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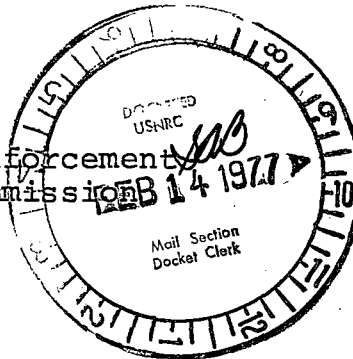
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Public Service Electric and Gas Company Salem Nuclear Generating Station
P.O. Box #168 Hancocks Bridge, New Jersey 08038

February 7, 1977

50-~~272~~

Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



MONTHLY OPERATING REPORT
SALEM GENERATING STATION

In compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, the following Monthly Operating Reports for the month of January 1977 are being sent to you. (original plus 10 copies)

Appendix B - Average Daily Unit Power Level
Appendix C - Operating Data Report
Appendix D - Unit Shutdowns and Power Reductions

Additional copies of the Monthly Operating Reports are being sent to:

1. Mr. James P. O'Reilly (1 Copy)
Director of US NRC
Office of Inspection and Enforcement
Region I
631 Park Avenue
King of Prussia, PA 19406
2. Director, Office of Management Information (2 Copies)
and Program Control
US Nuclear Regulatory Commission
Washington, D.C. 20555

H. J. Heller

H. J. Heller
Manager - Salem Generating Station

Enc.

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The Energy People

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-272
UNIT Salem No.1
DATE 2-7-77
COMPLETED BY J.M. ZUPKO JR.
TELEPHONE 609-365-7000 EXT. 507

MONTH January

DAY AVERAGE DAILY POWER LEVEL
(MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>38</u>
9	<u>17</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY AVERAGE DAILY POWER LEVEL
(MWE-NET)

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>61</u>
21	<u>111</u>
22	<u>23</u>
23	<u>0</u>
24	<u>24</u>
25	<u>173</u>
26	<u>226</u>
27	<u>229</u>
28	<u>147</u>
29	<u>119</u>
30	<u>238</u>
31	<u>220</u>

APPENDIX C
OPERATING DATA REPORT

DOCKET NO. 50-272
UNIT Salem No. 1
DATE 2-7-77
COMPLETED BY J.M. Zupko Jr.
TELEPHONE 609-365-7000 Ext.507

OPERATING STATUS

1. REPORTING PERIOD: January 1976 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3398 MAX. DEPEND. CAPACITY (MWe-NET): To Be Determined
DESIGN ELECTRICAL RATING (MWe-Net): 1090
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): None
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>481.5</u>	<u>481.5</u>	<u>783.1</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
7. HOURS GENERATOR ON LINE	<u>256.1</u>	<u>256.1</u>	<u>296.4</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>246014</u>	<u>246014</u>	<u>294711</u>
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	<u>53,290</u>	<u>53,290</u>	<u>56,520</u>
11. NET ELECTRICAL ENERGY GENERATED (MWH)	<u>28,375</u>	<u>28,375</u>	<u>28,375</u>
12. REACTOR SERVICE FACTOR	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
13. REACTOR AVAILABILITY FACTOR	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
14. UNIT SERVICE FACTOR	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
15. UNIT AVAILABILITY FACTOR	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
16. UNIT CAPACITY FACTOR (Using MDC)	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
17. UNIT CAPACITY FACTOR (Using Design MWe)	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>
18. UNIT FORCED OUTAGE RATE	<u>N/A</u>	<u>N/A</u>	<u>N/A</u>

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): 4-2-77
Modification of Low Pressure Turbine Blading, Eight Weeks.

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):	FORECAST	ACHIEVED
INITIAL CRITICALITY	<u>9-30-76</u>	<u>12-11-76</u>
INITIAL ELECTRICITY	<u>11-01-76</u>	<u>12-25-76</u>
COMMERCIAL OPERATION	<u>12-20-76</u>	

APPENDIX D

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-272

UNIT NAME Salem No. 1

DATE 2-7-77

COMPLETED BY J.M. Zupko Jr.

TELEPHONE 609-365-7000 Ext. 507

REPORT MONTH January 1977

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
76-4	770101	F	15.1	A	3	12 Stm. Gen. Feedwater Regulating Valve
77-1	770102	F	13.0	H	3	11, 12 Stm. Gen. Feed Pump Trip
77-2	770102	S	16.0	B	3	Test Procedure-Loss of Offsite Pwr.
77-3	770103	S	112.0	B	3	Test Procedure-Steam Dump Control
77-4	770108	F	12.2	H	3	11 Stm. Gen. Feed Pump Trip
77-5	770109	F	17.6	B	1	Turbine Intercept Valve Leak
77-6	770110	F	29.5	H	3	12 Stm. Gen. Feed Pump Trip
77-7	770111	F	203.7	B	3	11, 12 Stm. Gen. Feed Pump Trip
77-8	770122	F	56.7	H	3	12 Stm. Gen. Feed Pump Trip
77-9	770125	F	2.8	H	3	Loss of Generator Excitation
77-10	770128	F	3.1	H	3	12 Stm. Gen. Flow Calibration - Procedural Error.
77-11	770128	F	3.2	H	3	Frozen Impulse Pressure Feeler Line - Turbine Trip.
77-12	770129	F	3.0	H	3	11, 12 Stm. Gen. Feed Pump Trip

SUMMARY: