

ATTACHMENT 2C

MPS3 EAL TECHNICAL BASES DOCUMENT (Marked-up)

**Dominion Energy Nuclear Connecticut, Inc. (DENC)
Millstone Power Station Unit 3 and ISFSI**

**Emergency Action Level Technical Bases Document
Millstone Power Station – Unit 3**

(Marked-up)

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1.0 INTRODUCTION

This document provides an explanation and rationale for each Emergency Action Level (EAL) included in the NEI 99-01, Revision 6, EAL Upgrade Project for Millstone Power Station Unit 3 (MPS3). It should be used to facilitate review of the MPS3 EALs and provide historical documentation for future reference. Decision-makers responsible for implementation of MP-26-EPI-FAP06, Classification and PARs, may use this document as a technical reference in support of EAL interpretation. This information may assist the Director of Station Emergency Operations (DSEO)/Assistant Director Technical Support (ADTS) in making classifications, particularly those involving judgment or multiple events. The basis information may also be useful in training and for explaining event classifications to off-site officials.

The expectation is that emergency classifications are to be made as soon as conditions are present and recognizable for the classification, but within 15 minutes or less in all cases of conditions present. Use of this document for assistance is not intended to delay the emergency classification.

Since the information in a basis document can affect emergency classification decision-making (e.g., the DSEO/ADTS refers to it during an event), the NRC staff expects that changes to the basis document will be evaluated in accordance with the provisions of 10 CFR 50.54(q). For Dominion Energy sites, a 10 CFR 50.54(q)(3) screening/evaluation will be performed to evaluate changes to this document.

Dominion Energy fleet procedure CM-AA-400, "10 CFR 50.59 and 10 CFR 72.48 – Changes, Tests and Experiments" provides a method to determine the impacts to licensing basis documents when changes are proposed to procedures, including changes to Abnormal Operating Procedures (AOPs) and Emergency Operating Procedures (EOPs). The 50.59/72.48 applicability review specifically requires that the effect of a proposed procedure change on the Emergency Plan (and associated EALs) be reviewed/assessed. When impacts to the Emergency Plan are identified, a separate review in accordance to 10 CFR 50.54(q) will be performed to determine the acceptability of the proposed procedure change.

2.0 DISCUSSION

2.1 Background

EALs are the plant-specific indications, conditions or instrument readings that are utilized to classify emergency conditions defined in the Millstone Power Station Emergency Plan.

In 1992, the NRC endorsed NUMARC/NESP-007, "Methodology for Development of Emergency Action Levels" as an alternative guidance to the original Standard Review Plan and NUREG-0654 EAL schemes.

NEI 99-01 (NUMARC/NESP-007), Revisions 4 and 5, were subsequently issued for industry implementation. Enhancements over earlier revisions included:

- Consolidating the system malfunction initiating conditions and example emergency action levels which address conditions that may be postulated to occur during plant shutdown conditions.

- Initiating conditions and example emergency action levels that fully address conditions that may be postulated to occur at permanently Defueled Stations and Independent Spent Fuel Storage Installations (ISFSIs).
- Simplifying the fission product barrier EAL threshold for a Site Area Emergency.

Subsequently, Revision 6 of NEI 99-01 has been issued which incorporates resolutions to numerous implementation issues including the NRC EAL Frequently Asked Questions (FAQs). Using NEI 99-01 Revision 6, "Methodology for the Development of Emergency Action Levels for Non-Passive Reactors," November 2012 (ref. 4.1.1), MPS3 conducted an EAL implementation upgrade project that produced the EALs discussed herein.

2.2 Fission Product Barriers

Fission product barrier thresholds represent threats to the defense in depth design concept that precludes the release of radioactive fission products to the environment. This concept relies on multiple physical barriers, any one of which, if maintained intact, precludes the release of significant amounts of radioactive fission products to the environment.

Many of the EALs derived from the NEI methodology are fission product barrier threshold based. That is, the conditions that define the EALs are based upon thresholds that represent the loss or potential loss of one or more of the three fission product barriers. "Loss" and "Potential Loss" signify the relative damage and threat of damage to the barrier. A "Loss" threshold means the barrier no longer assures containment of radioactive materials. A "Potential Loss" threshold implies a greater probability of barrier loss and reduced certainty of maintaining the barrier.

The primary fission product barriers are:

- A. Fuel Clad Barrier (FC): The Fuel Clad Barrier consists of the cladding material that contains the fuel pellets.
- B. Reactor Coolant System Barrier (RCS): The RCS Barrier includes the RCS primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary isolation valves.
- C. Containment Barrier (CTMT): The Containment Barrier includes the containment building and connections up to and including the outermost containment isolation valves. This barrier also includes the main steam, feedwater, and blowdown line extensions outside the containment building up to and including the outermost secondary side isolation valve. Containment Barrier thresholds are used as criteria for escalation of the Emergency Classification Level (ECL) from an Alert to a Site Area Emergency or a General Emergency.

2.3 Fission Product Barrier Classification Criteria

The following criteria are the bases for event classification related to fission product barrier loss or potential loss:

Alert:

Any loss or any potential loss of either Fuel Clad or RCS Barrier

Site Area Emergency:

Loss or potential loss of any two barriers

General Emergency:

Loss of any two barriers and loss or potential loss of the third barrier

2.4 EAL Organization

The MPS3 EAL scheme includes the following features:

- Division of the EAL set into three broad groups:
 - EALs applicable under any plant operational modes – This group would be reviewed by the EAL-user any time emergency classification is considered.
 - EALs applicable only under hot operational modes – This group would only be reviewed by the EAL-user when the plant is in Hot Shutdown, Hot Standby, Startup, or Power Operation mode.
 - EALs applicable only under cold operating modes – This group would only be reviewed by the EAL-user when the plant is in Cold Shutdown, Refueling or Defueled mode.

The purpose of the groups is to avoid review of hot condition EALs when the plant is in a cold condition and avoid review of cold condition EALs when the plant is in a hot condition. This approach significantly minimizes the total number of EALs that must be reviewed by the EAL-user for a given plant condition, reduces EAL-user reading burden and, thereby, speeds identification of the EAL that applies to the emergency.

- Within each group, assignment of EALs to categories and subcategories:

Category and subcategory titles are selected to represent conditions that are operationally significant to the EAL-user. The MPS3 EAL categories are aligned to and represent the NEI 99-01 "Recognition Categories." Subcategories are used in the MPS3 scheme as necessary to further divide the EALs of a category into logical sets of possible emergency classification thresholds. The MPS3 EAL categories and subcategories are listed below.

The EALs are pre-determined, site-specific, observable thresholds for determining whether an Initiating Condition (IC) has occurred and that an EAL threshold was met or exceeded. Thus failure to evaluate the IC and EAL together could result in an incorrect declaration.

The primary tool for determining the emergency classification level is the EAL Classification Matrix. The user of the EAL Classification Matrix may (but is not required to) consult the EAL technical bases in order to obtain additional information concerning the EALs under classification consideration. The user should consult Section 3.0 and Attachment 1 of this document for such information.

EAL Groups, Categories and Subcategories

EAL Group/Category	EAL Subcategory
<u>Any Operating Mode:</u>	
R – Abnormal Rad Levels / Rad Effluent	1 – Radiological Effluent 2 – Irradiated Fuel Event 3 – Area Radiation Levels
H – Hazards and Other Conditions Affecting Plant Safety	1 – Security 2 – Seismic Event 3 – Natural or Technological Hazard 4 – Fire 5 – Hazardous Gas 6 – Control Room Evacuation 7 – DSEO/ADTS Judgment
E – Independent Spent Fuel Storage Installation (ISFSI)	1 – Confinement Boundary
<u>Hot Conditions:</u>	
M – System Malfunction	1 – Loss of Emergency AC Power 2 – Loss of Vital DC Power 3 – Loss of Control Room Indications 4 – RCS Activity 5 – RCS Leakage 6 – RPS Failure 7 – Loss of Communications 8 – Containment Failure 9 – Hazardous Event Affecting Safety Systems
F – Fission Product Barrier Degradation	None
<u>Cold Conditions:</u>	
C – Cold Shutdown / Refueling System Malfunction	1 – RCS Level 2 – Loss of Emergency AC Power 3 – RCS Temperature 4 – Loss of Vital DC Power 5 – Loss of Communications 6 – Hazardous Event Affecting Safety Systems

2.5 Technical Bases Information

EAL technical bases are provided in Attachment 1 for each EAL according to EAL group (Any, Hot, Cold), EAL category (R, C, E, F, H and M) and EAL subcategory. A summary explanation of each category and subcategory is given at the beginning of the technical bases discussions of the EALs included in the category. For each EAL, the following information is provided:

Category Letter & Title

Subcategory Number & Title

Initiating Condition (IC)

Site-specific description of the generic IC given in NEI 99-01 Rev. 6.

EAL Identifier (enclosed in rectangle)

Each EAL is assigned a unique identifier to support accurate communication of the emergency classification to onsite and offsite personnel. Four characters define each EAL identifier:

1. First character (letter): Corresponds to the EAL category as described above (R, C, E, F, H or M)
2. Second character (letter): The emergency classification (G, S, A or U)
 - G = General Emergency
 - S = Site Area Emergency
 - A = Alert
 - U = Unusual Event
3. Third character (number): Subcategory number within the given category. Subcategories are sequentially numbered beginning with the number one (1). If a category does not have a subcategory, this character is assigned the number one (1).
4. Fourth character (number): The numerical sequence of the EAL within the EAL subcategory. If the subcategory has only one EAL, it is given the number one (1).

Classification (enclosed in rectangle):

General Emergency (G), Site Area Emergency (S), Alert (A) or Unusual Event (U).

EAL (enclosed in rectangle)

Exact wording of the EAL as it appears in the EAL Classification Matrix.

Notes (as applicable)

Mode Applicability

One or more of the following plant operating conditions comprise the mode to which each EAL is applicable: 1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown, 5 - Cold Shutdown, 6 - Refueling, DEF - Defueled, or All. (See Section 2.6 for operating mode definitions)

Definition(s):

If the EAL wording contains a defined term, the definition of the term is included in this section. These definitions can also be found in Section 5.1.

Basis:

An EAL basis section that provides MPS3-relevant information concerning the EAL as well as a description of the rationale for the EAL as provided in NEI 99-01 Rev. 6.

Reference(s):

Source documentation from which the EAL is derived

2.6 Operational Mode Applicability

MODE	K_{eff}	THERMAL POWER *	T_{AVG}
1. Power Operation	≥ 0.99	$> 5\%$	$\geq 350^{\circ}\text{F}$
2. Startup	≥ 0.99	$\leq 5\%$	$\geq 350^{\circ}\text{F}$
3. Hot Standby	< 0.99	0	$\geq 350^{\circ}\text{F}$
4. Hot Shutdown	< 0.99	0	$> 200^{\circ}\text{F} \text{ \& } < 350^{\circ}\text{F}$
5. Cold Shutdown	< 0.98	0	$\leq 200^{\circ}\text{F}$
6. Refueling **	≤ 0.95	0	$\leq 140^{\circ}\text{F}$
Defueled	NA	NA	NA – no fuel in reactor vessel

* Excluding Decay Heat

** Fuel in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed

The plant operating mode that exists at the time that the event occurs (prior to any protective system or operator action being initiated in response to the condition) should be compared to the mode applicability of the EALs. If a lower or higher plant operating mode is reached before the emergency classification is made, the declaration shall be based on the mode that existed at the time the event occurred.

3.0 GUIDANCE ON MAKING EMERGENCY CLASSIFICATIONS

3.1 General Considerations

When making an emergency classification, the DSEO/ADTS must consider all information having a bearing on the proper assessment of an Initiating Condition (IC). This includes the EAL plus any associated Operational Mode Applicability, Notes, and the informing basis information. In the Category F matrices, EALs are based on loss or potential loss of Fission Product Barrier thresholds.

3.1.1 Classification Timeliness

NRC regulations require the licensee to establish and maintain the capability to assess, classify, and declare an emergency condition within 15 minutes after the availability of indications to plant operators that an emergency action level has been exceeded and to promptly declare the emergency condition as soon as possible following identification of the appropriate emergency classification level. The NRC staff has provided guidance on implementing this requirement in NSIR/DPR-ISG-01, "Interim Staff Guidance, Emergency Planning for Nuclear Power Plants" (ref. 4.1.8).

3.1.2 Valid Indications

All emergency classification assessments shall be based upon valid indications, reports or conditions. A valid indication, report, or condition, is one that has been verified through appropriate means such that there is no doubt regarding the indicator's operability, the condition's existence, or the report's accuracy.

An indication, report, or condition is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

3.1.3 Imminent Conditions

For ICs and EALs that have a stipulated time duration (e.g., 15 minutes, 30 minutes, etc.), the DSEO/ADTS should not wait until the applicable time has elapsed, but should declare the event as soon as it is determined that the condition has exceeded, or will likely exceed, the applicable time. If an ongoing radiological release is detected and the release start time is unknown, it should be assumed that the release duration specified in the IC/EAL has been exceeded, absent data to the contrary.

3.1.4 Planned vs. Unplanned Events

A planned work activity that results in an expected event or condition which meets or exceeds an EAL does not warrant an emergency declaration provided that: 1) the activity proceeds as planned, and 2) the plant remains within the limits imposed by the operating license. Such activities include planned work to test, manipulate, repair, maintain or modify a system or component. In these cases, the controls associated with the planning, preparation and execution of the work will ensure that compliance is maintained with all aspects of the operating license provided that the activity proceeds and concludes as expected. Events or conditions of this type may be subject to the reporting requirements of 10CFR 50.72 (ref. 4.1.4).

3.1.5 Classification Based on Analysis

The assessment of some EALs is based on the results of analyses that are necessary to ascertain whether a specific EAL threshold has been exceeded (e.g., dose assessments, chemistry sampling, RCS leak rate calculation, etc.). For these EALs, the wording of the EAL or associated basis discussion will identify the necessary analysis. In these cases, the 15-minute declaration period starts with the availability of the analysis results that show the threshold to be exceeded (i.e., this is the time that the EAL information is first available). The NRC expects licensees to establish the capability to initiate and complete EAL-related analyses within a reasonable period of time (e.g., maintain the necessary expertise on-shift).

3.1.6 DSEO/ADTS Judgment

While the EALs have been developed to address a full spectrum of possible events and conditions which may warrant emergency classification, a provision for classification based on operator/management experience and judgment is still necessary. The NEI 99-01 EAL scheme provides the DSEO/ADTS with the ability to classify events and conditions based upon judgment using EALs that are consistent with the Emergency Classification Level (ECL) definitions (refer to Category H). The DSEO/ADTS will need to determine if the effects or consequences of the event or condition reasonably meet or exceed a particular ECL definition. A similar provision is incorporated in the Fission Product Barrier Tables; judgment may be used to determine the status of a fission product barrier.

3.2 Classification Methodology

To make an emergency classification, the user will compare an event or condition (i.e., the relevant plant indications and reports) to an EAL(s) and determine if the EAL has been met or exceeded. The evaluation of an EAL must be consistent with the related Operating Mode Applicability and Notes. If an EAL has been met or exceeded, the associated IC is likewise met, the emergency classification process "clock" starts, and the ECL must be declared in accordance with plant procedures no later than 15 minutes after the process "clock" started.

When assessing an EAL that specifies a time duration for the potentially classifiable condition, the "clock" for the EAL time duration runs concurrently with the emergency classification process "clock." For a full discussion of this timing requirement, refer to NSIR/DPR-ISG-01 (ref. 4.1.8).

3.2.1 Classification of Multiple Events and Conditions

When multiple emergency events or conditions are present, the user will identify all met or exceeded EALs. The highest applicable ECL identified during this review is declared. For example:

- If an Alert EAL and a Site Area Emergency EAL are met, whether at one unit or at two units, a Site Area Emergency should be declared.

There is no "additive" effect from multiple EALs meeting the same ECL. For example:

- If two Alert EALs are met, whether at one unit or at two units, an Alert should be declared.

Related guidance concerning classification of rapidly escalating events or conditions is provided in Regulatory Issue Summary (RIS) 2007-02, *Clarification of NRC Guidance for Emergency Notifications During Quickly Changing Events* (ref. 4.1.2).

3.2.2 Consideration of Mode Changes During Classification

The mode in effect at the time that an event or condition occurred, and prior to any plant or operator response, is the mode that determines whether or not an IC is applicable. If an event or condition occurs, and results in a mode change before the emergency is declared, the emergency classification level is still based on the mode that existed at the time that the event or condition was initiated (and not when it was declared). Once a different mode is reached, any new event or condition, not related to the original event or condition, requiring emergency classification should be evaluated against the ICs and EALs applicable to the operating mode at the time of the new event or condition.

