEAR POWER PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.2-17

Loss of Normal Feedwater With Power, Nuclear Power and Pressurizer Pressure Versus Time

Figure 15.2-18 Loss of Normal Feedwater With Power, Pressurizer Water Volume and Pressurizer Steam Relief Rate Versus Time





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R.E. GINNA NUCLEAR POWER PLANT
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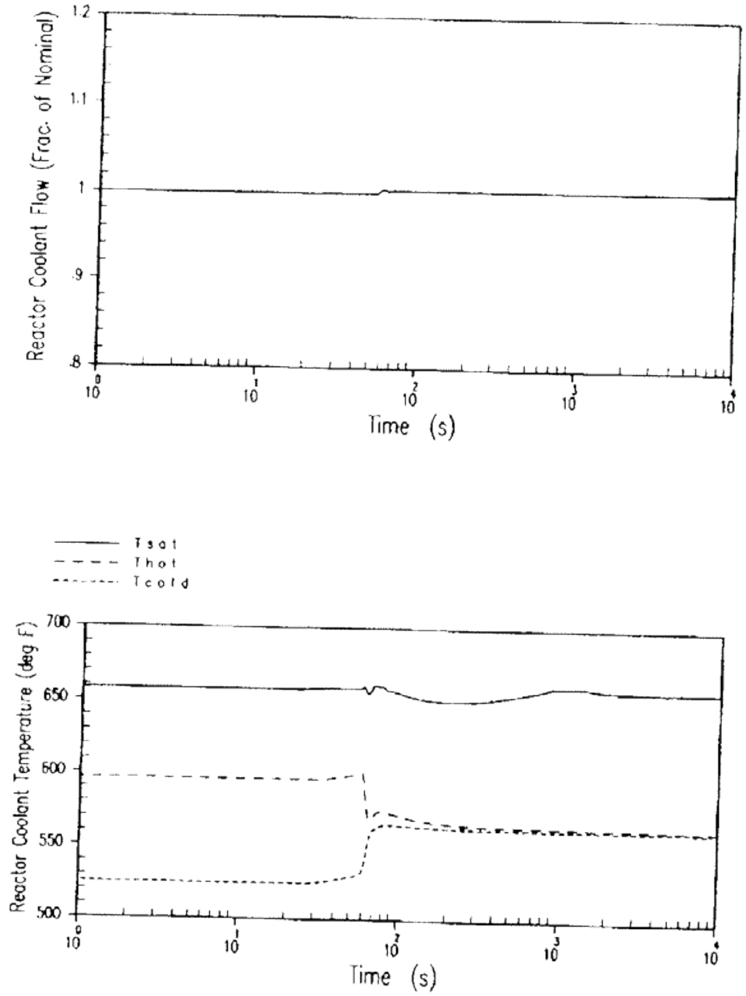
Figure 15.2-18

Loss of Normal Feedwater With Power, Pressurizer Water Volume and Pressurizer Steam Relief Rate Versus Time

Figure 15.2-19 Loss of Normal Feedwater With Power, Reactor Coolant Flow and Core Inlet/ Outlet Temperatures Versus Time



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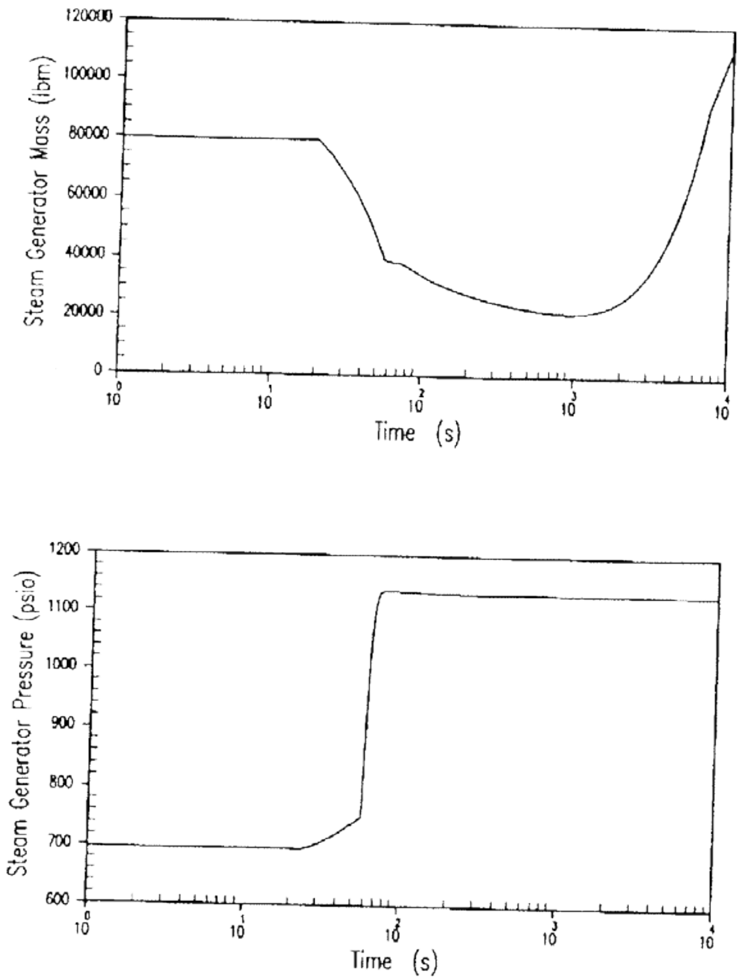
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.2-19

Loss of Normal Feedwater With Power, Reactor Coolant Flow and Core Inlet/Outlet Temperatures Versus Time

Figure 15.2-20 Loss of Normal Feedwater With Power, Steam Generator Mass and Steam Generator Pressure Versus Time



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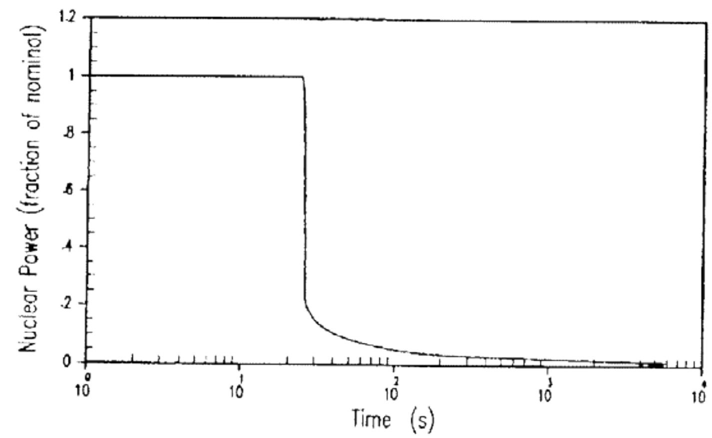
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.2-20

Loss of Normal Feedwater With Power, Steam Generator Mass and Steam Generator Pressure Versus Time

Figure 15.2-21 Feedline Break With Offsite Power; Nuclear Power and Pressurizer Pressure Versus Time



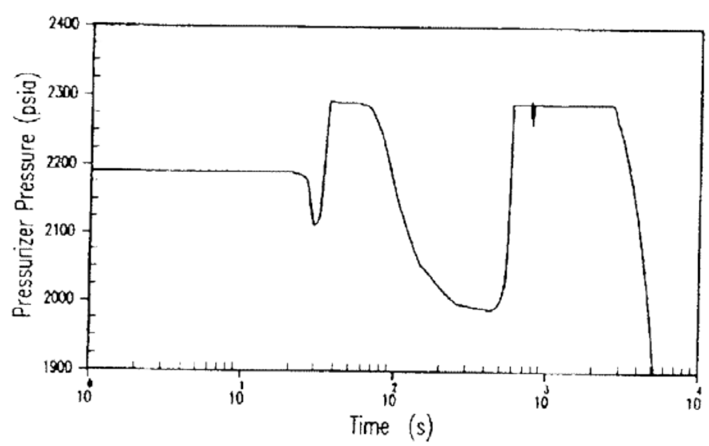


Figure 15.2-22 Feedline Break With Offsite Power; Pressurizer Water Volume and Pressurizer Steam Relief Rate Versus Time

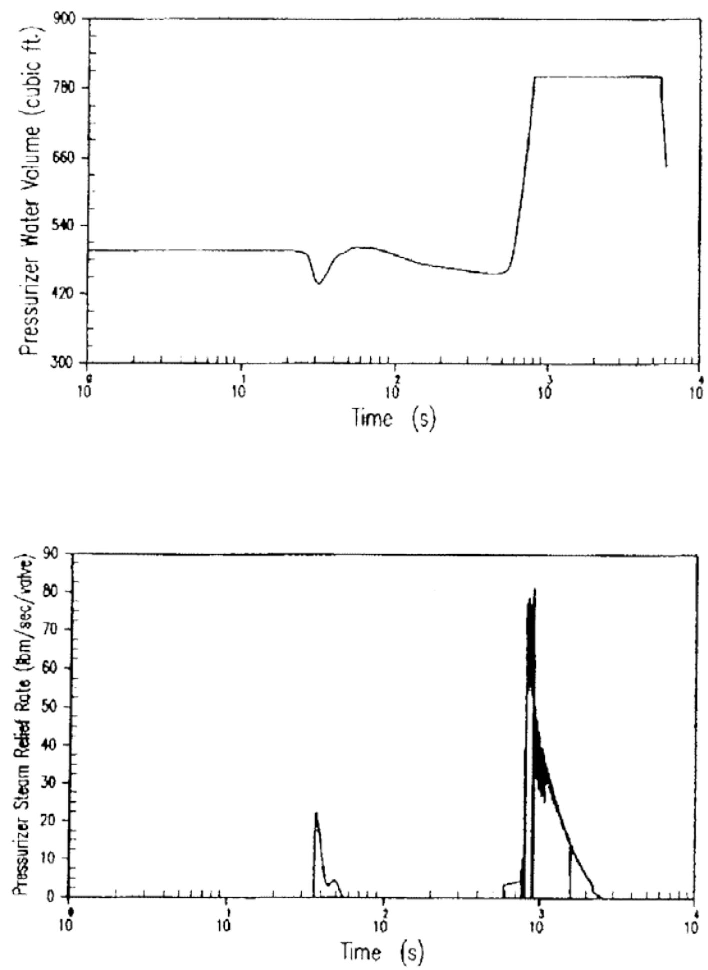
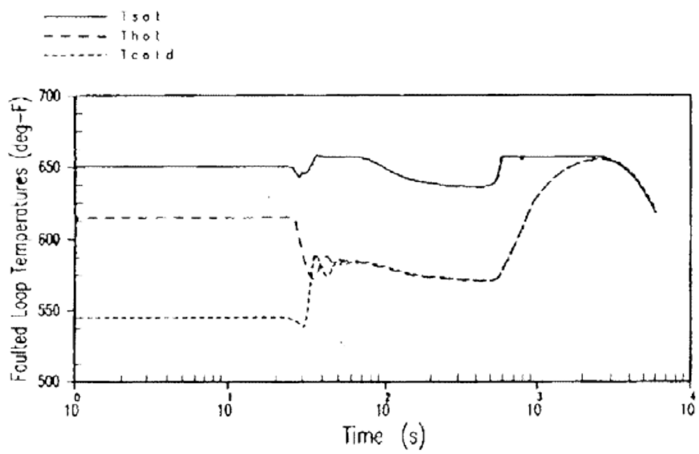


Figure 15.2-23 Feedline Break With Offsite Power; Cold Leg, Hot Leg and Saturation Temperatures Versus Time



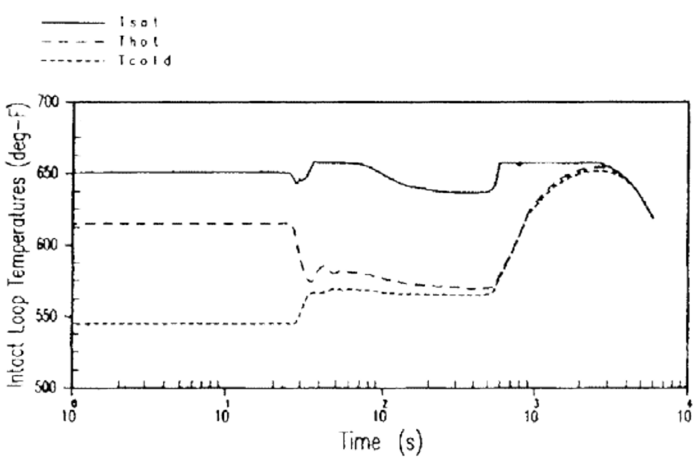


Figure 15.2-24 Feedline Break With Offsite Power; Steam Generator Mass and Steam Generator Pressure Versus Time

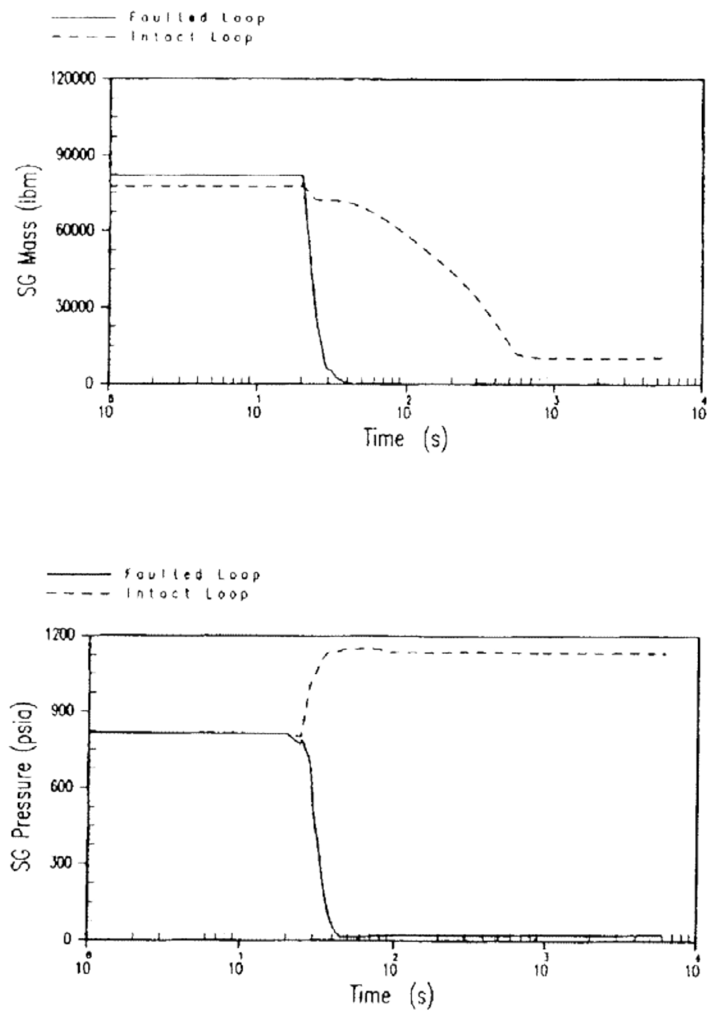


Figure 15.2-25 Feedline Break With Offsite Power; Feedwater Mass Flow Rates Versus Time

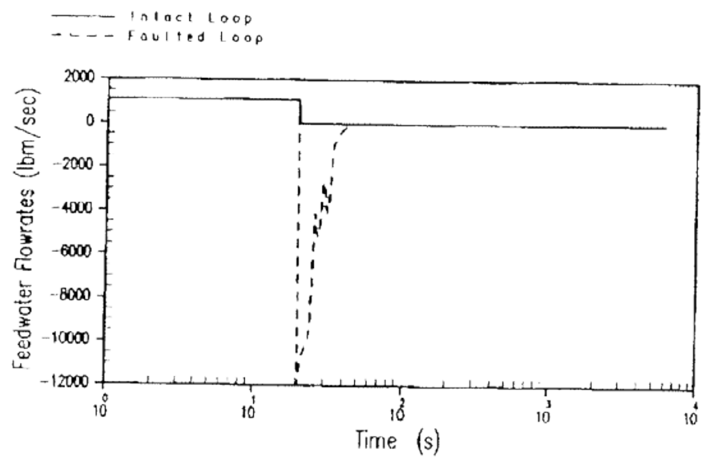


Figure 15.2-26 Feedline Break Without Offsite Power; Nuclear Power and Pressurizer Pressure Versus Time

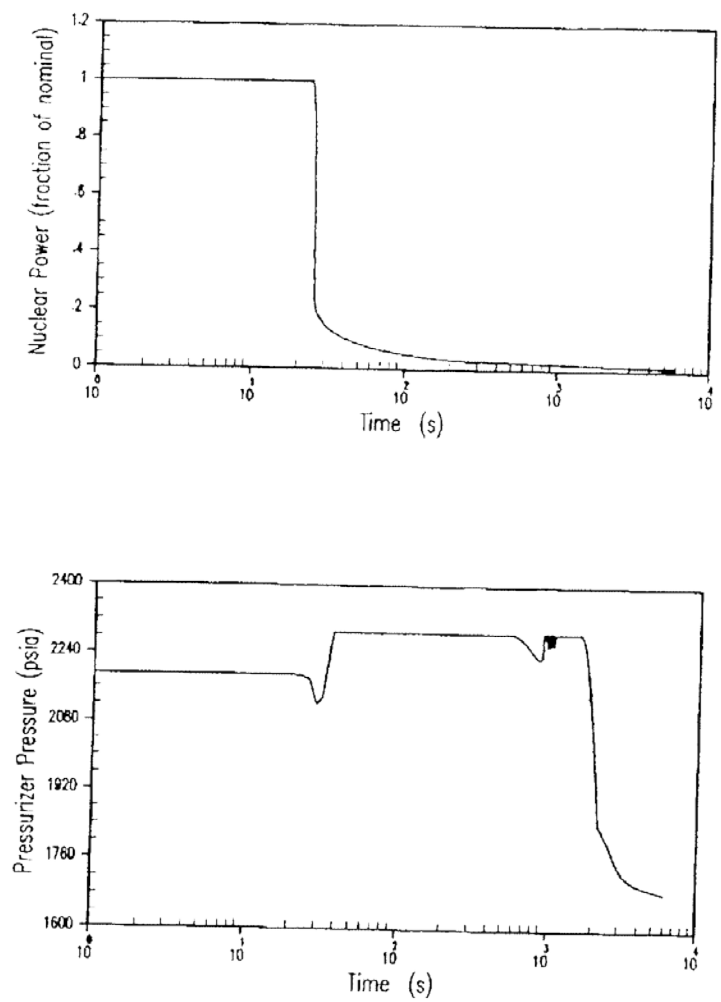
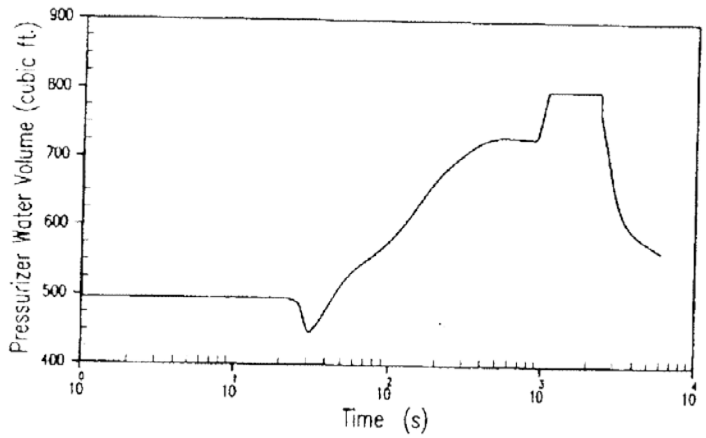


Figure 15.2-27 Feedline Break Without Offsite Power; Pressurizer Water Volume and Pressurizer Steam Relief Rate Versus Time



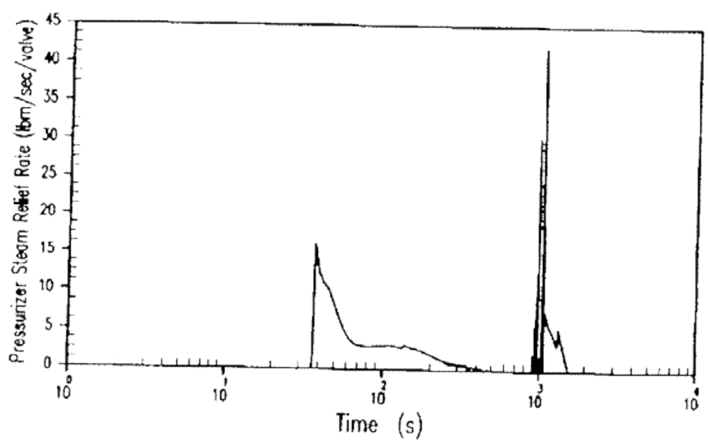
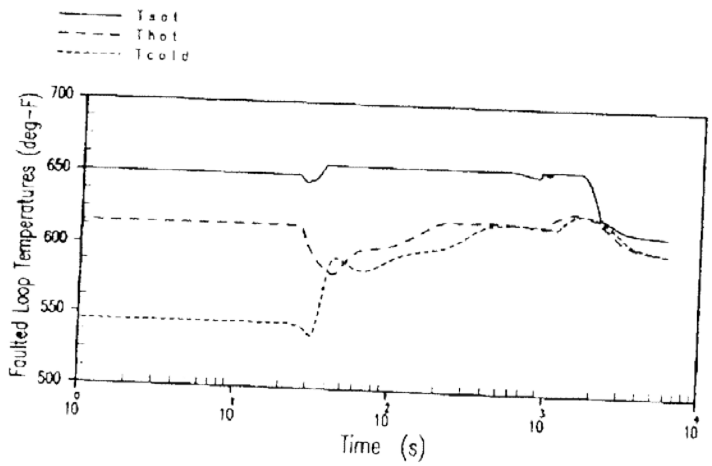


Figure 15.2-28 Feedline Break Without Offsite Power; Cold Leg, Hot Leg and Saturation Temperatures Versus Time



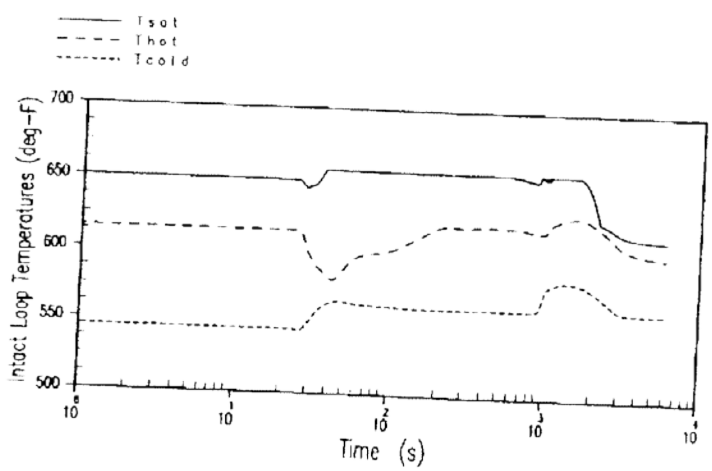
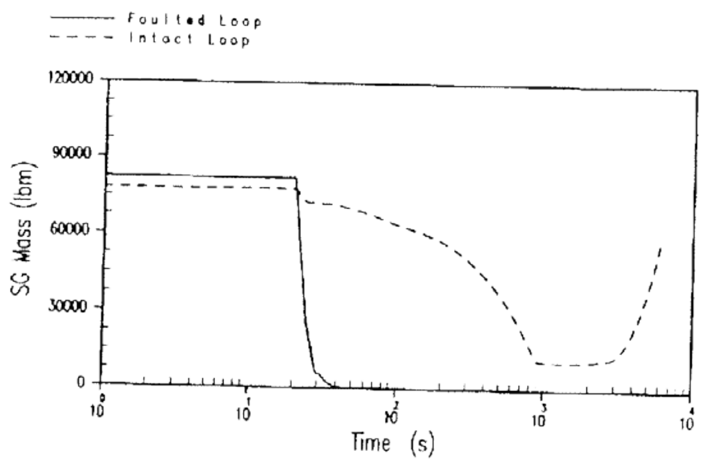


Figure 15.2-29 Feedline Break Without Offsite Power; Steam Generator Mass and Steam Generator Pressure Versus Time



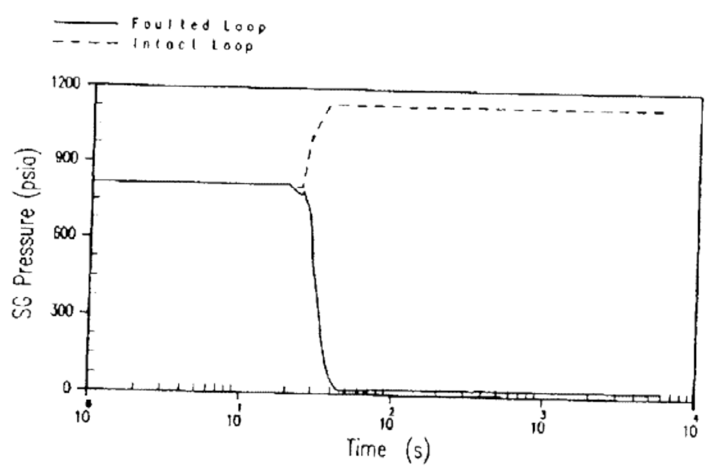


Figure 15.2-30 Feedline Break Without Offsite Power; Feedwater Mass Flow Rates Versus Time

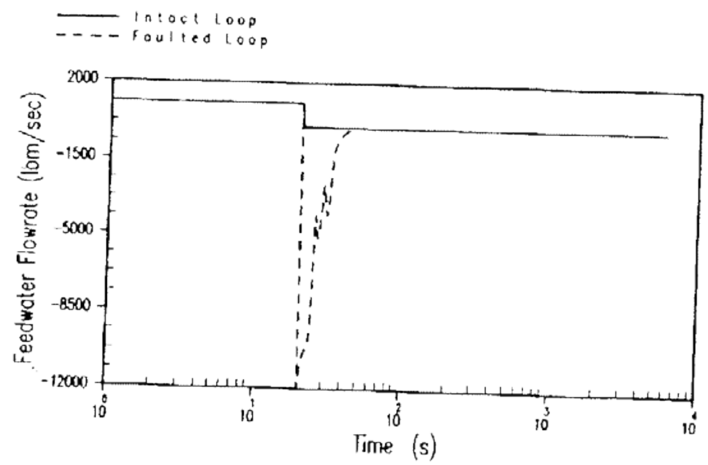
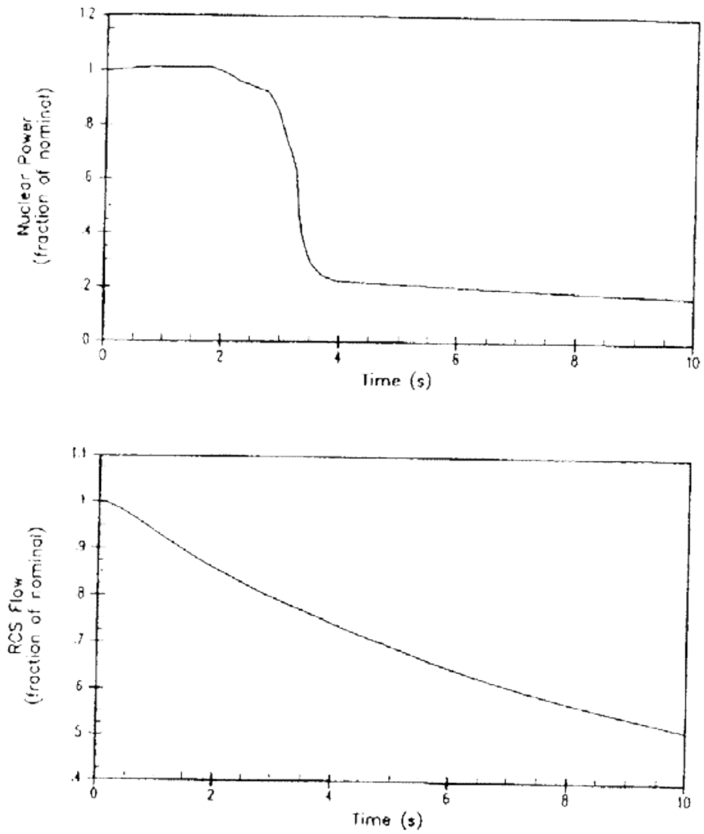


Figure 15.3-1   Full Loss of Flow (Undervoltage), Nuclear Power and RCS Flow Versus Time



ROCHESTER GAS AND ELECTRIC COMPANY
R. E. GINNA NUCLEAR POWER PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.3-1
Full Loss of Flow (Undervoltage)
Nuclear Power and RCS Flow Versus Time

Figure 15.3-1a Full Loss of Flow (Underfrequency), Nuclear Power and RCS Flow Versus Time

