

PSE&G REGULATORY DOCKET FILE

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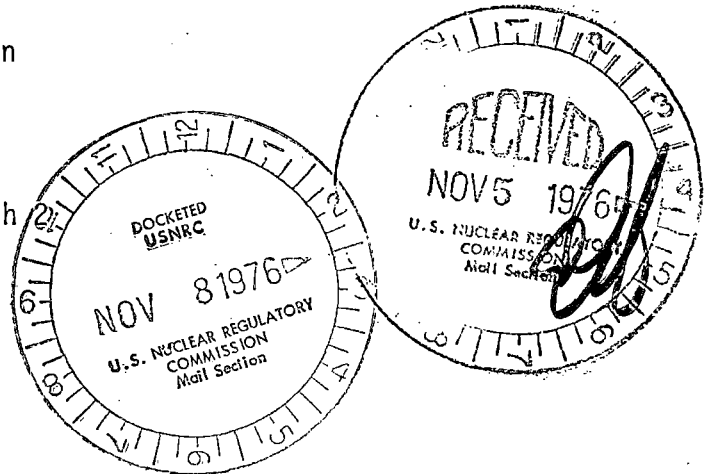
October 29, 1976

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. Karl Kniel, Chief
Light Water Reactor Branch

Gentlemen:

REACTOR PRESSURE VESSEL SUPPORTS
ASYMMETRIC CAVITY PRESSURE LOADING
NO. 1 AND 2 UNITS
SALEM NUCLEAR GENERATING STATION
DOCKET NOS. 50-272 AND 50-311



Reference: (1) Letter to D. B. Vassallo (USNRC) from R. L. Mittl (PSE&G) dated March 1, 1976, this subject.

In the Reference 1 letter, the status of the Salem plants regarding the asymmetric cavity loading concern was discussed. Specifically, it was stated that motion limiters (bumpers) had been installed on the reactor coolant loop piping in order to limit postulated break sizes and hence reduce resulting forces on the reactor vessel supports. Results of analyses to conclusively demonstrate continued reactor pressure vessel support integrity were expected in October 1976.

The aforementioned analyses have been actively pursued. Input data for the three phases of the force generation calculation (i.e., asymmetric cavity forces, loop piping forces, and reactor vessel internals) have been generated and preliminary runs of computer calculations of the asymmetric cavity forces have been made. Results obtained to date confirm the confidence expressed in our Reference 1 letter as to the adequacy of the Salem reactor vessel supports. Formal calculation of the other two components of the force calculation, time history superposition, or detailed component evaluations have not yet been made.

As you are aware, the subject concern has been the object of substantial review and discussion within both the NRC and the nuclear industry. A dichotomy of approach has arisen, such that some plant owners are pursuing detailed analytical justification of their designs, while others, convinced of the justifiability of their designs, are proposing a program of augmented In-service Inspections as an active rather than passive step in addressing the subject concern.

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Public Service has been reviewing both approaches. Inasmuch as the aforementioned preliminary analysis continues to provide confidence that the Salem supports are adequate, we believe that an augmented ISI Program should be considered as an alternate to further analytical treatment. We are placing emphasis on defining an augmented ISI Program which could be adopted following NRC acceptance of the augmented inspection alternate as proposed by the nuclear industry.

Very truly yours,



R. L. Mittl
General Manager - Projects
Engineering and Construction
Department