For events that occur in Cold Shutdown or Refueling, escalation is via EALs that are applicable in the Cold Shutdown or Refueling modes, even if Hot Shutdown (or a higher mode) is entered during the subsequent plant response. In particular, the fission product barrier EALs are applicable only to events that initiate in the Hot Shutdown mode or higher.

3.2.3 Classification of Imminent Conditions

Although EALs provide specific thresholds, the DSEO/ADTS must remain alert to events or conditions that could lead to meeting or exceeding an EAL within a relatively short period of time (i.e., a change in the ECL is IMMINENT). If, in the judgment of the DSEO/ADTS, meeting an EAL is IMMINENT, the emergency classification should be made as if the EAL has been met. While applicable to all emergency classification levels, this approach is particularly important at the higher emergency classification levels since it provides additional time for implementation of protective measures.

3.2.4 Emergency Classification Level Upgrading and Downgrading

An ECL may be downgraded when the event or condition that meets the highest IC and EAL no longer exists, and other site-specific downgrading requirements are met. If downgrading the ECL is deemed appropriate, the new ECL would then be based on a lower applicable IC(s) and EAL(s). The ECL may also simply be terminated.

As noted above, guidance concerning classification of rapidly escalating events or conditions is provided in RIS 2007-02 (ref. 4.1.2).

3.2.5 Classification of Short-Lived Events

Event-based ICs and EALs define a variety of specific occurrences that have potential or actual safety significance. By their nature, some of these events may be short-lived and, thus, over before the emergency classification assessment can be completed. If an event occurs that meets or exceeds an EAL, the associated ECL must be declared regardless of its continued presence at the time of declaration. Examples of such events include an earthquake or a failure of the reactor protection system to automatically scram the reactor followed by a successful manual scram.

3.2.6 Classification of Transient Conditions

Many of the ICs and/or EALs employ time-based criteria. These criteria will require that the IC/EAL conditions be present for a defined period of time before an emergency declaration is

warranted. In cases where no time-based criterion is specified, it is recognized that some transient conditions may cause an EAL to be met for a brief period of time (e.g., a few seconds to a few minutes). The following guidance should be applied to the classification of these conditions.

EAL momentarily met during expected plant response - In instances in which an EAL is briefly met during an expected (normal) plant response, an emergency declaration is not warranted provided that associated systems and components are operating as expected, and operator actions are performed in accordance with procedures.

EAL momentarily met but the condition is corrected prior to an emergency declaration – If an operator takes prompt manual action to address a condition, and the action is successful in correcting the condition prior to the emergency declaration, then the applicable EAL is not considered met and the associated emergency declaration is not required. For illustrative purposes, consider the following example:

An ATWS occurs and the high pressure ECCS systems fail to automatically start. The plant enters an inadequate core cooling condition (a potential loss of both the Fuel Clad and RCS Barriers). If an operator manually starts a high pressure ECCS system in accordance with an EOP step and clears the inadequate core cooling condition prior to an emergency declaration, then the classification should be based on the ATWS only.

It is important to stress that the 15-minute emergency classification assessment period (process clock) is not a “grace period” during which a classification may be delayed to allow the performance of a corrective action that would obviate the need to classify the event. Emergency classification assessments must be deliberate and timely, with no undue delays. The provision discussed above addresses only those rapidly evolving situations when an operator is able to take a successful corrective action prior to the DSEO/ADTS completing the review and steps necessary to make the emergency declaration. This provision is included to ensure that any public protective actions resulting from the emergency classification are truly warranted by the plant conditions.

3.2.7 After-the-Fact Discovery of an Emergency Event or Condition

In some cases, an EAL may be met but the emergency classification was not made at the time of the event or condition. This situation can occur when personnel discover that an event or condition existed which met an EAL, but no emergency was declared, and the event or condition no longer exists at the time of discovery. This may be due to the event or condition not being recognized at the time or an error that was made in the emergency classification process.

In these cases, no emergency declaration is warranted; however, the guidance contained in NUREG-1022 (ref. 4.1.3) is applicable. Specifically, the event should be reported to the NRC in accordance with 10CFR 50.72 (ref. 4.1.4) within one hour of the discovery of the undeclared event or condition. The licensee should also notify appropriate State and local agencies in accordance with the agreed upon arrangements.

3.2.8 Retraction of an Emergency Declaration

Guidance on the retraction of an emergency declaration reported to the NRC is discussed in NUREG-1022 (ref. 4.1.3).

4.0 REFERENCES

4.1 Developmental

- 4.1.1 NEI 99-01, Revision 6, "Methodology for the Development of Emergency Action Levels for Non-Passive Reactors," (ADAMS Accession No. ML12326A805)
- 4.1.2 RIS 2007-02, "Clarification of NRC Guidance for Emergency Notifications During Quickly Changing Events", February 2, 2007.
- 4.1.3 NUREG-1022, "Event Reporting Guidelines: 10CFR50.72 and 50.73"
- 4.1.4 10 CFR 50.72, "Immediate Notification Requirements for Operating Nuclear Power Reactors"
- 4.1.5 10 CFR 50.73, "License Event Report System"
- 4.1.6 MPS3 Technical Specifications Table 1.2, "Operational Modes"
- 4.1.7 MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
- 4.1.8 NSIR/DPR-ISG-01, "Interim Staff Guidance, Emergency Planning for Nuclear Power Plants"
- 4.1.9 MPS Emergency Plan
- 4.1.10 NUH-003, "UFSAR for the Horizontal Modular Storage System for Irradiated Fuel"
- 4.1.11 MPS2 ETE-NAF-2010-0004, "MPS Unit 2 ISFSI 10 CFR72.212 Evaluation Report"
- 4.1.12 MPS3 FSAR Chapter 9.1, "Fuel Storage and Handling"
- 4.1.13 SP 3613F.3-001, "Containment Boundary During Movement of Fuel Within The Containment Building"

4.2 Implementing

- 4.2.1 MP-26-EPI-FAP06, "Classification and PARs"
- 4.2.2 NEI 99-01 Rev. 6 to MPS3 EAL Comparison Matrix
- 4.2.3 MPS3 EAL Matrix

5.0 DEFINITIONS, ACRONYMS & ABBREVIATIONS

5.1 Definitions (ref. 4.1.1 except as noted)

Selected terms used in Initiating Condition, EAL statements and EAL bases are set in all capital letters (e.g., ALL CAPS). These are defined terms that have specific meanings as used in this document. The definitions of these terms are provided below.

ALERT

Events are in progress, or have occurred, which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. **Any** releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.

CONFINEMENT BOUNDARY

The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielding Canister (DSC) (ref. 4.1.10).

CONTAINMENT CLOSURE

The procedurally defined conditions or actions taken to secure containment (~~Primary or Secondary~~) and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist (ref. 4.1.13):

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

EMERGENCY ACTION LEVEL (EAL)

A pre-determined, site-specific, observable threshold for an INITIATING CONDITION that, when met or exceeded, places the plant in a given emergency classification level.

EMERGENCY CLASSIFICATION LEVEL (ECL)

One of a set of names or titles established by the US Nuclear Regulatory Commission (NRC) for grouping off-normal events or conditions according to (1) potential or actual effects or consequences, and (2) resulting onsite and offsite response actions. The emergency classification levels, in ascending order of severity, are:

- ~~Notification of Unusual Event (NOUE)~~
- Alert
- Site Area Emergency (SAE)
- General Emergency (GE)

EXPLOSION

A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding, arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FAULTED

The term applied to a steam generator that has a steam leak on the secondary side of sufficient size to cause an uncontrolled drop in steam generator pressure or the steam generator to become completely depressurized.

FIRE

Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FISSION PRODUCT BARRIER THRESHOLD

A pre-determined, site-specific, observable threshold indicating the loss or potential loss of a fission product barrier.

FLOODING

A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

GENERAL EMERGENCY

Events are in progress or have occurred which involve actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity or HOSTILE ACTION that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA PAG exposure levels offsite for more than the immediate site area.

HOSTAGE

A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION

An act toward a ~~NPP~~ MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on the ~~NPP~~ MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

HOSTILE FORCE

One or more individuals who are engaged in a determined assault, overtly or by stealth and deception, equipped with suitable weapons capable of killing, maiming, or causing destruction.

IMMINENT

The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

IMPEDE(D)

Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

INDEPENDENT SPENT FUEL STORAGE INSTALLATION (ISFSI)

A complex that is designed and constructed for the interim storage of spent nuclear fuel and other radioactive materials associated with spent fuel storage.

INITIATING CONDITION (IC)

An event or condition that aligns with the definition of one of the four emergency classification levels by virtue of the potential or actual effects or consequences.

~~Normal Levels~~

~~As applied to radiological IC/EALs, the highest reading in the past twenty-four hours excluding the current peak value.~~

OWNER CONTROLLED AREA (OCA)

The area within the SITE BOUNDARY including the PROTECTED AREA (ref. 4.1.9).

PROJECTILE

An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA

The area within the Millstone Power Station security fence (ref. 4.1.9).

REFUELING PATHWAY

Refueling cavity, fuel transfer canal, and spent fuel pool (SFP), but **not** including the reactor vessel, comprise the refueling pathway (ref. 4.1.12).

RUPTURED

The condition of a steam generator in which primary-to-secondary leakage is of sufficient magnitude to require a safety injection.

SAFETY SYSTEM

A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;

- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

SECURITY CONDITION

Any security event as listed in the approved security contingency plan that constitutes a threat/compromise to site security, threat/risk to site personnel, or a potential degradation to the level of safety of the plant. A Security Condition does **not** involve a HOSTILE ACTION.

SITE AREA EMERGENCY

Events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage or malicious acts; (1) toward site personnel or equipment that could lead to the likely failure of or; (2) that prevent effective access to equipment needed for the protection of the public. **Any** releases are **not** expected to result in exposure levels which exceed EPA PAG exposure levels beyond the SITE BOUNDARY.

SITE BOUNDARY

That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA (ref. 4.1.7)

UNISOLABLE

An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED

A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

UNUSUAL EVENT

Events are in progress or have occurred which indicate a potential degradation in the level of safety of the plant or indicate a security threat to facility protection has been initiated. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs.

VALID

An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

VISIBLE DAMAGE

Damage to a SAFETY SYSTEM train that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected SAFETY SYSTEM train.

~~Damage to a component or structure that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected component or structure.~~

5.2 Abbreviations/Acronyms

°F	Degrees Fahrenheit
°	Degrees
μCi	Micro Curie
AC	Alternating Current
ADTS	Assistant Director Technical Support
AFW	Auxiliary Feedwater
AOP	Abnormal Operating Procedure
ARM	Area Radiation Monitor
ATWS	Anticipated Transient Without Scram
CDA	Containment Depressurization Actuation
CDE	Committed Dose Equivalent
CET	Core Exit Thermocouple
CFR	Code of Federal Regulations
CIAS	Containment Isolation Actuation Signal
CPM	Counts Per Minute
CSFST	Critical Safety Function Status Check
CTMT	Containment
DBA	Design Basis Accident
DEF	Defueled
DC	Direct Current
D/G	Diesel Generator
DSEO	Director of Station Emergency Operations
EAL	Emergency Action Level
ECCS	Emergency Core Cooling System
ECL	Emergency Classification Level
EDG	Emergency Diesel Generator
EOF	Emergency Operations Facility
EOP	Emergency Operating Procedure
EPA	Environmental Protection Agency
EPI	Emergency Plan Implementing
FAA	Federal Aviation Administration
FBI	Federal Bureau of Investigation
FC	Fuel Clad Barrier
FEMA	Federal Emergency Management Agency
FSAR	Final Safety Analysis Report
GE	General Emergency
GPM	Gallons Per Minute

HR.....	Heat Removal
Hr.	Hour
HSM	Horizontal Storage Module
IC	Initiating Condition
ICC.....	Inadequate Core Cooling
ISFSI	Independent Spent Fuel Storage Installation
K_{eff}	Effective Neutron Multiplication Factor
LCO.....	Limiting Condition of Operation
LER	Licensee Event Report
LOCA	Loss of Coolant Accident
LPSI	Low Pressure Safety Injection
LRW	Liquid Radwaste
LWR	Light Water Reactor
MCB	Main Control Board
Min.	Minute
MPH	Miles Per Hour
MPS	Millstone Power Station
mR, mRem, mrem, mREM	milli-Roentgen Equivalent Man
MW.....	Megawatt
NEI	Nuclear Energy Institute
NEIC	National Earthquake Information Center
NPP	Nuclear Power Plant
NRC	Nuclear Regulatory Commission
NSSS	Nuclear Steam Supply System
NORAD	North American Aerospace Defense Command
OBE	Operating Basis Earthquake
OCA	Owner Controlled Area
PAG	Protective Action Guideline
PORV	Power Operated Relief Valve
PSIG	Pounds per Square Inch Gauge
PTS	Pressurized Thermal Shock
QSS	Quench Spray System
R	Roentgen
RCS	Reactor Coolant System
Rem, rem, REM	Roentgen Equivalent Man
REMODCM ...	Radiological Effluent Monitoring Manual/Off-site Dose Calculation Manual
RPS.....	Reactor Protection System
RSS.....	Recirculation Spray System
RVLIS.....	Reactor Vessel Level Instrumentation System

RVLMS..... Reactor Vessel Level Monitoring System
SBO Station Blackout
SCIP..... Security Contingency Plan Implementing Procedures
SCBA Self-Contained Breathing Apparatus
SFP Spent Fuel Pool (Pit)
SG Steam Generator
SI Safety Injection
SM Shift Manager
SPDS Safety Parameter Display System
SRO Senior Reactor Operator
SRV Safety Relief Valve
TC (T/C) Thermocouple
TEDE Total Effective Dose Equivalent
TAF Top of Active Fuel
TIC Thermally Induced Current
TS Technical Specifications
TSC Technical Support Center
UE Unusual Event
UFSAR Updated Final Safety Analysis Report
USGS United States Geological Survey

6.0 MPS3-TO-NEI 99-01, Rev. 6 EAL CROSS-REFERENCE

This cross-reference is provided to facilitate association and location of a MPS3 EAL within the NEI 99-01 IC/EAL identification scheme. Further information regarding the development of the MPS3 EALs based on the NEI guidance can be found in the EAL Comparison Matrix.

MPS3	NEI 99-01, Rev. 6	
	IC	Example EAL
RU1.1	AU1	1
RU1.2	AU1	2
RU1.3	AU1	3
RU1.4	AU1	1
RU1.5	AU1	1
RU2.1	AU2	1
RA1.1	AA1	1
RA1.2	AA1	2
RA1.3	AA1	3
RA1.4	AA1	4
RA2.1	AA2	1
RA2.2	AA2	2
RA2.3	AA2	3
RA3.1	AA3	1
RA3.2	AA3	2
RS1.1	AS1	1
RS1.2	AS1	2
RS1.3	AS1	3
RS2.1	AS2	1
RG1.1	AG1	1
RG1.2	AG1	2

MPS3	NEI 99-01, Rev. 6	
EAL	IC	Example EAL
RG1.3	AG1	3
RG2.1	AG2	1
CU1.1	CU1	1
CU1.2	CU1	2
CU2.1	CU2	1
CU3.1	CU3	1
CU3.2	CU3	2
CU4.1	CU4	1
CU5.1	CU5	1, 2, 3
CA1.1	CA1	1
CA1.2	CA1	2
CA2.1	CA2	1
CA3.1	CA3	1, 2
CA6.1	CA6	1
CS1.1	CS1	1
CS1.2	CS1	2
CS1.3	CS1	3
CG1.1	CG1	1
CG1.2	CG1	2
EU1.1	EU1	1
FA1.1	FA1	1
FS1.1	FS1	1
FG1.1	FG1	1
HU1.1	HU1	1, 2, 3

MPS3	NEI 99-01, Rev. 6	
EAL	IC	Example EAL
HU2.1	HU2	1
HU3.1	HU3	1
HU3.2	HU3	2
HU3.3	HU3	3
HU3.4	HU3	4
HU4.1	HU4	1
HU4.2	HU4	2
HU4.3	HU4	3
HU4.4	HU4	4
HU7.1	HU7	1
HA1.1	HA1	1, 2
HA5.1	HA5	1
HA6.1	HA6	1
HA7.1	HA7	1
HS1.1	HS1	1
HS6.1	HS6	1
HS7.1	HS7	1
HG7.1	HG7	1
MU1.1	SU1	1
MU3.1	SU2	1
MU4.1	SU3	1
MU4.2	SU3	2
MU5.1	SU4	1, 2, 3
MU6.1	SU5	1

**Millstone Power Station Unit 3
Emergency Action Level Technical Bases Document**

Serial No. 18-364
Docket Nos. 50-423; 72-47
Enclosure 3; Attachment 2C

MPS3	NEI 99-01, Rev. 6	
EAL	IC	Example EAL
MU6.2	SU5	2
MU7.1	SU6	1, 2, 3
MU8.1	SU7	1, 2
MA1.1	SA1	1
MA3.1	SA2	1
MA6.1	SA5	1
MA9.1	SA9	1
MS1.1	SS1	1
MS2.1	SS8	1
MS6.1	SS5	1
MG1.1	SG1	1
MG2.1	SG8	1

7.0 ATTACHMENTS

- 7.1 Attachment 1, Emergency Action Level Technical Bases
- 7.2 Attachment 2, Safe Operation & Shutdown Areas Tables R-2 & H-2 Bases

Category R – Abnormal Rad Release / Rad Effluent

EAL Group: ANY (EALs in this category are applicable to any plant condition, hot or cold.)