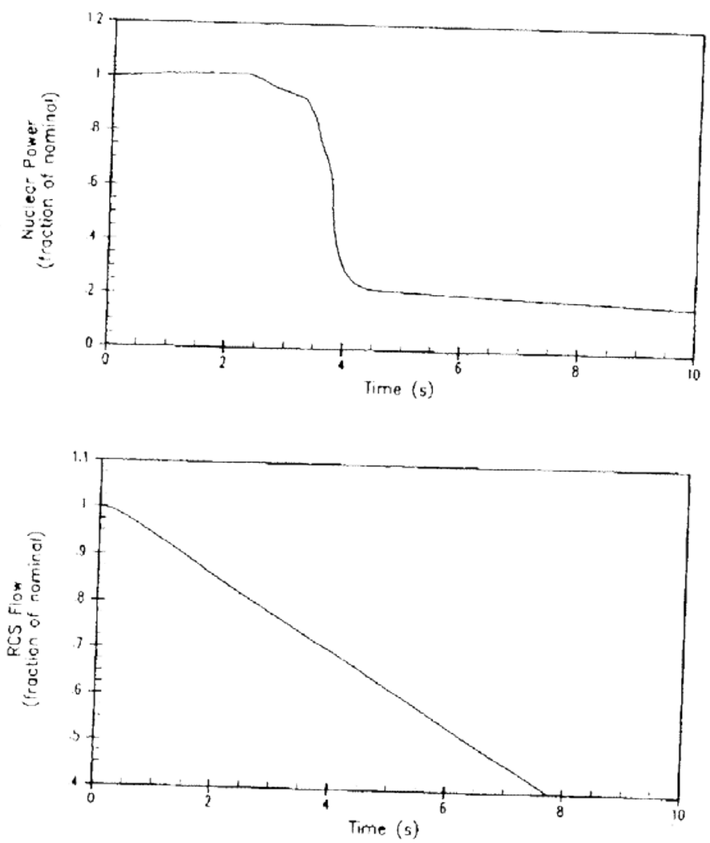


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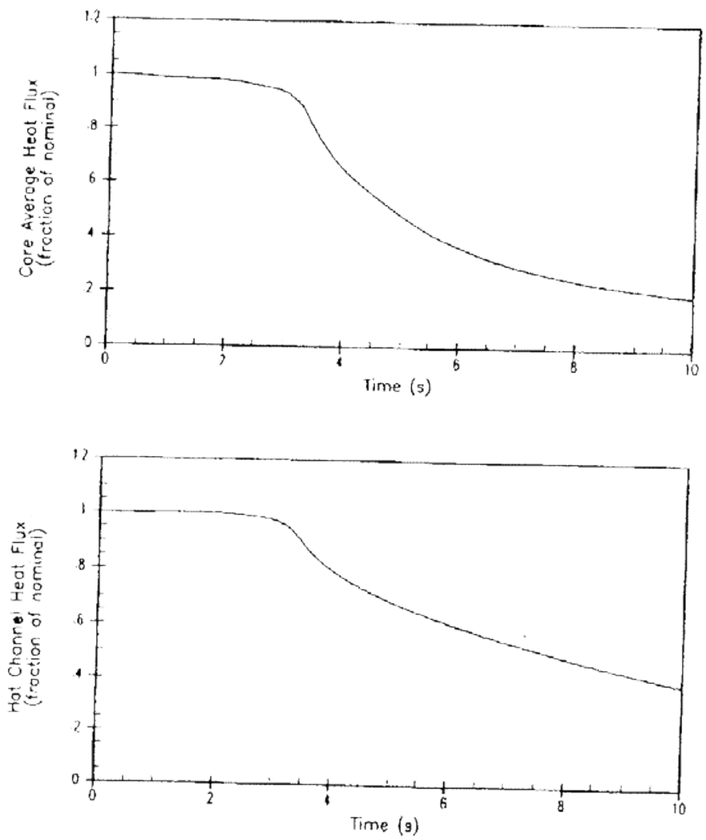
Figure 15.3-1a Full Loss of Flow (Underfrequency) Nuclear Power and RCS Flow Versus Time

Figure 15.3-2 Full Loss of Flow (Undervoltage), Core Average and Hot Channel Heat Flux Versus Time



ROCHESTER GAS AND ELECTRIC COMPANY
R. E. GINNA NUCLEAR POWER PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.3-2
Full Loss of Flow (Undervoltage)
Core Average and Hot Channel Heat Flux Versus Time

Figure 15.3-2a Full Loss of Flow (Underfrequency), Core Average and Hot Channel Heat Flux Versus Time

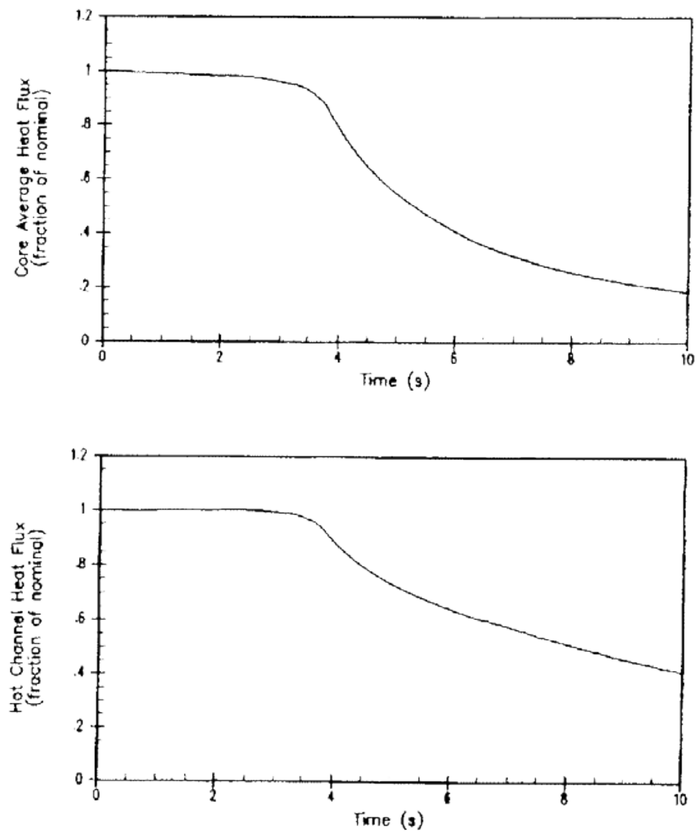


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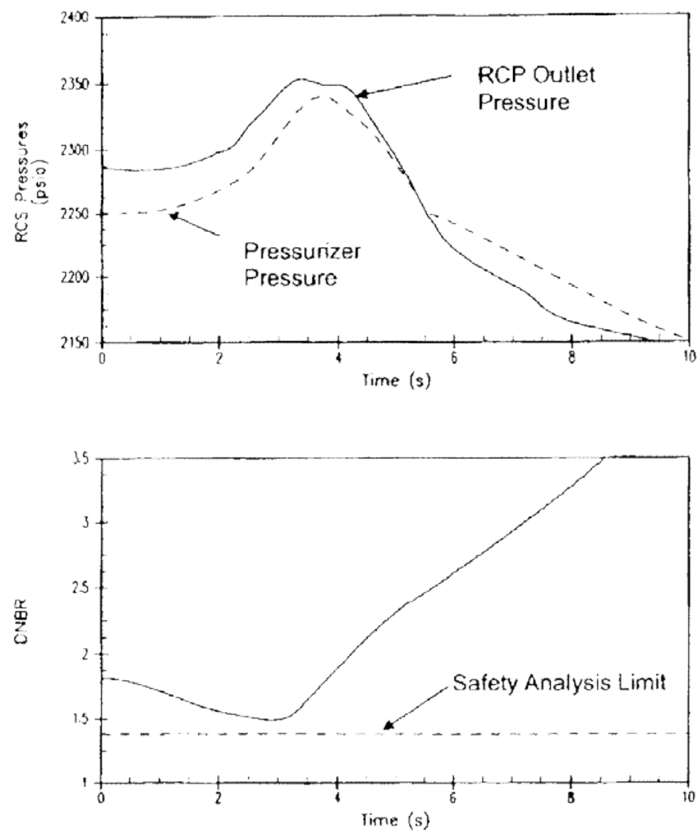
Figure 15.3-2a Full Loss of Flow (Underfrequency) Core Average and Hot Channel Heat Flux Versus Time

Figure 15.3-3 Full Loss of Flow (Undervoltage), RCS Pressures and DNBR Versus Time



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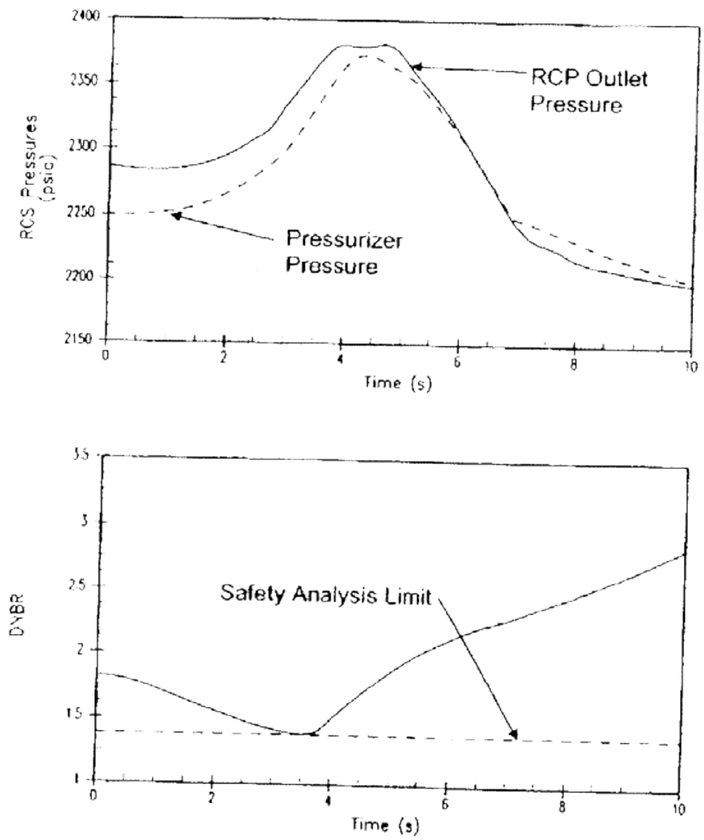
R. E. GINNA NUCLEAR POWER PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.3-3
Full Loss of Flow (Undervoltage)
RCS Pressures and DNBR Versus Time

Figure 15.3-3a Full Loss of Flow (Underfrequency), DNBR and Reactor Coolant System Pressures Versus Time



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UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.3-3a
Full Loss of Flow (Underfrequency)
RCS Pressures and DNBR Versus Time

Figure 15.3-4 Partial Loss of Flow, Nuclear Power and RCS Flow Versus Time

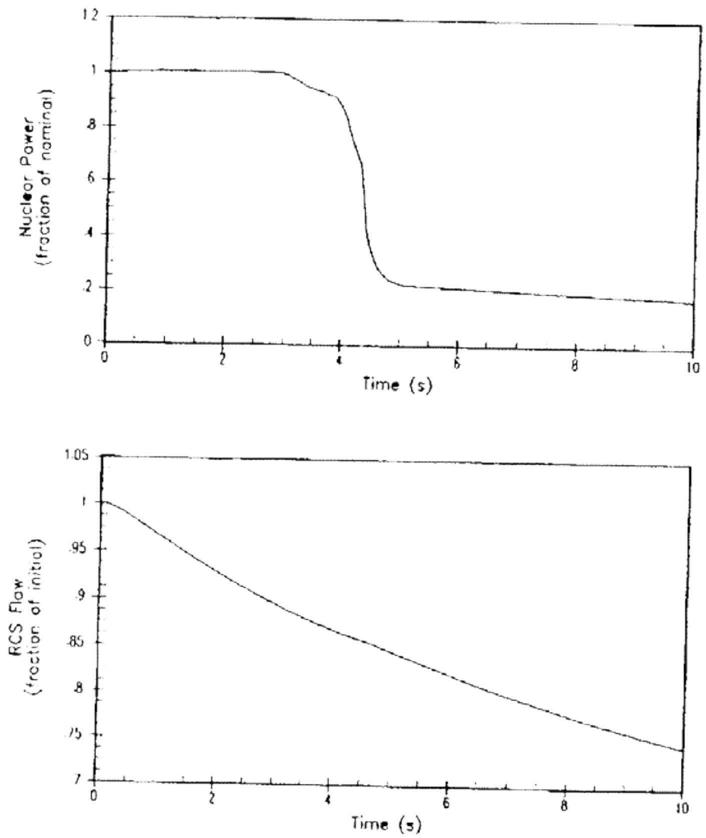

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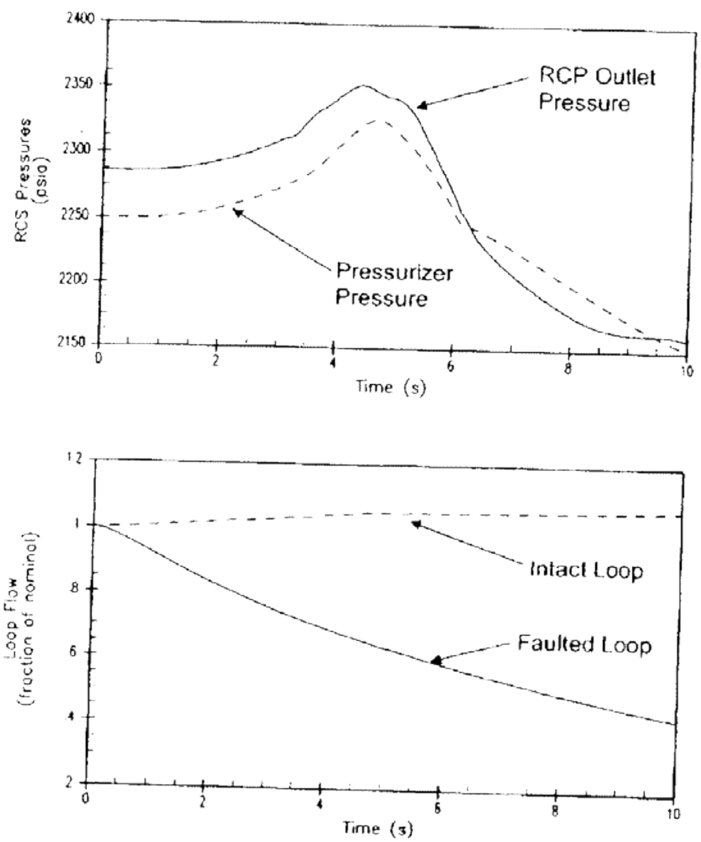
Figure 15.3-4 Partial Loss of Flow Nuclear Power and RCS Flow Versus Time

Figure 15.3-5 Partial Loss of Flow, RCS Pressures and RCS Loop Flows Versus Time



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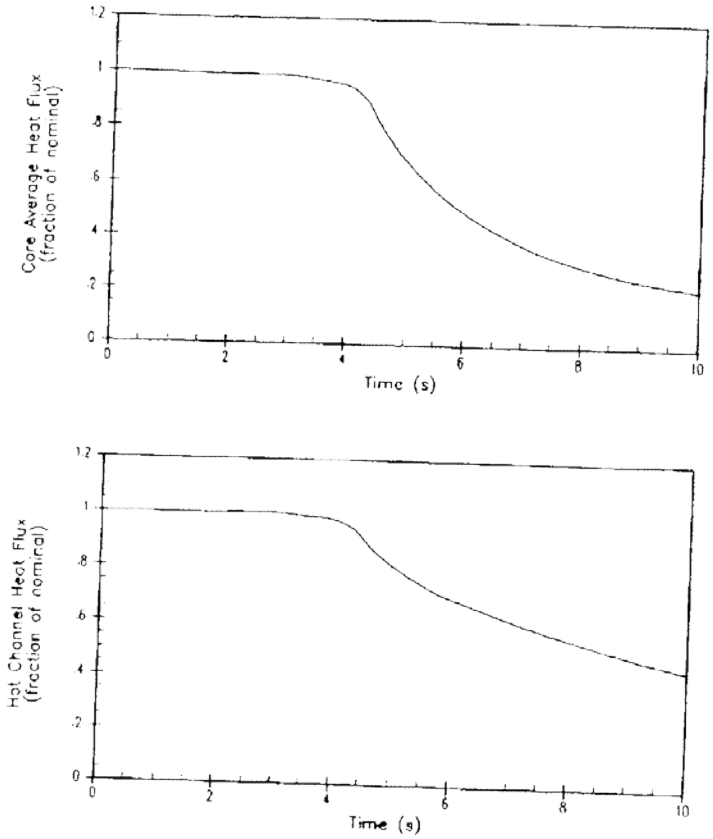
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.3-5
Partial Loss of Flow
RCS Pressures and RCS Loop Flows Versus Time

Figure 15.3-6 Partial Loss of Flow, Core Average and Hot Channel Heat Flux Versus Time



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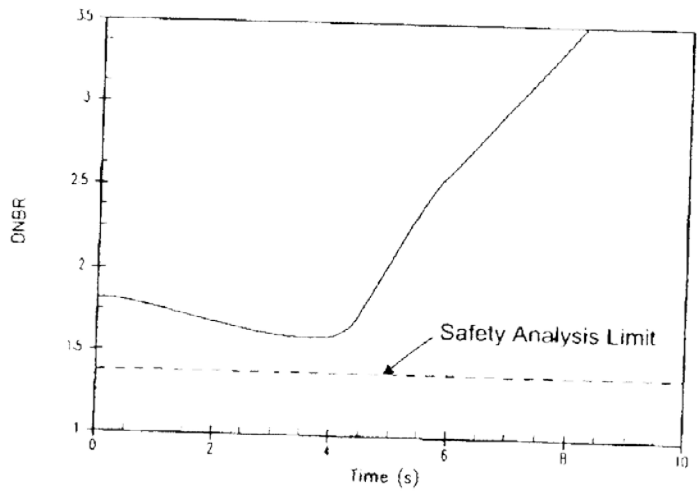
R. E. GINNA NUCLEAR POWER PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.3-6
Partial Loss of Flow
Core Average and Hot Channel Heat Flux Versus Time

Figure 15.3-7 Partial Loss of Flow, DNBR Versus Time



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UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.3-7
Partial Loss of Flow
DNBR Versus Time

Figure 15.3-8 Locked Rotor, RCS Pressures and RCS Loop Flows Versus Time

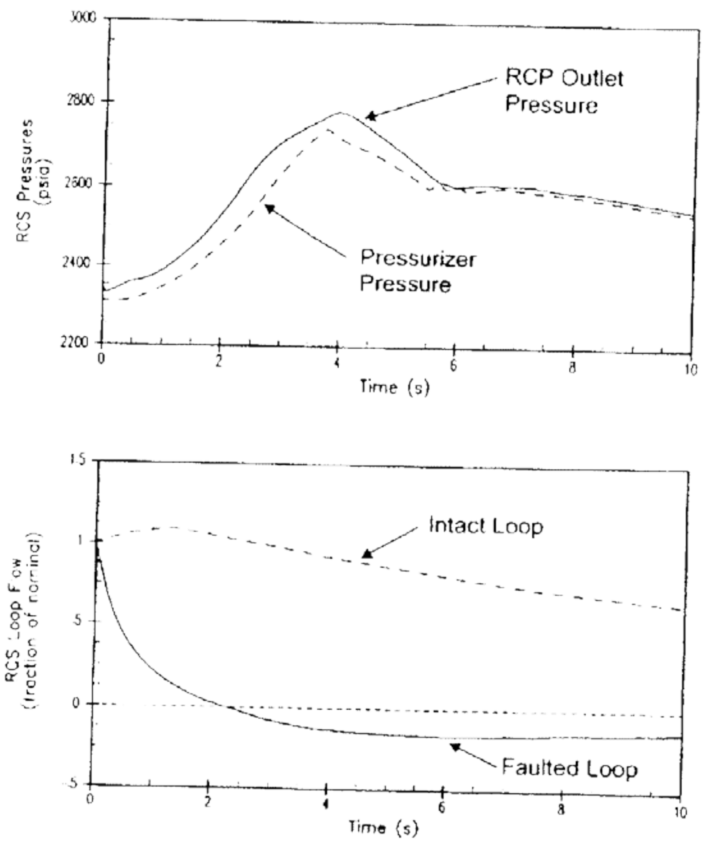

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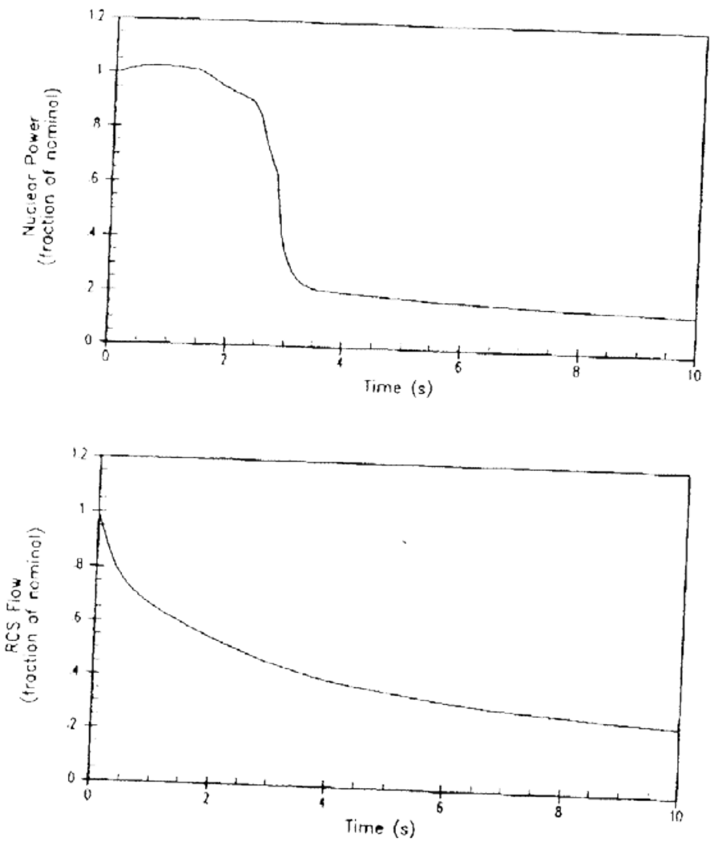
Figure 15.3-8 Locked Rotor RCS Pressures and RCS Loop Flows Versus Time

Figure 15.3-9 Locked Rotor, Nuclear Power and RCS Flow Versus Time



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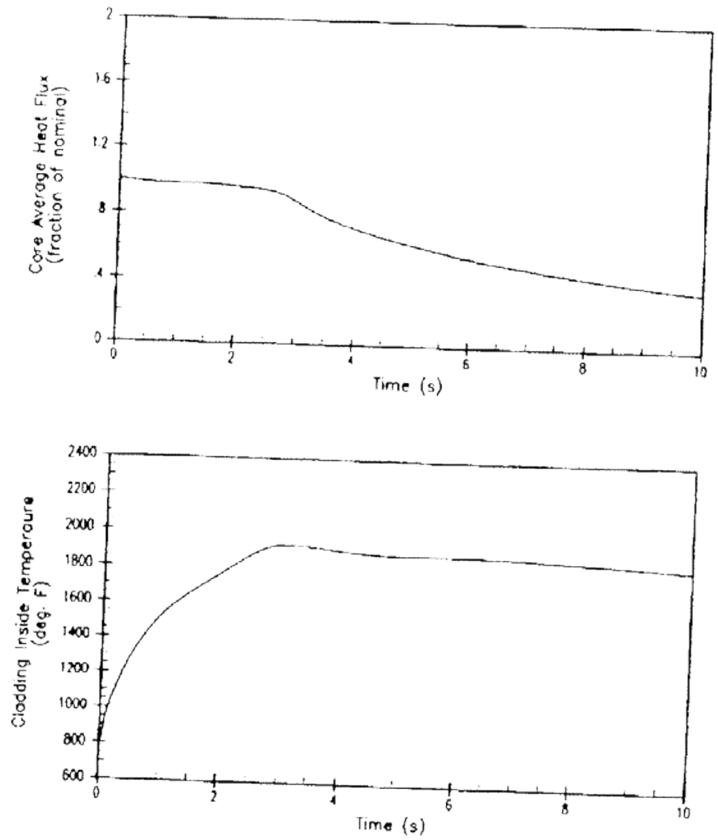
R. E. GINNA NUCLEAR POWER PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.3-9
Locked Rotor
Nuclear Power and RCS Flow Versus Time

Figure 15.3-10 Locked Rotor, Core Average Heat Flux and Cladding Inside Temperature Versus Time



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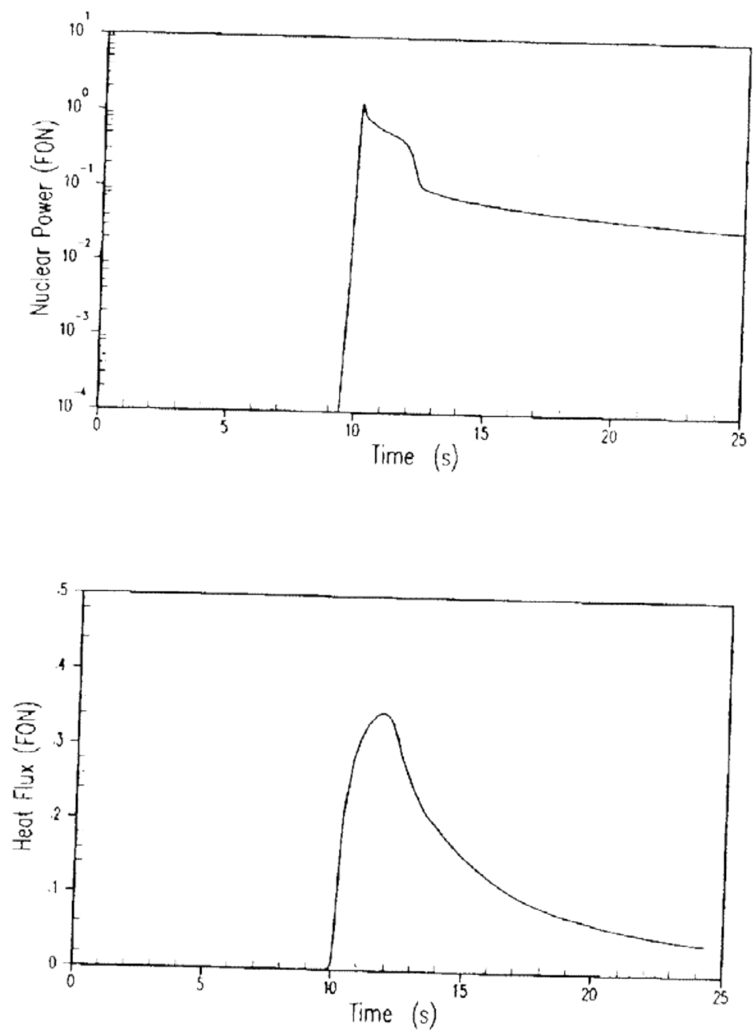
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.3-10
Locked Rotor
Core Average Heat Flux and Cladding Inside Temperature Versus Time

Figure 15.4-1 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal From Subcritical Conditions, Heat Flux and Nuclear Power Versus Time (422V+Fuel)



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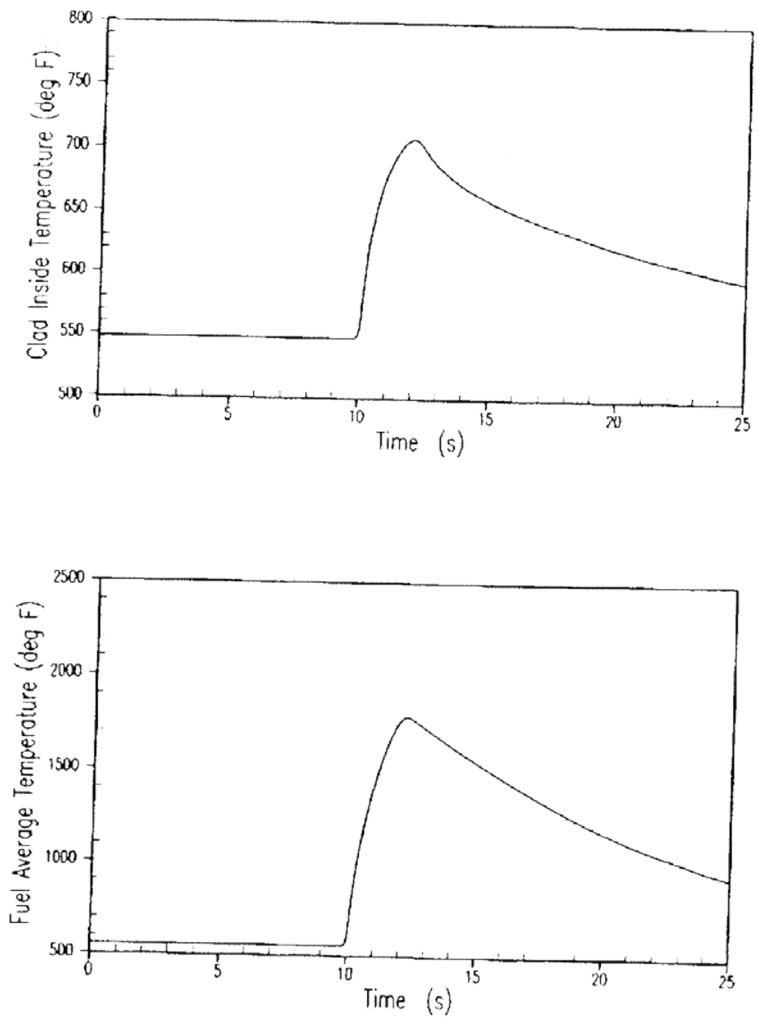
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.4-1
Uncontrolled Rod Cluster Control Assembly Bank Withdrawal From Subcritical Conditions, Heat Flux and Nuclear Power Versus Time (422V+ Fuel)

Figure 15.4-2 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal From Subcritical Conditions, Clad Inside and Fuel Average Temperature Versus Time (422V+Fuel)



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UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.4-2
Uncontrolled Rod Cluster Control Assembly Bank Withdrawal From Subcritical Conditions, Clad Inside and Fuel Average Temperature Versus Time (422V+ Fuel)

Figure 15.4-3 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal at Power, Minimum Feedback, 100 pcm/sec, Nuclear Power and Heat Flux Versus Time

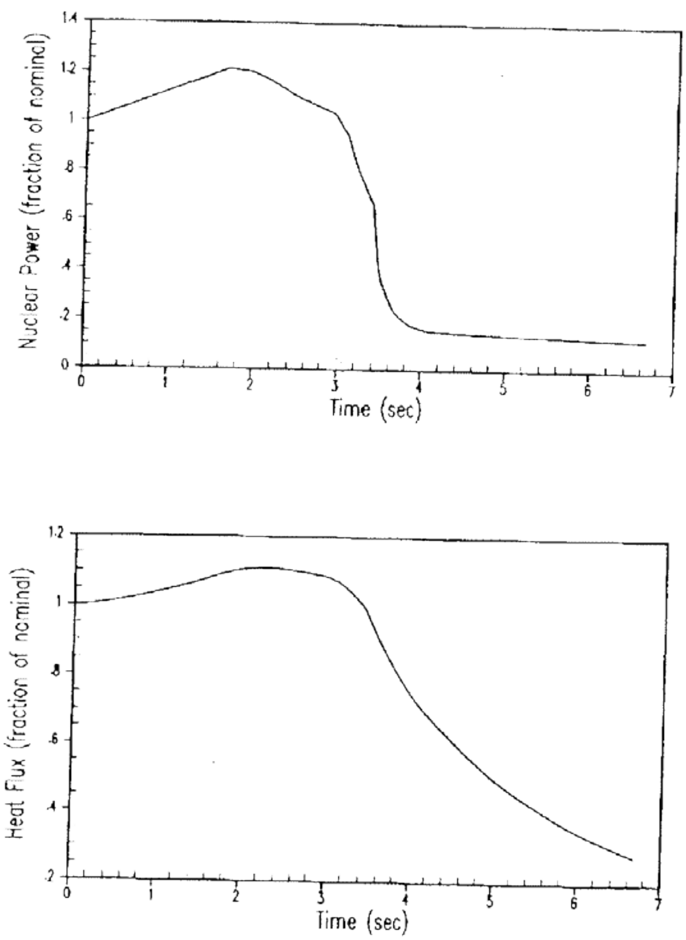


Figure 15.4-4 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal at Power, Minimum Feedback, 100 pcm/sec, Pressurizer Pressure and Pressurizer Water Volume Versus Time

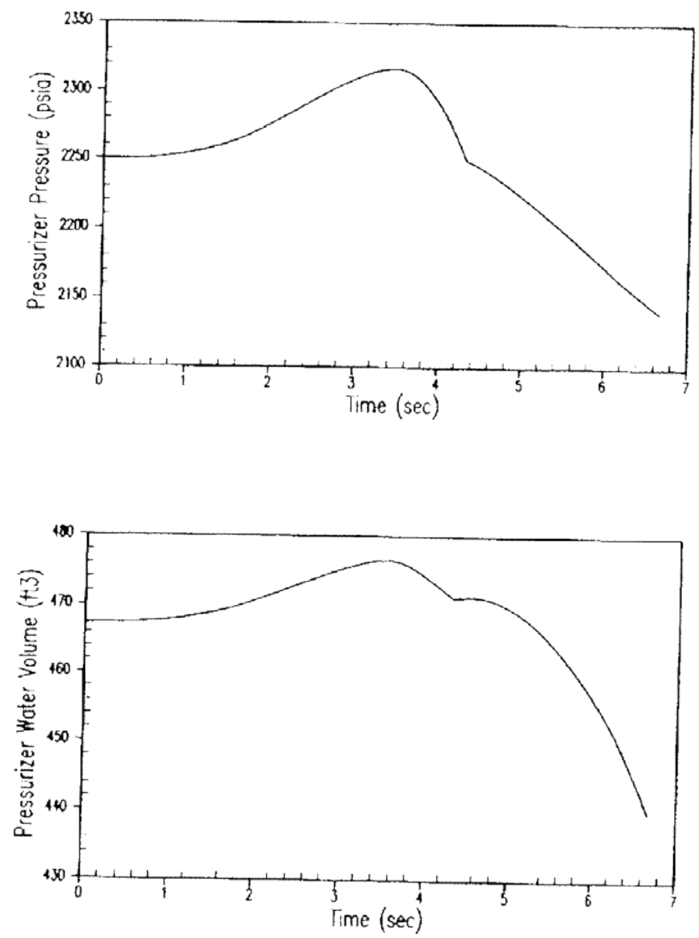
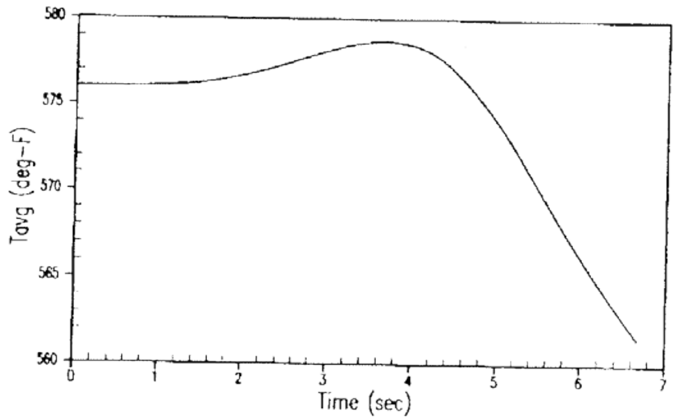


Figure 15.4-5 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal at Power, Minimum Feedback, 100 pcm/sec, T_{avg} and DNBR Versus Time



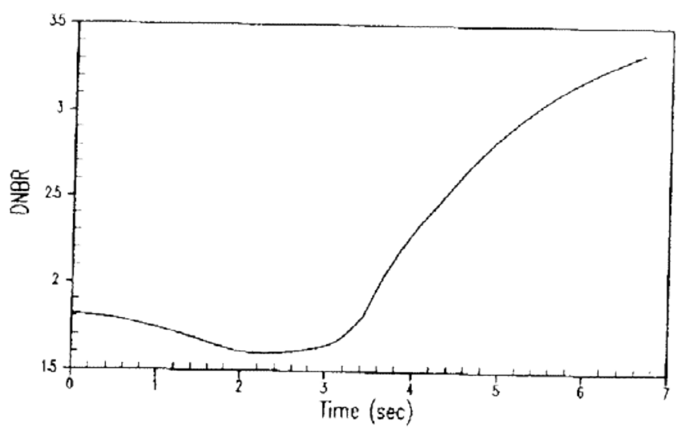


Figure 15.4-6 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal at Power, Maximum Feedback, 5 pcm/sec, Nuclear Power and Heat Flux Versus Time

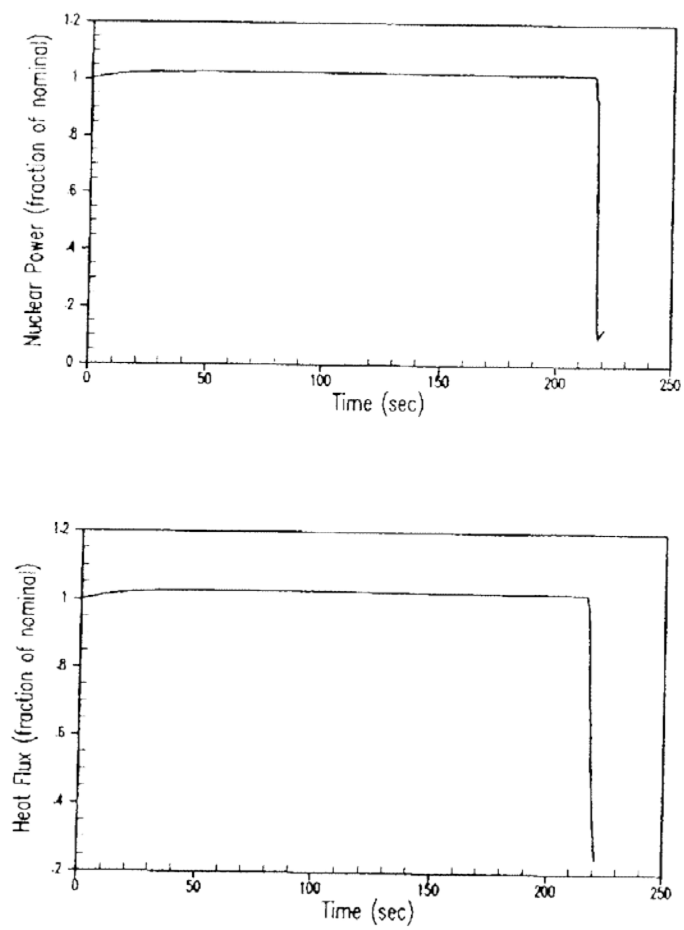
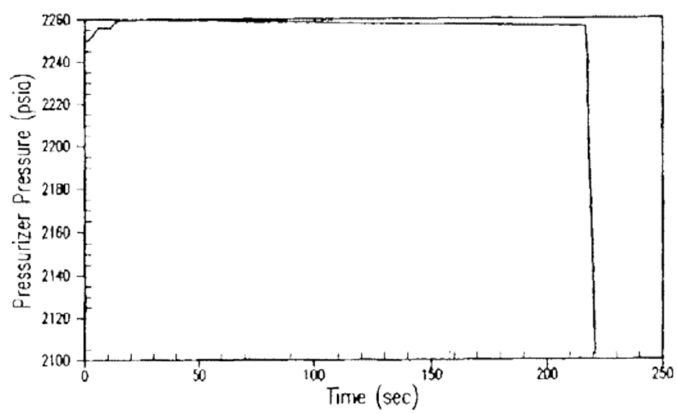


Figure 15.4-7 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal at Power, Maximum Feedback, 5 pcm/sec, Pressurizer Water Volume and Pressurizer Pressure Versus Time



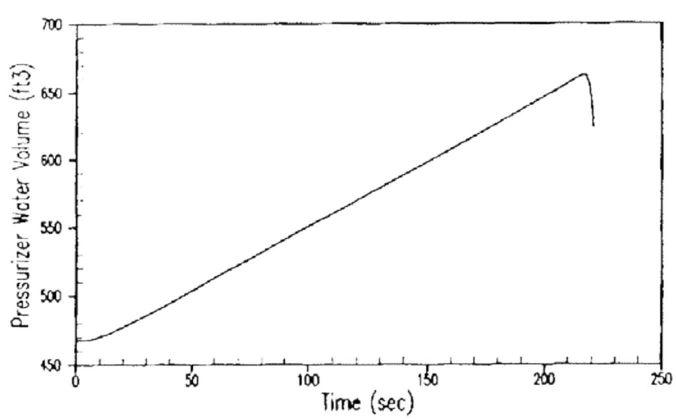


Figure 15.4-8 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal at Power, Maximum Feedback, 5 pcm/sec, T_{AVG} and DNBR Versus Time

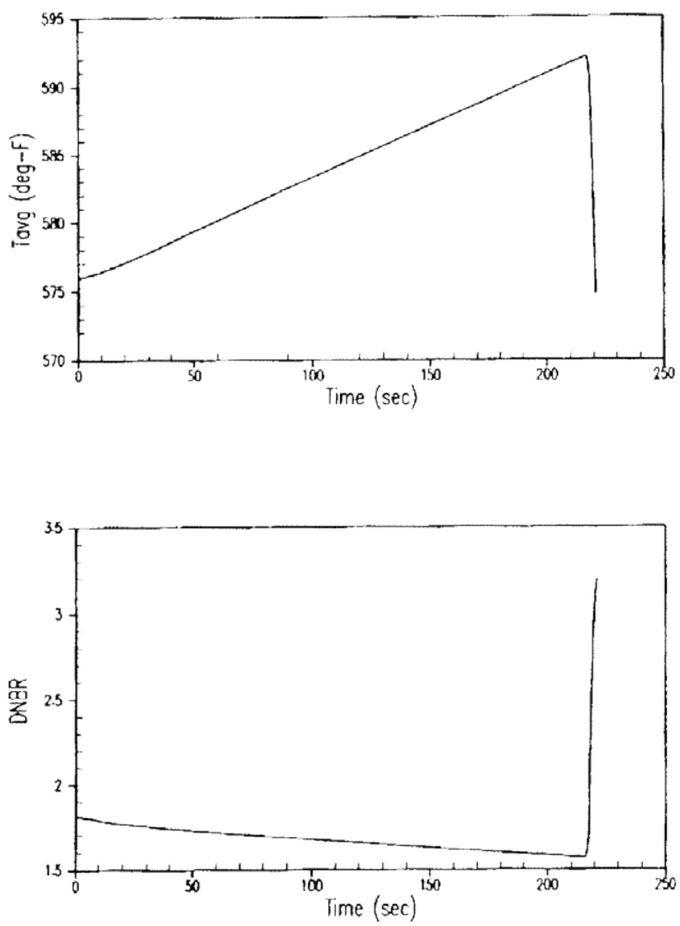


Figure 15.4-9 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal at Power, Maximum and Maximum Feedback, DNBR Versus Reactivity Insertion Rate

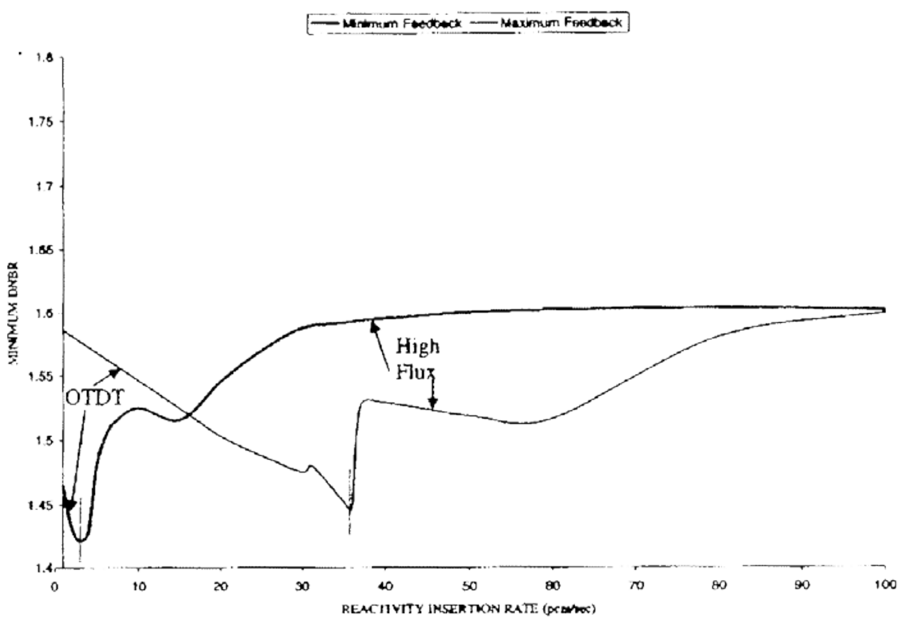


Figure 15.4-10 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal from 60% Power, Minimum and Maximum Feedback, DNBR Versus Reactivity Insertion Rate

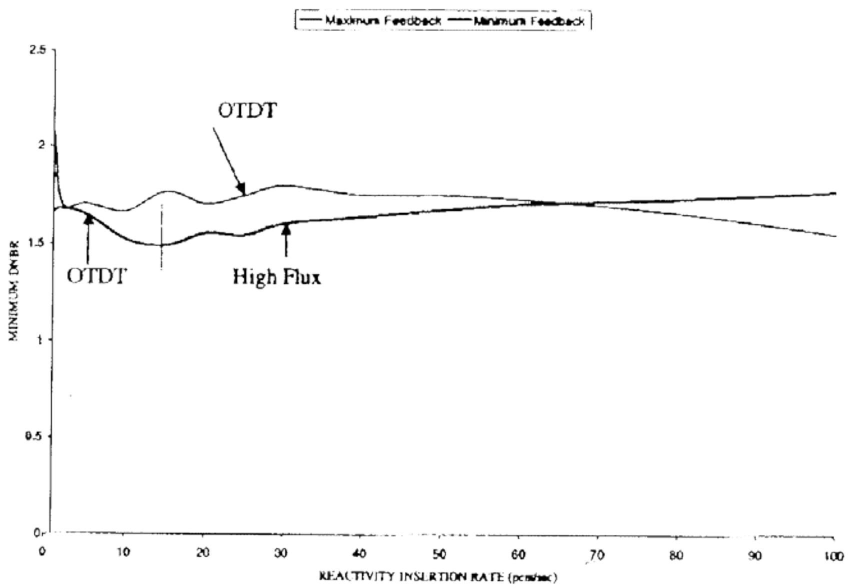


Figure 15.4-11 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal from 10% Power, Minimum and Maximum Feedback, DNBR Versus Reactivity Insertion Rate

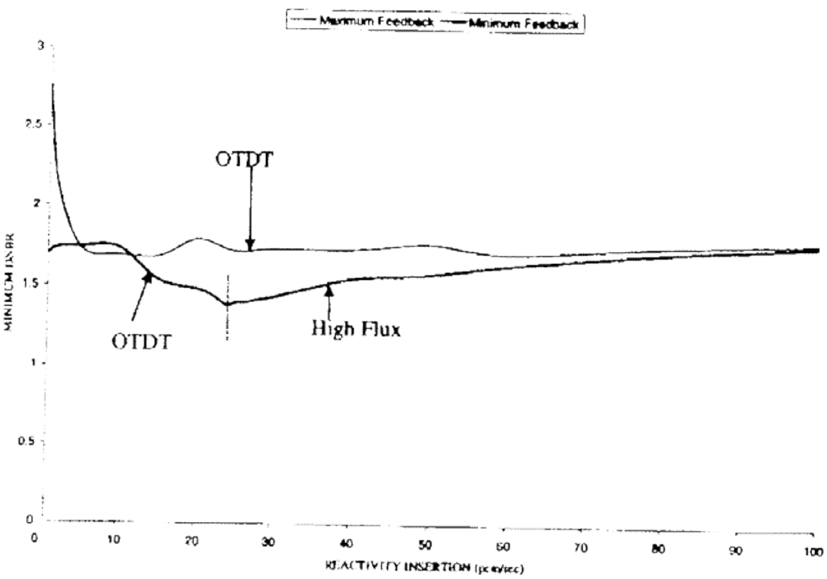
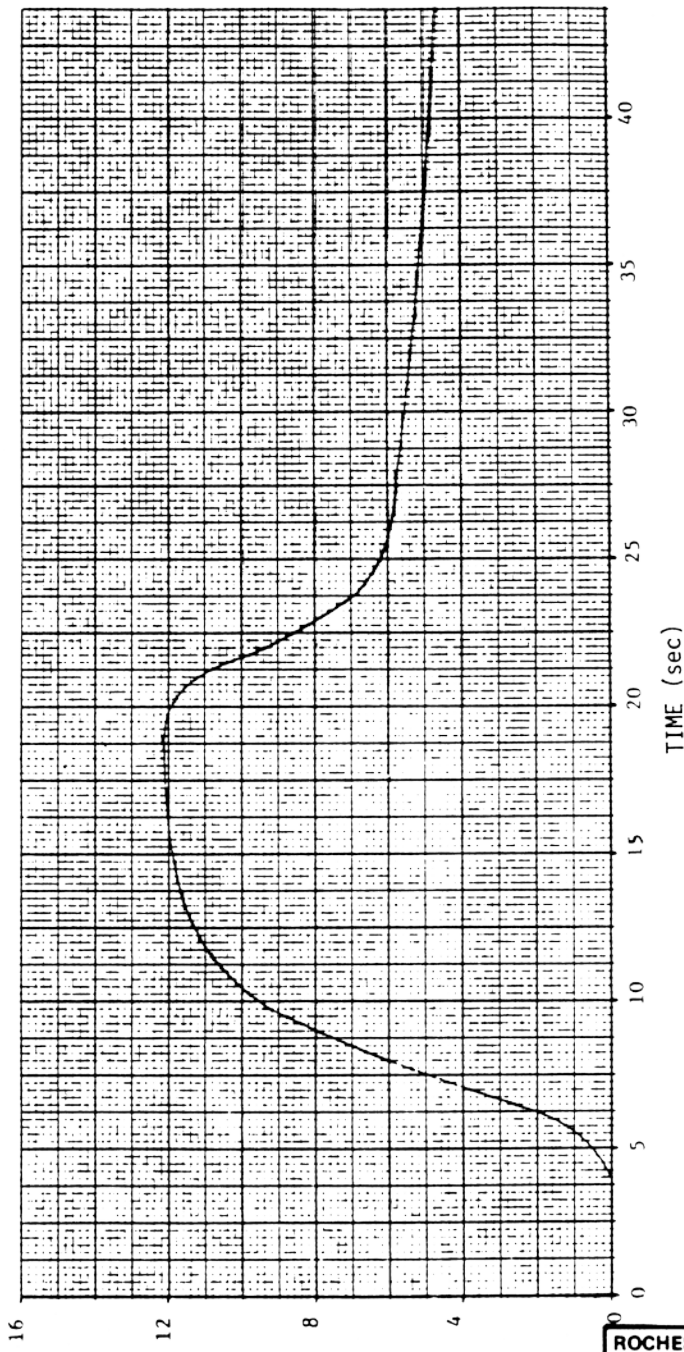


Figure 15.4-12 Startup of an Inactive Coolant Loop, Nuclear Power Versus Time

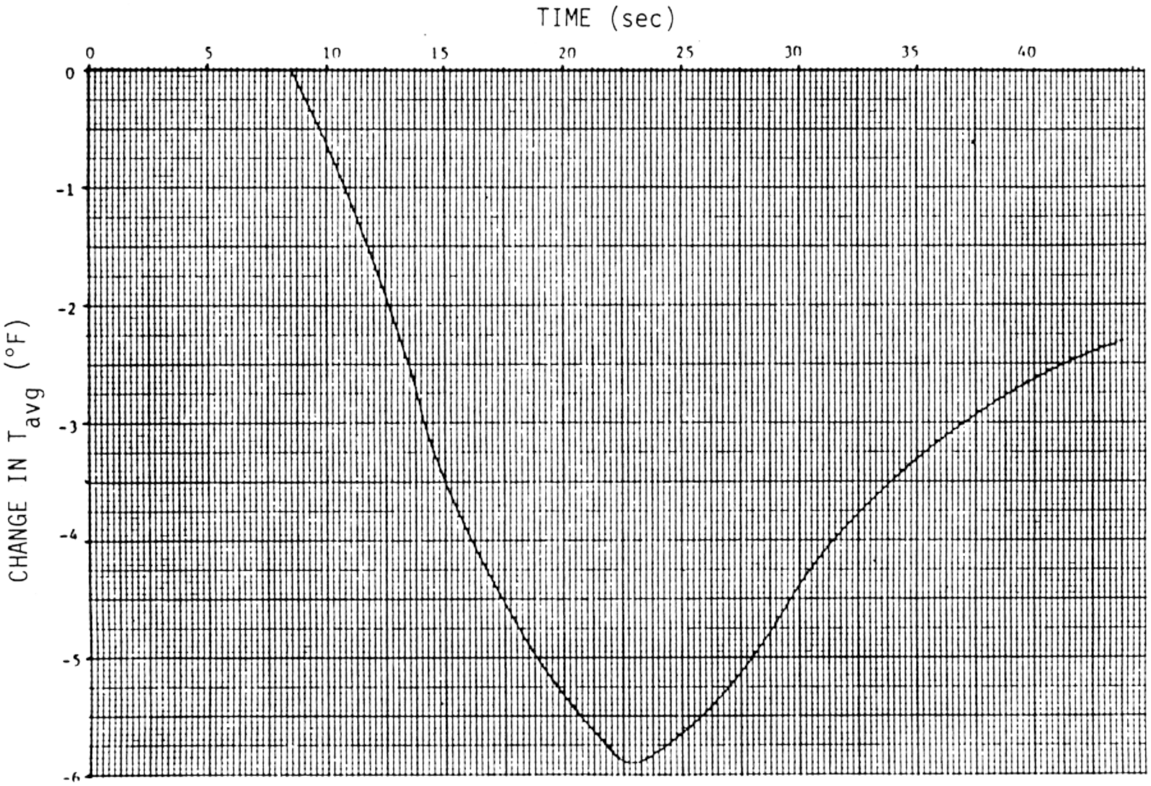


ROCHESTER GAS AND ELECTRIC CORPORATION
R.E. GINNA NUCLEAR POWER PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.4-12

Startup of an Inactive Coolant Loop, Nuclear Power Versus Time

Figure 15.4-13 Startup of an Inactive Coolant Loop, T_{AVG} Versus Time

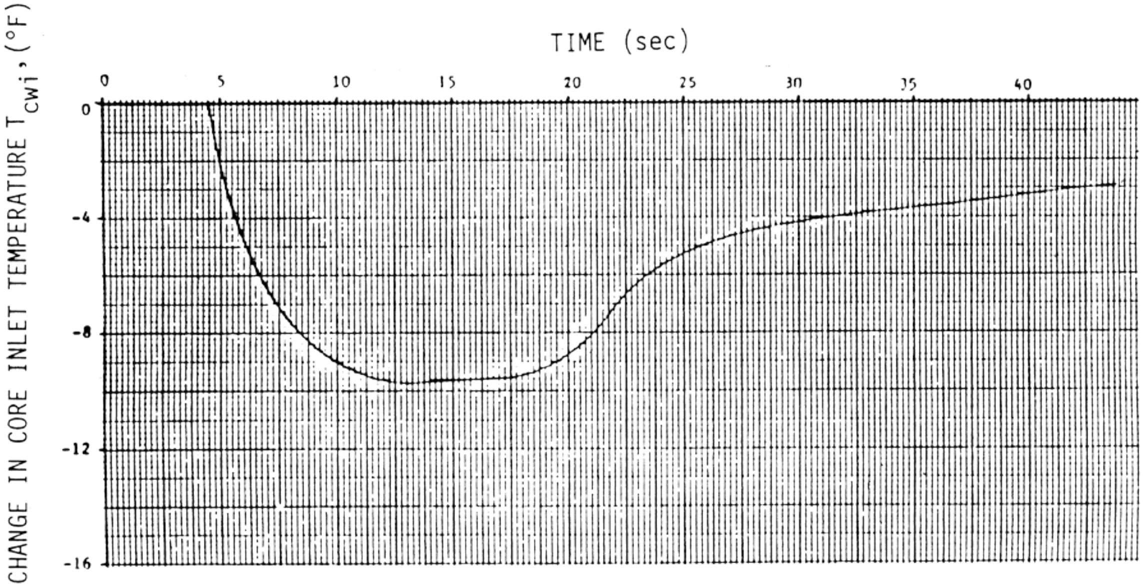


ROCHESTER GAS AND ELECTRIC CORPORATION
R.E. GINNA NUCLEAR POWER PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.4-13

Startup of an Inactive Coolant Loop, T_{avg} Versus Time

Figure 15.4-14 Startup of an Inactive Coolant Loop, Core Inlet Temperature Versus Time

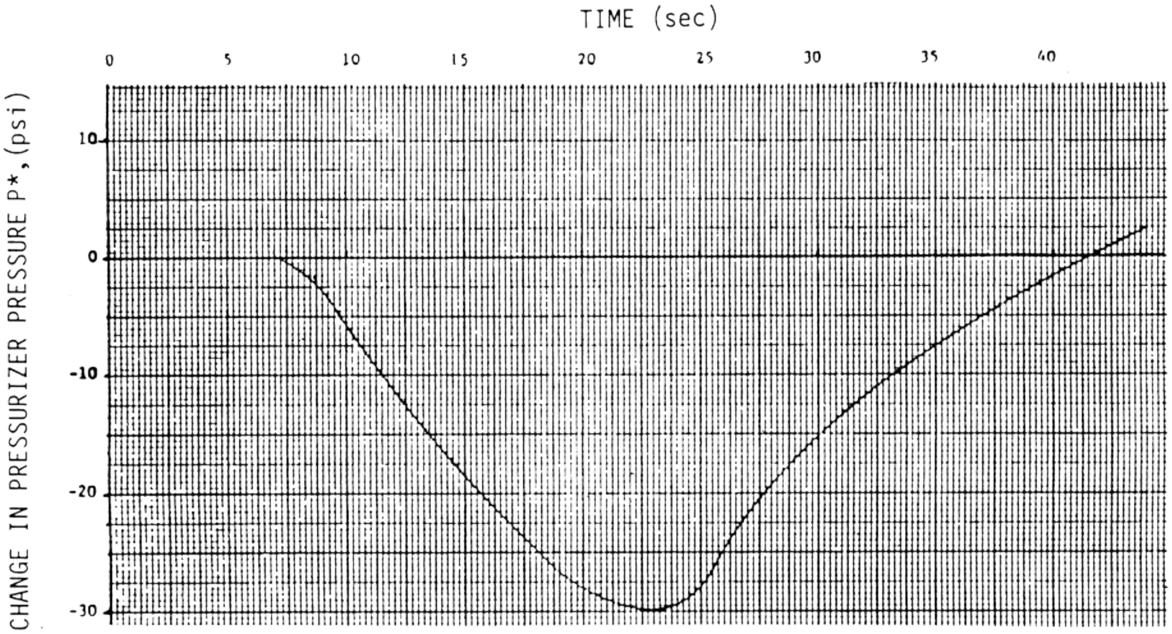


ROCHESTER GAS AND ELECTRIC CORPORATION
R.E. GINNA NUCLEAR POWER PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.4-14

Startup of an Inactive Coolant Loop, Core Inlet Temperature Versus Time

Figure 15.4-15 Startup of an Inactive Coolant Loop, Pressurizer Pressure Versus Time



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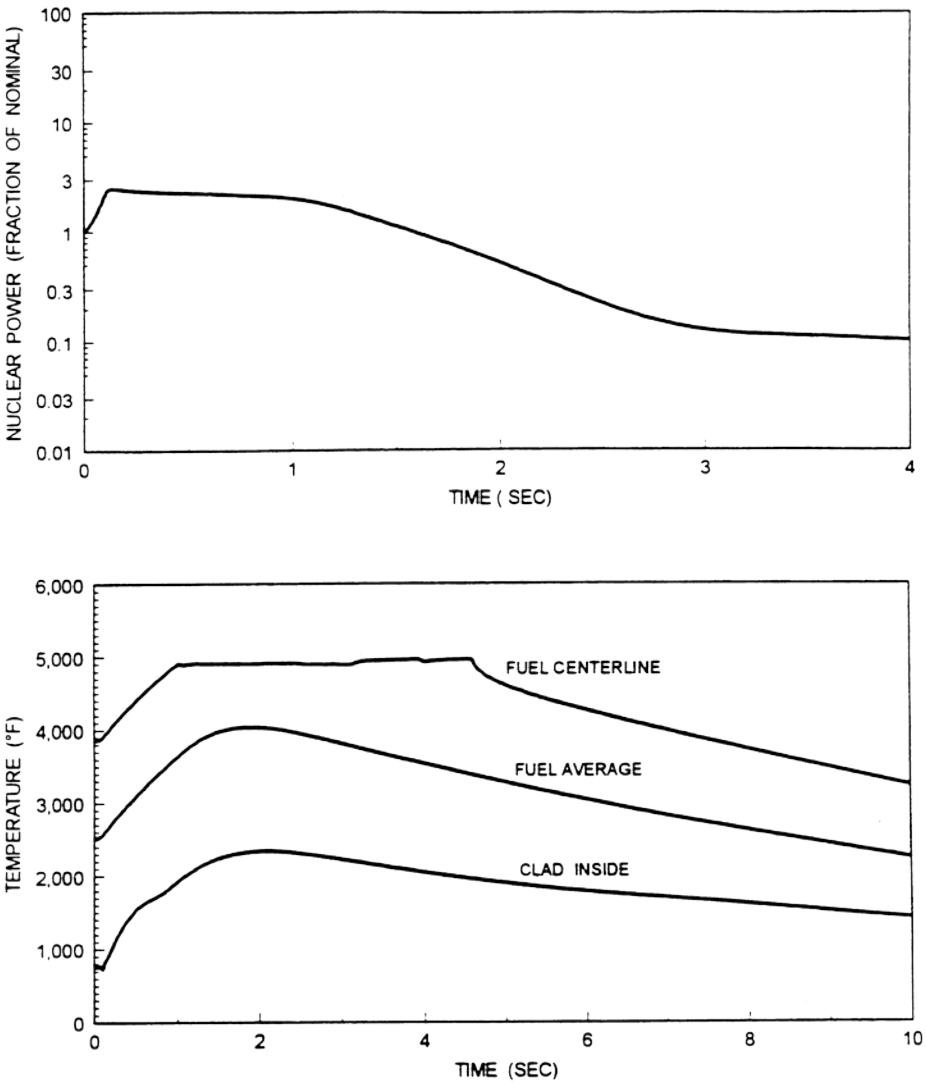
R.E. GINNA NUCLEAR POWER PLANT
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.4-15

Startup of an Inactive Coolant Loop, Pressurizer Pressure Versus Time

Figure 15.4-16 Rod Cluster Control Assembly Ejection Beginning-of-Life, Full Power, Fuel and Clad Temperature and Nuclear Power Versus Time



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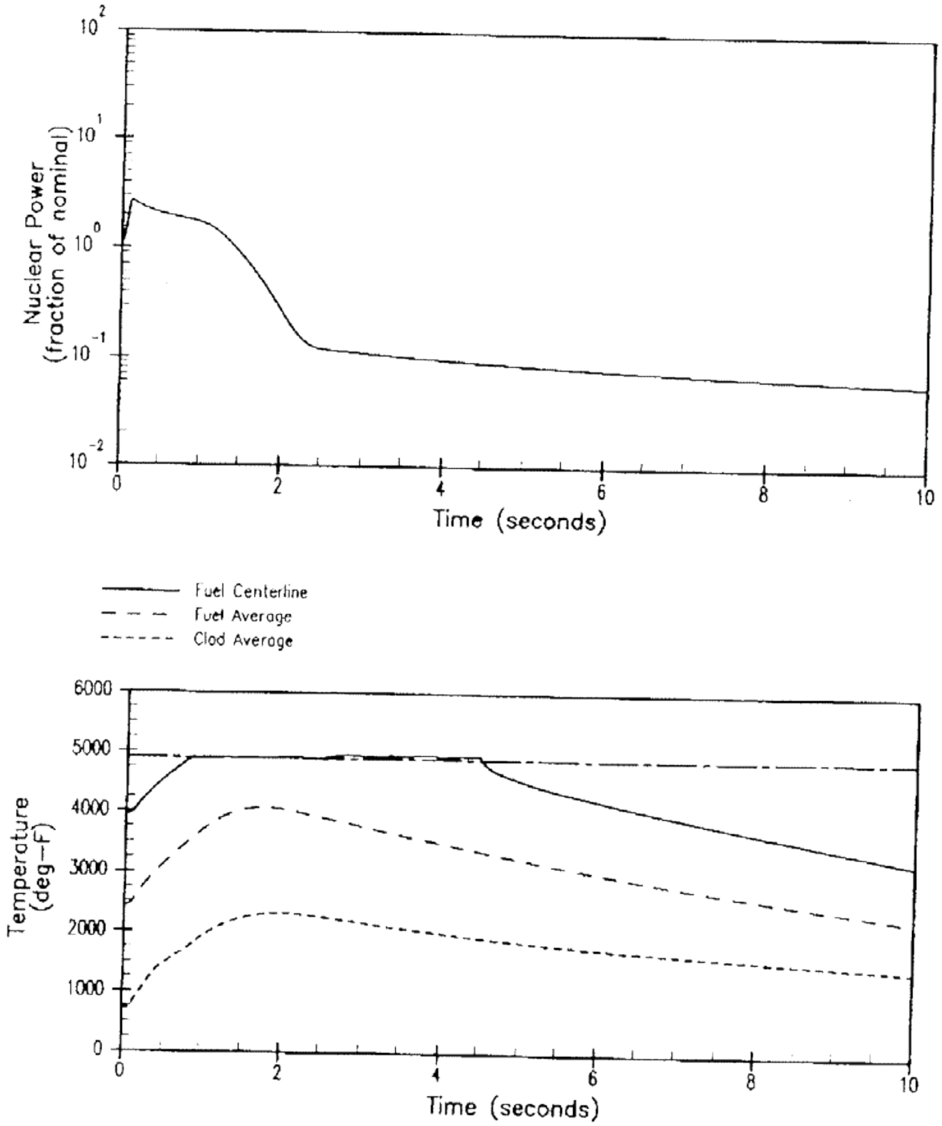
UPDATED FINAL SAFETY ANALYSIS REPORT

Figure 15.4-16
Rod Cluster Control Assembly Ejection Beginning-of-Life, Full Power, Fuel and Clad Temperature and Nuclear Power Versus Time

REV 10 12/93

Figure 15.4-16aRod Cluster Control Assembly Ejection, Beginning of Life, Full Power, Fuel and Clad Temperature and Nuclear Power Versus Time



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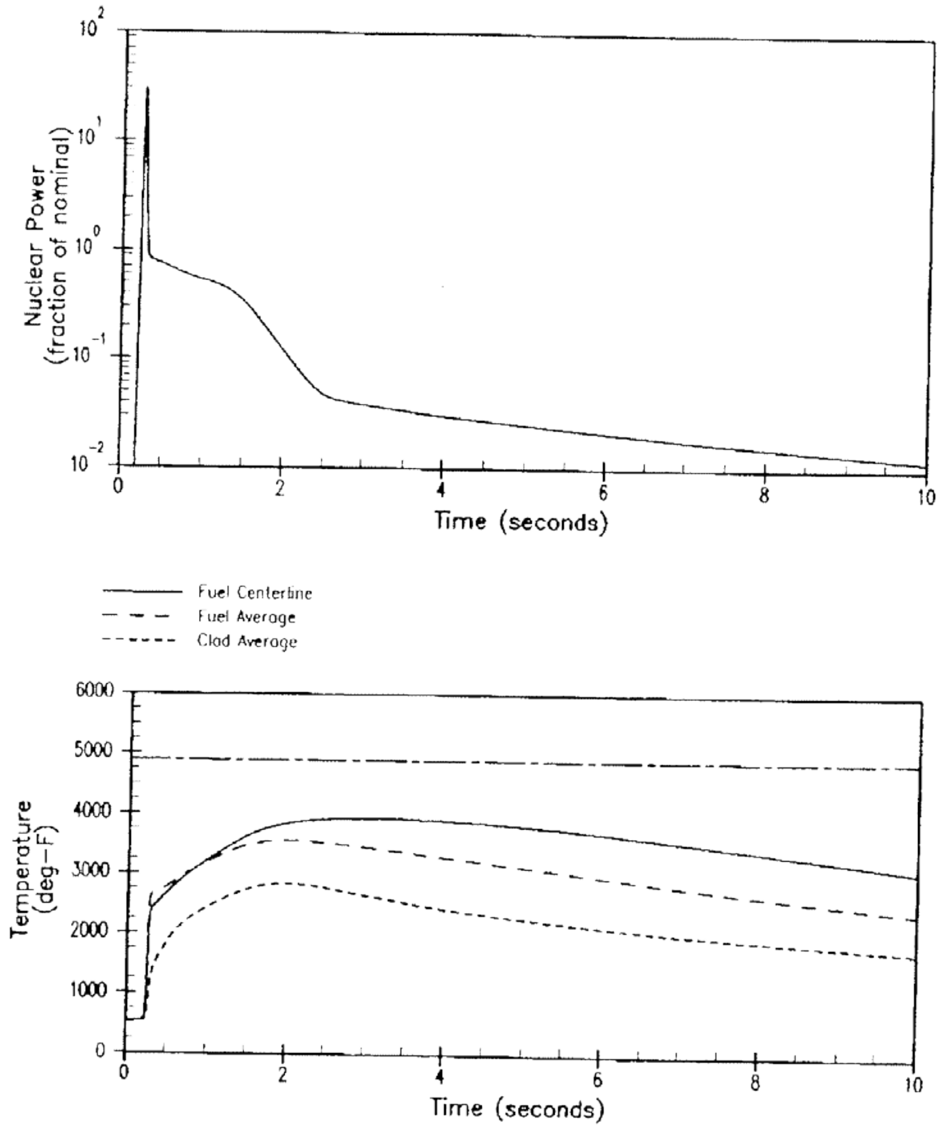
Updated Final Safety Analysis Report

Figure 15.4-16a
Rod Cluster Control Assembly Ejection, Beginning of Life, Full Power, Fuel and Clad Temperature and Nuclear Power versus Time

Figure 15.4-16b Rod Cluster Control Assembly Ejection, Beginning of Life, Zero Power, Fuel and Clad Temperature and Nuclear Power Versus Time



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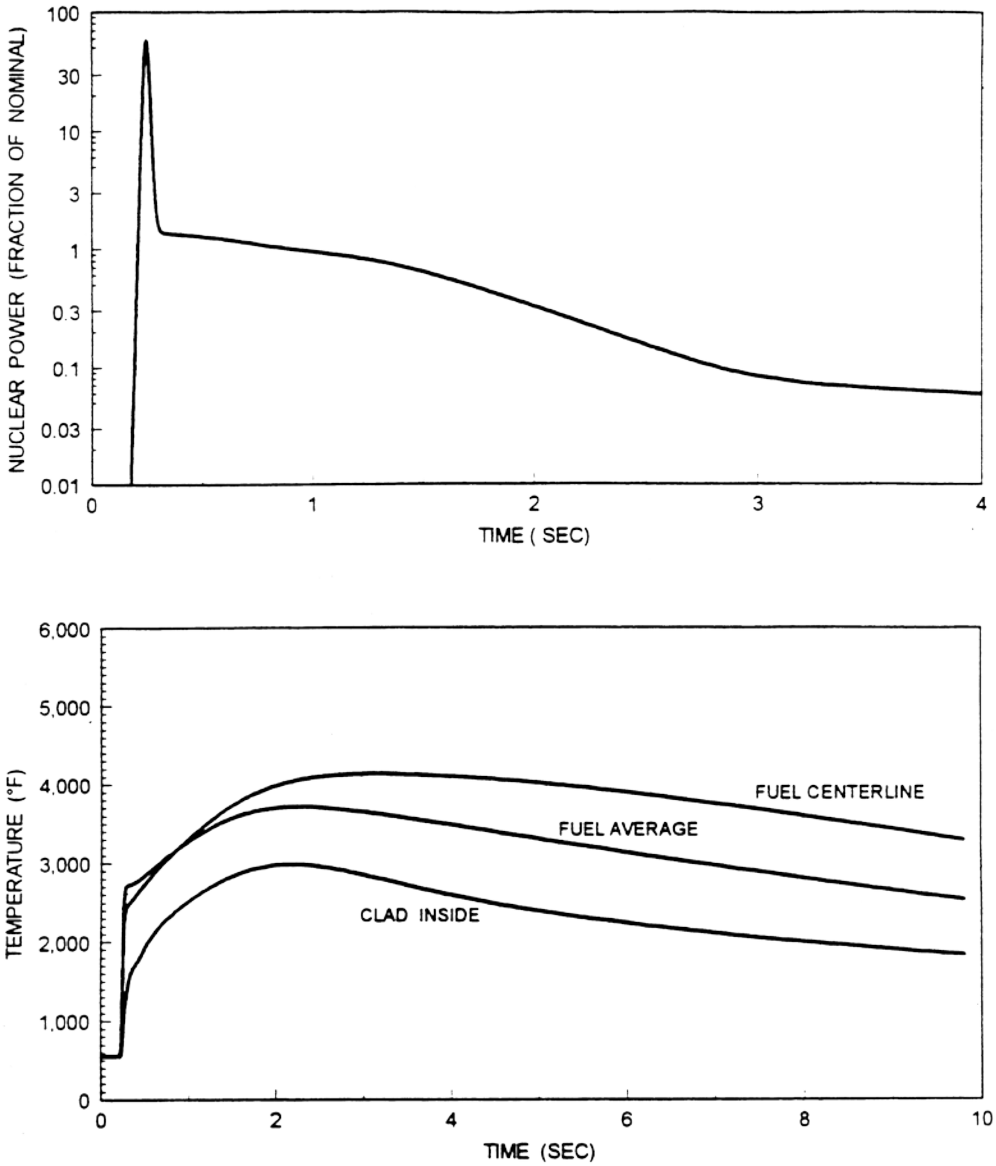
Updated Final Safety Analysis Report

Figure 15.4−16b
Rod Cluster Control Assembly Ejection, Beginning of Life, Zero Power, Fuel and Clad Temperature and Nuclear Power versus Time

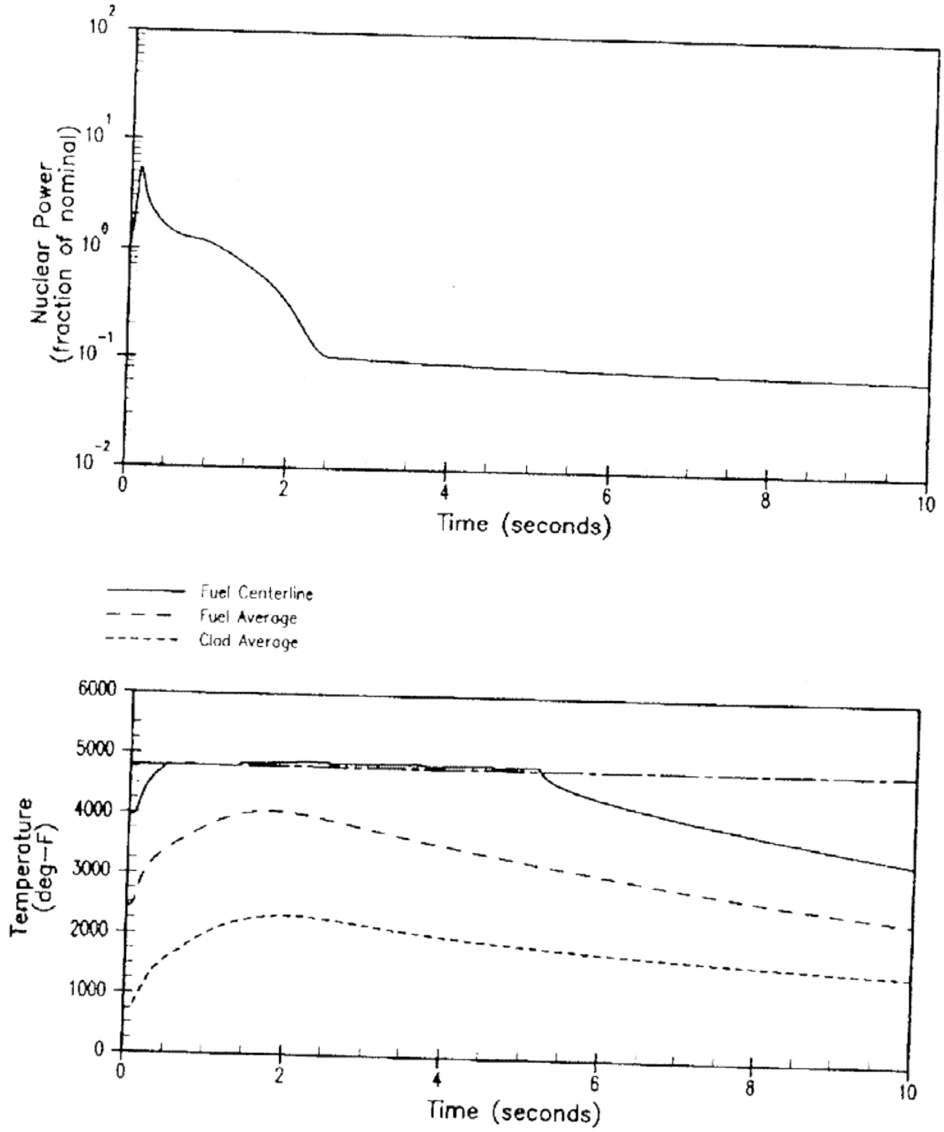
Figure 15.4-17 Rod Cluster Control Assembly Ejection Beginning-of-Life, Zero Power, Fuel and Clad Temperature and Nuclear Power Versus Time



ROCHESTER GAS AND ELECTRIC CORPORATION R. E. GINNA NUCLEAR POWER PLANT UPDATED FINAL SAFETY ANALYSIS REPORT
Figure 15.4-17 Rod Cluster Control Assembly Ejection Beginning-of-Life, Zero Power, Fuel and Clad Temperature and Nuclear Power Versus Time

REV 10 12/93

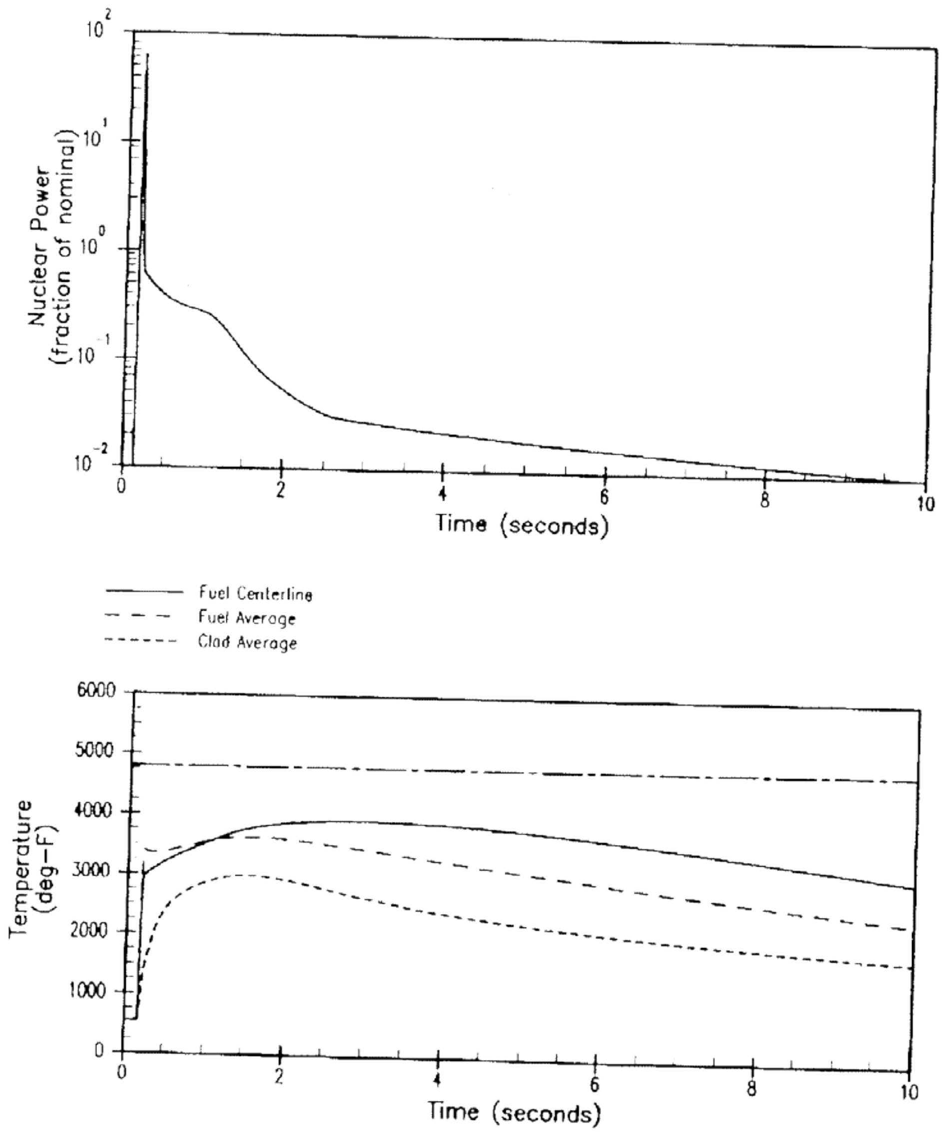
Figure 15.4-17a Rod Cluster Control Assembly Ejection, End of Life, Full Power, Fuel and Clad Temperature and Nuclear Power Versus Time



Rochester Gas and Electric Corporation
R. E. Ginna Nuclear Power Plant
Updated Final Safety Analysis Report

Figure 15.4-17a
Rod Cluster Control Assembly Ejection, End of Life, Full Power, Fuel and Clad Temperature and Nuclear Power versus Time

Figure 15.4-17b Rod Cluster Control Assembly Ejection, End of Life, Zero Power, Fuel and Clad Temperature and Nuclear Power Versus Time



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Updated Final Safety Analysis Report

Figure 15.4–17b
Rod Cluster Control Assembly Ejection, End of Life, Zero Power, Fuel and Clad Temperature and Nuclear Power versus Time

Figure 15.4-18 Rod Cluster Control Assembly Drop Heat Flux and Nuclear Power Versus Time

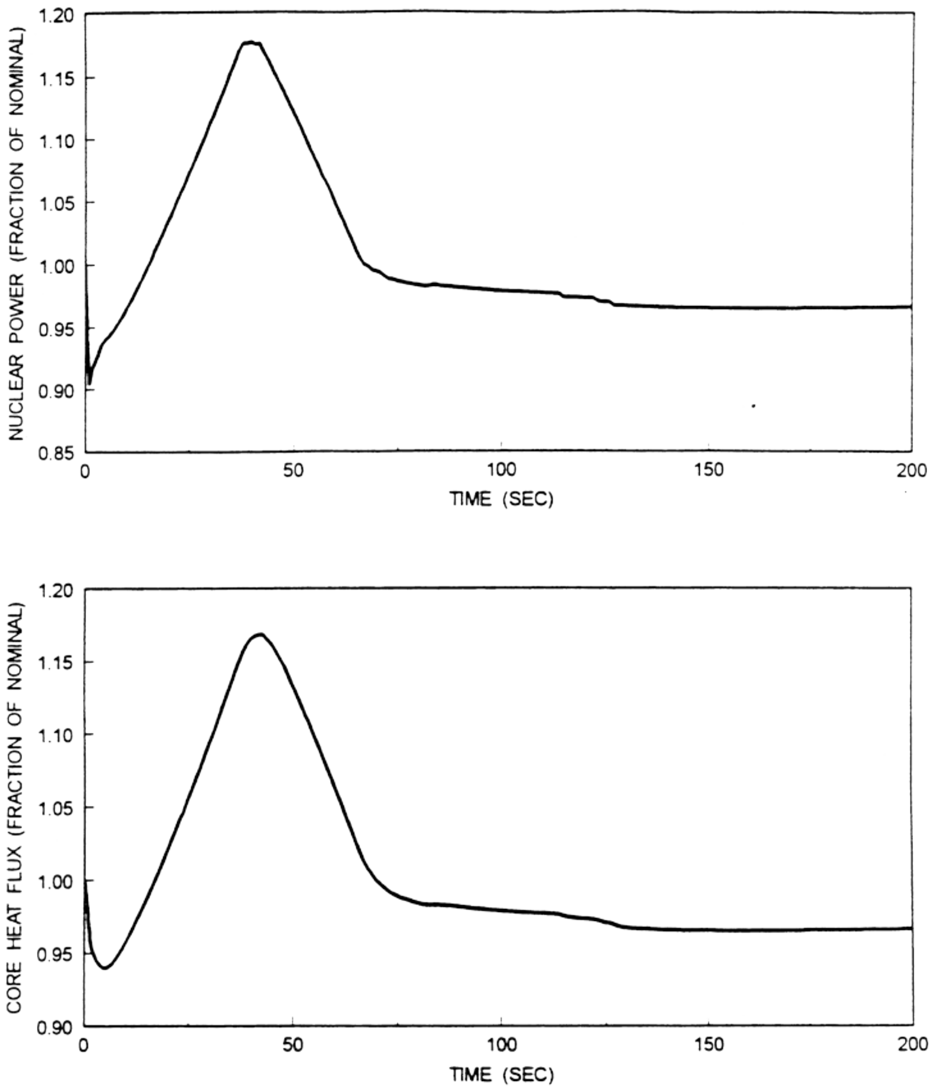

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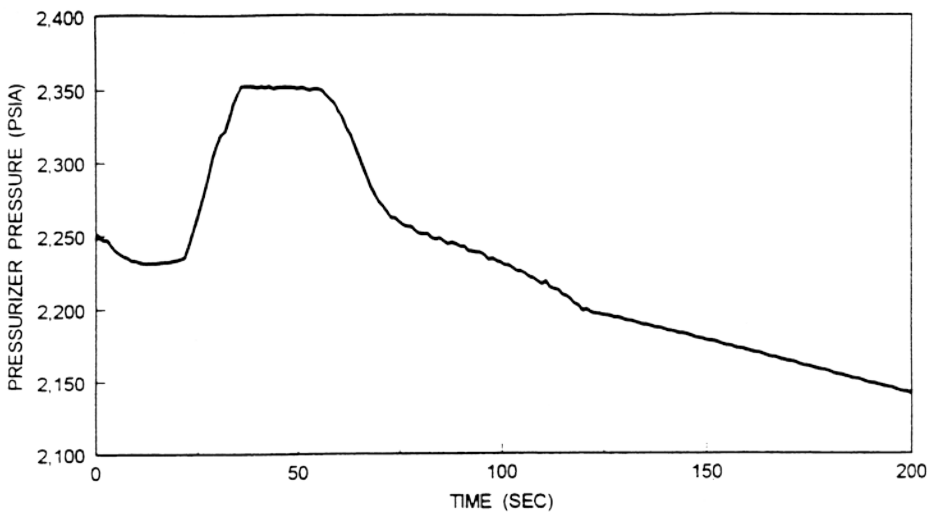
Figure 15.4-18

Rod Cluster Control Assembly Drop Heat Flux and Nuclear Power Versus Time

REV 10 12/93

Figure 15.4-19 Rod Cluster Control Assembly Drop Pressurizer Pressure and Core Average Temperature Versus Time



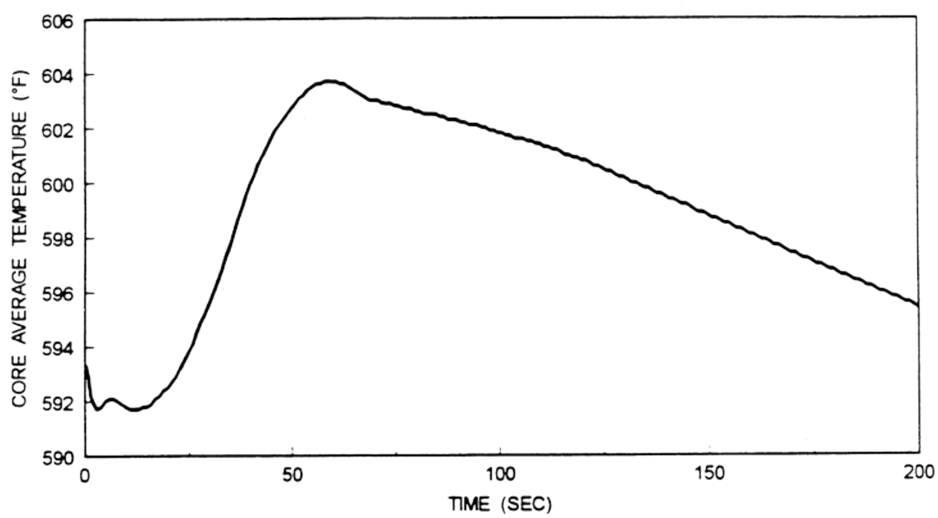


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Figure 15.4-19

Rod Cluster Control Assembly Drop Pressurizer Pressure and Core Average Temperature Versus Time

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Figure 15.4-20 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal from 8% Power (RCS Pressure Case), Minimum Feedback, 31 pcm/sec, Nuclear Power and Heat Flux versus Time

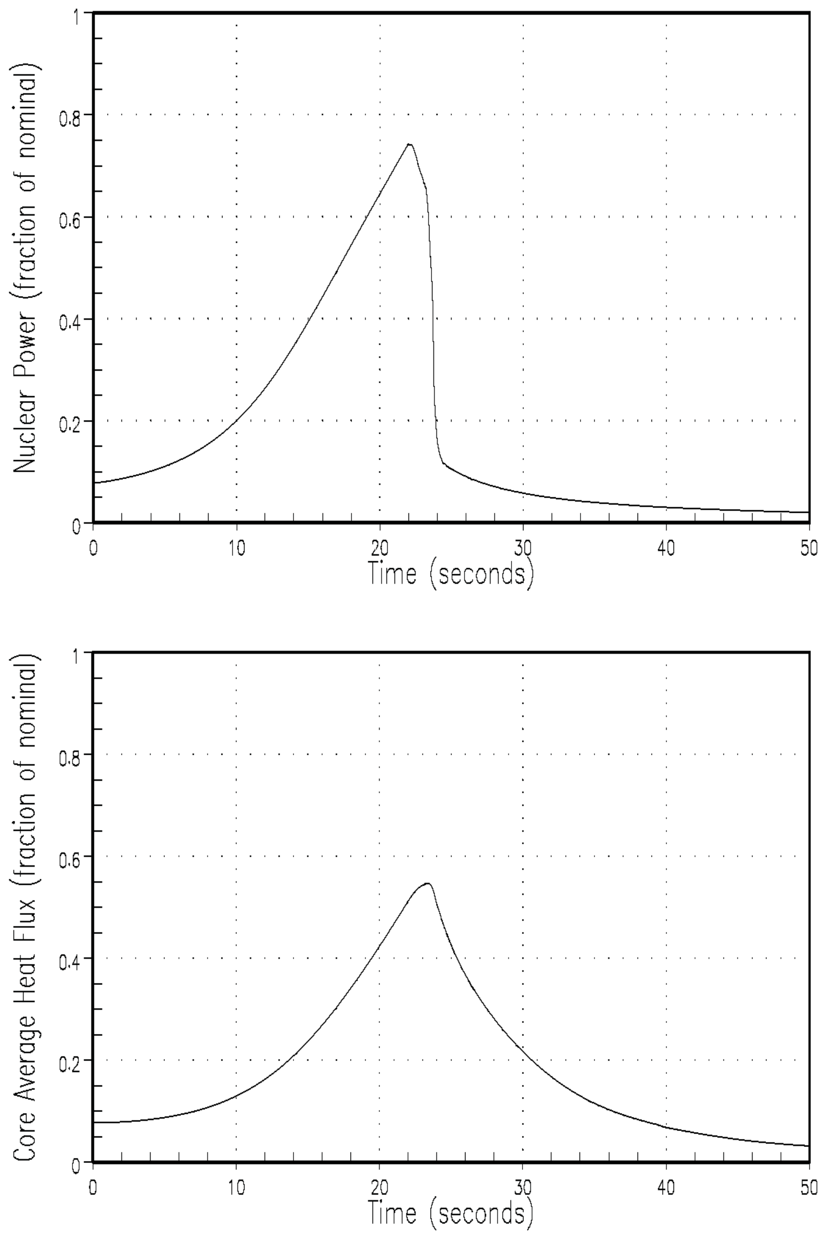


Figure 15.4-21 Uncontrolled Rod Cluster Control Assembly Bank Withdrawal from 8% Power (RCS Pressure Case), Minimum Feedback, 31 pcm/sec, RCS Pressures and T_{avg} versus Time

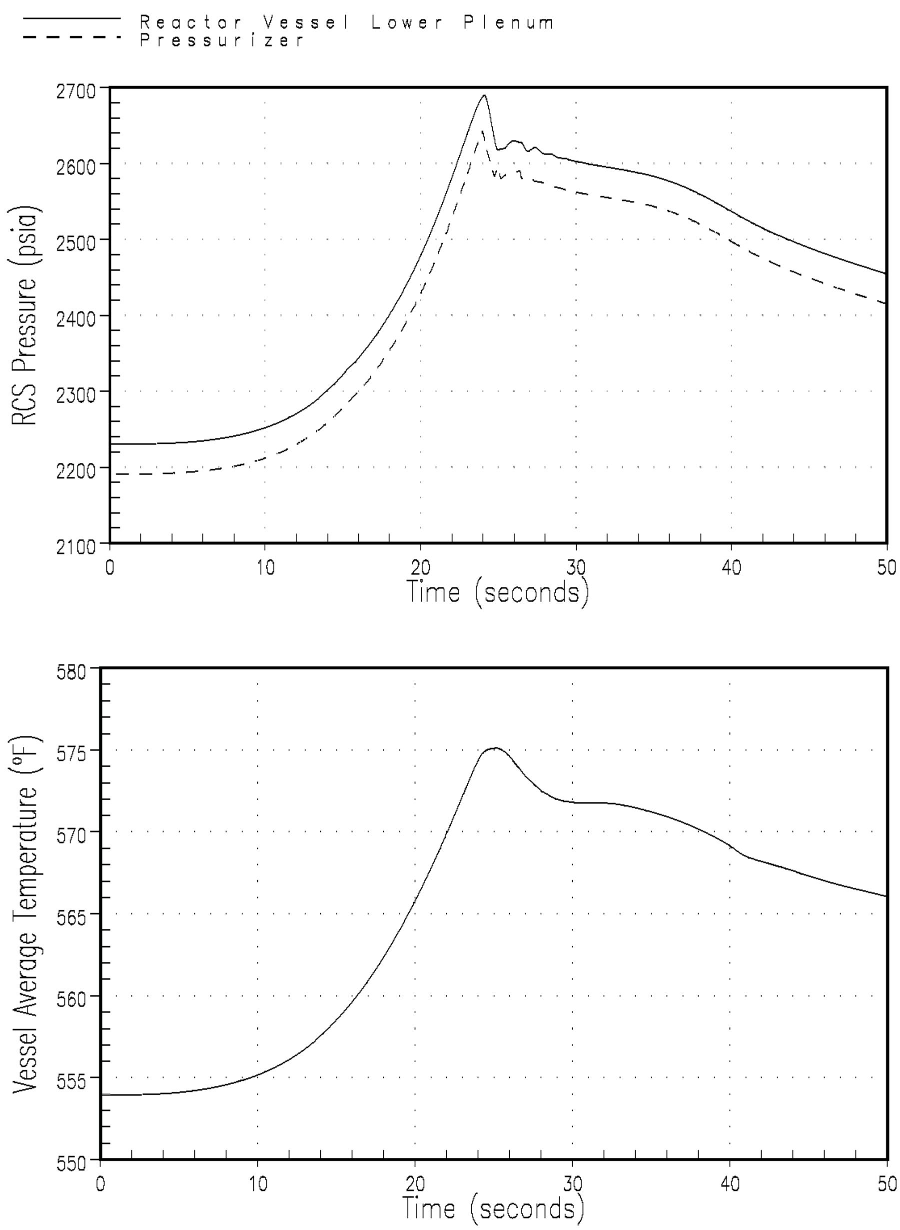
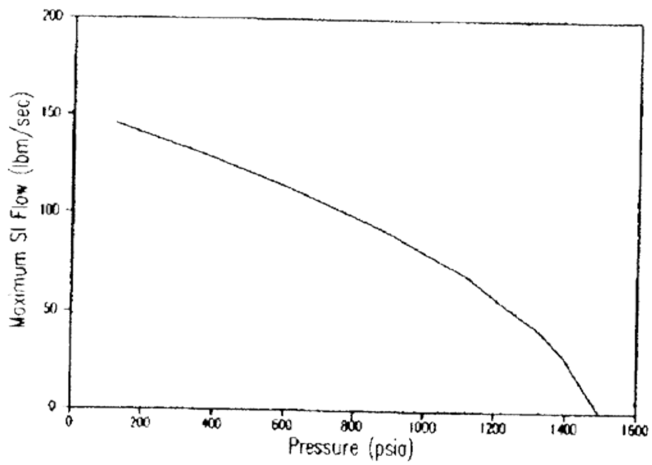


Figure 15.6-1 Steam Generator Tube Rupture (Overfill), Maximum Safety Injection Flow Versus Pressure



R.E. GINNA NUCLEAR POWER PLANT UPDATED FINAL SAFETY ANALYSIS REPORT
Figure 15.6-1 Steam Generator Tube Rupture (Overfill), Maximum Safety Injection Flow Versus Pressure

Figure 15.6-1a RCS Depressurization, Nuclear Power Versus Time

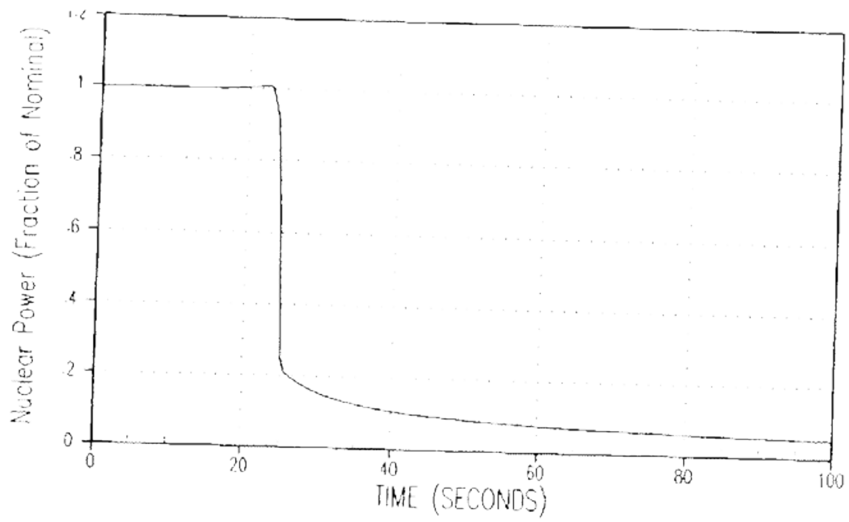


Figure 15.6-1a
RCS Depressurization
Nuclear Power versus Time

Figure 15.6-1b RCS Pressurization, Pressurizer Pressure Versus Time

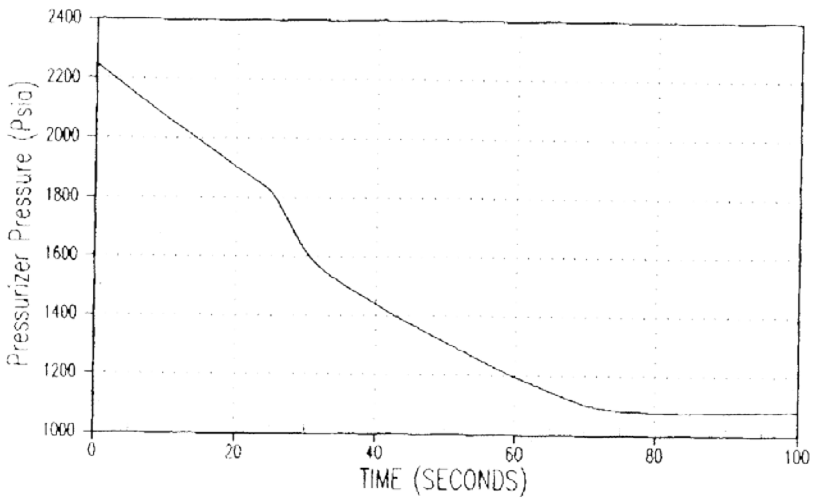


Figure 15.6-1b
RCS Depressurization
Pressurizer Pressure versus Time

Figure 15.6-1c RCS Depressurization, Indicated Loop Average Temperature Versus Time

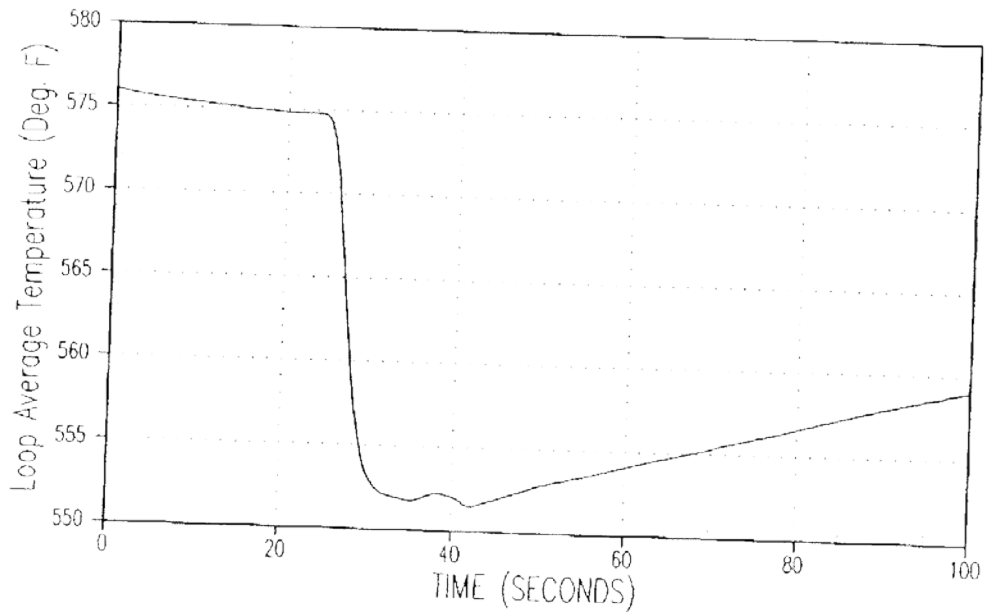


Figure 15.6-1c
RCS Depressurization
Indicated Loop Average Temperature versus Time

Figure 15.6-1d RCS Depressurization, DNBR Versus Time

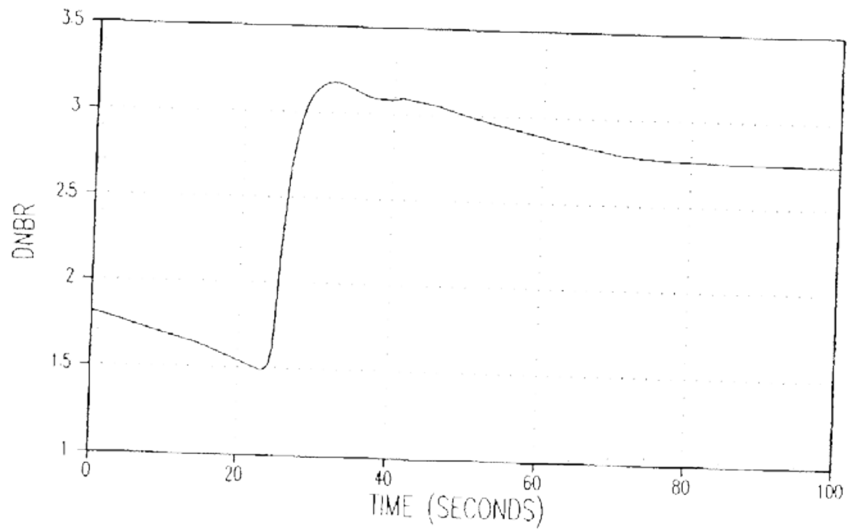


Figure 15.6-1d
RCS Depressurization
DNBR versus Time

Figure 15.6-2 SGTR (Overfill), Pressurizer Level and Pressurizer Pressure Versus Time

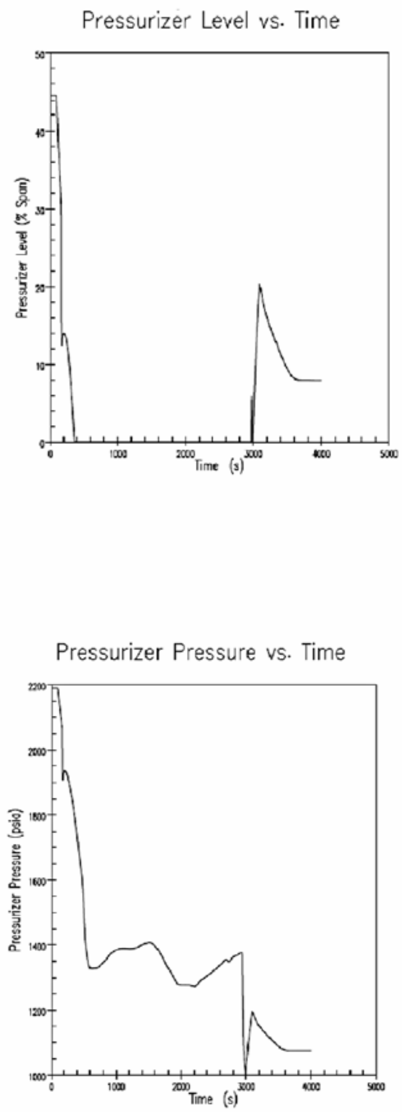


Figure 15.6-3 SGTR (Overfill), Secondary Pressure and Steam Generator Liquid Mass Versus Time

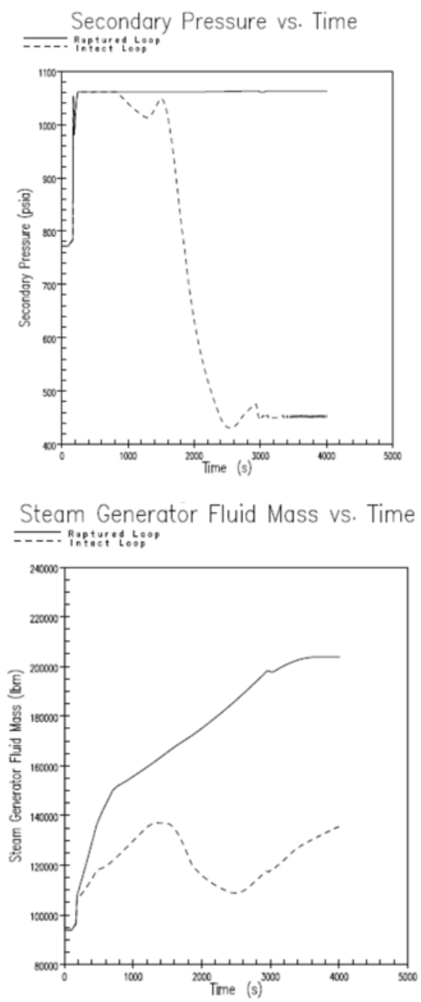


Figure 15.6-4 SGTR (Overfill), Hot and Cold Leg Temperatures for Intact and Ruptured Steam Generators Versus Time

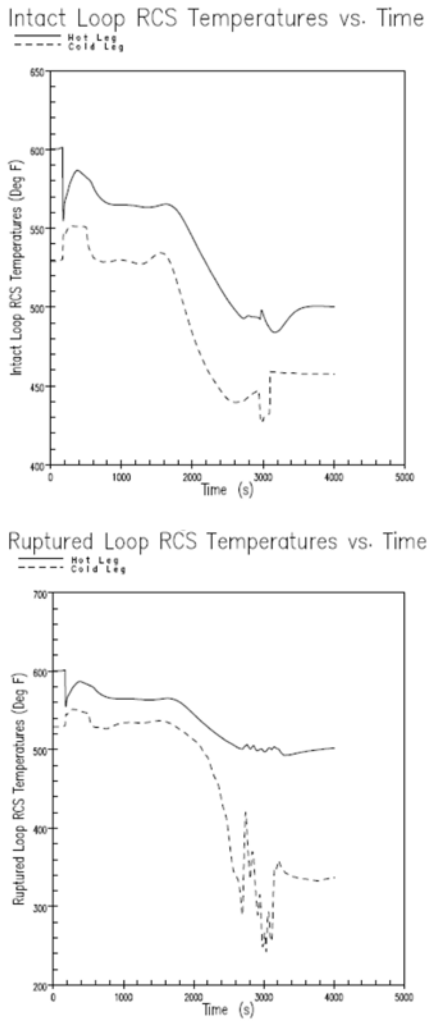


Figure 15.6-5 SGTR (Overfill), Total Primary to Secondary Leakage and Total Integrated Primary to Secondary Leakage Versus Time

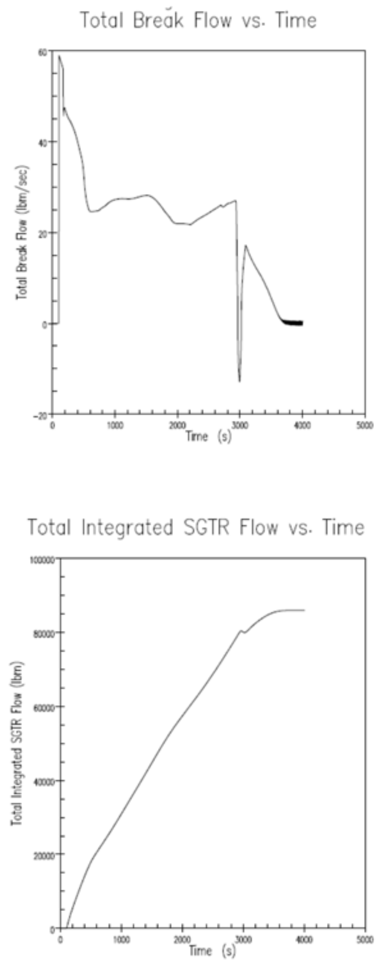


Figure 15.6-6 SGTR (Overfill), Steam Generator Relief Flow and Integrated Steam Generator Relief Flow Versus Time

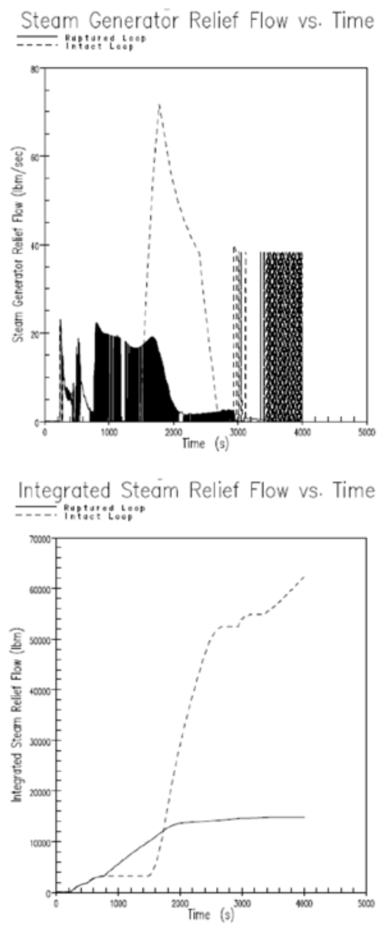


Figure 15.6-7 SGTR (Overfill), Steam Generator Water Volume Versus Time

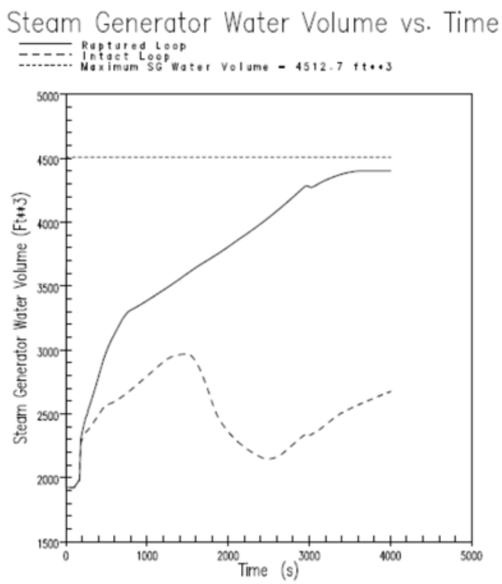


Figure 15.6-8 SGTR (Dose), Pressurizer Level and Pressurizer Pressure Versus Time

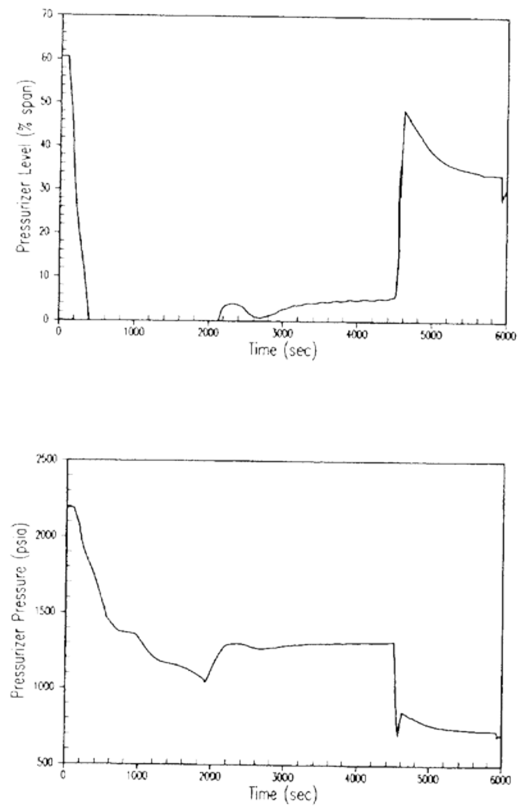


Figure 15.6-8
SGTR (Dose), Pressurizer Level and
Pressurizer Pressure vs. Time

Figure 15.6-9 SGTR (Dose), Secondary Pressure and Steam Generator Liquid Mass Versus Time

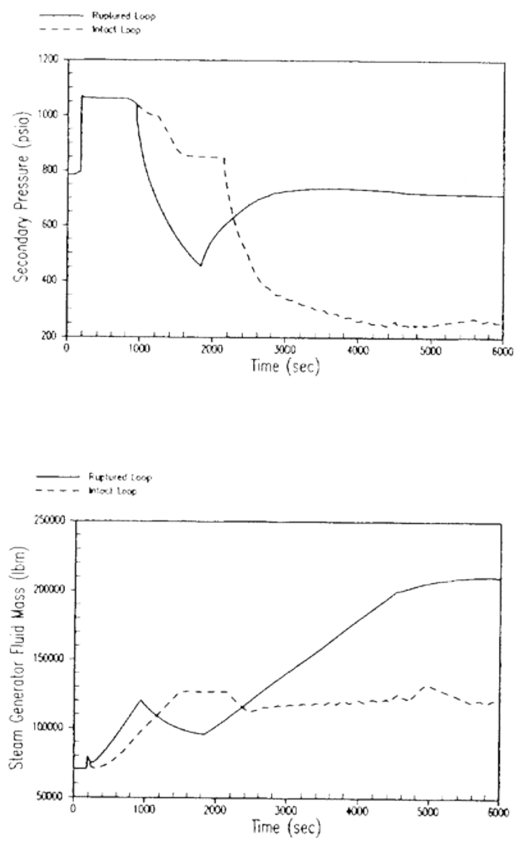


Figure 15.6-9
SGTR (Dose), Secondary Pressure and Steam Generator Liquid Mass vs. Time

Figure 15.6-10 SGTR (Dose), Hot and Cold Leg Temperatures for Intact and Ruptured Steam Generators Versus Time

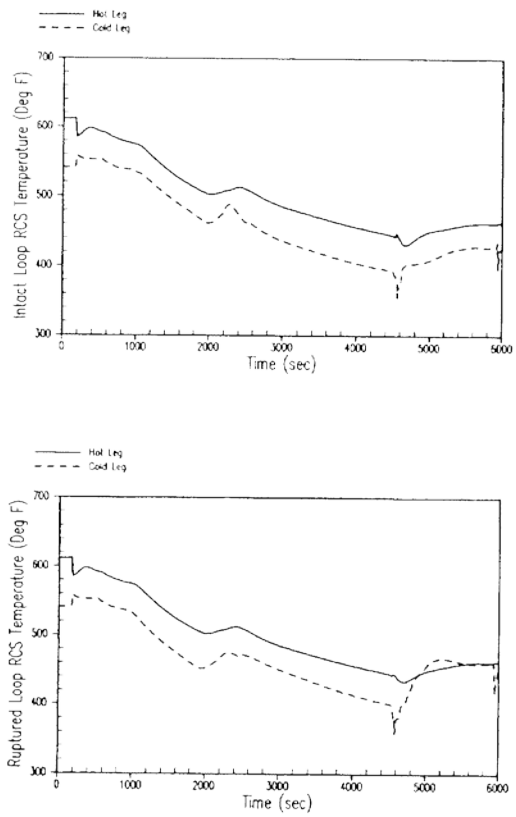
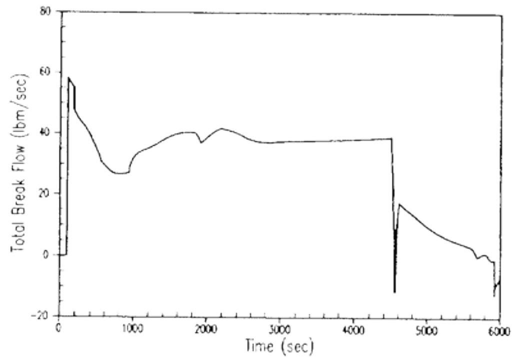


