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REGULATORY DOCKET FILE COPY

Public Service Electric and Gas Company P.O. Box 168 Hancocks Bridge, New Jersey 08038

Salem Nuclear Generating Station

August 10, 1978

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

US NRC
DISTRIBUTION SERVICES
BRANCH

1978 AUG 17 AM 11 42

REGISTRATION
SERVICES UNIT

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In compliance with section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Summary of Safety Related Maintenance
Operating Summary
Refueling Information

Sincerely yours,

H. J. Midura

H. J. Midura
Manager - Salem Generating Station

LKM:jcm

cc: Mr. Boyce H. Grier
Director of U. S. NRC
Office of Inspection and Enforcement
Region I
631 Park Avenue
King of Prussia, Pa. 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



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A008
5/1

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-272

UNIT Salem #1

DATE August 10, 1978

COMPLETED BY L. K. Miller

TELEPHONE 609-365-7000 X507

MONTH July, 1978

DAY AVERAGE DAILY POWER LEVEL (MWe-NET)

1	737
2	123
3	557
4	623
5	963
6	1024
7	1088
8	946
9	1040
10	996
11	865
12	1037
13	1082
14	962
15	1047
16	883

DAY AVERAGE DAILY POWER LEVEL (MWE-NET)

17	1068
18	286
19	0
20	0
21	226
22	975
23	1067
24	1054
25	834
26	913
27	1021
28	990
29	1050
30	1038
31	1061

50-272

DOCKET NO.:

DATE : August 10, 1978

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

OPERATING STATUS

1. Unit Name: Salem #1
2. Reporting Period: July, 1978
3. Licensed Thermal Power (Mwt): 3338
4. Nameplate Rating (Gross MWe): 1135
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reason:
None

Notes:

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: None

	This Month	Year to Date	Cumulative
11. Hours In Reporting Period	744	5087	9528
12. Number Of Hours Reactor Was Critical	666	2601.5	5144.7
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	652.9	2420.2	4850.8
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,933,495.2	7,168,869.4	13,864,089.4
17. Gross Electrical Energy Generated (MWH)	642,850	2,426,490	4,612,800
18. Net Electrical Energy Generated (MWH)	611,725	2,292,516	4,350,734
19. Unit Service Factor	87.8	47.6	50.9
20. Unit Availability Factor	87.8	47.6	50.9
21. Unit Capacity Factor (Using MDC Net)	76.2	41.8	42.3
22. Unit Capacity Factor (Using DER Net)	75.4	41.3	41.9
23. Unit Forced Outage Rate	14.7	52.6	38.6
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY

INITIAL ELECTRICITY

COMMERCIAL OPERATION

Forecast
9/30/76

11/1/76

12/20/76

Achieved
12/11/76

12/25/76

6/30/77

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1978DOCKET NO.: 50-272UNIT NAME: Salem #1DATE: August 10, 1978COMPLETED BY: L.K. MillerTELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
78-59	7-1-78	F	0	B	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainers.
78-60	7-1-78	F	0	B	4	- - - -	HH	Filter	Clean 12 Heater Drain and Condensate Pump Suction Strainers.
78-61	7-1-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainers.
78-62	7-1-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainers.
78-63	7-1-78	F	0	A	4	- - - -	HH	Filter	Clean Feed Pump and Condensate Pump Suction Strainers.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure(Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error(Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

(Load Reduction)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report(LER) File
(NUREG-0161)