Many EALs are based on actual or potential degradation of fission product barriers because of the elevated potential for offsite radioactivity release. Degradation of fission product barriers though is not always apparent via non-radiological symptoms. Therefore, direct indication of elevated radiological effluents or area radiation levels are appropriate symptoms for emergency classification.

At lower levels, abnormal radioactivity releases may be indicative of a failure of containment systems or precursors to more significant releases. At higher release rates, offsite radiological conditions may result which require offsite protective actions. Elevated area radiation levels in plant may also be indicative of the failure of containment systems or preclude access to plant vital equipment necessary to ensure plant safety.

Events of this category pertain to the following subcategories:

1. Radiological Effluent

Direct indication of effluent radiation monitoring systems provides a rapid assessment mechanism to determine releases in excess of classifiable limits. Projected offsite doses, actual offsite field measurements or measured release rates via sampling indicate doses or dose rates above classifiable limits.

2. Irradiated Fuel Events

Conditions indicative of a loss of adequate shielding or damage to irradiated fuel may preclude access to vital plant areas or result in radiological releases that warrant emergency classification.

3. Area Radiation Levels

Sustained general area radiation levels which may preclude access to areas required to safely operate and shutdown the plant also warrant emergency classification.

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1a – Radiological Effluent (Liquid)
Initiating Condition: Release of liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.1 Unusual Event

Reading on **EITHER** of the following radiation monitors > 2x the “alarm” setpoint for ≥60 min. (Notes 1, 2, 3)

- 3SSR-RE08 SG Blowdown
- 3DAS-RE50 Turbine Building Floor Drain

(Notes 1, 2, 3)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator’s operability, the condition’s existence, or the report’s accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any ~~gaseous or liquid~~ radiological release, monitored or un-monitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions

alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

~~EAL #1—This EAL addresses normally occurring continuous radioactivity releases from monitored gaseous or liquid effluent pathways (ref. 1).~~

~~EAL #2—This EAL addresses radioactivity releases that cause effluent radiation monitor readings to exceed 2 times the limit established by a radioactivity discharge permit. This EAL will typically be associated with planned batch releases from non-continuous release pathways (e.g., radwaste, waste gas).~~

~~EAL #3—This EAL addresses uncontrolled gaseous or liquid releases that are detected by sample analyses or environmental surveys, particularly on unmonitored pathways (e.g., spills of radioactive liquids into storm drains, heat exchanger leakage in river water systems, etc.).~~

Escalation of the emergency classification level would be via IC AA1RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1a – Radiological Effluent (Liquid)
Initiating Condition: Release of liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.2 Unusual Event

Reading on **EITHER** of the following effluent radiation monitors > 2 x the “alarm” setpoint established by a current radioactivity discharge permit for ≥ 60 min.

- 3LWS-RE70 Liquid Waste Effluent
- 3CND-RE07 Waste Neutralization Sump

(Notes 1, 2, 3)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator’s operability, the condition’s existence, or the report’s accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

3CND-RE07 Waste Neutralizer Discharge radiation monitor can only be read on the RMS Console (3-RMS-CNSL2).

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous or liquid radiological release, monitored or un-monitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL. ~~EAL #1—This EAL addresses normally occurring continuous radioactivity releases from monitored gaseous or liquid effluent pathway~~

~~EAL #2—This EAL addresses radioactivity releases that cause effluent radiation monitor readings to exceed 2 times the limit established by a radioactivity discharge permit. This EAL will~~ Such releases are typically be associated with planned batch releases from non-continuous release pathways (e.g., radwaste, waste gas) (ref. 1, 2).

~~EAL #3—This EAL addresses uncontrolled gaseous or liquid releases that are detected by sample analyses or environmental surveys, particularly on unmonitored pathways (e.g., spills of radioactive liquids into storm drains, heat exchanger leakage in river water systems, etc.).~~

Escalation of the emergency classification level would be via IC AA1RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. RP 17-24, "Millstone Abnormal Rad Release Liquid EAL Thresholds based on NEI 99-01, Rev. 6"
3. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1a – Radiological Effluent (Liquid)
Initiating Condition: Release of liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.3 Unusual Event

Sample analysis for a liquid release indicates a concentration or release rate
> 2 x the allocated REMODCM limits for ≥ 60 min. (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous or liquid radiological release, monitored or un-monitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

~~EAL #1—This EAL addresses normally occurring continuous radioactivity releases from monitored gaseous or liquid effluent pathways.~~

~~EAL #2—This EAL addresses radioactivity releases that cause effluent radiation monitor readings to exceed 2 times the limit established by a radioactivity discharge permit. This EAL will typically be associated with planned batch releases from non-continuous release pathways (e.g., radwaste, waste gas).~~

~~EAL #3—This EAL addresses uncontrolled gaseous or liquid releases that are detected by sample analyses or environmental surveys, particularly on unmonitored pathways (e.g., spills of radioactive liquids into storm drains, heat exchanger leakage in river-ocean water systems, etc.).~~

Escalation of the emergency classification level would be via IC AA1RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1b – Radiological Effluent (Gaseous)

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1 mrem TEDE

EAL:

RU1.4 Unusual Event

Reading on **any** Table R-1 effluent radiation monitor > column "UE" for ≥ 60 min.
(Notes 1, 2, 3)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Table R-1 Unit 3 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Ventilation Vent Normal/High Range (HVR10A/B)	5.9E+00 $\mu\text{Ci/cc}$	5.9E-01 $\mu\text{Ci/cc}$	5.9E-02 $\mu\text{Ci/cc}$	5.9E-03 $\mu\text{Ci/cc}$
Stack (SLCRS) Normal/High Range (HVR19A/B)	3.6E+02 $\mu\text{Ci/cc}$	3.6E+01 $\mu\text{Ci/cc}$	3.6E+00 $\mu\text{Ci/cc}$	3.6E-01 $\mu\text{Ci/cc}$
Main Steam Release (MSS75-78)	5.3E+01 $\mu\text{Ci/cc}$	5.3E+00 $\mu\text{Ci/cc}$	5.3E-01 $\mu\text{Ci/cc}$	N/A
Aux FDWTR Pump Mntr (MSS79)	1.6E+01 $\mu\text{Ci/cc}$	1.6E+00 $\mu\text{Ci/cc}$	1.6E-01 $\mu\text{Ci/cc}$	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous or liquid radiological release, monitored or un-monitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

~~EAL #1—This EAL addresses normally occurring continuous radioactivity releases from monitored gaseous effluent pathways (ref. 1, 2).~~

~~EAL #2—This EAL addresses radioactivity releases that cause effluent radiation monitor readings to exceed 2 times the limit established by a radioactivity discharge permit. This EAL will typically be associated with planned batch releases from non-continuous release pathways (e.g., radwaste, waste gas).~~

~~EAL #3—This EAL addresses uncontrolled gaseous or liquid releases that are detected by sample analyses or environmental surveys, particularly on unmonitored pathways (e.g., spills of radioactive liquids into storm drains, heat exchanger leakage in river water systems, etc.).~~

The basis for the gaseous Unusual Event IC and associated thresholds has been revised to correspond to any unplanned release of gaseous effluent radioactivity to the environment that will result in greater than 1 mrem TEDE. This Unusual Event gaseous release criterion is being used consistently across all operating nuclear units at Dominion Energy nuclear stations at Millstone, North Anna and Surry. The reason this alternative criterion is required is due to the fact that for some effluent gaseous release pathways, the resulting calculated Unusual Event threshold following the NEI 99-01 guidance of two times the site-specific effluent release limit would result in a Unusual Event threshold value greater than the corresponding calculated ALERT threshold based on exceeding 10 mrem TEDE. For the other gaseous release pathways that did not show an incongruent relationship when compared to the ALERT threshold, many showed Unusual Event values essentially equivalent to 1 mrem TEDE when applying the guidance in NEI 99-01 of a value set at two times the site-specific effluent release limit. The fact that, (1) many of the gaseous release pathway Unusual Event values following

NEI 99-01 guidance were essentially equivalent to 1 mrem TEDE, (2) application of an alternative definition set at a value of 1 mrem TEDE results in a more limiting value for those release paths that showed incongruent comparison to the corresponding ALERT threshold, and (3) Unusual Event criterion set at a value ten (10) times lower than the ALERT threshold provides a logical and consistent escalation between each classification level, provides justification for the Unusual Event criterion of 1 mrem TEDE. This single Initiating Condition (IC) definition for gaseous releases at the Unusual Event level is being applied to maintain consistency across the Dominion Energy nuclear fleet and to reduce confusion and human error potential if two different (IC) definitions were applied.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Due to the fact that there are no REMODCM limits on steam safeties or auxiliary feedwater exhausts and the limited ability for these respective radiation monitors to detect low level activity in these steam line configurations, the UE classification thresholds for the steam safeties and auxiliary feedwater exhaust are being labeled N/A (not applicable).

Escalation of the emergency classification level would be via IC AA4RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. RP 18-03, "MP3 Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01, Rev. 6"
3. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1b – Radiological Effluent (Gaseous)
Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1 mrem TEDE

EAL:

RU1.5 Unusual Event

Sample analysis for a gaseous release indicates a concentration or release rate > 2 x the allocated REMODCM limits for ≥ 60 min. (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous or liquid radiological release, monitored or un-monitored, ~~including those for which a radioactivity discharge permit is normally prepared.~~

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

Calculation RP 18-03 (ref. 2) demonstrates how a release rate limit based on 2 x the allocated REMODCM limit will produce essentially 1 mrem TEDE assuming most prevalent meteorological dispersion.

Most prevalent meteorology represents conditions that would most likely to exist (based on most prevalent stability class and average wind speed within that stability class). Dispersion based on most prevalent meteorology differs from that assumed in the REMODCM which uses annual average meteorology. Dispersion based on actual meteorological conditions at the time of the emergency (most prevalent) can be 10 – 20 times higher than the annual average dispersion prescribed for use in an ODCM.

~~EAL #1— This EAL addresses normally occurring continuous radioactivity releases from monitored gaseous or liquid effluent pathways.~~

~~EAL #2— This EAL addresses radioactivity releases that cause effluent radiation monitor readings to exceed 2 times the limit established by a radioactivity discharge permit. This EAL will typically be associated with planned batch releases from non-continuous release pathways (e.g., radwaste, waste gas).~~

~~EAL #3— This EAL addresses uncontrolled gaseous or liquid releases that are detected by sample analyses or environmental surveys, particularly on unmonitored pathways (e.g., spills of radioactive liquids into storm drains, heat exchanger leakage in river water systems, etc.).~~

Escalation of the emergency classification level would be via IC AA1RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. RP 18-03, "MP3 Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
3. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.1 Alert

Reading on **any** Table A-1 effluent radiation monitor > column "ALERT" for ≥ 15 min.
(Notes 1, 2, 3, 4)

- Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.
- Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.
- Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.
- Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Table R-1 Unit 3 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Ventilation Vent Normal/High Range (HVR10A/B)	5.9E+00 $\mu\text{Ci/cc}$	5.9E-01 $\mu\text{Ci/cc}$	5.9E-02 $\mu\text{Ci/cc}$	5.9E-03 $\mu\text{Ci/cc}$
Stack (SLCRS) Normal/High Range (HVR19A/B)	3.6E+02 $\mu\text{Ci/cc}$	3.6E+01 $\mu\text{Ci/cc}$	3.6E+00 $\mu\text{Ci/cc}$	3.6E-01 $\mu\text{Ci/cc}$
Main Steam Release (MSS75-78)	5.3E+01 $\mu\text{Ci/cc}$	5.3E+00 $\mu\text{Ci/cc}$	5.3E-01 $\mu\text{Ci/cc}$	N/A
Aux FDWTR Pump Mntr (MSS79)	1.6E+01 $\mu\text{Ci/cc}$	1.6E+00 $\mu\text{Ci/cc}$	1.6E-01 $\mu\text{Ci/cc}$	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem TEDE while the 50 mrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Escalation of the emergency classification level would be via IC AS4RS1.

Reference(s):

1. RP 18-03 MP3, "Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.2 Alert

Dose assessment using actual meteorology indicates doses > 10 mrem TEDE at or beyond the SITE BOUNDARY (Note 4)

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem TEDE while the 50 mrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Actual meteorology is specifically identified since it gives the most accurate dose assessment.
Actual meteorology (including forecasts) should be used whenever possible.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

Escalation of the emergency classification level would be via IC AS4RS1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.3 Alert

Analysis of a liquid effluent sample indicates a concentration or release rate that would result in doses > 10 mrem TEDE at or beyond the SITE BOUNDARY for 60 min. of exposure (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem TEDE while the 50 mrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

This EAL is assessed per the REMODCM (ref. 2). REMODCM software can be used to produce a dose to the maximum individual.

Escalation of the emergency classification level would be via IC AS4RS1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AA1

-Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.4 Alert

Field survey results indicate closed window dose rates > 10 mR/hr expected to continue for ≥60 min. at or beyond the SITE BOUNDARY (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem ~~TEDE~~ while the 50 mrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

Escalation of the emergency classification level would be via IC AS4RS1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-26-EPI-FAP04, "Emergency Operations Facility Activation and Operation"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 100 mrem TEDE

EAL:

RS1.1 Site Area Emergency

Reading on **any** Table R-1 effluent radiation monitor > column "SAE" for ≥ 15 min.
(Notes 1, 2, 3, 4)

- Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.
- Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.
- Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.
- Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available

Table R-1 Unit 3 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Ventilation Vent Normal/High Range (HVR10A/B)	5.9E+00 $\mu\text{Ci/cc}$	5.9E-01 $\mu\text{Ci/cc}$	5.9E-02 $\mu\text{Ci/cc}$	5.9E-03 $\mu\text{Ci/cc}$
Stack (SLCRS) Normal/High Range (HVR19A/B)	3.6E+02 $\mu\text{Ci/cc}$	3.6E+01 $\mu\text{Ci/cc}$	3.6E+00 $\mu\text{Ci/cc}$	3.6E-01 $\mu\text{Ci/cc}$
Main Steam Release (MSS75-78)	5.3E+01 $\mu\text{Ci/cc}$	5.3E+00 $\mu\text{Ci/cc}$	5.3E-01 $\mu\text{Ci/cc}$	N/A
Aux FDWTR Pump Mntr (MSS79)	1.6E+01 $\mu\text{Ci/cc}$	1.6E+00 $\mu\text{Ci/cc}$	1.6E-01 $\mu\text{Ci/cc}$	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to 10% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude are associated with the failure of plant systems needed for the protection of the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 10% of the EPA PAG of 1,000 mrem ~~TEDE while the 500 mrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.~~

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Escalation of the emergency classification level would be via IC AG4RG1.

Reference(s):

1. RP 18-03, "MP3 Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AS1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 100 mrem TEDE

EAL:

RS1.2 Site Area Emergency

Dose assessment using actual meteorology indicates doses > 100 mrem TEDE at or beyond the SITE BOUNDARY (Note 4)

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to 10% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude are associated with the failure of plant systems needed for the protection of the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 10% of the EPA PAG of 1,000 mrem ~~TEDE~~ while the 500 mrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments

using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Actual meteorology is specifically identified since it gives the most accurate dose assessment. Actual meteorology (including forecasts) should be used whenever possible.