Figure 15.6-10
SGTR (Dose), Hot and Cold Leg Temperatures for Intact and Ruptured Steam Generators vs. Time

Figure 15.6-11 SGTR (Dose), Total Primary to Secondary Leakage and Total Integrated Primary to Secondary Leakage Versus Time



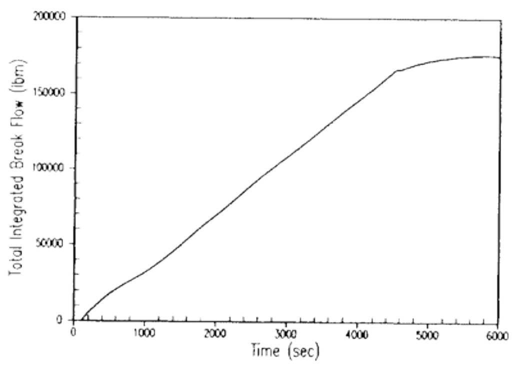


Figure 15.6-11
SGTR (Dose), Total Primary to Secondary Leakage and Total Integrated Primary to Secondary Leakage vs. Time

Figure 15.6-12 SGTR (Dose), Steam Generator Relief Flow and Integrated Steam Generator Relief Flow Versus Time

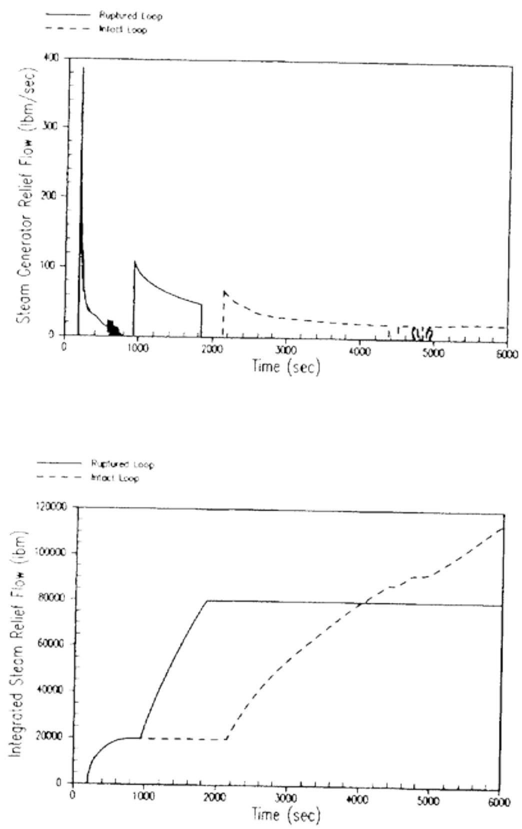


Figure 15.6-12
SGTR (Dose), Steam Generator Relief Flow and Integrated Steam Generator Relief Flow vs. Time

Figure 15.6-13 SGTR (Dose), Steam Generator Water Volume Versus Time

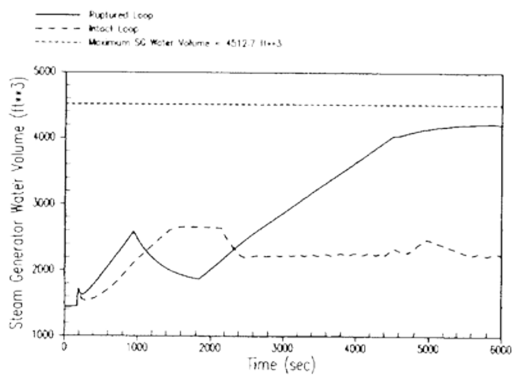
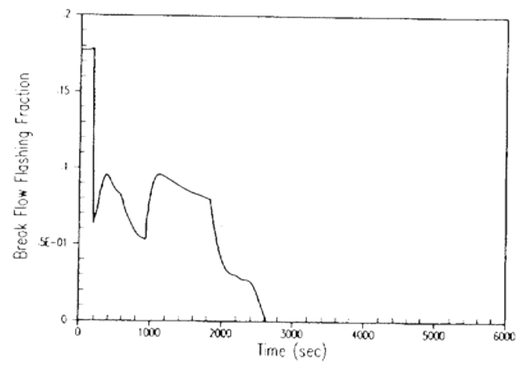


Figure 15.6-13
SGTR (Dose), Steam Generator Water Volume vs. Time

Figure 15.6-14 SGTR (Dose), Tube Rupture Flow Flashing Fraction and Integrated Flashed Break Versus Time



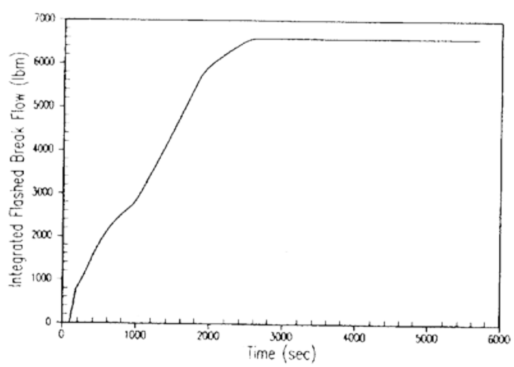


Figure 15.6-14
SGTR (Dose), Tube Rupture Flow Flashing Fraction and Integrated Flashed Break Flow vs. Time

Figure 15.6-15 Small Break LOCA - 2-Inch Break, Pressurizer Pressure Versus Time

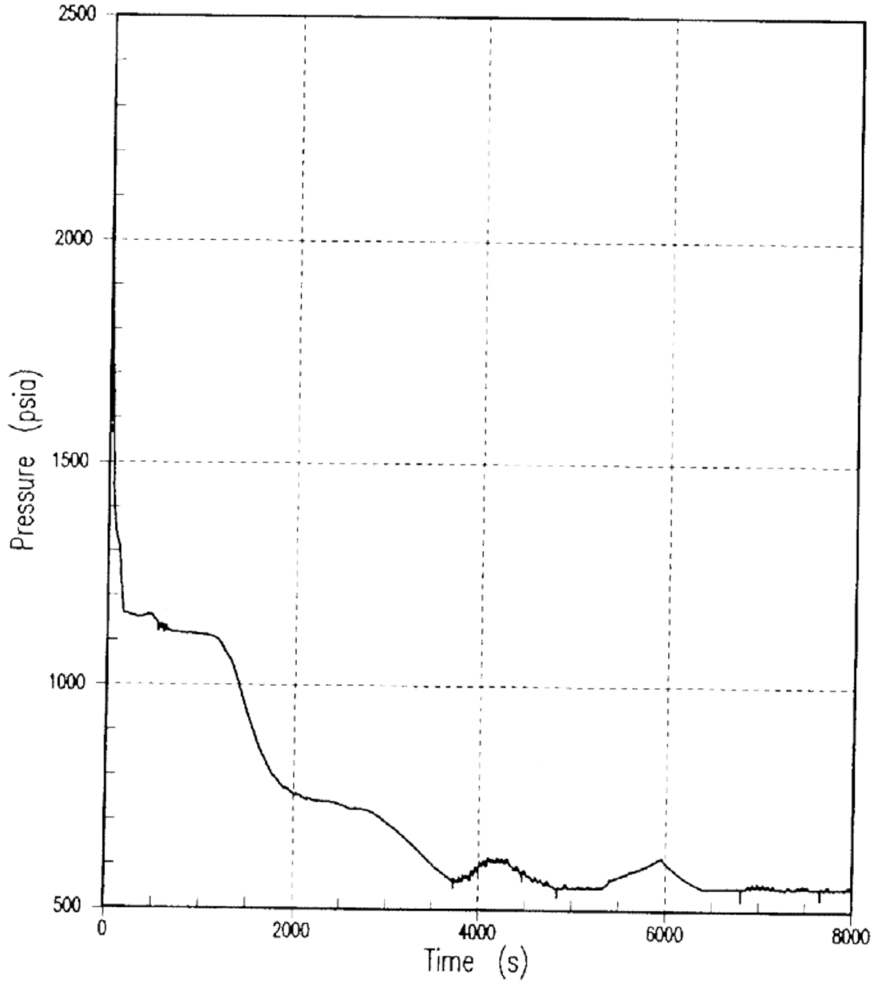


Figure 15.6-16 Small Break LOCA - 2-Inch Break, Core Mixture Level Versus Time

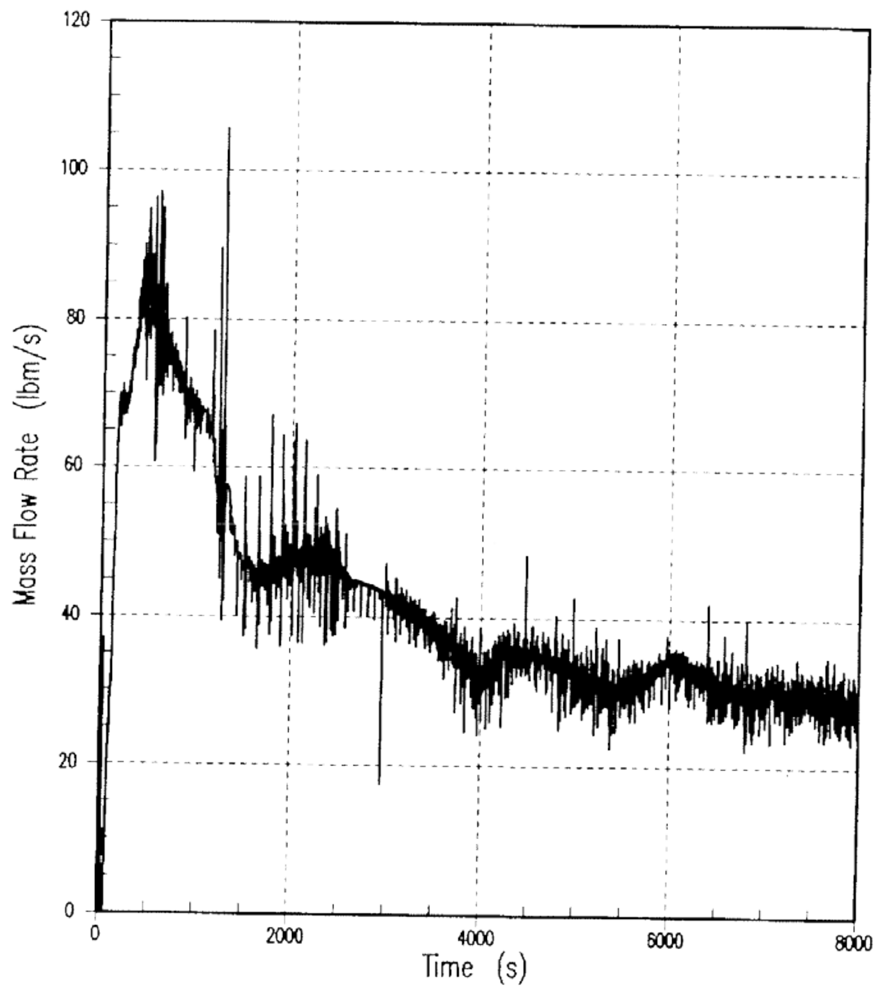


Figure 15.6-17 Small Break LOCA - 2-Inch High Break, Peak Cladding Temperature at PCT Elevation Versus Time

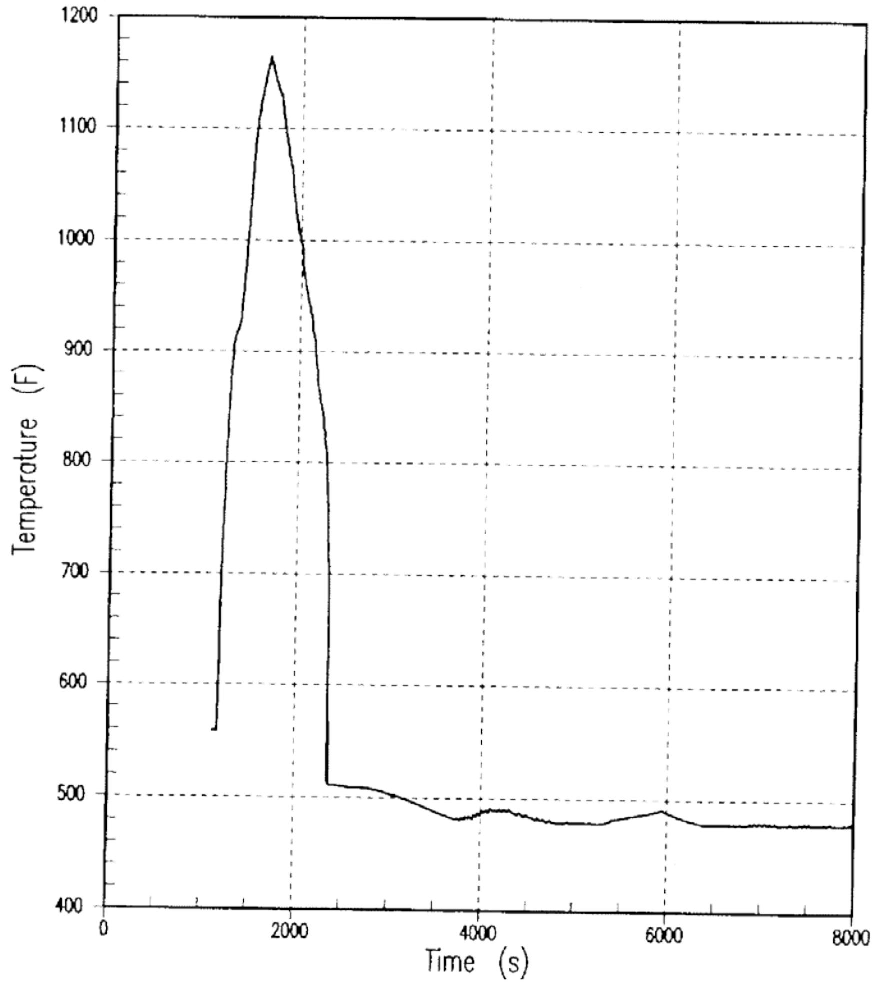


Figure 15.6-18 Small Break LOCA - 2-Inch High Break, Core Exit Vapor Flow Versus Time

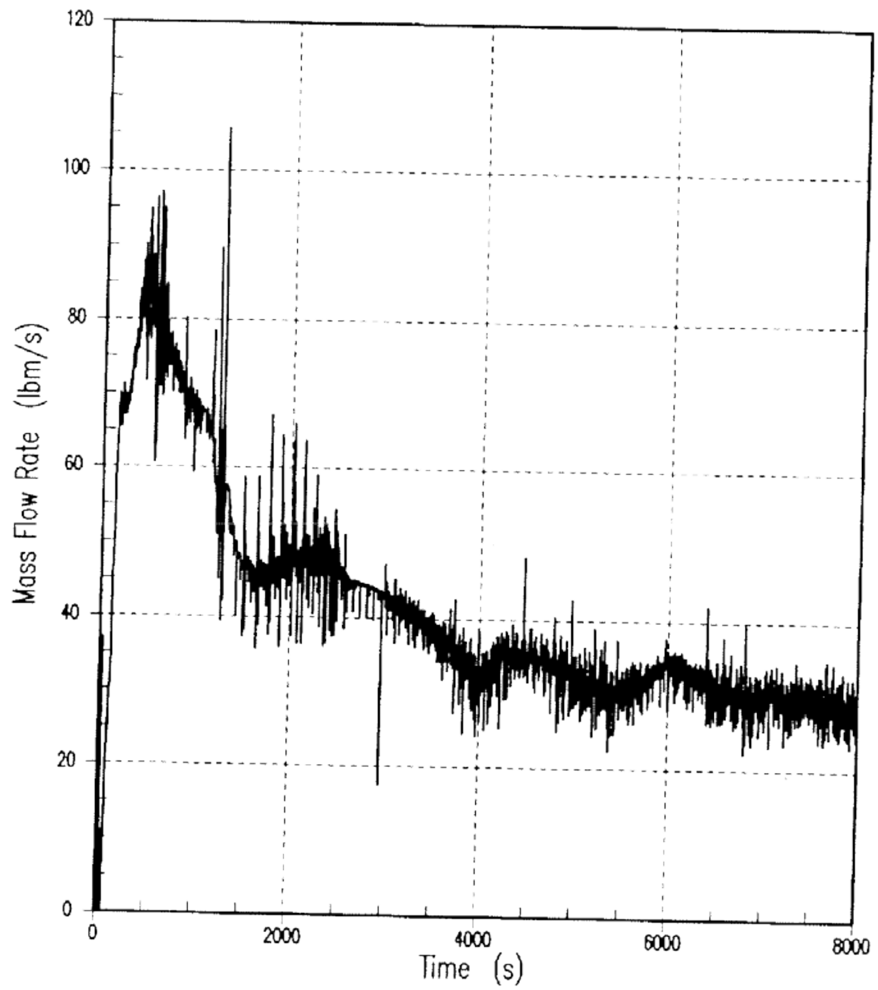


Figure 15.6-19 Small Break LOCA - 2-Inch Break, Hot Rod Heat Transfer Coefficient at PCT Elevation Versus Time

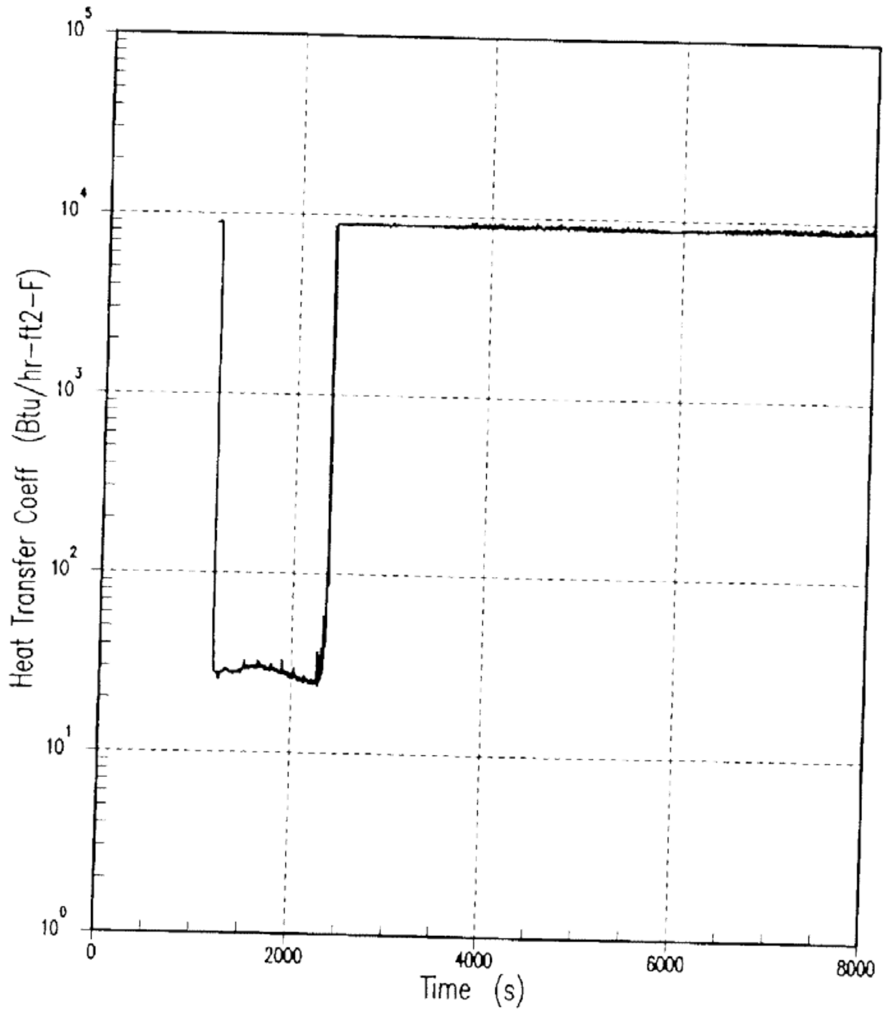


Figure 15.6-20 Small Break LOCA - 2-Inch Break, Fluid Temperature at PCT Elevation Versus Time

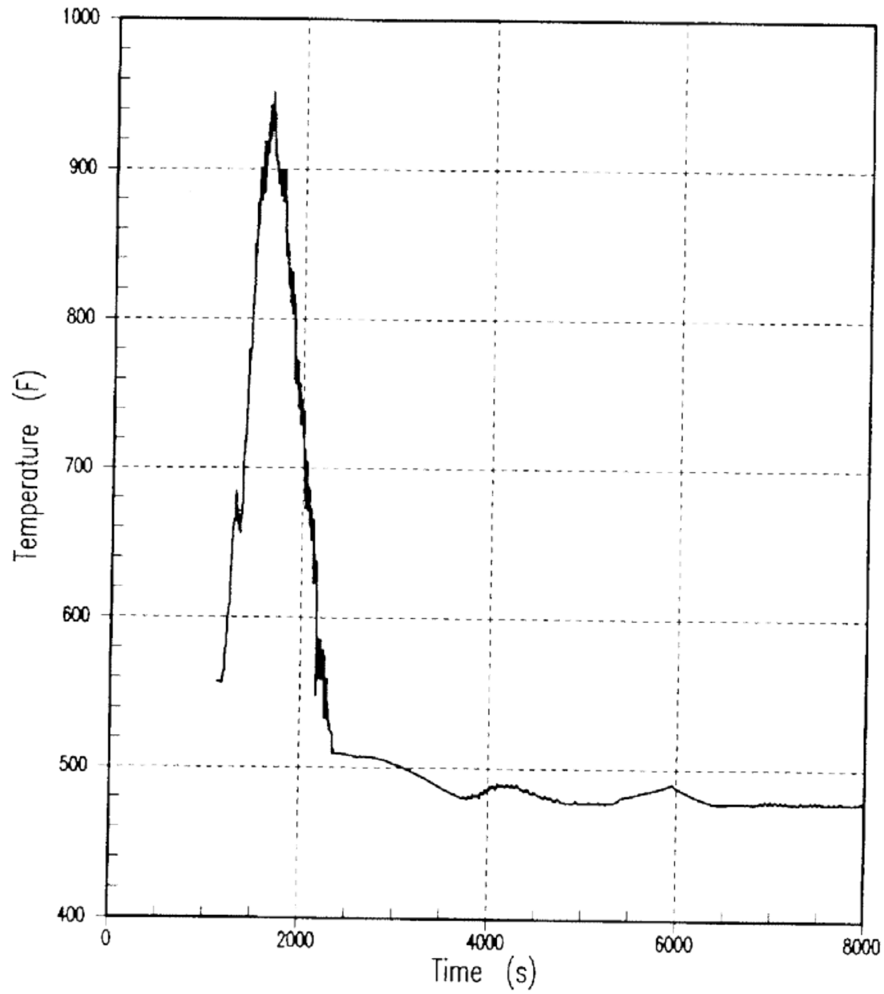


Figure 15.6-21 Small Break LOCA - Axial Power Distribution, Heat Rate Versus Core Elavation

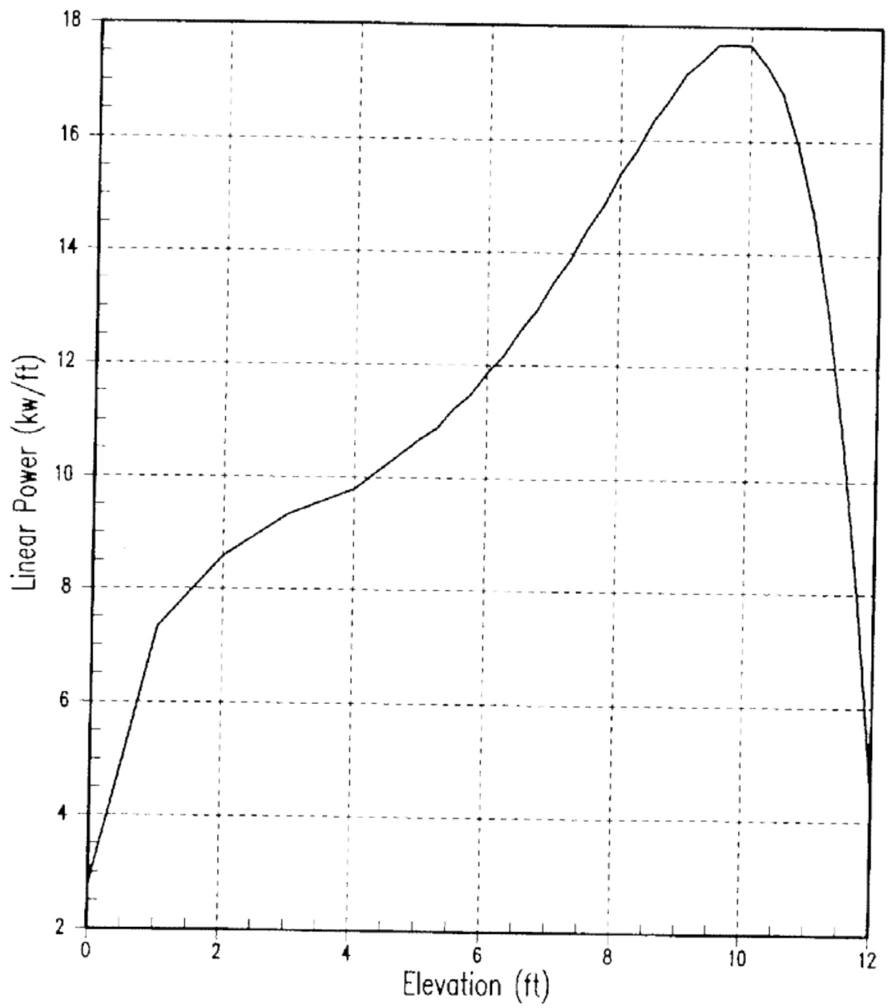


Figure 15.6-22 Small Break LOCA - 1.5-Inch Break, Pressurizer Pressure Versus Time

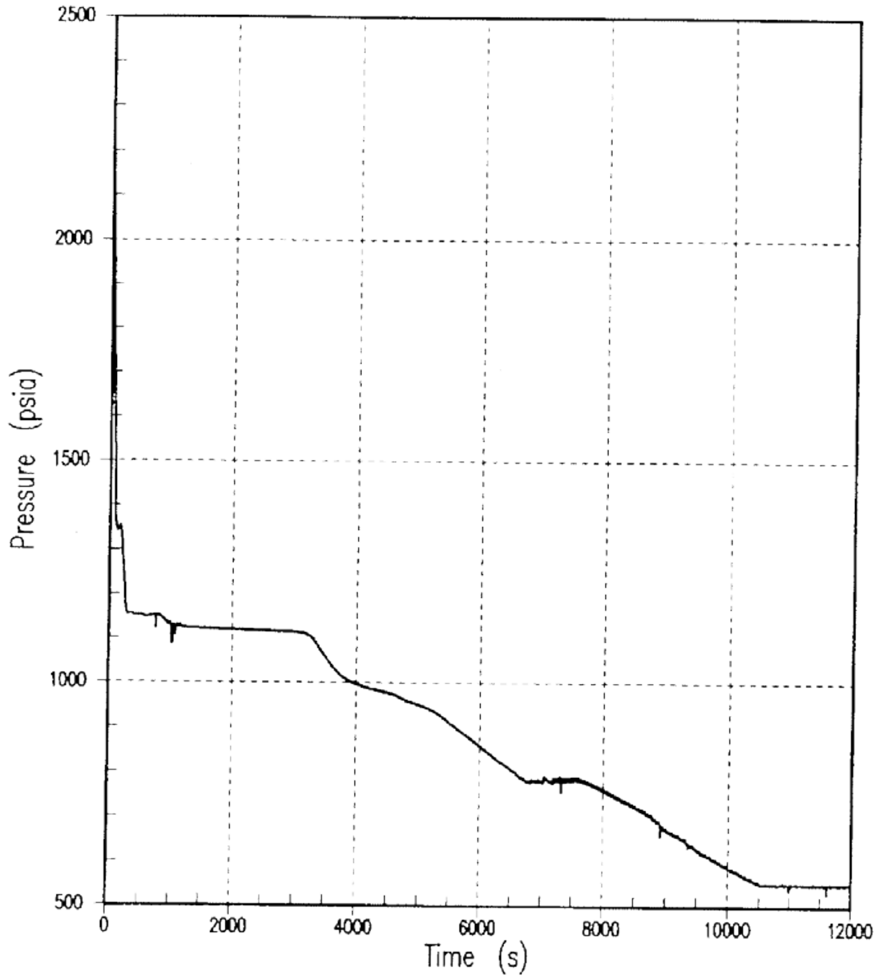


Figure 15.6-23 Small Break LOCA - 3-Inch High Break, Pressurizer Pressure Versus Time

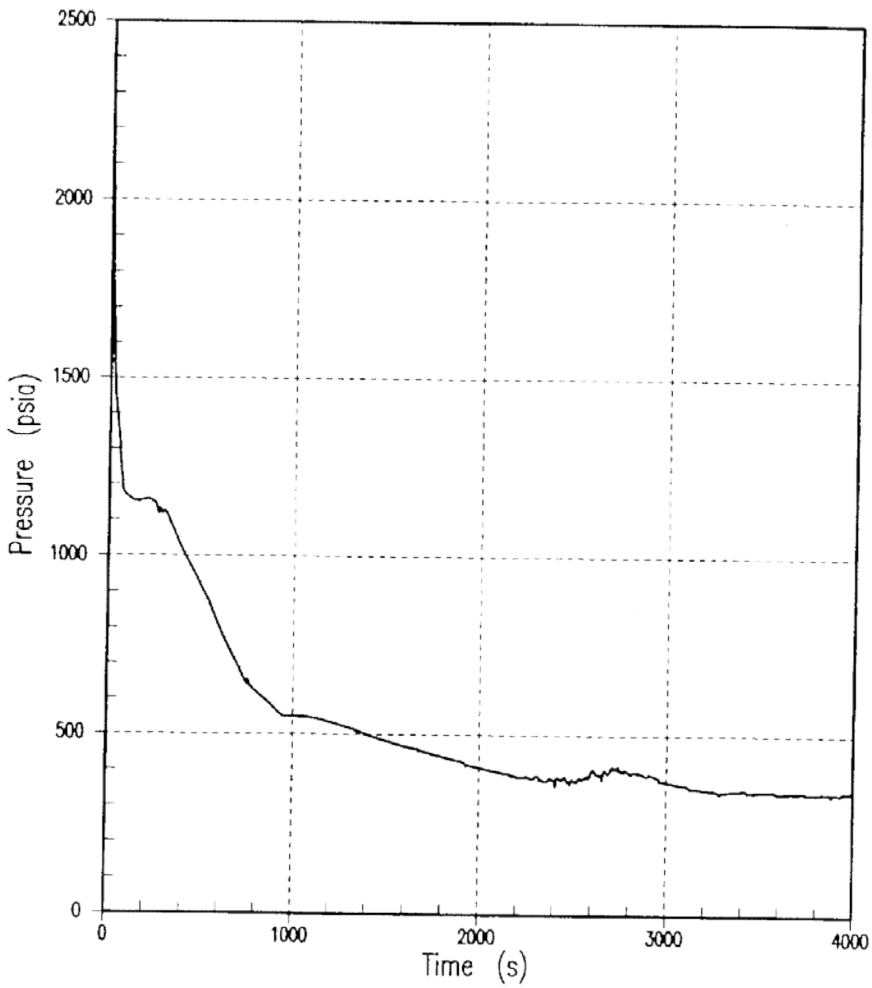


Figure 15.6-24 Small Break LOCA - 1.5-Inch Break, Core Mixture Level Versus Time

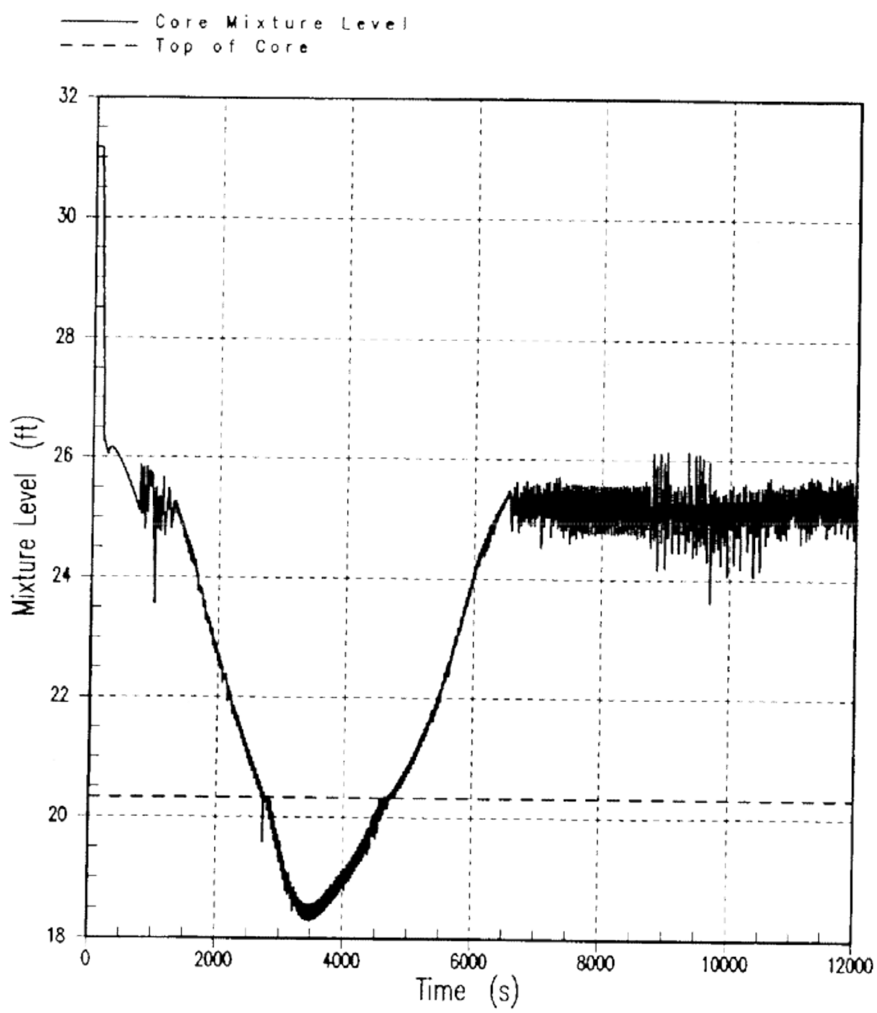


Figure 15.6-25 Small Break LOCA - 3-Inch Break, Core Mixture Level Versus Time

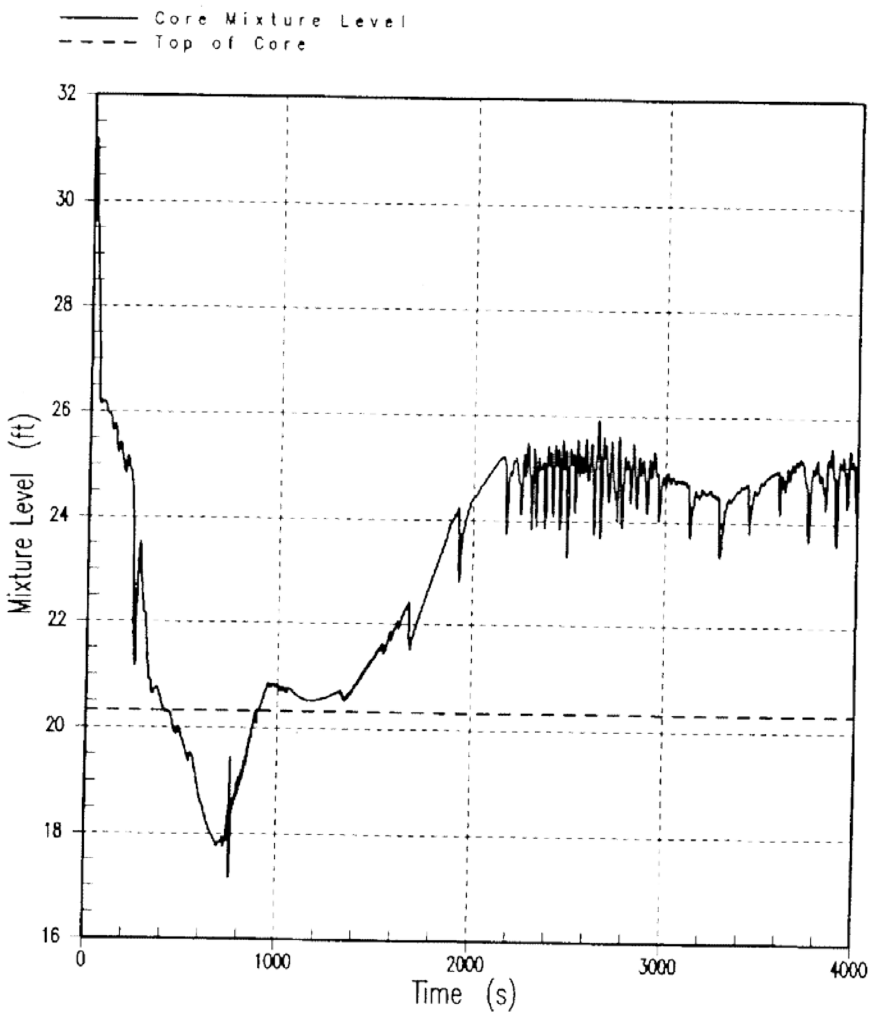


Figure 15.6-26 Small Break LOCA - 1.5-Inch Break, Peal Cladding Temperature at PCT Elevation Versus Time

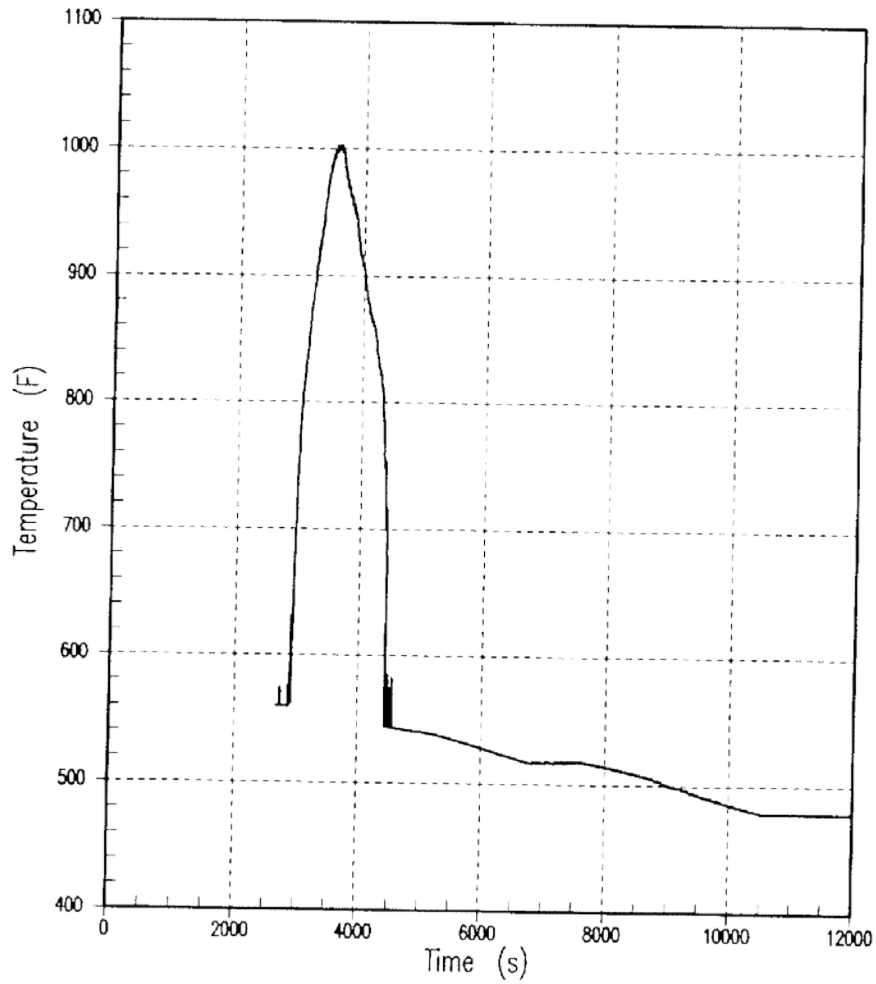


Figure 15.6-27 Small Break LOCA - 3-Inch Break, Peak Cladding Temperature at PCT Elevation Versus Time

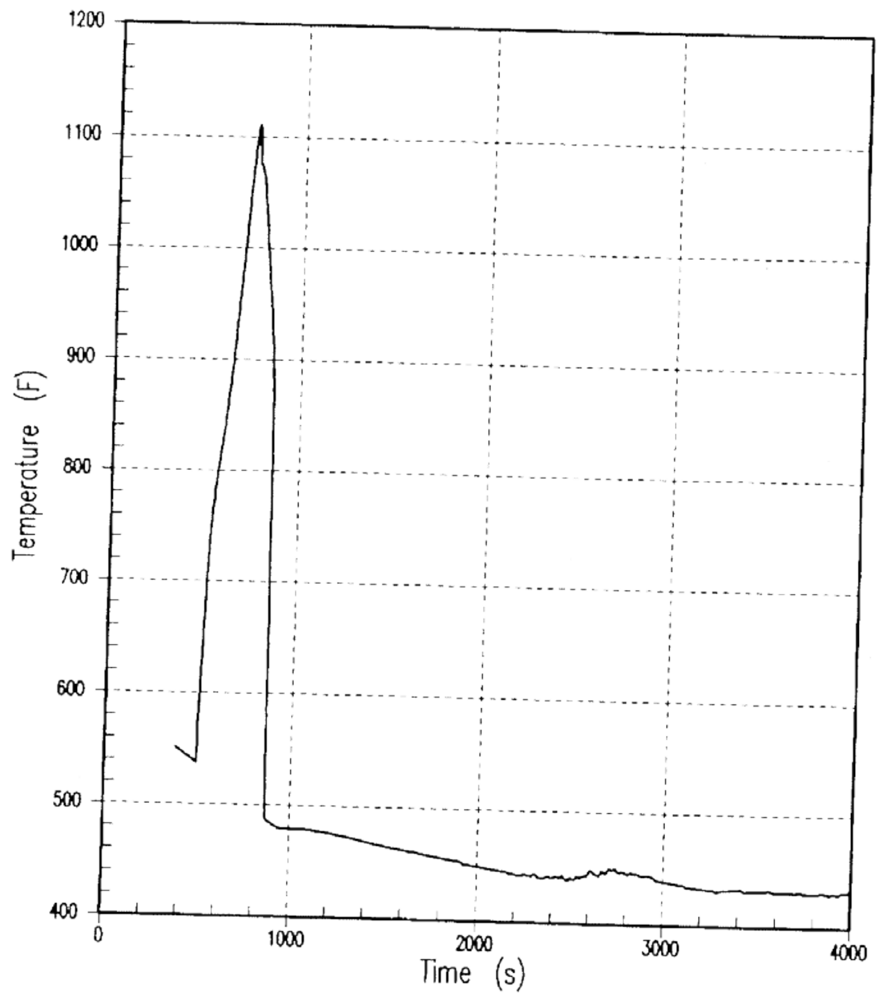


Figure 15.6-28 Figure Deleted

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Figure 15.6-29 Figure Deleted

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Figure 15.6-30 Figure Deleted

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Figure 15.6-31 R.E. Ginna Vessel Model Noding Diagram[1]

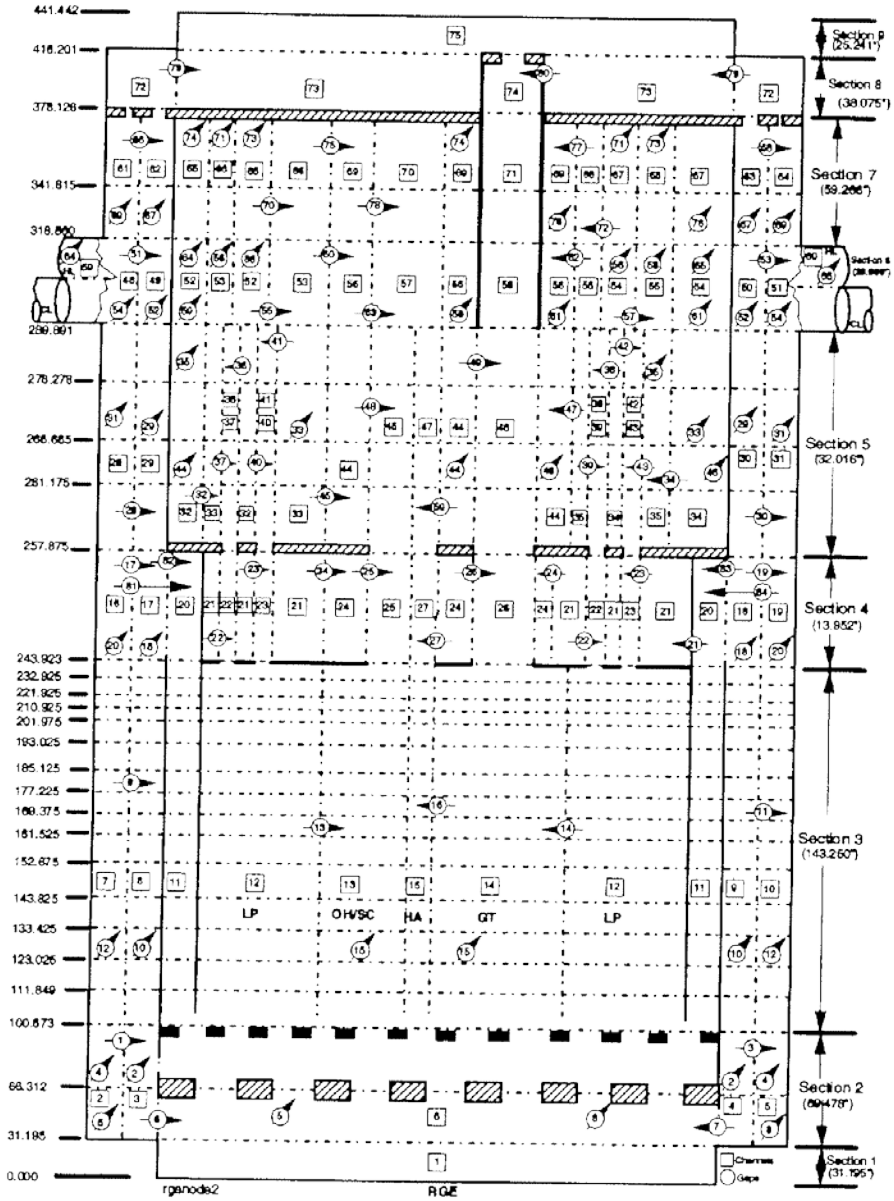
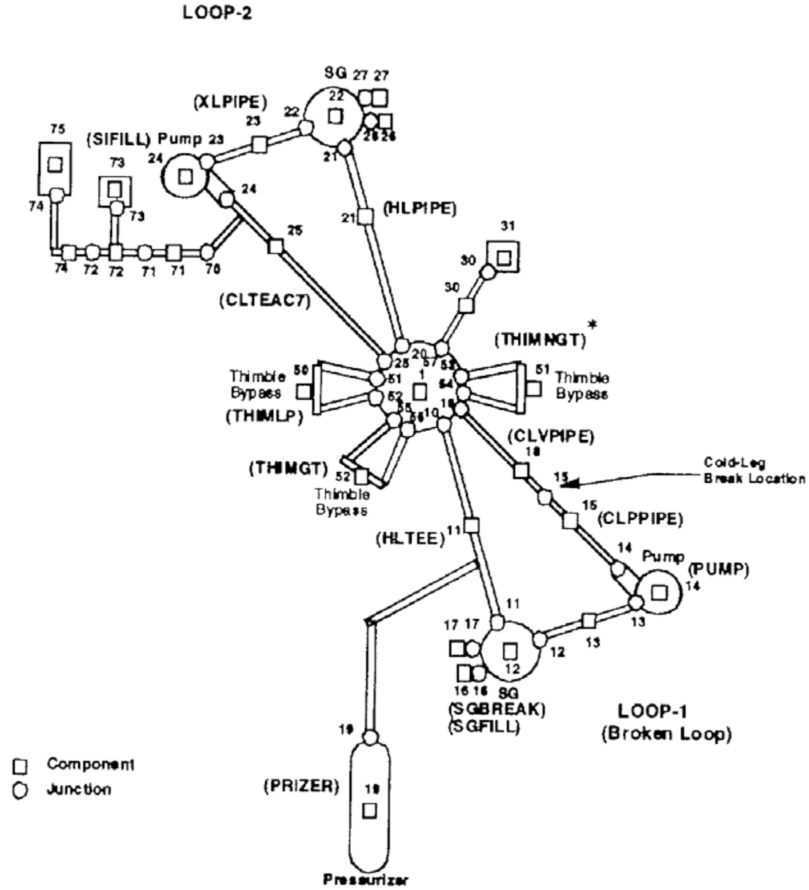


Figure 15.6-31 R. E. Ginna Vessel Model Noding Diagram[1]

[1] Note that Gaps 21, 51, and 53 are blocked, however, they appear on the noding diagram for completeness.

Figure 15.6-32 R.E. Ginna Loop Model Noding Diagram



*THIMNGT = Non Guide Tube Thimbles

Figure 15.6-32 R. E. Ginna Loop Model Noding Diagram

Figure 15.6-33 R.E. Ginna Initial Transient Axial Power Distributions

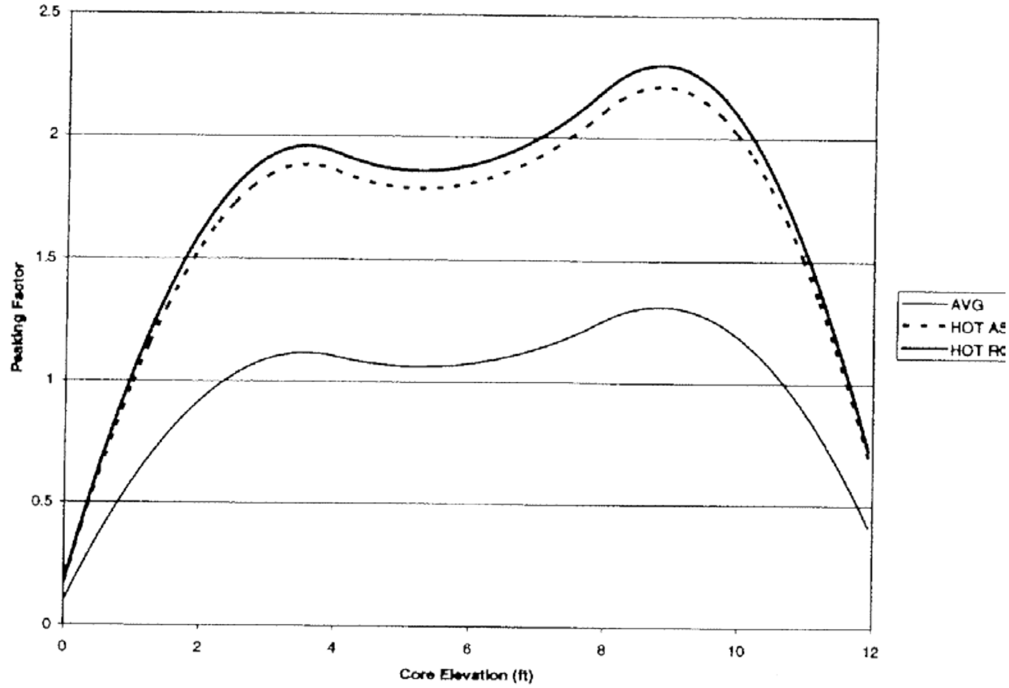


Figure 15.6-33 R. E. Ginna Initial Transient Axial Power Distributions

Figure 15.6-34 Containment Pressure Used for the R.E. Ginna Best-Estimate Large Break LOCA Initial Transient

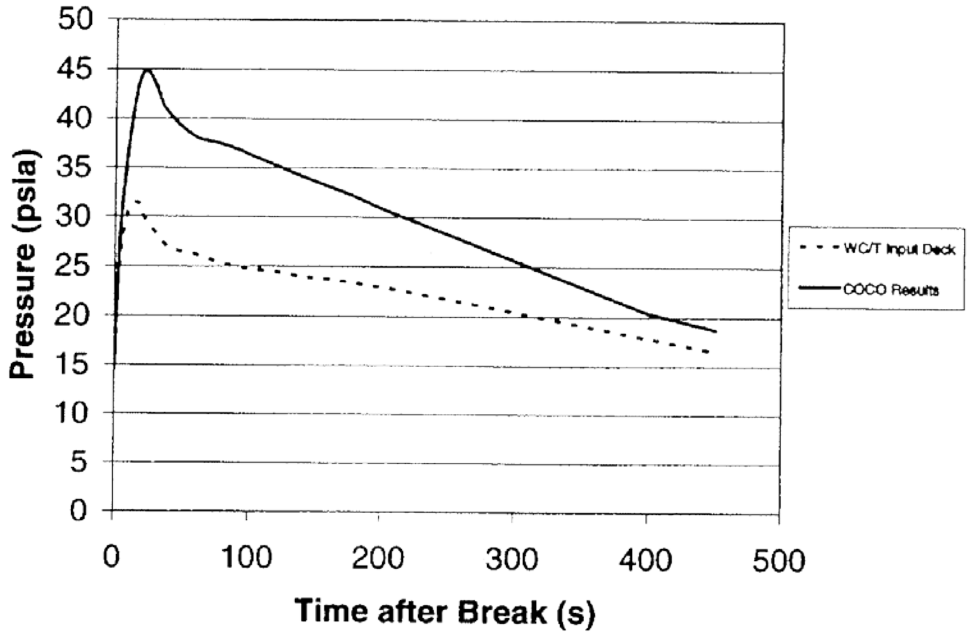


Figure 15.6-34 Containment Pressure Used for the R. E. Ginna Best-Estimate Large Break LOCA Initial Transient