⁵
Exhibit 1-Same
Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1978DOCKET NO.: 50-272UNIT NAME: Salem #1DATE: August 10, 1978COMPLETED BY: L. K. MillerTELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
78-64	7-1-78	F	15.26	A	2	- - - -	HA	Turbin	Manual Reator Trip Due to Turbine E.H. Control Malfunction.
78-65	7-3-78	F	0	A	4	- - - -	HH	Valvex	Repair 11 Steam Generator Feed Pump Recirc. Diaphragm.
78-66	7-3-78	F	0	A	4	- - - -	HH	Valvex	Repair 11 Steam Generator Feed Pump Recirc. Diaphragm.
78-67	7-5-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-68	7-5-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-69	7-5-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-70	7-7-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction strainer.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1978DOCKET NO.: 50-272UNIT NAME: Salem #1DATE: August 10, 1978COMPLETED BY: L.K. MillerTELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
78-71	7-7-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-72	7-8-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-73	7-8-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-74	7-8-78	F	0	A	4	- - - -	HH	Filter	Clean 11, 12, and 13 Condensate Pump Suction Strainers.
78-75	7-9-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-76	7-9-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-77	7-9-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1978DOCKET NO.: 50-272UNIT NAME: Salem #1DATE: August 10, 1978COMPLETED BY: L. K. MillerTELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
78-78	7-10-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-79	7-10-78	F	0	A	4	- - - -	HH	Filter	Clean Steam Generator Feed Pump and Condensate Pump Suction Strainers.
78-80	7-11-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-81	7-12-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-82	7-13-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump and Condensate Pump Suction Strainers.
78-83	7-14-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-84	7-15-78	F	0	A	4	- - - -	HH	Filter	Claen 12 Heater Drain Pump Suction Strainer.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1978DOCKET NO.: 50-272UNIT NAME: Salem #1DATE: August 10, 1978COMPLETED BY: L. K. MillerTELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
78-85	7-17-78	F	0	A	4	- - - -	HH	Filter	Clean 12 Heater Drain Pump Suction Strainer.
78-86	7-18-78	F	97.36	A	3	- - - -	EB	Transf.	Reactor Trip Due to Under Frequency on 4 KV Group Buses 1F and 1H.
78-87	7-22-78	F	0	A	4	- - - -	HH	Filter	Clean 11 and 13 Condensate Pump Suction Strainers.
78-88	7-22-78	F	0	A	4	- - - -	HH	Filter	Clean 12B Circulating Water Screen.
78-89	7-24-78	F	0	A	4	- - - -	HH	Filter	Clean Condensate Pump Suction Strainer.
78-90	7-25-78	F	0	A	4	- - - -	HC	HTEXCH	11A Condenser Leak.
78-91	7-25-78	F	0	A	4	- - - -	HH	Filter	Clean 11 and 12 Steam Generator Feed Pump Suction Strainers.
78-92	7-26-78	F	0	A	4	- - - -	HC	HTEXCH	11A Condenser Leak.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1978DOCKET NO.: 50-272UNIT NAME: Salem #1DATE: August 10, 1978COMPLETED BY: L. K. MillerTELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
78-93	7-26-78	F	0	A	4	- - - -	HF	Filter	12B Circulator Screens Clogged.
78-94	7-27-78	F	0	A	4	- - - -	HC	HTEXCH	11A Circulator Condenser Leak.
78-95	7-27-78	F	0	A	4	- - - -	HH	Filter	Clean Condensate Pump Suction Strainer.
78-96	7-28-78	F	0	A	4	- - - -	RC	XXXXXX	Xenon Oscillations
78-97	7-29-78	F	0	A	4	- - - -	RC	XXXXXX	Xenon Oscillations
78-98	7-29-78	F	0	A	4	- - - -	RC	XXXXXX	Xenon Oscillations
78-99	7-30-78	F	0	A	4	- - - -	RC	XXXXXX	Xenon Oscillations
78-100	7-31-78	F	0	A	4	- - - -	HH	Filter	Clean Steam Generator Feed Pump Suction Strainers.

MAJOR PLANT MODIFICATIONS
REPORT MONTH June 1978

DOCKET NO: 50-272

UNIT NAME: Salem I

DATE: August 10, 1978

COMPLETED BY: L. K. Miller

TELEPHONE: (609) 365-7000 X507

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
ED-0058	Main Steam Piping	Modify Turning Vanes
ED-0143	1A-125VDC Battery	Change cells 2, 8, 36, 50, & 56
ED-0143	1B-125VDC Battery	Change cells 7 & 47
ED-0160	Fire Protection	Relocate C.R. Fire Detectors
ED-0169	Station Power	Install Wattmeter Transducer on SPT and Aux. Transformer
ED-0364	Service Water	Install Modified Service Water Pumps. 11, 15, & 16 completed.
1-EC-0378	Condenser	Install Experimental Tubes.
1-EC-0385	Diesel Generators	Install Modified Turbochargers

* DESIGN CHANGE REQUEST

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MAJOR PLANT MODIFICATIONS

REPORT MONTH July, 1978DOCKET NO.: 50-272UNIT NAME: Salem IDATE: August 10, 1978COMPLETED BY: L. K. MillerTELEPHONE: (609) 365-7000X-507

*DCR NO.	10CFR50.59. SAFETY EVALUATION
EC-0058	Functional operation is unchanged. The design change will improve reliability. The basis of the technical specifications are not affected.
ED-0143	Change of battery cell external connections in no way alters the capacity or performance data listed in the SAR or Technical Specs.
ED-0160	Not Safety Related and does not effect any safety related systems.
ED-0169	Safety related equipment will not be affected.
ED-0364	A revised seismic analysis has been done on the new aluminum bronze column pipe. The impeller modification produced a flatter head-capacity curve without affecting design flow point. System still performs original design function. Probability of accidents has not increased, nor has the occurrence of new accidents been made possible. The Tech. Spec bases are unaffected.
1-EC-0378	The modification of this system does not affect any presently performed safety analysis nor does it create any new hazards. The basis of the technical specifications are not affected.
1-EC-0385	This modification is being made to reduce the potential for further failure of the Emergency Diesel Generation turbo changers. This modification will not reduce the margin of safety in any Technical Specification; cause an accident or malfunction not previously analyzed in the FSAR, nor increase the probability of occurrence or consequence of such an accident.

SUMMARY OF SAFETY RELATED MAINTENANCE

UNIT NAME: Salem #1

DEPARTMENT Maintenance

DATE: August 10, 1978

REPORT MONTH July, 1978

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
OD-9215	Valve 12SW24	Valve leaks through.	Replaced diaphragm.
OD-9199	Valve 13RC16	Packing leak.	Tightened packing nut.
OD-9027	Valve 1CV2	Flange leak.	Replaced bonnet and seat gasket.
OP-0280	1C Diesel Gen. Turbo Charger.	Replace with rebuilt unit.	Replaced unit.
OD-8160	Valve 11MS171	Packing leak.	Repacked valve.
OD-9253	#11 Chiller	Freezes up.	Recharged unit.
OD-8814	1C Diesel	Diesel trips on over crank	Lubricated Governor Linkage.
OD-8143	15 Service Water Pump.	Does not pump.	Replaced Pump.
OD-9366	#11 and #12 Concentrate Pump	Heat trace circuit in alarm condition.	Replaced temp. sensing elements.
OD-8498	#11 Chiller Compressor	Compressor Short Cycles and trips on freeze protect.	Replaced Mechanical Seal
OD-8961	Valve 11SW24	Ruptured diaphragm	Replaced diaphragm.

SUMMARY OF SAFETY RELIABILITY MAINTENANCE

DEPARTMENT MaintenanceREPORT MONTH July, 1978

DOCKET NO.: 50-272

UNIT NAME: Salem #1

DATE: August 10, 1978

COMPLETED BY: L.K. Miller

TELEPHONE: 609-365-7000 X507

WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
OD-9363	#15 Service Water Pump Strainer.	Drive motor shaft gland leaks.	Repacked strainer.
OD-8698	1C Diesel	Pipe leak.	Replaced pipe cap.
OD-8475	1C Diesel	11 Start Air Compressor head leaks.	Tightened head bolts.
OD-7435	11 Chiller	Inadvertant trips.	Charged #11 Chiller.
OD-9048	#15 Service Wtr. Pump.	Pump leak.	Installed new pump.
MD-2805	1A Diesel	Jacket water heater failure alarm.	Recalibrated TD-7343
MD-2810	1C 4KV Vital Bus	Under voltage relay failed to meet Tech. Spec.	Recalibrated relay.
OD-9303	1A and B 125V. Batteries.	Voltage too high.	Lowered voltage to correct level
OD-9301	28 VDC Battery Charger.	Voltage too high	Calibrated voltage meter.
OP-0312	#15 Service Wtr. Pump.	Replace pump IAW DCR ED-0364-15.	Installed pump.
OD-8529	Valve 11NT22	Leaks at threaded connection.	Replaced gasket.

SUMMARY OF SAFETY RELATED MAINTENANCE

DEPARTMENT MaintenanceREPORT MONTH July, 1978

DOCKET NO.: 50-272

UNIT NAME: Salem #1DATE: August 10, 1978COMPLETED BY: L. K. MillerTELEPHONE: 609-365-7000 X507

WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
OP-0313	#16 Service Water Pump	Replace pump DCR ED-0364-16.	Replaced pump.
OD-8869	#16 Service Water Pump	Flex line on sealing water to stuffing box leaks.	Replaced flex line.
MD-2813	Valve 11SW24	Diaphragm leak	Replaced diaphragm.
MD-2677	1B Battery	Replace cells as per DCR ED-0143.	Replaced cells.
OP-0308	#11 Service Water Pump.	Replace Pump as per DCR ED-0364-11.	Replaced Pump
MD-2676	1A Battery	Change cells DCR ED-0143	Replaced cells.
MD-2814	#1 Unit Cast Handling Crane.	Repair cable take up reel.	Rewound reel.
MD-2168	#12 RHR Room Cooler	Pipe leaks.	Replaced pipe.

SUMMARY OF SAFETY RELATED MAINTENANCE

DEPARTMENT PerformanceREPORT MONTH July, 1978

DOCKET NO.: 50-272

UNIT NAME: Salem #1

DATE: August 10, 1978

COMPLETED BY: L. K. Miller

TELEPHONE: 609-365-7000 X507

WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
OD-9355	APD Unit	No motion in filter drive system.	Changed filter screen.
PD-4882	1PC948A/D	High output.	Adjusted setpoint.
PD-4888	#11 Loop AT/TARG	Out of spec.	Calibrated.
PD-4896	1LC106A,B, & C.	Out of spec.	Adjusted setpoints.
PD-4897	RMS 1R1B	Detector problem.	Replaced detector tubes.
OD-9291	#11 Accumulator	Level Chan. "A" erratic, Indic. "Sticks".	Replaced strain gauge on transmitter repaired and calibrated indicator.
OD-9260	#11 Accum. Chan.A	Increasing level while Chan. B is constant.	Replace strain gauge on transmitter repaired and calibrated indicator.
OD-9117	Accumulator	Calibrate 11 and 12 Accumulator Level.	Recalibrated 934A, 935A, 935B, 934B.
OD-9395	Pressurizer Level Channels.	Level Channels all indicate diff	Compound Bezel reading adjusted to original voltage.
OD-9201	RMS 1R-13-C	Inlet & outlet valves do not operate.	Calibrated.

SUMMARY OF SAFETY RELATED MAINTENANCE

DEPARTMENT PerformanceREPORT MONTH July, 1978DOCKET NO.: 50-272UNION NAME: Salem #1DATE: August 10, 1978COMPLETED BY: L. K. MillerTELEPHONE: 609-365-7000 X507

WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
OD-9068	#13 Aux. Feed Water Pump.	No start indication in control room.	Adjusted limit switch.
PD-4862	LFT-513	Verify operation.	Recalibrated.
PD-4524	11SGFP Turbine	Determine cause of oscillations.	Modified speed control circuit corrected by DCR FC0380.
RE-131	AFDM	Reset target ΔT TO -3.7%	Target ΔT Set to -3.7%.
OP-0359	RMS R11A	Reset alarm according to DCR EC-0390.	Reset R11A trip point.
PD-4853	Spare RMS Recorder	Recorder not printing.	Replaced clutch and cleaned contact.
OD-9202	11 Fan Coil Unit Filter.	Roughing filter indicator shows closed.	Opened air supply to solenoid.
OD-9259	EH Speed Reference Channel.	Channel failed.	Replaced comparator.
PD-4869	11C548C	Low Level 14 SG Channel III tripped intermittently.	Replaced capacitors.
PD-4878	11 Steam Gen. Level.	Intermittent Low-Low Level partial trip on computer.	Replaced Capacitors and adjusted output volts to 111VAC.
PD-4646	#13 Stop Valve.	Limit switch is not open.	Oiled roller and switch.
PD-4835	Status Panel	While tripping 1BS414 other lights on status lights were effected.	Replaced MS Capsule for #11 Loop Channel I RCP low flow.

SUMMARY OF SAFETY RELIEF MAINTENANCE

DOCKET NO.: 50-272

UNIT NAME: Salem #1

DEPARTMENT Performance

DATE: August 10, 1978

REPORT MONTH July, 1978

COMPLETED BY: L.K. Miller

TELEPHONE: 609-365-7000 X507

WORK ORDER NO.	EQUIPMENT	FAILURE DESCRIPTION	CORRECTIVE ACTION
PD-4876	1BS948B	Set point drifts.	Installed new module.
PD-4872	1BS457B	Out of Spec.	Reset set point.
PD-4870	Rx Prot. Console Alarm	Temp alarm did not clear after functional test.	Replaced power supply capacitor.
PD-4865	RMS 1RL3C	With detector disconnected, high alarm is on.	Replaced alarm module.
PD-4861	1FT512	Verify operation of transmitter.	Functionally checked.
PD-4858	11SG Chan. II Steam Flow.	Verify operation of steam flow Channel II.	Functionally checked.
PD-4857	11SG Stm. Flow Channel I.	Verify operation of steam flow Channel I.	Functionally checked.
OD-9240	Reactor Coolant TAVG-TREF Alarm.	Alarm comes up without 3° Deviation.	Replaced power supply capacitor.
OD-9154	14 Steam Generator Level.	Level deviation alarm for no apparent reason.	Replaced capacitors on 11M-505-E
OD-9077	Valve 11MS171	Valve leaking.	Stroked valve.
OD-8943	Pressurizer Press. Ch. I.	Alarmed at 330 PSI	Performed channel calibration.
OD-8930	PT403	Control Room Indicator reading incorrect.	Calibrated summator.

SALEM I OPERATING SUMMARY

JULY, 1978

- 7-1 At 0350 a load increase from 70% was started. The unit reached 90% at 0636. At 0941 an E-H System Failure Alarm was received, reactor power dropped 8% and the alarm cleared. The E-H System was placed in manual-control. A load reduction was started at 1050 in order to clean condensate pump suction strainers. At 1500 a load increase was started from 63% power. At 1625 with the E-H System in manual, the controller failed high. Reactor power jumped 12% and was finally stopped by the turbine governor valve position limiter. A load reduction was then initiated using the valve position limiter to lower the load. At 2233 the E-H System failed low and the governor valves went closed. The operator manually tripped the reactor.
- 7-2 The reactor was taken critical at 0057 and the unit was synchronized at 1344. Reactor power was then increased and reached 64% at 2400.
- 7-3 The Load Dispatcher placed emergency minimum loading into effect at the request of the PJM interconnection. The unit output was limited to 50%. At 0545 the emergency minimum loading was cancelled and the unit began a load increase. Power was increased to 70% and held to clean a condensate pump suction strainer. At 1036 a load decrease was started to reduce power to 50% to clean a feed pump suction strainer and repair a feed pump recirculation valve. Power was held at 50% the rest of the day.
- 7-4 At 0825 the load dispatcher requested a load increase from 50% to 70%. The unit reached 70% at 1535 and remained here the rest of the day.

- 7-5 At 0212 a load increase was started to 100% power. At 1435 power was decreased to 90% to clean a heater drain pump suction strainer. At 1700 a load increase was started to 100%. At 2130 a load decrease to 70% was started to clean a condensate pump suction strainer.
- 7-6 At 0407 a load increase was started to 100%. Stabilized reactor power at approximately 99% for the remainder of the day.
- 7-7 Reactor power stabilized at 99%. At 1125 load was reduced to clean heater drain pump strainer. At approximately 1300 reactor power at 90%. Strainer cleaned and load was being increased to 100% at 1320. Reactor power was at 100% at 1630. Load was reduced from 99% to 90% to clean heater drain pump strainer at 2045. Load was at 90% at 2230. At 2245 load was increasing to 100% after HDP strainer was cleaned.
- 7-8 Load was increasing to 100% from 94%. At 0140 reactor power is 99.5%. At 0245 reactor power was decreasing from 99.5% to 90% to clean heater drain pump strainers. Reactor power was at 90% by 0415. Heater drain pump was returned to service and increased load to 100% at 0425. Reactor power was 100% at 0613. Load was decreased from 99.5% to 90% for heater drain pump maintenance at 0915. Reactor power was 90% at 1115. Heater drain pump is put in service and reactor power is restored to 100% at 1130. At 1400 reactor power is 100%. At 1500 load is reduced from 99% to 70% to clean condensate pump strainer. Reactor power is at 70% at 1900. Strainer is cleaned load is increasing to 100% at 2145.
- 7-9 Load is at 74% and increasing to 100%. Load is at 99% at 0400. At 0525 load was decreased from 99% to 90% for maintenance on heater drain pumps. Load was at 99% at 0605. HDP was put in service and load was increased from 90% to 100% at 0630.