Escalation of the emergency classification level would be via IC AG1RG1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AS1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 100 mrem TEDE

EAL:

RS1.3 Site Area Emergency

Field survey results indicate closed window dose rates > 100 mR/hr expected to continue for ≥60 min. at or beyond the SITE BOUNDARY (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to 10% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude are associated with the failure of plant systems needed for the protection of the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 10% of the EPA PAG of 1,000 mrem ~~TEDE while the 500-mrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.~~

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

Escalation of the emergency classification level would be via IC AG1RG1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"

2. MP-26-EPI-FAP04, "Emergency Operations Facility Activation and Operation"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AS1

Category: R₁ – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE

EAL:

RG1.1 General Emergency

Reading on **any** Table R-1 effluent radiation monitor > column "GE" for ≥15 min.
(Notes 1, 2, 3, 4)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Table R-1 Unit 3 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Ventilation Vent Normal/High Range (HVR10A/B)	5.9E+00 µCi/cc	5.9E-01 µCi/cc	5.9E-02 µCi/cc	5.9E-03 µCi/cc
Stack (SLCRS) Normal/High Range (HVR19A/B)	3.6E+02 µCi/cc	3.6E+01 µCi/cc	3.6E+00 µCi/cc	3.6E-01 µCi/cc
Main Steam Release (MSS75-78)	5.3E+01 µCi/cc	5.3E+00 µCi/cc	5.3E-01 µCi/cc	N/A
Aux FDWTR Pump Mntr (MSS79)	1.6E+01 µCi/cc	1.6E+00 µCi/cc	1.6E-01 µCi/cc	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude will require implementation of protective actions for the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at the EPA PAG of 1,000 mrem ~~TEDE while the 5,000 mrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.~~

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Reference(s):

1. RP 18-03 MP3, "Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AG1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE

EAL:

RG1.2 General Emergency

Dose assessment using actual meteorology indicates doses > 1,000 mrem TEDE at or beyond the SITE BOUNDARY (Note 4)

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude will require implementation of protective actions for the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at the EPA PAG of 1,000 mrem TEDE while the 5,000 mrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments

using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Actual meteorology is specifically identified since it gives the most accurate dose assessment. Actual meteorology (including forecasts) should be used whenever possible.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AG1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE

EAL:

RG1.3 General Emergency

Field survey results indicate closed window dose rates > 1,000 mR/hr expected to continue for ≥ 60 min. at or beyond the SITE BOUNDARY (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude will require implementation of protective actions for the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at the EPA PAG of 1,000 mrem ~~TEDE while the 5,000 mrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.~~

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-26-EPI-FAP04, "Emergency Operations Facility Activation and Operation"

3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AG1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: UNPLANNED loss of water level above irradiated fuel

EAL:

RU2.1 Unusual Event

UNPLANNED water level drop in the REFUELING PATHWAY as indicated by **any** of the following:

- FUEL POOL LEVEL LO alarm MB1A 3-4
- FUEL POOL WATER LEVEL LO alarm FP 1-3
- Report of dropping level in refueling cavity or SFP

AND

UNPLANNED rise in corresponding area radiation levels as indicated by **any** of the following radiation monitors:

- RMS01 Manipulator Crane
- RMS41/42 Fuel Drop
- RMS08 SFP Bridge
- RMS36 Fuel Pool

Mode Applicability:

All

Definition(s):

UNPLANNED- A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

REFUELING PATHWAY- Refueling cavity, fuel transfer canal, and spent fuel pool (SFP), but **not** including the reactor vessel, comprise the refueling pathway.

Basis:

This IC addresses a decrease in water level above irradiated fuel sufficient to cause elevated radiation levels. This condition could be a precursor to a more serious event and is also indicative of a minor loss in the ability to control radiation levels within the plant. It is therefore a potential degradation in the level of safety of the plant.

A water level decrease will be primarily determined by indications from available level instrumentation. Other sources of level indications may include reports from plant personnel (e.g., from a refueling crew) or video camera observations (if available). A significant drop in the water level may also cause a loss of SFP cooling suction flow and an increase in the radiation levels of adjacent areas that can be detected by monitors in those locations.

The specified radiation monitors are those expected to see increase area radiation levels as a result of a loss of REFUELING PATHWAY inventory (ref. 1, 2, 3, 4). Increasing radiation indications on these monitors in the absence of indications of decreasing REFUELING PATHWAY level are not classifiable under this EAL.

The effects of planned evolutions should be considered. For example, a refueling bridge area radiation monitor reading may increase due to planned evolutions such as lifting of the reactor vessel head or movement of a fuel assembly. Note that this EAL is applicable only in cases where the elevated reading is due to an unplanned loss of water level.

A drop in water level above irradiated fuel within the reactor vessel may be classified in accordance ~~Recognition~~ Category C during the Cold Shutdown and Refueling modes.

Escalation of the emergency classification level would be via IC AA2RA2.

Reference(s):

1. AOP 3572, "Failure of Refueling Cavity Seal"
2. EOP 3505A, "Loss of Spent Fuel Pool Cooling"
3. EOP 3502, "Fuel Handling Accident"
4. OP 3362, "Radiation Monitor System Display and Control System"
5. NEI 99-01 AU2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Significant lowering of water level above, or damage to, irradiated fuel

EAL:

RA2.1 Alert

IMMINENT uncover of irradiated fuel in the REFUELING PATHWAY

Mode Applicability:

All

Definition(s):

CONFINEMENT BOUNDARY- The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielding Canister (DSC).

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

REFUELING PATHWAY- Refueling cavity, fuel transfer canal, and spent fuel pool (SFP), but **not** including the reactor vessel, comprise the refueling pathway.

Basis:

This IC addresses events that have caused IMMINENT or actual damage to an irradiated fuel assembly, or a significant lowering of water level within the spent fuel pool REFUELING PATHWAY (see *Developer Notes*). These events present radiological safety challenges to plant personnel and are precursors to a release of radioactivity to the environment. As such, they represent an actual or potential substantial degradation of the level of safety of the plant.

~~This IC applies to~~ For irradiated fuel that is licensed for dry storage, this EAL applies up to the point that the loaded storage cask is sealed. Once sealed, damage to a loaded cask causing loss of the CONFINEMENT BOUNDARY is classified in accordance with IC E-HU1.

Escalation of the emergency would be based on either ~~Recognition~~ Category A-R or C ICs.

EAL #1

This EAL escalates from AU2-RU2.1 in that the loss of level, in the affected portion of the REFUELING PATHWAY, is of sufficient magnitude to have resulted in uncover of irradiated fuel. Indications of irradiated fuel uncover may include direct or indirect visual observation (e.g., reports from personnel or camera images), as well as significant changes in water and radiation levels, or other plant parameters. Computational aids may also be used (e.g., a boil-off curve). Classification of an event using this EAL should be based on the totality of available indications, reports and observations.

While an area radiation monitor could detect an increase in a dose rate due to a lowering of water level in some portion of the REFUELING PATHWAY, the reading may not be a reliable indication of whether or not the fuel is actually uncovered. To the degree possible, readings should be considered in combination with other available indications of inventory loss.

A drop in water level above irradiated fuel within the reactor vessel may be classified in accordance with ~~Recognition Category C~~ during the Cold Shutdown and Refueling modes. ~~EAL #2~~

~~—— This EAL addresses a release of radioactive material caused by mechanical damage to irradiated fuel. Damaging events may include the dropping, bumping or binding of an assembly, or dropping a heavy load onto an assembly. A rise in readings on radiation monitors should be considered in conjunction with in-plant reports or observations of a potential fuel damaging event (e.g., a fuel handling accident).~~

~~—— EAL #3~~

~~—— Spent fuel pool water level at this value is within the lower end of the level range necessary to prevent significant dose consequences from direct gamma radiation to personnel performing operations in the vicinity of the spent fuel pool. This condition reflects a significant loss of spent fuel pool water inventory and thus it is also a precursor to a loss of the ability to adequately cool the irradiated fuel assemblies stored in the pool.~~

~~—— Escalation of the emergency classification level would be via ICs AS1 or AS2 (see AS2 Developer Notes).~~

Reference(s):

1. NEI 99-01 AA2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Significant lowering of water level above, or damage to, irradiated fuel
EAL:

RA2.2 Alert

Damage to irradiated fuel resulting in a release of radioactivity

AND

VALID alarm on **any** of the following radiation monitors:

- CMS22 CTMT G/A
- RM01 Manipulator Crane
- RMS41/42 Fuel Drop
- HVR17 Fuel Bldg
- RMS08 SFP Bridge
- RMS36 Fuel Pool

Mode Applicability:

All

Definition(s):

CONFINEMENT BOUNDARY- The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielded Canister (DSC).

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

The specified radiation monitors are those expected to see increased area radiation levels as a result of damage to irradiated fuel (ref. 1, 2, 3, 4).

This ~~IC-EAL~~ addresses events that have caused ~~imminent or actual~~ damage to an irradiated fuel assembly, ~~or a significant lowering of water level within the spent fuel pool (see Developer Notes).~~ These events present radiological safety challenges to plant personnel and are precursors to a release of radioactivity to the environment. As such, they represent an actual or potential substantial degradation of the level of safety of the plant.

~~This IC applies to~~For irradiated fuel that is licensed for dry storage, this EAL applies up to the point that the loaded storage cask is sealed. Once sealed, damage to a loaded cask causing loss of the CONFINEMENT BOUNDARY is classified in accordance with IC E-HU1.

~~_____ EAL # This EAL escalates from AU2 in that the loss of level, in the affected portion of the REFUELING PATHWAY, is of sufficient magnitude to have resulted in uncover of irradiated fuel. Indications of irradiated fuel uncover may include direct or indirect visual observation (e.g., reports from personnel or camera images), as well as significant changes in water and radiation levels, or other plant parameters. Computational aids may also be used (e.g., a boil-off curve). Classification of an event using this EAL should be based on the totality of available indications, reports and observations.~~

~~_____ While an area radiation monitor could detect an increase in a dose rate due to a lowering of water level in some portion of the REFUELING PATHWAY, the reading may not be a reliable indication of whether or not the fuel is actually uncovered. To the degree possible, readings should be considered in combination with other available indications of inventory loss.~~

~~_____ A drop in water level above irradiated fuel within the reactor vessel may be classified in accordance Recognition Category C during the Cold Shutdown and Refueling modes.~~

This EAL addresses a release of radioactive material caused by mechanical damage to irradiated fuel. Damaging events may include the dropping, bumping or binding of an assembly, or dropping a heavy load onto an assembly. A rise in readings on radiation monitors should be considered in conjunction with in-plant reports or observations of a potential fuel damaging event (e.g., a fuel handling accident). ~~EAL #3 Spent fuel pool water level at this value is within the lower end of the level range necessary to prevent significant dose consequences from direct gamma radiation to personnel performing operations in the vicinity of the spent fuel pool. This condition reflects a significant loss of spent fuel pool water inventory and thus it is also a precursor to a loss of the ability to adequately cool the irradiated fuel assemblies stored in the pool.~~

Escalation of the emergency would be based on either Recognition Category A ~~R~~ or C ICs. Escalation of the emergency classification level would be via ICs AS1 or AS2 (see AS2 Developer Notes).

~~_____~~

Reference(s):

1. AOP 3572, "Failure of Refueling Cavity Seal"
2. EOP 3505A, "Loss of Spent Fuel Pool Cooling"
3. EOP 3502, "Fuel Handling Accident"
4. OP 3362, "Radiation Monitor System Display and Control System"
5. NEI 99-01 AA2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Significant lowering of water level above, or damage to, irradiated fuel

EAL:

RA2.3 Alert

Lowering of spent fuel pool level to 10 ft. (Level 2) on 3SFC-LI55A or 3SFC-LI55B

Mode Applicability:

All

Definition(s):

None

Basis:

~~———This IC EAL addresses events that have caused IMMINENT or actual damage to an irradiated fuel assembly, or a significant lowering of water level within the spent fuel pool (see *Developer Notes*). These events present radiological safety challenges to plant personnel and are precursors to a release of radioactivity to the environment. As such, they represent an actual or potential substantial degradation of the level of safety of the plant. This IC applies to irradiated fuel that is licensed for dry storage up to the point that the loaded storage cask is sealed. Once sealed, damage to a loaded cask causing loss of the CONFINEMENT BOUNDARY is classified in accordance with IC E-HU1.~~

~~———Escalation of the emergency would be based on either Recognition Category A or C ICs. EAL #This EAL escalates from AU2 in that the loss of level, in the affected portion of the REFUELING PATHWAY, is of sufficient magnitude to have resulted in uncover of irradiated fuel. Indications of irradiated fuel uncover may include direct or indirect visual observation (e.g., reports from personnel or camera images), as well as significant changes in water and radiation levels, or other plant parameters. Computational aids may also be used (e.g., a boil-off curve). Classification of an event using this EAL should be based on the totality of available indications, reports and observations.~~

~~———While an area radiation monitor could detect an increase in a dose rate due to a lowering of water level in some portion of the REFUELING PATHWAY, the reading may not be a reliable indication of whether or not the fuel is actually uncovered. To the degree possible, readings should be considered in combination with other available indications of inventory loss.~~

~~———A drop in water level above irradiated fuel within the reactor vessel may be classified in accordance Recognition Category C during the Cold Shutdown and Refueling modes.~~

~~This EAL addresses a release of radioactive material caused by mechanical damage to irradiated fuel. Damaging events may include the dropping, bumping or binding of an assembly, or dropping a heavy load onto an assembly. A rise in readings on radiation monitors should be considered in conjunction with in plant reports or observations of a potential fuel damaging event (e.g., a fuel handling accident).~~

EAL #3Spent fuel pool water level at this value is within the lower end of the level range

necessary to prevent significant dose consequences from direct gamma radiation to personnel performing operations in the vicinity of the spent fuel pool. This condition reflects a significant loss of spent fuel pool water inventory and thus it is also a precursor to a loss of the ability to adequately cool the irradiated fuel assemblies stored in the pool.

Escalation of the emergency classification level would be via ICs AS4 RS1 or ARS2 (~~see AS2 Developer Notes~~).

Post-Fukushima order EA-12-051 required the installation of reliable SFP level indication (3SFC-LI55A or 3SFC-LI55B) capable of identifying normal level (Level 1 – EL 47 ft. 1 in.), SFP level 10 ft. above the top of the fuel racks (Level 2 – EL 35 ft. 11 in.) and SFP level at 1 ft. above the top of the fuel racks (Level 3 – EL 26 ft. 11 in.) (ref. 1, 2).

Reference(s):

1. ETE-CPR-2012-0008, "Millstone Unit 3 – Beyond Design Basis FLEX Strategy Basis Document and Final Integrated Plan"
2. DC MP3-13-01012, "Beyond Design Basis Spent Fuel Pool Level Instrumentation, Millstone Unit 3"
3. NEI 99-01 AA2

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 2 – Irradiated Fuel Events
Initiating Condition: Spent fuel pool level at the top of the fuel racks
EAL:

RS2.1 Site Area Emergency

Lowering of spent fuel pool level to 1 ft. (Level 3) on 3SFC-LI55A or 3SFC-LI55B

Mode Applicability:

All

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

This ~~IC-EAL~~ addresses a significant loss of spent fuel pool inventory control and makeup capability leading to IMMINENT fuel damage. This condition entails major failures of plant functions needed for protection of the public and thus warrant a Site Area Emergency declaration.

It is recognized that this IC would likely not be met until well after another Site Area Emergency IC was met; however, it is included to provide classification diversity.

Escalation of the emergency classification level would be via IC AG1-RG1 or AG2-ARG2.

Post-Fukushima order EA-12-051 required the installation of reliable SFP level indication (3SFC-LI55A or 3SFC-LI55B) capable of identifying normal level (Level 1 – EL 47 ft. 1 in.), SFP level 10 ft. above the top of the fuel racks (Level 2 – EL 35 ft. 11 in.) and SFP level at 1 ft. above the top of the fuel racks (Level 3 – EL 26 ft. 11 in.) (ref. 1, 2).

Reference(s):

1. ETE-CPR-2012-0008, "Millstone Unit 3 – Beyond Design Basis FLEX Strategy Basis Document and Final Integrated Plan"
2. DC MP3-13-01012, "Beyond Design Basis Spent Fuel Pool Level Instrumentation, Millstone Unit 3"
3. NEI 99-01 AS2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Spent fuel pool level **cannot** be restored to at least the top of the fuel racks for 60 minutes or longer

EAL:

RG2.1 General Emergency

Spent fuel pool level **cannot** be restored to at least 1 ft. (Level 3) on 3SFC-LI55A or 3SFC-LI55B for ≥ 60 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

All

Definition(s):

None

Basis:

This ~~IC~~-EAL addresses a significant loss of spent fuel pool inventory control and makeup capability leading to a prolonged uncover of spent fuel. This condition will lead to fuel damage and a radiological release to the environment.

It is recognized that this ~~IC~~-EAL would likely not be met until well after another General Emergency ~~IC~~-EAL was met; however, it is included to provide classification diversity.

Post-Fukushima order EA-12-051 required the installation of reliable SFP level indication (3SFC-LI55A or 3SFC-LI55B) capable of identifying normal level (Level 1 – EL 47 ft. 1 in.), SFP level 10 ft. above the top of the fuel racks (Level 2 – EL 35 ft. 11 in.) and SFP level at 1 ft. above the top of the fuel racks (Level 3 – EL 26 ft. 11 in.) (ref. 1, 2).

Reference(s):

1. ETE-CPR-2012-0008, "Millstone Unit 3 – Beyond Design Basis FLEX Strategy Basis Document and Final Integrated Plan"
2. DC MP3-13-01012, "Beyond Design Basis Spent Fuel Pool Level Instrumentation, Millstone Unit 3"
3. NEI 99-01 AG2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 3 – Area Radiation Levels

Initiating Condition: Radiation levels that IMPEDE access to equipment necessary for normal plant operations, cooldown or shutdown

EAL:

RA3.1 Alert

Dose rate > 15 mR/hr in **EITHER** of the following areas:

- Control Room
- Central Alarm Station

Mode Applicability:

All

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

Basis:

This IC addresses elevated radiation levels in certain plant rooms/areas sufficient to preclude or IMPEDE personnel from performing actions necessary to maintain normal plant operation, or to perform a normal plant cooldown and shutdown. As such, it represents an actual or potential substantial degradation of the level of safety of the plant. The Emergency Director/DSEO/ADTS should consider the cause of the increased radiation levels and determine if another IC may be applicable. ~~For EAL #2, an Alert declaration is warranted if entry into the affected room/area is, or may be, procedurally required during the plant operating mode in effect at the time of the elevated radiation levels. The emergency classification is not contingent upon whether entry is actually necessary at the time of the increased radiation levels. Access should be considered as impeded if extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., installing temporary shielding, requiring use of non routine protective equipment, requesting an extension in dose limits beyond normal administrative limits).~~

~~An emergency declaration is not warranted if any of the following conditions apply:~~

~~The plant is in an operating mode different than the mode specified for the affected room/area (i.e., entry is not required during the operating mode in effect at the time of the elevated radiation levels). For example, the plant is in Mode 1 when the radiation increase occurs, and the procedures used for normal operation, cooldown and shutdown do not require entry into the affected room until Mode 4.~~

~~The increased radiation levels are a result of a planned activity that includes compensatory measures which address the temporary inaccessibility of a room or area (e.g., radiography, spent filter or resin transfer, etc.).~~

~~The action for which room/area entry is required is of an administrative or record keeping nature (e.g., normal rounds or routine inspections).~~

~~The access control measures are of a conservative or precautionary nature, and would not actually prevent or impede a required action.~~

Areas that meet this threshold include the Control Room (CR) and the Central Alarm Station (CAS). The Control Room is monitored for excessive radiation by one detector, RMS-22 (ref. 1). The CAS is included in this EAL because of its importance to permitting access to areas required to assure safe plant operations.

~~Escalation of the emergency classification level would be via Recognition Category A, C or F ICs.~~

Reference(s):

1. AOP 3573, "Radiation Monitor Alarm Response"
2. NEI 99-01 AA3

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 3 – Area Radiation Levels

Initiating Condition: Radiation levels that IMPEDE access to equipment necessary for normal plant operations, cooldown or shutdown

EAL:

RA3.2 Alert

An UNPLANNED event results in radiation levels that prohibit or IMPEDE access to **any** Table R-2 room or area (Note 5)

Note 5: If the equipment in the listed room or area was already inoperable or out-of-service before the event occurred, then no emergency classification is warranted.

Table R-2 Safe Operation & Shutdown Rooms/Areas	
Room/Area	Mode
Aux. Building EI 43'	1, 2, 3
Aux. Building EI 24' East MCC/RCA EI 24' West MCC/RCA EI 24'	3, 4
Aux. Building EI 3' 8"	3
Containment ESF Building A RHR Cubicle ESF Building B RHR Cubicle ESF Building EI 36' ESF Building EI 24' East Switchgear West Switchgear	4

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

UNPLANNED- A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses elevated radiation levels in certain plant rooms/areas sufficient to preclude or IMPEDE personnel from performing actions necessary to maintain normal plant operation, or to perform a normal plant cooldown and shutdown. As such, it represents an actual or

potential substantial degradation of the level of safety of the plant. The Emergency Director ~~DSEO/ADTS~~ should consider the cause of the increased radiation levels and determine if another IC may be applicable.

For ~~EAL #2-RA3.2~~, an Alert declaration is warranted if entry into the affected room/area is, or may be, procedurally required during the plant operating mode in effect at the time of the elevated radiation levels. The emergency classification is not contingent upon whether entry is actually necessary at the time of the increased radiation levels. Access should be considered as IMPEDED if extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., installing temporary shielding, requiring use of non-routine protective equipment, requesting an extension in dose limits beyond normal administrative limits).

If the equipment in the listed room or area was already inoperable, or out-of-service, before the event occurred, then no emergency should be declared since the event will have no adverse impact beyond that already allowed by Technical Specifications at the time of the event.

The list of plant rooms or areas with entry-related mode applicability identified specify those rooms or areas that contain equipment which require a manual/local action as specified in operating procedures used for normal plant operation, cooldown and shutdown. Rooms or areas in which actions of a contingent or emergency nature would be performed (e.g., an action to address an off-normal or emergency condition such as emergency repairs, corrective measures or emergency operations) are not included. In addition, the list specifies the plant mode(s) during which entry would be required for each room or area (ref. 1).

An emergency declaration is **not** warranted if any of the following conditions apply:

- The plant is in an operating mode different than the mode specified for the affected room/area (i.e., entry is not required during the operating mode in effect at the time of the elevated radiation levels). For example, the plant is in Mode 1 when the radiation increase occurs, and the procedures used for normal operation, cooldown and shutdown do not require entry into the affected room until Mode 4.
- The increased radiation levels are a result of a planned activity that includes compensatory measures which address the temporary inaccessibility of a room or area (e.g., radiography, spent filter or resin transfer, etc.).
- The action for which room/area entry is required is of an administrative or record keeping nature (e.g., normal rounds or routine inspections).
- The access control measures are of a conservative or precautionary nature, and would not actually prevent or IMPEDE a required action.

~~Escalation of the emergency classification level would be via Recognition Category A, C or F ICs.~~

Reference(s):

1. Attachment 2, "Safe Operation & Shutdown Areas Tables R-2 & H-2 Bases"
2. NEI 99-01 AA3

Category C – Cold Shutdown / Refueling System Malfunction

EAL Group: Cold Conditions (RCS temperature $\leq 200^{\circ}\text{F}$); EALs in this category are applicable only in one or more cold operating modes.

Category C EALs are directly associated with cold shutdown or refueling system safety functions. Given the variability of plant configurations (e.g., systems out-of-service for maintenance, containment open, reduced AC power redundancy, time since shutdown) during these periods, the consequences of any given initiating event can vary greatly. For example, a loss of decay heat removal capability that occurs at the end of an extended outage has less significance than a similar loss occurring during the first week after shutdown. Compounding these events is the likelihood that instrumentation necessary for assessment may also be inoperable. The cold shutdown and refueling system malfunction EALs are based on performance capability to the extent possible with consideration given to RCS integrity, CONTAINMENT CLOSURE, and fuel clad integrity for the applicable operating modes (5 - Cold Shutdown, 6 - Refueling, DEF – Defueled).

The events of this category pertain to the following subcategories:

1. RCS Level

RCS water level is directly related to the status of adequate core cooling and, therefore, fuel clad integrity.

2. Loss of Emergency AC Power

Loss of vital plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of onsite and offsite power sources for 4.16 KV emergency buses.

3. RCS Temperature

Uncontrolled or inadvertent temperature or pressure rises are indicative of a potential loss of safety functions.

4. Loss of Vital DC Power

Loss of emergency plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of power to or degraded voltage on the 125V DC vital buses.

5. Loss of Communications

Certain events that degrade plant operator ability to effectively communicate with essential personnel within or external to the plant warrant emergency classification.

6. Hazardous Event Affecting Safety Systems

Certain hazardous natural and technological events may result in **VISIBLE DAMAGE** to or degraded performance of safety systems warranting classification.

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: UNPLANNED loss of RCS inventory

EAL:

CU1.1 Unusual Event

UNPLANNED loss of reactor coolant results in RCS water level < a required lower limit for ≥15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

UNPLANNED-. A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

RCS water level less than a required lower limit is meant to be less than the lower end of the level control band being procedurally maintained for the current condition or evolution.

With the plant in Cold Shutdown, RCS water level is normally maintained within a pressurizer level control band (ref. 1). However, if RCS level is being controlled below the normal pressurizer level control band, or if level is being maintained in a designated band in the reactor vessel it is the inability to maintain level above the low end of the designated control band due to a loss of inventory resulting from a leak in the RCS that is the concern.

With the plant in Refueling mode, RCS water level is normally maintained at or above the reactor vessel flange (ref. 2, 3).

This IC addresses the inability to restore and maintain water level to a required minimum level (or the lower limit of a level band), or a loss of the ability to monitor ~~(reactor vessel/RCS [PWR] or RPV [BWR])~~ level concurrent with indications of coolant leakage. Either of these conditions is considered to be a potential degradation of the level of safety of the plant.

Refueling evolutions that decrease RCS water inventory are carefully planned and controlled. An UNPLANNED event that results in water level decreasing below a procedurally required limit warrants the declaration of an Unusual Event due to the reduced water inventory that is available to keep the core covered.

This EAL-#1 recognizes that the minimum required ~~(reactor vessel/RCS [PWR] or RPV [BWR])~~ level can change several times during the course of a refueling outage as different plant configurations and system lineups are implemented. This EAL is met if the minimum level, specified for the current plant conditions, cannot be maintained for 15 minutes or longer. The minimum level is typically specified in the applicable operating procedure but may be specified in another controlling document.

The 15-minute threshold duration allows sufficient time for prompt operator actions to restore and maintain the expected water level. This criterion excludes transient conditions causing a brief lowering of water level.

~~———— EAL #2 addresses a condition where all means to determine (reactor vessel/RCS [PWR] or RPV [BWR]) level have been lost. In this condition, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels. Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the (reactor vessel/RCS [PWR] or RPV [BWR]).~~

Continued loss of RCS inventory may result in escalation to the Alert emergency classification level via either IC CA1 or CA3.

Reference(s):

1. OP 3208, "Plant Cooldown"
2. OP 3216, "RCS Drain (ICCE)"
3. OP 3210B, "Refueling Operations"
4. NEI 99-01 CU1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: UNPLANNED loss of RCS inventory

EAL:

CU1.2 Unusual Event

RCS water level **cannot** be monitored

AND EITHER:

- UNPLANNED rise in **any** Table C-1 sump or tank level due to a loss of RCS inventory
- Visual observation of UNISOLABLE RCS leakage

Table C-1 Sumps/Tanks

- PRT
- CDTT
- PDTT
- Containment Sump
- Unidentified Leakage Sump
- Auxiliary Bldg. Sump
- ESF Bldg. Sump

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED-. A parameter changes or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses the inability to restore and maintain water level to a required minimum level (or the lower limit of a level band), or a loss of the ability to monitor (reactor vessel/RCS [PWR] or RPV [BWR]) level concurrent with indications of coolant leakage. Either of these conditions is considered to be a potential degradation of the level of safety of the plant.

Refueling evolutions that decrease RCS water inventory are carefully planned and controlled. An UNPLANNED event that results in water level decreasing below a procedurally required limit warrants the declaration of an Unusual Event due to the reduced water inventory that is available to keep the core covered.

~~EAL #1 recognizes that the minimum required (reactor vessel/RCS [PWR] or [BWR]) level can change several times during the course of a refueling outage as different plant configurations and system lineups are implemented. This EAL is met if the minimum level, specified for the current plant conditions, cannot be maintained for 15 minutes or longer. The minimum level is typically specified in the applicable operating procedure but may be specified in another controlling document.~~

~~The 15-minute threshold duration allows sufficient time for prompt operator actions to restore and maintain the expected water level. This criterion excludes transient conditions causing a brief lowering of water level.~~

This EAL #2 addresses a condition where all means to determine (reactor vessel/RCS [PWR] or [BWR]) level have been lost. In this condition, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels (Table C-1) (ref. 1, 2, 3). Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the (reactor vessel/RCS [PWR] or [BWR]).

Continued loss of RCS inventory may result in escalation to the Alert emergency classification level via either IC CA1 or CA3.

Reference(s):

1. OP 3216, "RCS Drain (ICCE)"
2. EOP 3505, "Loss of Shutdown Cooling and/or RCS Inventory"
3. MPS3 FSAR Section 5.2, "Integrity of Reactor Coolant Pressure Boundary"
4. NEI 99-01 CU1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Significant Loss of RCS inventory

EAL:

CA1.1 Alert

Loss of RCS inventory as indicated by **EITHER**:

- < 82% RVLMS plenum level
- ≤ 15 in above centerline of hotleg on tygon tube indicator

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

None

Basis:

This IC addresses conditions that are precursors to a loss of the ability to adequately cool irradiated fuel (i.e., a precursor to a challenge to the fuel clad barrier). This condition represents a potential substantial reduction in the level of plant safety.

For this EAL #1, a lowering of RCS water level below ~~(site-specific level)~~ ft the specified value(s) indicates that operator actions have not been successful in restoring and maintaining ~~(reactor vessel/RCS [PWR] or RPV [BWR])~~ water level. The heat-up rate of the coolant will increase as the available water inventory is reduced. A continuing decrease in water level will lead to core uncover. The specified levels are the lowest RCS levels that supports continued decay heat removal pump (RHR) operations (ref. 1, 2).

Although related, this EAL is concerned with the loss of RCS inventory and not the potential concurrent effects on systems needed for decay heat removal (e.g., loss of a Residual Heat Removal suction point). An increase in RCS temperature caused by a loss of decay heat removal capability is evaluated under IC CA3.

A decreasing RCS level can be indicated by (ref. 1, 2):

- Tygon tube level indicator
- ICC Reactor Vessel Level Monitoring System (RVLMS)

~~For EAL #2, the inability to monitor (reactor vessel/RCS [PWR] or RPV [BWR]) level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation. If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels. Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the (reactor vessel/RCS [PWR] or RPV [BWR]).~~

~~The 15 minute duration for the loss of level indication was chosen because it is half of the EAL duration specified in IC CS1~~

~~If RCS the (reactor vessel/RCS [PWR] or RPV [BWR]) inventory water level continues to lower, then escalation to Site Area Emergency would be via IC CS1.~~

Reference(s):

1. EOP 3505, "Loss of Shutdown Cooling and/or RCS Inventory"
2. OP 3216, "RCS Drain (ICCE)"
3. NEI 99-01 CA1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Significant Loss of RCS inventory

EAL:

CA1.2 Alert

RCS water level **cannot** be monitored for ≥ 15 min. (Note 1)

AND EITHER

- UNPLANNED rise in **any** Table C-1 sump or tank level due to a loss of RCS inventory
- Visual observation of UNISOLABLE RCS leakage

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table C-1 Sumps/Tanks
<ul style="list-style-type: none"> • PRT • CDTT • PDTT • Containment Sump • Unidentified Leakage Sump • Auxiliary Bldg. Sump • ESF Bldg. Sump

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses conditions that are precursors to a loss of the ability to adequately cool irradiated fuel (i.e., a precursor to a challenge to the fuel clad barrier). This condition represents a potential substantial reduction in the level of plant safety.

~~For EAL #1, a lowering of water level below (site specific level) indicates that operator actions have not been successful in restoring and maintaining (reactor vessel/RCS [PWR] or RPV [BWR]) water level. The heat up rate of the coolant will increase as the available water inventory is reduced. A continuing decrease in water level will lead to core uncover.~~

~~Although related, EAL #1 is concerned with the loss of RCS inventory and not the potential concurrent effects on systems needed for decay heat removal (e.g., loss of a Residual Heat Removal suction point). An increase in RCS temperature caused by a loss of decay heat removal capability is evaluated under IC CA3.~~

For this EAL #2, the inability to monitor ~~(reactor vessel/RCS [PWR] or RPV [BWR])~~ level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation. If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels (ref. 1, 2). Sump and/or tank level (Table C-1) changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the ~~(reactor vessel/RCS [PWR] or RPV [BWR])~~ (ref 1, 2, 3).

The 15-minute duration for the loss of level indication was chosen because it is half of the EAL duration specified in IC CS1.

If the ~~(reactor vessel/RCS [PWR] or RPV [BWR])~~ inventory level continues to lower, then escalation to Site Area Emergency would be via IC CS1.

Reference(s):

1. OP 3216, "RCS Drain (ICCE)"
2. EOP 3505, "Loss of Shutdown Cooling and/or RCS Inventory"
3. MPS3 FSAR Section 5.2, "Integrity of Reactor Coolant Pressure Boundary"
4. NEI 99-01 CA1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Loss of RCS inventory affecting core decay heat removal capability

EAL:

CS1.1 Site Area Emergency

RVLMS plenum level reading 19% (Note 12)

Note 12: This EAL is only applicable if a RVLMS 19% sensor is operable.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

This IC addresses a significant and prolonged loss of ~~(reactor vessel/RCS [PWR] or RCS [BWR])~~ inventory control and makeup capability leading to IMMINENT fuel damage. The lost inventory may be due to a RCS component failure, a loss of configuration control or prolonged boiling of reactor coolant. These conditions entail major failures of plant functions needed for protection of the public and thus warrant a Site Area Emergency declaration.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in RCS level. If RCS/~~reactor vessel~~RCS level cannot be restored, fuel damage is probable.

~~Outage/shutdown contingency plans typically provide for re-establishing or verifying CONTAINMENT CLOSURE following a loss of heat removal or RCS inventory control functions. The difference in the specified RCS/reactor vessel levels of EALs 1.b and 2.b reflect the fact that with CONTAINMENT CLOSURE established, there is a lower probability of a fission product release to the environment.~~

~~In EAL 3.a, the 30-minute criterion is tied to a readily recognizable event start time (i.e., the total loss of ability to monitor level), and allows sufficient time to monitor, assess and correlate reactor and plant conditions to determine if core uncover has actually occurred (i.e., to account for various accident progression and instrumentation uncertainties). It also allows sufficient time for performance of actions to terminate leakage, recover inventory control/makeup equipment and/or restore level monitoring.~~

~~The inability to monitor (reactor vessel/RCS [PWR] or RCS [BWR]) level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation. If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels. Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the (reactor vessel/RCS [PWR] or RCS [BWR]).~~

Millstone 3 includes in its Inadequate Core Cooling instrumentation a reactor vessel level monitoring system (RVLMS) that is displayed to the operators and can measure discrete reactor vessel water levels from the top of the reactor vessel head to the top of the core plate (19% sensor - approximately 10 inches above the top of the active fuel). The bottom of this instrument's span in the reactor vessel plenum is the lowest available reactor vessel level indicator and is used in this EAL to represent approximately the top of active fuel (ref. 1).

RVLMS is only required to be operable in modes 1, 2 and 3 (ref. 2). For plant conditions in which RVLMS is disconnected or otherwise inoperable, such as in the Refueling mode, classification should be made based on CS1.2 when RCS water level cannot be monitored.

Thise EALs addresses concerns raised by Generic Letter 88-17, Loss of Decay Heat Removal; SECY 91-283, Evaluation of Shutdown and Low Power Risk Issues; NUREG-1449, Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States; and NUMARC 91-06, Guidelines for Industry Actions to Assess Shutdown Management.

Escalation of the emergency classification level would be via ICs CG1 or AG4RG1.

Reference(s):

1. MPS3 FSAR Section 7.5, "Safety Related Display Instrumentation"
2. MPS3 Technical Specifications Section 3.3.3.6, "Accident Monitoring Instrumentation"
3. EOP 35 FR-C.2, "Response to Degraded Core Cooling Bases"
4. Calculation W3-517-981 RE, "Millstone 3 EOP Setpoint Documentation"
5. NEI 99-01 CS1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Loss of RCS inventory affecting core decay heat removal capability

EAL:

CS1.2 Site Area Emergency

RCS level **cannot** be monitored for ≥ 30 min. (Note 1)

AND

Core uncover is indicated by **any** of the following:

- UNPLANNED rise in **any** Table C-1 sump or tank level of sufficient magnitude to indicate core uncover
- Visual observation of UNISOLABLE RCS leakage of sufficient magnitude to indicate core uncover
- **Any** CTMT area radiation monitor reading > 3 R/hr (Refueling Mode)
- Erratic source range monitor indications

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table C-1 Sumps/Tanks
<ul style="list-style-type: none"> • PRT • CDTT • PDTT • Containment Sump • Unidentified Leakage Sump • Auxiliary Bldg. Sump • ESF Bldg. Sump

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses a significant and prolonged loss of ~~(reactor vessel/RCS [PWR] or RPV [BWR])~~ inventory control and makeup capability leading to IMMINENT fuel damage. The lost inventory may be due to a RCS component failure, a loss of configuration control or prolonged boiling of reactor coolant. These conditions entail major failures of plant functions needed for protection of the public and thus warrant a Site Area Emergency declaration.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in reactor vessel level. If RCS/~~reactor vessel~~ level cannot be restored, fuel damage is probable.

~~Outage/shutdown contingency plans typically provide for re-establishing or verifying CONTAINMENT CLOSURE following a loss of heat removal or RCS inventory control functions. The difference in the specified RCS/reactor vessel levels of EALs 1.b and 2.b reflect the fact that with CONTAINMENT CLOSURE established, there is a lower probability of a fission product release to the environment.~~

In this EAL-3.a, the 30-minute criterion is tied to a readily recognizable event start time (i.e., the total loss of ability to monitor level), and allows sufficient time to monitor, assess and correlate reactor and plant conditions to determine if core uncover has actually occurred (i.e., to account for various accident progression and instrumentation uncertainties). It also allows sufficient time for performance of actions to terminate leakage, recover inventory control/makeup equipment and/or restore level monitoring.

The inability to monitor ~~(reactor vessel/RCS [PWR] or RPV [BWR])~~ level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation (ref. 1, 2). If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels (Table C-1). Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the ~~(reactor vessel/RCS [PWR] or RPV [BWR])~~ (ref. 1, 2, 3).

If the make-up rate to the RCS unexplainably rises above the pre-established rate, a loss of RCS inventory may be occurring even if the source of the leakage cannot be immediately identified. Visual observation of leakage from systems connected to the RCS that cannot be isolated could also be indicative of a loss of RCS inventory.

In the Refueling mode, as water level in the reactor vessel lowers, the dose rate above the core will rise. The dose rate due to this core shine should result in on-scale indications of > 3 R/hr on CTMT area radiation monitors (ref. 4).

Post-TMI accident studies indicated that the installed PWR nuclear instrumentation will operate erratically when the core is uncovered and that this should be used as a tool for making such determinations.

~~These~~ This EALs addresses concerns raised by Generic Letter 88-17, Loss of Decay Heat Removal; SECY 91-283, Evaluation of Shutdown and Low Power Risk Issues; NUREG-1449, Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States; and NUMARC 91-06, Guidelines for Industry Actions to Assess Shutdown Management.

Escalation of the emergency classification level would be via IC CG1 or AG1

Reference(s):

1. OP 3216, "RCS Drain (ICCE)"
2. EOP 3505, "Loss of Shutdown Cooling and/or RCS Inventory"
3. MPS3 FSAR Section 5.2, "Integrity of Reactor Coolant Pressure Boundary"
4. Calculation RA-0078, "Verification of Rad Monitor Response to Core Uncovery"
5. Millstone Unit 3 Radiation Monitor Manual
6. NEI 99-01 CS1

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 1 – RCS Level
Initiating Condition: Loss of RCS inventory affecting fuel clad integrity with containment challenged

EAL:

CG1.1 General Emergency

RVLMS plenum level reading 19% for ≥ 30 min. (Notes 1, 12)

AND

Any Containment Challenge indication, Table C-2

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 6: If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, declaration of a General Emergency is **not** required.

Note 12: This EAL is only applicable if a RVLMS 19% sensor is operable.

Table C-2 Containment Challenge Indications

- CONTAINMENT CLOSURE **not** established (Note 6)
- CTMT hydrogen concentration > 4%
- UNPLANNED increase in CTMT pressure

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

Millstone 3 includes in its Inadequate Core Cooling instrumentation a reactor vessel level monitoring system (RVLMS) that is displayed to the operators and can measure discrete reactor vessel water levels from the top of the reactor vessel head to the top of the core plate (19% sensor - approximately 10 inches above the top of the active fuel). The bottom of this instrument's span in the reactor vessel plenum is the lowest available reactor vessel level indicator and is used in this EAL to represent approximately the top of active fuel (ref. 1, 3).

RVLMS is only required to be operable in modes 1, 2 and 3 (ref. 2). For plant conditions in which RVLMS is disconnected or otherwise inoperable, such as in the Refueling mode, classification should be made based on CG1.2 when RCS water level cannot be monitored.

This IC addresses the inability to restore and maintain ~~reactor vessel~~RCS level above the top of active fuel with containment challenged. This condition represents actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity. Releases can be reasonably expected to exceed EPA PAG exposure levels offsite for more than the immediate site area.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in reactor vessel level. If RCS/~~reactor vessel~~ level cannot be restored, fuel damage is probable.

Three conditions are associated with a challenge to containment's capability to serve as an effective barrier to fission product release:

1. With CONTAINMENT CLOSURE not established, there is a high potential for a direct and unmonitored release of radioactivity to the environment. If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, then declaration of a General Emergency is not required.
2. The existence of an explosive mixture means, at a minimum, that the containment atmospheric hydrogen concentration is sufficient to support a hydrogen burn (i.e., at the lower deflagration limit of 4%). A hydrogen burn will raise containment pressure and could result in collateral equipment damage leading to a loss of containment integrity. It therefore represents a challenge to containment integrity.

In the early stages of a core uncover event, it is unlikely that hydrogen buildup due to a core uncover could result in an explosive gas mixture in containment. However, containment monitoring and/or sampling should be performed to verify this assumption and a General Emergency declared if it is determined that hydrogen concentration has exceeded the minimum necessary to support a hydrogen burn (4%) (ref. 4). If all installed hydrogen gas monitors are out-of-service during an event leading to fuel cladding damage, it may not be possible to obtain a containment hydrogen gas concentration reading as ambient conditions within the containment will preclude personnel access. During periods when installed containment hydrogen gas monitors are out-of-service, operators may use the other listed indications to assess whether or not containment is challenged.

3. Any UNPLANNED rise in containment pressure in the Cold Shutdown or Refueling mode indicates a potential challenge of CONTAINMENT CLOSURE capability due to likely reduced pressure capability of the containment (temporary barriers installed for CONTAINMENT CLOSURE). UNPLANNED containment pressure rise indicates CONTAINMENT CLOSURE cannot be assured and the containment cannot be relied upon as a barrier to fission product release.

~~In EAL 2.b, the 30-minute criterion is tied to a readily recognizable event start time (i.e., the total loss of ability to monitor level), and allows sufficient time to monitor, assess and correlate reactor and plant conditions to determine if core uncover has actually occurred (i.e., to account for various accident progression and instrumentation uncertainties). It also allows sufficient time for performance of actions to terminate leakage, recover inventory control/makeup equipment and/or restore level monitoring.~~

~~The inability to monitor (reactor vessel/RCS [PWR] or RPV [BWR]) level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation. If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels. Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the (reactor vessel/RCS [PWR] or RPV [BWR]).~~

This EAL addresses concerns raised by Generic Letter 88-17, *Loss of Decay Heat Removal*; SECY 91-283, *Evaluation of Shutdown and Low Power Risk Issues*; NUREG-1449, *Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States*; and NUMARC 91-06, *Guidelines for Industry Actions to Assess Shutdown Management*.

Reference(s):

1. MPS3 FSAR Section 7.5, "Safety Related Display Instrumentation"
2. MPS3 Technical Specifications Section 3.3.3.6, "Accident Monitoring Instrumentation"
3. EOP 35 FR-C.2, "Response to Degraded Core Cooling Bases"
4. MP-CALC-ENG-PRA98NQA-01555S3, "SAMG – Computational Aids and Setpoints"
5. NEI 99-01 CG1

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 1 – RCS Level
Initiating Condition: Loss of RCS inventory affecting fuel clad integrity with containment challenged

EAL:

CG1.2 General Emergency

RCS level **cannot** be monitored for ≥ 30 min. (Note 1)

AND

Core uncover is indicated by **any** of the following:

- UNPLANNED rise in **any** Table C-1 sump or tank level of sufficient magnitude to indicate core uncover
- Visual observation of UNISOLABLE RCS leakage of sufficient magnitude to indicate core uncover
- **Any** CTMT area radiation monitor reading > 3 R/hr (Refueling Mode)
- Erratic source range monitor indications

AND

Any Containment Challenge indication, Table C-2

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 6: If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, declaration of a General Emergency is **not** required.

Table C-1 Sumps/Tanks

- PRT
- CDTT
- PDTT
- Containment Sump
- Unidentified Leakage Sump
- Auxiliary Bldg. Sump
- ESF Bldg. Sump

Table C-2 Containment Challenge Indications

- CONTAINMENT CLOSURE **not** established (Note 6)
- CTMT hydrogen concentration > 4%
- UNPLANNED increase in CTMT pressure

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses the inability to restore and maintain reactor vessel/RCS level above the top of active fuel with containment challenged. This condition represents actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity. Releases can be reasonably expected to exceed EPA PAG exposure levels offsite for more than the immediate site area.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in reactor vessel level. If RCS/reactor vessel level cannot be restored, fuel damage is probable.

The inability to monitor (reactor vessel/RCS [PWR] or RCS [BWR]) level may be caused by instrumentation and/or power failures, or water level dropping below the range of available

instrumentation (ref. 1, 2). If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels. Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the ~~(reactor vessel/RCS [PWR] or RPV [BWR])~~ (ref. 2, 3, 4).

If the make-up rate to the RCS unexplainably rises above the pre-established rate, a loss of RCS inventory may be occurring even if the source of the leakage cannot be immediately identified. Visual observation of leakage from systems connected to the RCS that cannot be isolated could also be indicative of a loss of RCS inventory.

In the Refueling mode, as water level in the reactor vessel lowers, the dose rate above the core will rise. The dose rate due to this core shine should result in on-scale indications on area radiation monitors. A reading > 3 R/hr on any CTMT area radiation monitor such as RMS-4A and RMS-5A is indicative of likely core uncover while in the Refueling mode. Although RMS-4A and RMS-5A are not in direct line of sight of the reactor core when the vessel head is removed, scatter from the air and containment dome would result in readings of approximately 3 R/hr when water level is at the top of active fuel (ref. 4, 5).

Post-TMI accident studies indicated that the installed PWR nuclear instrumentation will operate erratically when the core is uncovered and that this should be used as a tool for making such determinations.

~~In EAL 2.b, t~~ The 30-minute criterion is tied to a readily recognizable event start time (i.e., the total loss of ability to monitor level), and allows sufficient time to monitor, assess and correlate reactor and plant conditions to determine if core uncover has actually occurred (i.e., to account for various accident progression and instrumentation uncertainties). It also allows sufficient time for performance of actions to terminate leakage, recover inventory control/makeup equipment and/or restore level monitoring.

Three conditions are associated with a challenge to containment's capability to serve as an effective barrier to fission product release:

1. With CONTAINMENT CLOSURE not established, there is a high potential for a direct and unmonitored release of radioactivity to the environment. If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, then declaration of a General Emergency is not required.
2. The existence of an explosive mixture means, at a minimum, that the containment atmospheric hydrogen concentration is sufficient to support a hydrogen burn (i.e., at the lower deflagration limit of 4%). A hydrogen burn will raise containment pressure and could result in collateral equipment damage leading to a loss of containment integrity. It therefore represents a challenge to containment integrity.

In the early stages of a core uncover event, it is unlikely that hydrogen buildup due to a core uncover could result in an explosive gas mixture in containment. However, containment monitoring and/or sampling should be performed to verify this assumption and a General Emergency declared if it is determined that hydrogen concentration has exceeded the minimum necessary to support a hydrogen burn (4%) (ref. 6). If all installed hydrogen gas monitors are out-of-service during an event leading to fuel cladding damage, it may not be possible to obtain a containment hydrogen gas

concentration reading as ambient conditions within the containment will preclude personnel access. During periods when installed containment hydrogen gas monitors are out-of-service, operators may use the other listed indications to assess whether or not containment is challenged.

3. Any UNPLANNED rise in containment pressure in the Cold Shutdown or Refueling mode indicates a potential challenge of CONTAINMENT CLOSURE capability due to likely reduced pressure capability of the containment (temporary barriers installed for CONTAINMENT CLOSURE). UNPLANNED containment pressure rise indicates CONTAINMENT CLOSURE cannot be assured and the containment cannot be relied upon as a barrier to fission product release.

Thise EALs addresses concerns raised by Generic Letter 88-17, *Loss of Decay Heat Removal*; SECY 91-283, *Evaluation of Shutdown and Low Power Risk Issues*; NUREG-1449, *Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States*; and NUMARC 91-06, *Guidelines for Industry Actions to Assess Shutdown Management*.

Reference(s):

1. OP 3216, "RCS Drain (ICCE)"
2. EOP 3505, "Loss of Shutdown Cooling and/or RCS Inventory"
3. MPS3 FSAR Section 5.2, "Integrity of Reactor Coolant Pressure Boundary"
4. Calculation RA-0078, "Verification of Rad Monitor Response to Core Uncovery"
5. Millstone Unit 3 "Radiation Monitor Manual"
6. MP-CALC-ENG-PRA98NQA-01555S3, "SAMG – Computational Aids and Setpoints"
7. NEI 99-01 CG1

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 2 – Loss of Emergency AC Power
Initiating Condition: Loss of **all** but one AC power source to emergency buses for 15 minutes or longer

EAL:

CU2.1 Unusual Event

AC power capability, Table C-3, to 4.16 kV emergency buses 34C and 34D reduced to a single power source for ≥ 15 min. (Note 1)

AND

Any additional single power source failure will result in loss of **all** AC power to SAFETY SYSTEMS

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table C-3 AC Power Sources	
Offsite	
<ul style="list-style-type: none"> • Unit 3 Normal Station Service Transformer A (NSST) • Unit 3 Reserve Station Service Transformer (RSST) 	
Onsite	
<ul style="list-style-type: none"> • EDG A • EDG B • SBO Diesel Generator (if already aligned) 	

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling, DEF - Defueled

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

Table C-3 provides a list of offsite and onsite AC electrical power sources credited for this EAL. The AC power source (SBO Diesel Generator) annotated "(if already aligned)" requires more than 15 minutes to establish and therefore is only credited if the source was already aligned at the time of AC power loss.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC describes a significant degradation of offsite and onsite AC power sources such that any additional single failure would result in a loss of all AC power to SAFETY SYSTEMS. In this condition, the sole AC power source may be powering one, or more than one, train of safety-related equipment.

When in the cold shutdown, refueling, or defueled mode, this condition is not classified as an Alert because of the increased time available to restore another power source to service. Additional time is available due to the reduced core decay heat load, and the lower temperatures and pressures in various plant systems. Thus, when in these modes, this condition is considered to be a potential degradation of the level of safety of the plant.

An "AC power source" is a source recognized in AOPs and EOPs, and capable of supplying required power to an emergency bus. Some examples of this condition are presented below.

- A loss of all offsite power with a concurrent failure of all but one emergency power source (e.g., an onsite diesel generator).
- A loss of all offsite power and loss of all emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being back-fed from the unit main generator transformer.
- A loss of emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being back-fed from an offsite power source.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of power.

The subsequent loss of the remaining single power source would escalate the event to an Alert in accordance with IC CA2.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.

- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

This cold condition EAL is equivalent to the hot condition EAL MA1.1.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS FSAR Section 8.2, "Off Site Power System"
3. MPS FSAR Section 8.3, "On Site Power System"
4. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
5. NEI 99-01 CU2

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 2 – Loss of Emergency AC Power

Initiating Condition: Loss of **all** offsite and **all** onsite AC power to emergency buses for 15 minutes or longer

EAL:

CA2.1 Alert

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 34C and 34D for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling, DEF - Defueled

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators, provided it can be aligned within the 15 minute classification criteria.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC addresses a total loss of AC power that compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink.

When in the cold shutdown, refueling, or defueled mode, this condition is not classified as a Site Area Emergency because of the increased time available to restore an emergency bus to service. Additional time is available due to the reduced core decay heat load, and the lower temperatures and pressures in various plant systems. Thus, when in these modes, this condition represents an actual or potential substantial degradation of the level of safety of the plant.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs CS1 or AS1RS1.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A. MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

This cold condition EAL is equivalent to the hot condition EAL MS1.1.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS FSAR Section 8.2, "Off Site Power System"
3. MPS FSAR Section 8.3, "On Site Power System"
4. EOP 3501, "Loss of All AC Power (Mode 5, 6 and Zero)"
5. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
6. NEI 99-01 CU2

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 3 – RCS Temperature

Initiating Condition: UNPLANNED increase in RCS temperature

EAL:

CU3.1 Unusual Event

UNPLANNED rise in RCS temperature to > 200°F

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

UNPLANNED-. A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

In the absence of reliable RCS temperature indication caused by the loss of decay heat removal capability, classification should be based on time to boil data when in Mode 6 or the RCS is not intact in Mode 5 (ref. 1). If the RCS is intact, classification should be based on the RCS pressure rise criteria of CA3.1. Guidance for calculating RCS time to 200°F is provided on the Shutdown Safety Assessment Checklist (ref. 3).

This ~~IC-EAL~~ addresses an UNPLANNED increase in RCS temperature above the Technical Specification cold shutdown temperature limit ~~or the inability to determine RCS temperature and level, and~~ represents a potential degradation of the level of safety of the plant (ref. 1). If the RCS is not intact and CONTAINMENT CLOSURE is not established during this event, the ~~Emergency Director/DSEO/ADTS~~ should also refer to ~~IC-EAL~~ CA3.1.

A momentary UNPLANNED excursion above the Technical Specification cold shutdown temperature limit when the heat removal function is available does not warrant a classification.

~~EAL #1~~This EAL This EAL involves a loss of decay heat removal capability, or an addition of heat to the RCS in excess of that which can currently be removed, such that reactor coolant

temperature cannot be maintained below the cold shutdown temperature limit specified in Technical Specifications. During this condition, there is no immediate threat of fuel damage because the core decay heat load has been reduced since the cessation of power operation.

During an outage, the level in the reactor vessel will normally be maintained at or above the reactor vessel flange. Refueling evolutions that lower water level below the reactor vessel flange are carefully planned and controlled. A loss of forced decay heat removal at reduced inventory may result in a rapid increase in reactor coolant temperature depending on the time after shutdown (ref. 3).

~~———— EAL #2 reflects a condition where there has been a significant loss of instrumentation capability necessary to monitor RCS conditions and operators would be unable to monitor key parameters necessary to assure core decay heat removal. During this condition, there is no immediate threat of fuel damage because the core decay heat load has been reduced since the cessation of power operation.~~

~~———— Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.~~

Escalation to Alert would be via IC CA1 based on an inventory loss or IC CA3 based on exceeding plant configuration-specific time criteria.

Reference(s):

1. Technical Specifications Table 1.2
2. EOP 3505, "Loss of Shutdown Cooling And/Or RCS Inventory"
3. OU-M3-201, "Shutdown Safety Assessment Checklist"
4. NEI 99-01 CU3

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 3 – RCS Temperature

Initiating Condition: UNPLANNED rise in RCS temperature

EAL:

CU3.2 Unusual Event

Loss of **all** RCS temperature and RCS water level indication for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6- Refueling

Definition(s):

CONTAINMENT CLOSURE - The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

Basis:

This ~~IC EAL~~ EAL addresses an UNPLANNED increase in RCS temperature above the Technical Specification cold shutdown temperature limit, or the inability to determine RCS temperature and level, ~~and~~ and represents a potential degradation of the level of safety of the plant. If the RCS is not intact and CONTAINMENT CLOSURE is not established during this event, the ~~Emergency Director~~ DSEO/ADTS should also refer to ~~IC EAL~~ CA3.1.

~~— A momentary UNPLANNED excursion above the Technical Specification cold shutdown temperature limit when the heat removal function is available does not warrant a classification.~~

~~— EAL #1 involves a loss of decay heat removal capability, or an addition of heat to the RCS in excess of that which can currently be removed, such that reactor coolant temperature cannot be maintained below the cold shutdown temperature limit specified in Technical Specifications. During this condition, there is no immediate threat of fuel damage because the core decay heat load has been reduced since the cessation of power operation.~~

~~— During an outage, the level in the reactor vessel will normally be maintained above the reactor vessel flange. Refueling evolutions that lower water level below the reactor vessel flange are carefully planned and controlled. A loss of forced decay heat removal at reduced~~

inventory may result in a rapid increase in reactor coolant temperature depending on the time after shutdown.

~~EAL #2~~This ~~EAL~~This EAL reflects a condition where there has been a significant loss of instrumentation capability necessary to monitor RCS conditions and operators would be unable to monitor key parameters necessary to assure core decay heat removal. During this condition, there is no immediate threat of fuel damage because the core decay heat load has been reduced since the cessation of power operation.

RCS level indications include (ref. 2):

- 3RCS-LT51A RHR A Transmitter
- 3RCS-LT51B RHR B Transmitter
- 3RCS-LT51C Letdown Transmitter
- ICC Reactor Vessel Level Monitoring System (RVLMS)
- Tygon Tube

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation to Alert would be via IC CA1 based on an inventory loss or IC CA3 based on exceeding plant configuration-specific time criteria.

Reference(s):

1. EOP 3505, "Loss of Shutdown Cooling And/Or RCS Inventory"
2. OP 3216, "Reactor Coolant System Drain (ICCE)"
3. NEI 99-01 CU3

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 3 – RCS Temperature

Initiating Condition: Inability to maintain plant in cold shutdown

EAL:

CA3.1 Alert

UNPLANNED rise in RCS temperature to > 200°F for > Table C-4 duration
(Notes 1, 13)

OR

UNPLANNED RCS pressure rise > 10 psi
(This pressure threshold does not apply during water-solid plant conditions)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded.

Note 13: If an RCS heat removal system is in operation within the applicable Table C-4 heat-up duration and RCS temperature is being reduced, the EAL is **not** applicable.

Table C-4 RCS Heat-up Duration Thresholds		
RCS Status	CONTAINMENT CLOSURE Status	Heat-up Duration
Intact <u>AND</u> not reduced inventory		60 min.
Not intact <u>OR</u> reduced inventory	Established	20 min.
	Not established	0 min.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

UNPLANNED-. A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

In the absence of reliable RCS temperature indication caused by the loss of decay heat removal capability, classification should be based on the RCS pressure rise criteria when the RCS is intact in Mode 4 or based on time to boil data when in Mode 5 or the RCS is not intact in Mode 4 (ref. 2, 3). Guidance for calculating RCS time to 200°F is provided on the Shutdown Safety Assessment Checklist (ref.4).

The RCS is considered to be at reduced inventory when RCS level is below 19 ft. 6 in. elevation (ref. 3).

This ~~IC~~ EAL addresses conditions involving a loss of decay heat removal capability or an addition of heat to the RCS in excess of that which can currently be removed. Either condition represents an actual or potential substantial degradation of the level of safety of the plant.

A momentary UNPLANNED excursion above the Technical Specification cold shutdown temperature limit when the heat removal function is available does not warrant a classification.

The RCS Heat-up Duration Thresholds table addresses an increase in RCS temperature when CONTAINMENT CLOSURE is established but the RCS is not intact, or RCS inventory is reduced (e.g., mid-loop operation in PWRs). The 20-minute criterion was included to allow time for operator action to address the temperature increase.

The RCS Heat-up Duration Thresholds table also addresses an increase in RCS temperature with the RCS intact. The status of CONTAINMENT CLOSURE is not crucial in this condition since the intact RCS is providing a high pressure barrier to a fission product release. The 60-minute time frame should allow sufficient time to address the temperature increase without a substantial degradation in plant safety.

Finally, in the case where there is an increase in RCS temperature, the RCS is not intact or is at reduced inventory ~~[PWR]~~, and CONTAINMENT CLOSURE is not established, no heat-up duration is allowed (i.e., 0 minutes). This is because 1) the evaporated reactor coolant may be released directly into the containment atmosphere and subsequently to the environment, and 2) there is reduced reactor coolant inventory above the top of irradiated fuel.

The RCS pressure rise threshold ~~EAL #2~~ provides a pressure-based indication of RCS heat-up in the absence of RCS temperature monitoring capability.

Escalation of the emergency classification level would be via IC CS1 or ~~AS4~~ RS1.

Reference(s):

1. Technical Specifications Table 1.2
2. EOP 3505, "Loss of Shutdown Cooling And/Or RCS Inventory"
3. OP 3216, "Reactor Coolant System Drain (ICCE)"
4. OU-M3-201, "Shutdown Safety Assessment Checklist"
5. NEI 99-01 CA3

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 4 – Loss of Vital DC Power

Initiating Condition: Loss of vital DC power for 15 minutes or longer

EAL:

CU4.1 Unusual Event

Indicated voltage is < 105 VDC on **required** vital 125 VDC battery bus 1 **OR** 2 for ≥15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis

The vital (Class 1E) 125 V DC power system consists of four physically and electrically separated channels (buses). Each bus has a 60 cell battery bank with a two-hour rating before reaching the minimum bus voltage of 105 Volts (1.75 volts/cell).

Battery buses 1 and 2 power reactor protection, engineered safety features, vital loads and instrumentation (ref. 1, 2, 3). Battery buses 3 and 4 are not credited for this EAL.

This IC addresses a loss of vital DC power which compromises the ability to monitor and control operable SAFETY SYSTEMS when the plant is in the cold shutdown or refueling mode. In these modes, the core decay heat load has been significantly reduced, and coolant system temperatures and pressures are lower; these conditions increase the time available to restore a vital DC bus to service. Thus, this condition is considered to be a potential degradation of the level of safety of the plant.

As used in this EAL, "required" means the vital DC buses necessary to support operation of the in-service, or operable, train or trains of SAFETY SYSTEM equipment. For example, if Train A is out-of-service (inoperable) for scheduled outage maintenance work and Train B is in-service (operable), then a loss of vital DC power affecting Train B would require the

declaration of an Unusual Event. A loss of vital DC power to Train A would not warrant an emergency classification.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Depending upon the event, escalation of the emergency classification level would be via IC CA1 or CA3, or an IC in ~~Recognition~~-Category AM.

This cold condition EAL is equivalent to the hot condition EAL MS2.1.

Reference(s):

1. MPS FSAR Section 8.3.2, "DC Power Systems"
2. AOP 3563, "Loss of DC Bus Power"
3. Technical Specifications Section 3.8.2.2, "D.C. Sources – Shutdown"
4. NEI 99-01 CU4

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 5 – Loss of Communications
Initiating Condition: Loss of **all** onsite or offsite communications capabilities
EAL:

CU5.1 Unusual Event

Loss of **all** Table C-5 onsite communication methods

OR

Loss of **all** Table C-5 State and local agency communication methods

OR

Loss of **all** Table C-5 NRC communication methods

Table C-5 Communication Methods			
System	Onsite	State/ Local	NRC
ENRS / ARCOS		X	
Station Radio System	X	X	
Plant Phone System	X	X	
Public Address System	X		
Gaitronics / Maintenance Jacks	X		
Federal Telephone System (ENS)			X
Commercial Telephone System		X	X
Satellite Phones		X	X
Dedicated Hotlines		X	

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling, DEF – Defueled

Definition(s):

None

Basis:

This IC addresses a significant loss of on-site or offsite communications capabilities. While not a direct challenge to plant or personnel safety, this event warrants prompt notifications to OROs State and local agencies and the NRC.

This IC should be assessed only when extraordinary means are being utilized to make communications possible (e.g., use of non-plant, privately owned equipment, relaying of on-site information via individuals or multiple radio transmission points, individuals being sent to offsite locations, etc.).

The first EAL condition EAL #1 addresses a total loss of the communications methods used in support of routine plant operations.

The second EAL condition EAL #2 addresses a total loss of the communications methods used to notify all OROs State and local agencies of an emergency declaration. The OROs State and local agencies referred to here are ~~(see Developer Notes)~~ the State of Connecticut and affected local communities.

The third EAL condition EAL #3 addresses a total loss of the communications methods used to notify the NRC of an emergency declaration.

This cold condition EAL is equivalent to the hot condition EAL MU7.1.

Reference(s):

1. MPS Emergency Plan Section 7.9, "Communication Systems"
2. MP-26-EPI-FAP07, "Notifications and Communications"
3. NEI 99-01 CU5

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 6 – Hazardous Event Affecting Safety Systems
Initiating Condition: Hazardous event affecting SAFETY SYSTEMS needed for the current operating mode

EAL:

CA6.1 Alert

The occurrence of **any** Table C-6 hazardous event

AND

Event damage has caused indications of degraded performance on one train of a SAFETY SYSTEM needed for the current operating mode

AND EITHER:

- Event damage has caused indications of degraded performance to the second train of the SAFETY SYSTEM needed for the current operating mode
- Event damage has resulted in VISIBLE DAMAGE to the second train of the SAFETY SYSTEM needed for the current operating mode

(Notes 9, 10)

Note 9: If the affected SAFETY SYSTEM train was already inoperable or out of service before the hazardous event occurred, then emergency classification is **not** warranted.