Figure 15.6-35 Peak Clad Temperature of the 5 rods for the Initial Transient

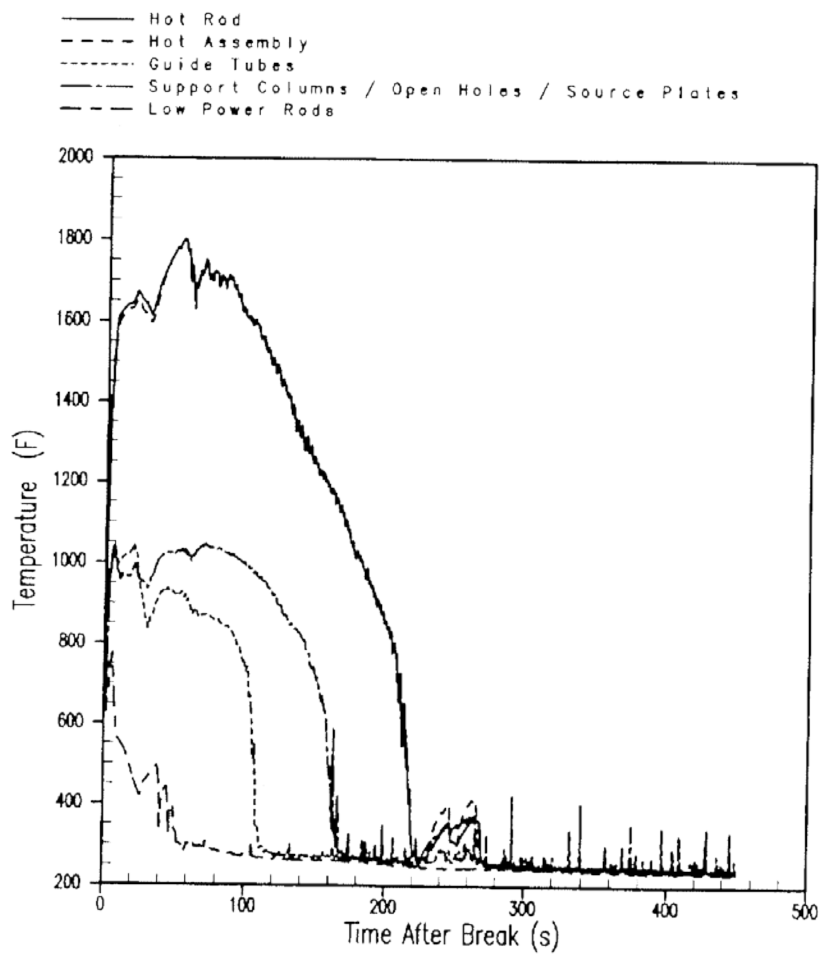


Figure 15.6-35 Peak Clad Temperature of the 5 rods for the Initial Transient

Figure 15.6-36 Split Break Flow for the Initial Transient

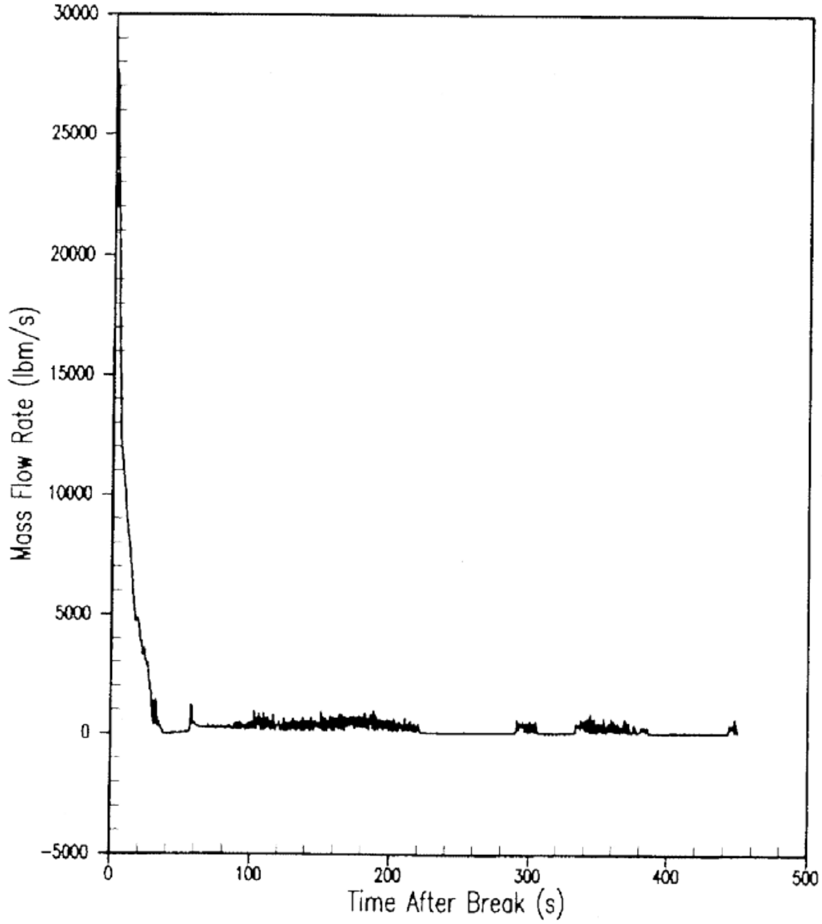


Figure 15.6-36 Split Break Flow for the Initial Transient

Figure 15.6-37 Total Flow at the Bottom of the Core for the Initial Transient

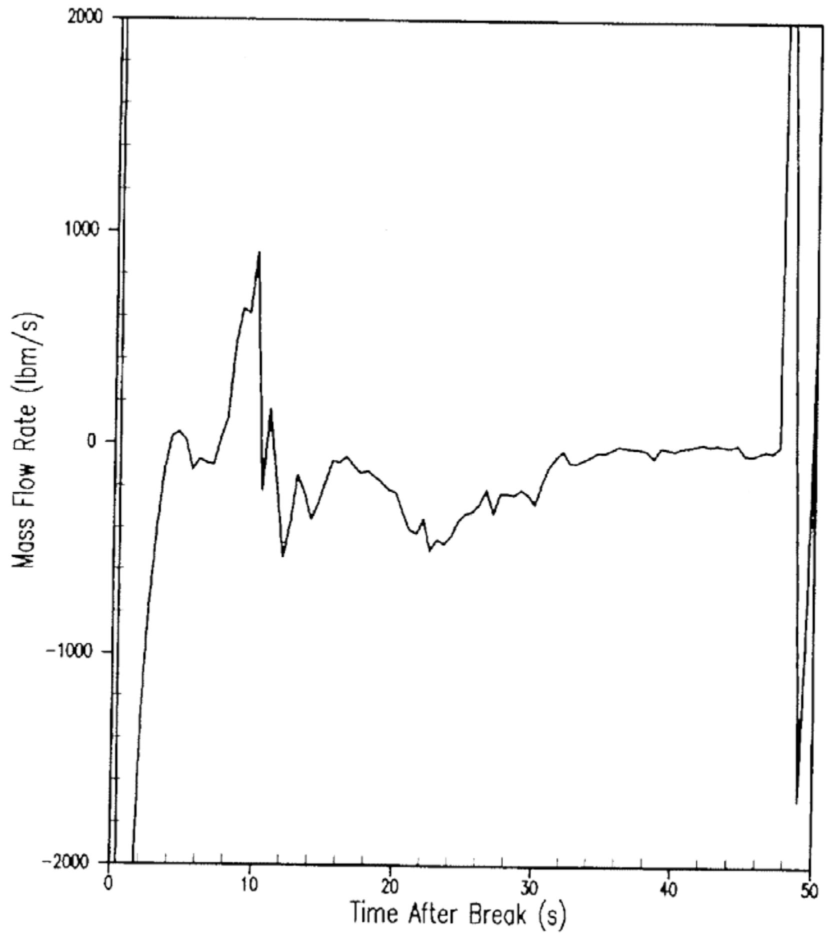


Figure 15.6-37 Total Flow at the Bottom of the Core for the Initial Transient

Figure 15.6-38 Accumulator Injection Flow for the Initial Transient

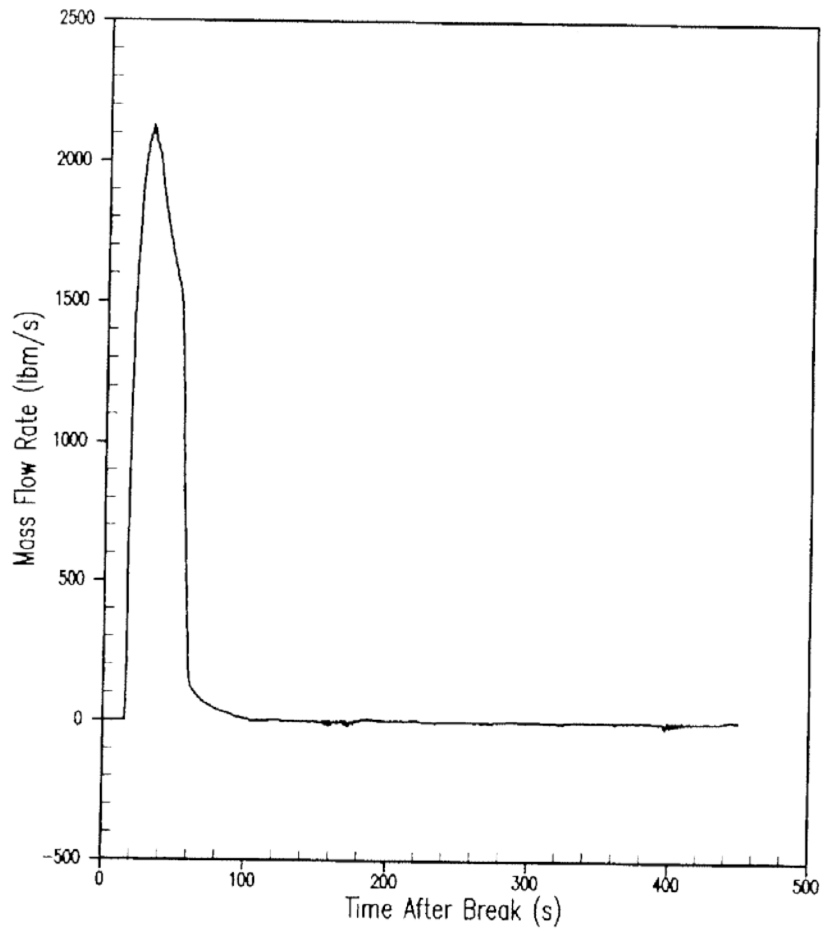


Figure 15.6-38 Accumulator Injection Flow for the Initial Transient

Figure 15.6-39 High Head Safety Injection Flow for the Initial Transient

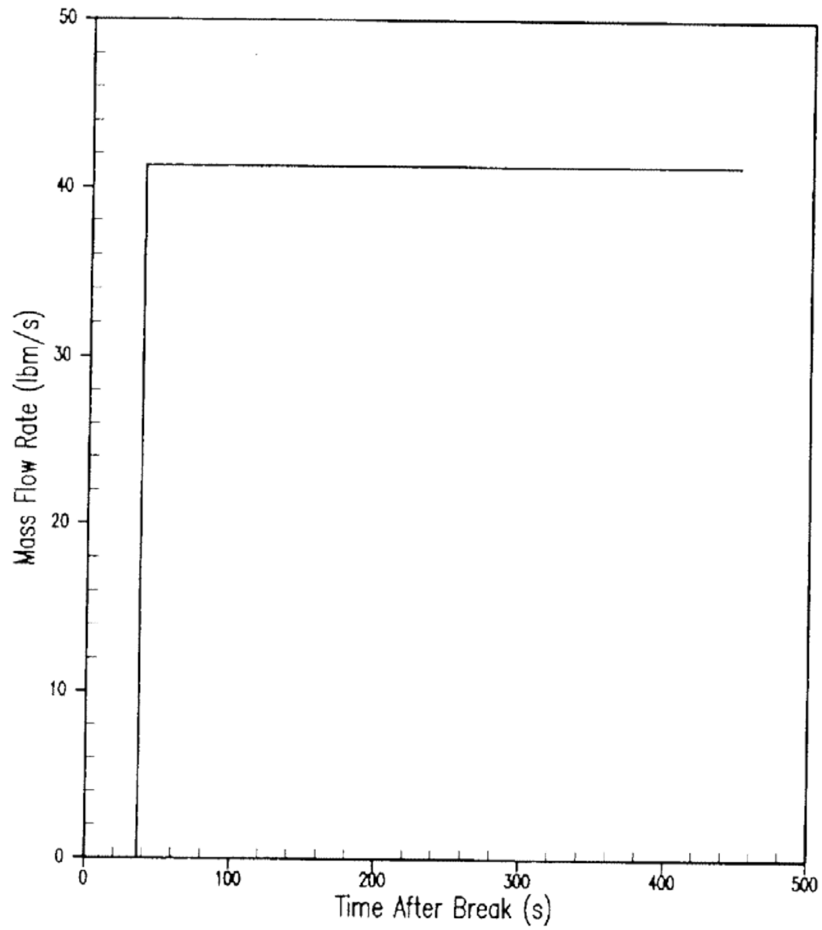


Figure 15.6-39 High Head Safety Injection Flow for the Initial Transient

Figure 15.6-40 Low Head Safety Injection Flow for the Initial Transient

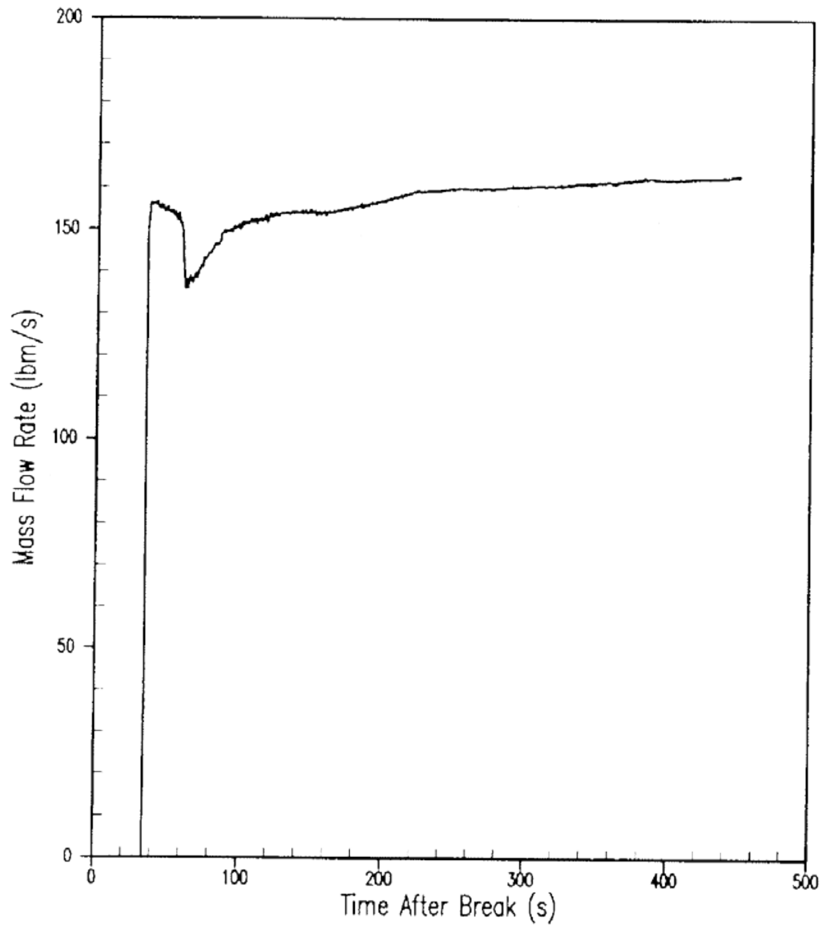


Figure 15.6-40 Low Head Safety Injection Flow for the Initial Transient

Figure 15.6-41 Average Collapsed Liquid Level in the Downcomer for the Initial Transient

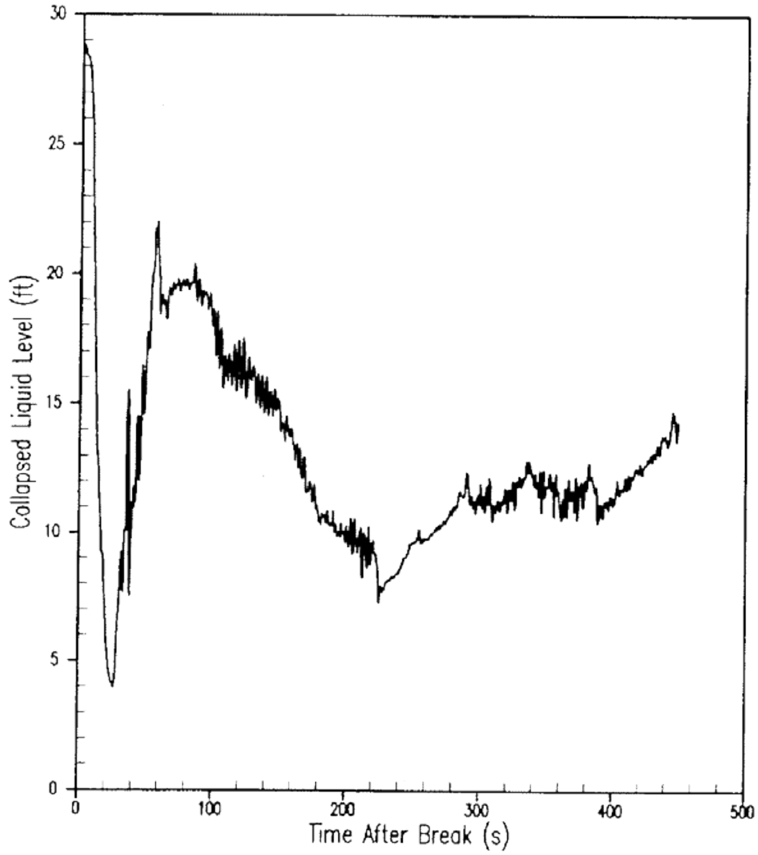


Figure 15.6-41 Average Collapsed Liquid Level in the Downcomer for the Initial Transient

Figure 15.6-42 Lower Plenum Collapsed Liquid Level for the Initial Transient

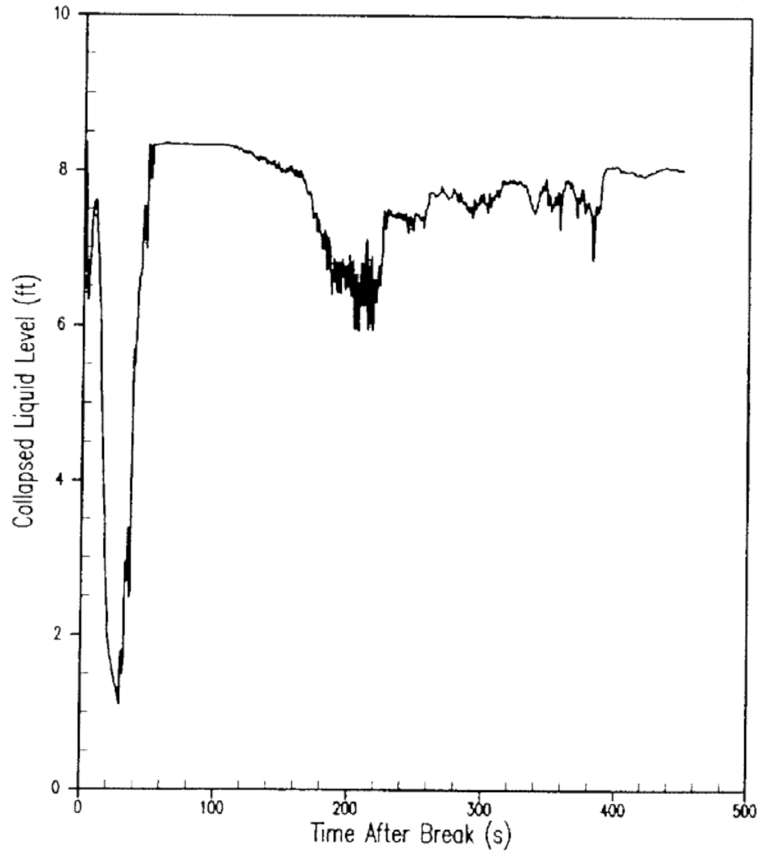


Figure 15.6-42 Lower Plenum Collapsed Liquid Level for the Initial Transient

Figure 15.6-43 Core Collapsed Liquid Levels for the Initial Transient

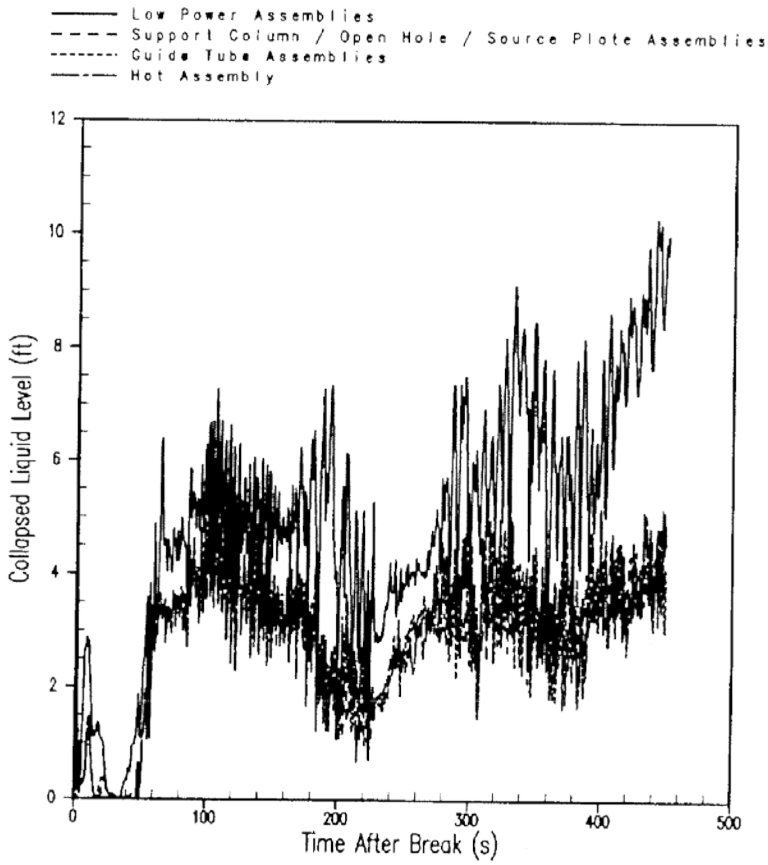


Figure 15.6-43 Core Collapsed Liquid Levels for the Initial Transient

Figure 15.6-44 Vessel Liquid Mass for the Initial Transient

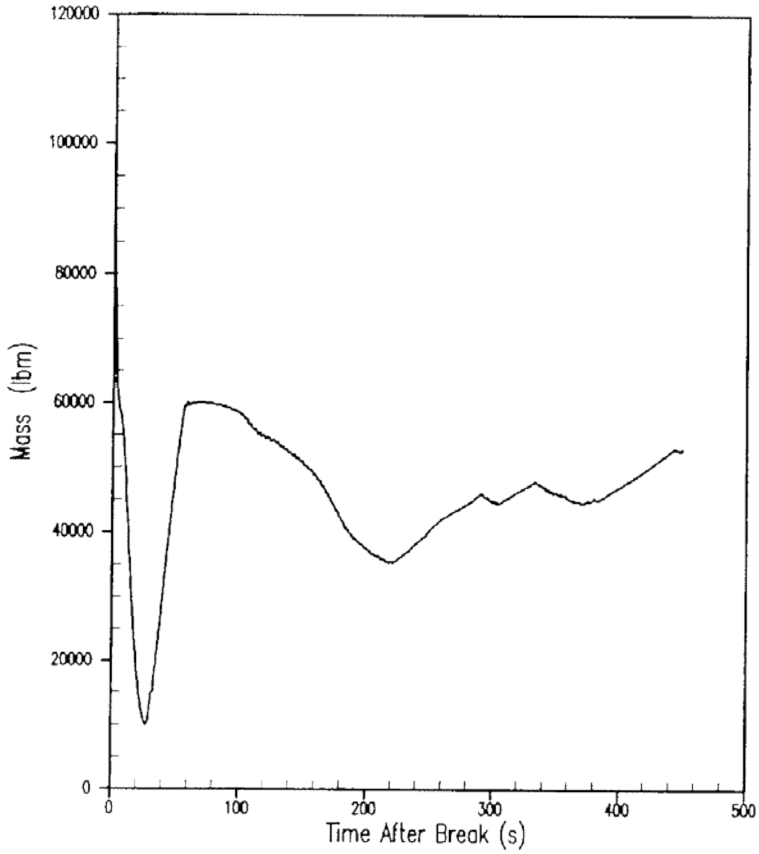


Figure 15.6-44 Vessel Liquid Mass for the Initial Transient

Figure 15.6-45 Pressurizer Pressure for the Initial Transient

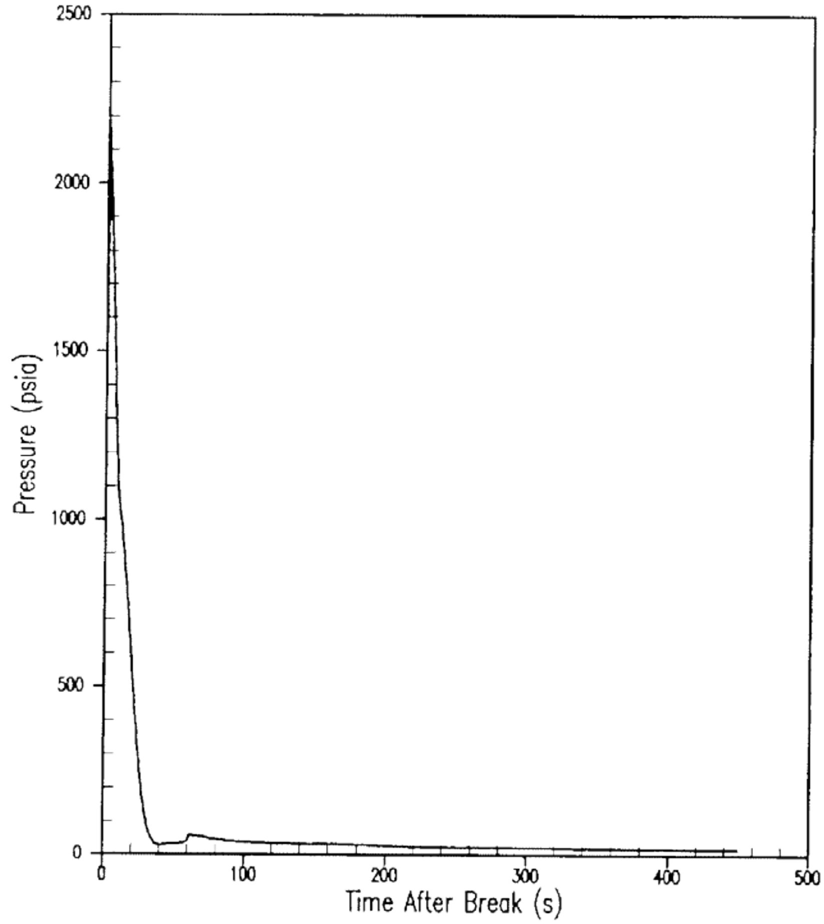


Figure 15.6-45 Pressurizer Pressure for the Initial Transient

Figure 15.6-46 Hot Rod Peak Clad Temperature and Elevation for the Initial Transient

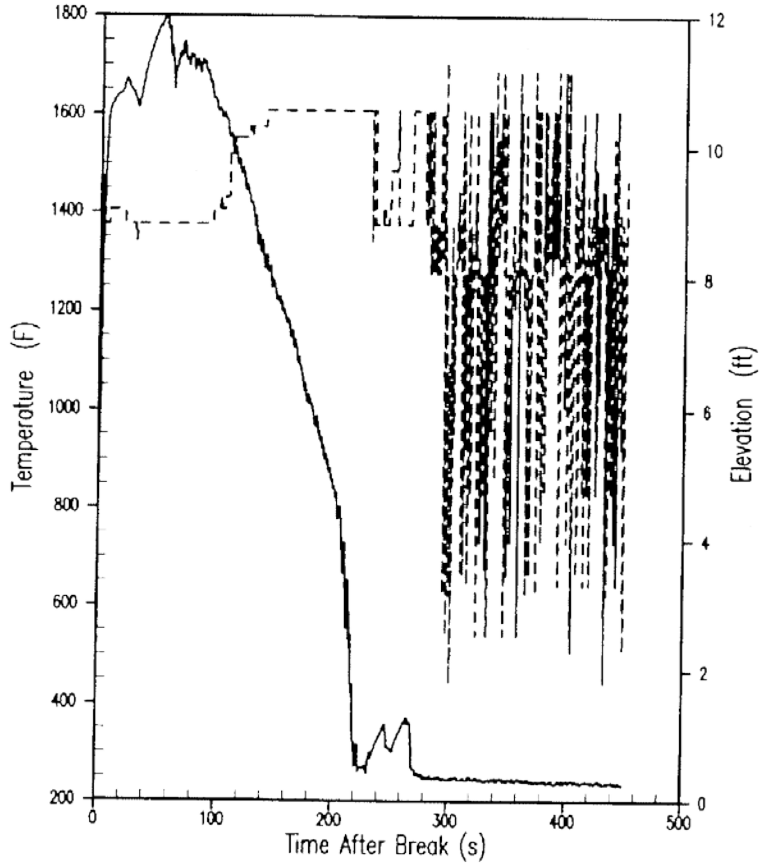


Figure 15.6-46 Hot Rod Peak Clad Temperature and Elevation for the Initial Transient

Figure 15.6-47 R.E. Ginna PBOT/PMID Analysis and Operating Limits

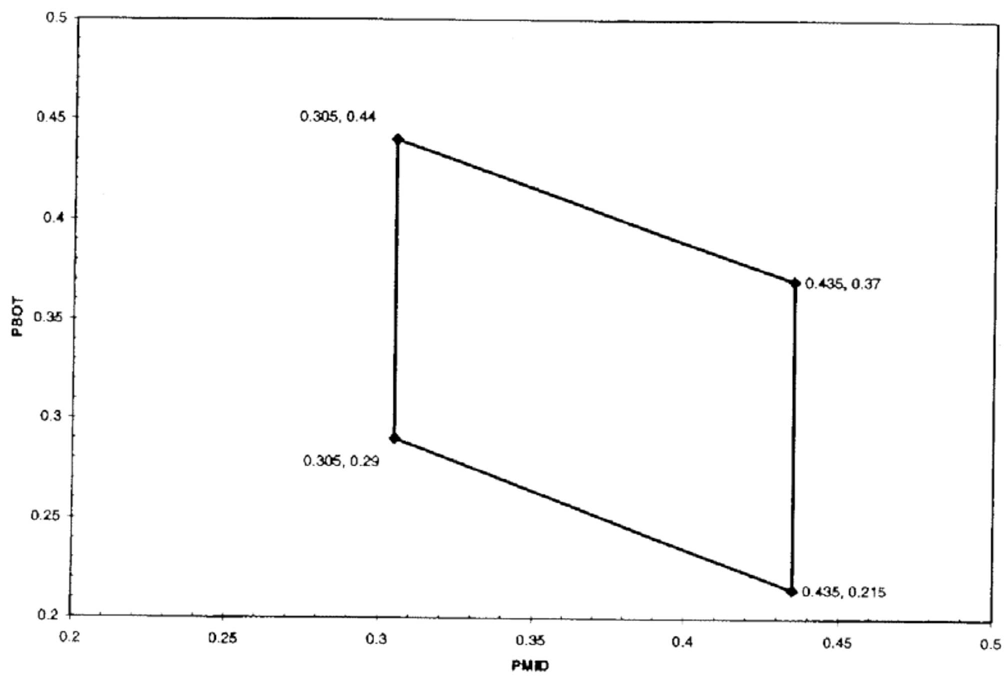


Figure 15.6-47 R. E. Ginna PBOT/PMID Analysis and Operating Limits

Figure 15.6-48 Lower Bound Containment Pressure for R.E. Ginna Analysis

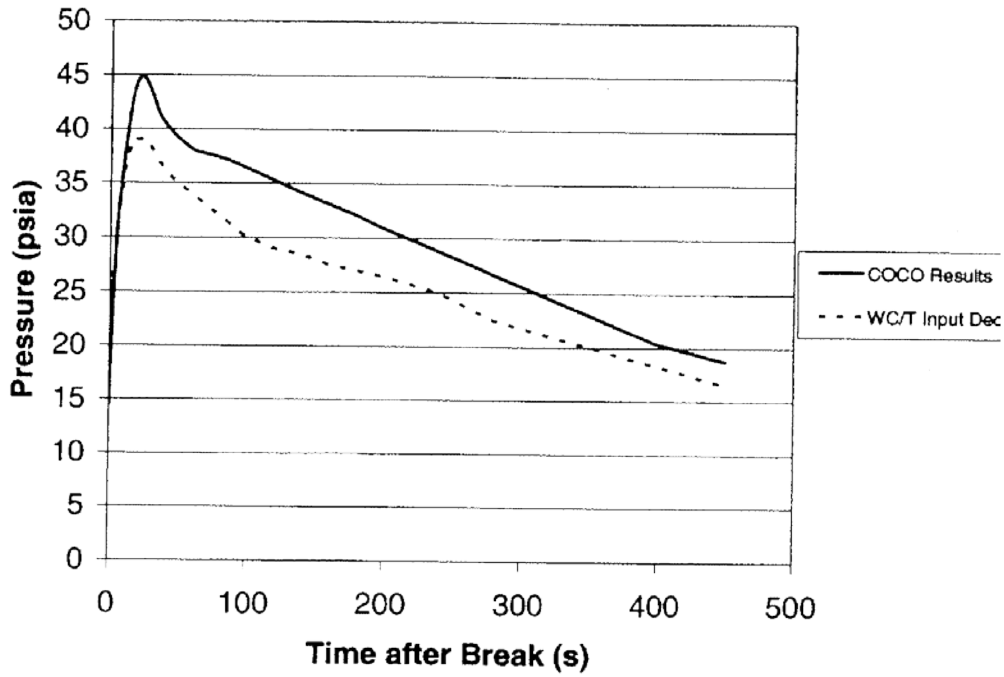


Figure 15.6-48 Lower Bound Containment Pressure for R. E. Ginna Analysis