- 7-9 Load increased to 94% at 0655. Load was decreased to 90% from 94% due to heater drain pump cleaning at 0750. Load was 90% at 0850. Heater drain pump is in service and load was increased from 90% to 100% at 0945. Load is 99% at 1200. Load was reduced at 1535 for heater drain pump maintenance from 99% to 90%. Load was 90% at 1657. Load remained at 90-92% for the remainder of the day.
- 7-10 Reactor power was at 91% due to cleaning of heater drain pump maintenance. Load was increased to 100% at 1745 from 91%. Load was being held at 96% at 1830 to watch feed pump strainer D/P. At 2200 commenced decreasing load to 50% from 96% to clean feed pump strainer.
- 7-11 Load is at 52% at 0400. Strainers were cleaned and load is increasing to 100% at 0540. Calorimetric taken at 1110 at 87% power. Reactor power at 99.5% at 1600. Decreasing load from 99% to 90% at 2300 for heater drain pump strainer cleaning.
- 7-12 Coming down in load to clean heater drain pump strainers. Power stabilized at 90% at 0030. The heater drain pump was returned to service and load was increased from 91% to 100% at 0350. Load was at 100% at 0545. Load was reduced from 99% to 90% at 2140 for heater drain pump maintenance. Load stabilized at 89% at 2350.
- 7-13 The heater drain pump was returned to service and load was increased from 92% to 100% at 0625. Load was reduced at 2145 from 96% to 90% to clean condensate pump strainers.
- 7-14 Load at 0300 was at 70% to clean condensate pump strainers. Strainers were cleaned and power was increasing at 0600 from 71% to 100%. Power was greater than 97% between 1100 and 2130. Power was greater than 97% between 1100 and 2130. Power was decreased for #12 heater drain pump strainer cleaning. At 2255 power was at 90.5% and returning to full power.

- 7-15 Power was increasing to 100% from 95%. Power was greater than 95% until 2210. Power was decreased to 50% to clean feed pump strainers.
- 7-16 Power was at 50% at 0530. Strainer was cleaned and power was increased to 100% at 0600. Power was greater than 95% from 1320 through the remainder of the day.
- 7-17 Plant power was being maintained greater than 95%. Reducing power from 99.5% at 2210 for maintenance on #12 Heater Drain Pump. Power was reduced to 93% at 2335.
- 7-18 Power was increased from 93% at 0001 to 100%. Reactor power was greater than 97% from 0110 to 0657. A reactor trip occurred at 0657 due to under frequency on group buses, causing all four (4) RCP's to trip. Prior to the trip the power level was 98.2%. Reactor was shutdown for the remainder of the day.
- 7-19 The plant was shutdown. Commencing of cooldown started at 0600 for minor plant maintenance activities. The reactor coolant system was 390°F. at 1100 for repairing of leaking check valve 1CV78. Commencing heatup of the plant at 1500.
- 7-20 Preparations being made for plant startup.
- 7-21 Reactor was critical at 0430. Boron was 974 ppm and rods were at 156 steps on Bank D. Turbine was synchronized at 1045. A power increase was started and the unit reached 93% power at 2400.
- 7-22 The unit reached 100% power at 0100. At 0300 a load reduction to 70% was started in order to clean condensate pump suction strainers. The unit returned to 100% load at 1418.

- 7-23 The unit operated at full load all day.
- 7-24 The unit operated at full load until 2148 when a load reduction to 70% was initiated to clean condensate pump suction strainer.
- 7-25 The unit returned to full load at 1100. At 1700 a load reduction to 50% was initiated to clean feed pump suction strainers.
- 7-26 The unit returned to full load at 0830 and operated here the remainder of the day.
- 7-27 At 0700 load was reduced to 94% when it was observed that the margin to trip for the overtemperature ΔT trip was reduced. It was later determined that a loose wire in a controller was the cause of the decreased margin. The unit operated at full load until 2138 when a load reduction to 70% was initiated to clean condensate pump suction strainers.
- 7-28 The unit returned to full power at 0748 and remained at full load until 7-31.
- 7-31 At 2108 a load reduction to 50% was started in order to clean feed pump suction strainers.

REFUELING INFORMATION

DOCKET NO.: 50-272UNIT: Salem No. 1

DATE: _____

COMPLETED BY: L.K. MillerTELEPHONE: 609-365-7000EXT. 507MONTH: July, 1978

1. Refueling information has changed from last month:

YES _____ NO X2. Scheduled date of next refueling: April 1, 19793. Scheduled date for restart following refueling: May 29, 19794. A. Will Technical Specification changes or other license
amendments be required? YES _____ NO _____NOT DETERMINED TO-DATE July, 1978B. Has the reload fuel design been reviewed by the Station Operating
Review Committee? YES _____ NO XIf no, when is it scheduled? January, 1979

5. Scheduled date(s) for submitting proposed licensing action:

February, 1979 if required.6. Important licensing considerations associated with refueling:
None

7. Number of Fuel Assemblies:

A. In-Core 193B. In Spent Fuel Storage 08. Present licensed spent fuel storage capacity: 264Future spent fuel storage capacity: 1,1709. Date of last refueling that can be discharged to the spent fuel
pool assuming the present licensed capacity: April, 1982