Note 10: If the hazardous event **only** resulted in VISIBLE DAMAGE, with **no** indications of degraded performance to at least one train of a SAFETY SYSTEM, then this emergency classification is **not** warranted.

Table C-6 Hazardous Events

- Seismic event (earthquake)
- Internal or external FLOODING event
- High winds or tornado strike
- FIRE
- EXPLOSION
- Other events with similar hazard characteristics as determined by the DSEO/ADTS

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

EXPLOSION - A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding, arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

VISIBLE DAMAGE - Damage to a SAFETY SYSTEM train that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected SAFETY SYSTEM train.

Basis:

This IC addresses a hazardous event that causes damage to SAFETY SYSTEMS needed for the current operating mode. In order to provide the appropriate context for consideration of an ALERT classification, the hazardous event must have caused indications of degraded SAFETY SYSTEM performance in one train, and there must be either indications of performance issues with the second SAFETY SYSTEM train or VISIBLE DAMAGE to the second train such that the potential exists for this second SAFETY SYSTEM train to have performance issues. In other words, in order for this EAL to be classified, the hazardous event must occur, at least one SAFETY SYSTEM train must have indications of degraded performance, and the second SAFETY SYSTEM train must have indications of degraded performance or VISIBLE DAMAGE such that the potential exists for performance issues. Note that this second SAFETY SYSTEM train is from the same SAFETY SYSTEM that has indications of degraded performance; commercial nuclear power plants are designed to be able to support single system issues without compromising public health and safety from radiological events.

Indications of degraded performance addresses damage to a SAFETY SYSTEM train that is in service/operation since indications for it will be readily available. The indications of degraded performance should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

VISIBLE DAMAGE addresses damage to a SAFETY SYSTEM train that is not in service/operation and that potentially could cause performance issues. Operators will make this determination based on the totality of available event and damage report information. This is intended to be a brief assessment not requiring lengthy analysis or quantification of the damage. This VISIBLE DAMAGE should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

~~This IC addresses a hazardous event that causes damage to a SAFETY SYSTEM, or a structure containing SAFETY SYSTEM components, needed for the current operating mode. This condition significantly reduces the margin to a loss or potential loss of a fission product barrier, and therefore represents an actual or potential substantial degradation of the level of safety of the plant.~~

~~EAL 1.b.1 addresses damage to a SAFETY SYSTEM train that is in service/operation since indications for it will be readily available. The indications of degraded performance should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.~~

~~EAL 1.b.2 addresses damage to a SAFETY SYSTEM component that is not in service/operation or readily apparent through indications alone, or to a structure containing SAFETY SYSTEM components. Operators will make this determination based on the totality of available event and damage report information. This is intended to be a brief assessment not requiring lengthy analysis or quantification of the damage.~~

An event affecting equipment common to two or more trains of a safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the common equipment) should be classified as an Alert under this EAL, as appropriate to the plant mode. By affecting the functionality of multiple trains of a safety system, the loss of the common equipment effectively meets the two-train impact criteria that underlie the EALs and bases.

An event affecting a single-train safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the one train) would not be classified under this EAL because the two-train impact criteria that underlie the EALs and bases would not be met. If an event affects a single-train safety system, then the emergency classification should be made based on plant parameters/symptoms meeting the EALs for another IC. Depending upon the circumstances, classification may also occur based on Shift Manager/DSEO/ADTS judgement.

An event that affects two trains of a safety system (e.g., one train has indications of degraded performance and the other VISIBLE DAMAGE) that also has one or more additional trains should be classified as an Alert under this EAL, as appropriate to the plant mode. This approach maintains consistency with the two-train impact criteria that underlie the EALs and bases, and is warranted because the event was severe enough to affect the functionality of two trains of a safety system despite plant design criteria associated with system and system train separation and protection. Such an event may have caused other plant impacts that are not immediately apparent.

Escalation of the emergency classification level would be via IC CS1 or AS4RS1.

This cold condition EAL is equivalent to the hot condition EAL MA8.1.

Reference(s):

Millstone Power Station Unit 3
Emergency Action Level Technical Bases Document
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1. AOP 3569, "Severe Weather Conditions"
2. AOP 3570, "Earthquake"
3. EP FAQ 2016-002
4. NEI 99-01 CA6

Category E – Independent Spent Fuel Storage Installation (ISFSI)

EAL Group: ANY (EALs in this category are applicable to any plant condition, hot or cold.)

An independent spent fuel storage installation (ISFSI) is a complex that is designed and constructed for the interim storage of spent nuclear fuel and other radioactive materials associated with spent fuel storage. A significant amount of the radioactive material contained within a canister must escape its packaging and enter the biosphere for there to be a significant environmental effect resulting from an accident involving the dry storage of spent nuclear fuel.

A Notification of Unusual Event is declared on the basis of the occurrence of an event of sufficient magnitude that a loaded cask CONFINEMENT BOUNDARY is damaged or violated.

The MPS ISFSI is located wholly within the MPS plant PROTECTED AREA. Therefore any security event related to the ISFSI is classified under Category H1 security event related EALs.

Category: ISFSI

Subcategory: Confinement Boundary

Initiating Condition: Damage to a loaded cask CONFINEMENT BOUNDARY

EAL:

EU1.1 Unusual Event

Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by an on-contact radiation reading on the surface of a loaded spent fuel cask (HSM) > **any** of the following:

- 1,700 mrem/hr on the HSM front bird screen
- 400 mrem/hr on the outside HSM door
- 12 mrem/hr on the HSM end shield wall

Mode Applicability:

All

Definition(s):

CONFINEMENT BOUNDARY- The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielded Canister (DSC).

INDEPENDENT SPENT FUEL STORAGE INSTALLATION (ISFSI): A complex that is designed and constructed for the interim storage of spent nuclear fuel and other radioactive materials associated with spent fuel storage.

Basis:

This IC addresses an event that results in damage to the CONFINEMENT BOUNDARY of a storage cask containing spent fuel. It applies to irradiated fuel that is licensed for dry storage beginning at the point that the loaded storage cask is sealed. The issues of concern are the creation of a potential or actual release path to the environment, degradation of one or more fuel assemblies due to environmental factors, and configuration changes which could cause challenges in removing the cask or fuel from storage.

The existence of "damage" is determined by radiological survey. The specified EAL threshold values correspond to 2 times the NUHOMS Horizontal Storage Module (HSM) technical specification external surface dose rate limits (ref. 2). The technical specification multiple of "2 times", which is also used in Recognition-Category A-R IC AU1RU1, is used here to distinguish between non-emergency and emergency conditions. The emphasis for this classification is the degradation in the level of safety of the spent fuel cask and not the magnitude of the associated dose or dose rate. It is recognized that in the case of extreme damage to a loaded cask, the fact that the "on-contact" dose rate limit is exceeded may be determined based on measurement of a dose rate at some distance from the cask (ref. 1, 2,3).

Security-related events for ISFSIs are covered under Category H ICs-HU1 and HA1.

Reference(s):

1. NUH-003, "UFSAR for the Horizontal Modular Storage System for Irradiated Fuel"
2. MPS2 ETE-NAF-2010-0004, "MPS Unit 2 ISFSI 10 CFR72.212 Evaluation Report Attachment 1 Section 5.4 HSM or HSM-H Dose Rate Evaluation Program"
3. RPM 2.5.9, "Dry Shielded Canisters (DSC) Surveys (ISFSI)"
4. NEI 99-01 E-HU1

Category F – Fission Product Barrier Degradation

EAL Group: Hot Conditions (RCS temperature > 200°F); EALs in this category are applicable only in one or more hot operating modes.

EALs in this category represent threats to the defense in depth design concept that precludes the release of highly radioactive fission products to the environment. This concept relies on multiple physical barriers any one of which, if maintained intact, precludes the release of significant amounts of radioactive fission products to the environment. The primary fission product barriers are:

- A. Fuel Clad Barrier (FC): The Fuel Clad Barrier consists of the cladding material that contains the fuel pellets.
- B. Reactor Coolant System Barrier (RCS): The RCS Barrier includes the RCS primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary isolation valves.
- C. Containment Barrier (CTMT): The Containment Barrier includes the containment building and connections up to and including the outermost containment isolation valves. This barrier also includes the main steam, feedwater, and blowdown line extensions outside the containment building up to and including the outermost secondary side isolation valve. Containment Barrier thresholds are used as criteria for escalation of the Emergency Classification Level (ECL) from an Alert to a Site Area Emergency or a General Emergency.

The EALs in this category require evaluation of the loss and potential loss thresholds listed in the fission product barrier matrix of Table F-1. "Loss" and "Potential Loss" signify the relative damage and threat of damage to the barrier. "Loss" means the barrier no longer assures containment of radioactive materials. "Potential Loss" means integrity of the barrier is threatened and could be lost if conditions continue to degrade. The number of barriers that are lost or potentially lost and the following criteria determine the appropriate emergency classification level:

Alert:

Any loss or any potential loss of either Fuel Clad or RCS Barrier

Site Area Emergency:

Loss or potential loss of any two barriers

General Emergency:

Loss of any two barriers and loss or potential loss of third barrier

The logic used for emergency classification based on fission product barrier monitoring should reflect the following considerations:

- The Fuel Clad Barrier and the RCS Barrier are weighted more heavily than the Containment Barrier.
- Unusual Event ICs associated with RCS and Fuel Clad Barriers are addressed under System Malfunction ICs.

- For accident conditions involving a radiological release, evaluation of the fission product barrier thresholds will need to be performed in conjunction with dose assessments to ensure correct and timely escalation of the emergency classification. For example, an evaluation of the fission product barrier thresholds may result in a Site Area Emergency classification while a dose assessment may indicate that an EAL for General Emergency IC AG1-~~RG1~~ has been exceeded.
- The fission product barrier thresholds specified within a scheme reflect plant-specific MPS3 design and operating characteristics.
- As used in this category, the term RCS leakage encompasses not just those types defined in Technical Specifications but also includes the loss of RCS mass to any location— inside the containment, an interfacing system, or outside of the containment. The release of liquid or steam mass from the RCS due to the as-designed/expected operation of a relief valve is not considered to be RCS leakage.
- At the Site Area Emergency level, EAL users should maintain cognizance of how far present conditions are from meeting a threshold that would require a General Emergency declaration. For example, if the Fuel Clad and RCS fission product barriers were both lost, then there should be frequent assessments of containment radioactive inventory and integrity. Alternatively, if both the Fuel Clad and RCS fission product barriers were potentially lost, the ~~Emergency Director~~ DSEO/ADTS would have more assurance that there was no immediate need to escalate to a General Emergency.

Category: Fission Product Barrier Degradation
Subcategory: N/A
Initiating Condition: Any loss or any potential loss of either Fuel Clad or RCS
EAL:

FA1.1 Alert

Any loss or any potential loss of **EITHER** Fuel Clad or RCS barrier (Table F-1)

Mode Applicability:

1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown

Definition(s):

None

Basis:

Fuel Clad, RCS and Containment comprise the fission product barriers. Table F-1 lists the fission product barrier thresholds, bases and references.

At the Alert classification level, Fuel Clad and RCS barriers are weighted more heavily than the Containment barrier. Unlike the Containment barrier, loss or potential loss of either the Fuel Clad or RCS barrier may result in the relocation of radioactive materials or degradation of core cooling capability. Note that the loss or potential loss of Containment barrier in combination with loss or potential loss of either Fuel Clad or RCS barrier results in declaration of a Site Area Emergency under EAL FS1.1

Reference(s):

1. NEI 99-01 FA1

Category: Fission Product Barrier Degradation
Subcategory: N/A
Initiating Condition: Loss or potential loss of **any** two barriers
EAL:

FS1.1 Site Area Emergency

Loss or potential loss of **any** two barriers (Table F-1)

Mode Applicability:

1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

Fuel Clad, RCS and Containment comprise the fission product barriers. Table F-1 lists the fission product barrier thresholds, bases and references.

At the Site Area Emergency classification level, each barrier is weighted equally. A Site Area Emergency is therefore appropriate for any combination of the following conditions:

- One barrier loss and a second barrier loss (i.e., loss - loss)
- One barrier loss and a second barrier potential loss (i.e., loss - potential loss)
- One barrier potential loss and a second barrier potential loss (i.e., potential loss - potential loss)

At the Site Area Emergency classification level, the ability to dynamically assess the proximity of present conditions with respect to the threshold for a General Emergency is important. For example, the existence of Fuel Clad and RCS Barrier loss thresholds in addition to offsite dose assessments would require continual assessments of radioactive inventory and Containment integrity in anticipation of reaching a General Emergency classification. Alternatively, if both Fuel Clad and RCS potential loss thresholds existed, they would have greater assurance that escalation to a General Emergency is less IMMINENT.

Reference(s):

1. NEI 99-01 FS1

Category: Fission Product Barrier Degradation

Subcategory: N/A

Initiating Condition: Loss of **any** two barriers and loss or potential loss of the third barrier

EAL:

FG1.1 General Emergency

Loss of **any** two barriers

AND

Loss or potential loss of the third barrier (Table F-1)

Mode Applicability:

1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown

Definition(s):

None

Basis:

Fuel Clad, RCS and Containment comprise the fission product barriers. Table F-1 lists the fission product barrier thresholds, bases and references.

At the General Emergency classification level each barrier is weighted equally. A General Emergency is therefore appropriate for any combination of the following conditions:

- Loss of Fuel Clad, RCS and Containment Barriers
- Loss of Fuel Clad and RCS Barriers with potential loss of Containment Barrier
- Loss of RCS and Containment Barriers with potential loss of Fuel Clad Barrier
- Loss of Fuel Clad and Containment Barriers with potential loss of RCS Barrier

Reference(s):

1. NEI 99-01 FG1

Table F-1 Fission Product Barrier Threshold Matrix & Bases

Table F-1 lists the threshold conditions that define the Loss and Potential Loss of the three fission product barriers (Fuel Clad, Reactor Coolant System, and Containment). The table is structured so that each of the three barriers occupies adjacent columns. Each fission product barrier column is further divided into two columns; one for Loss thresholds and one for Potential Loss thresholds.

The first column of the table (to the left of the Fuel Clad Loss column) lists the categories (types) of fission product barrier thresholds. The fission product barrier categories are:

- A. RCS or SG Tube Leakage
- B. Inadequate Heat removal
- C. CTMT Radiation / RCS Activity
- D. CTMT Integrity or Bypass
- E. DSEO/ADTS Judgment

Each category occupies a row in Table F-1 thus forming a matrix defined by the categories. The intersection of each row with each Loss/Potential Loss column forms a cell in which one or more fission product barrier thresholds appear. If NEI 99-01 does not define a threshold for a barrier Loss/Potential Loss, the word "None" is entered in the cell.

Thresholds are assigned sequential numbers within each barrier column beginning with number one.

If a cell in Table F-1 contains more than one numbered threshold, each of the numbered thresholds, if exceeded, signifies a Loss or Potential Loss of the barrier. It is not necessary to exceed all of the thresholds in a category before declaring a barrier Loss/Potential Loss.

Subdivision of Table F-1 by category facilitates association of plant conditions to the applicable fission product barrier Loss and Potential Loss thresholds. This structure promotes a systematic approach to assessing the classification status of the fission product barriers.

When equipped with knowledge of plant conditions related to the fission product barriers, the EAL-user first scans down the category column of Table F-1, locates the likely category and then reads across the fission product barrier Loss and Potential Loss thresholds in that category to determine if a threshold has been exceeded. If a threshold has not been exceeded, the EAL-user proceeds to the next likely category and continues review of the thresholds in the new category.

If the EAL-user determines that any threshold has been exceeded, by definition, the barrier is lost or potentially lost – even if multiple thresholds in the same barrier column are exceeded, only that one barrier is lost or potentially lost. The EAL-user must examine each of the three fission product barriers to determine if other barrier thresholds in the category are lost or potentially lost. For example, if containment radiation is sufficiently high, a Loss of the Fuel Clad and RCS Barriers and a Potential Loss of the Containment Barrier can occur. Barrier Losses and Potential Losses are then applied to the algorithms given in EALs FG1.1, FS1.1, and FA1.1 to determine the appropriate emergency classification.

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Table F-1 Fission Product Barrier Threshold Matrix

Category	Fuel Clad Barrier (FC)		Reactor Coolant System Barrier (RCS)		Containment Barrier (CTMT)	
	Loss	Potential Loss	Loss	Potential Loss	Loss	Potential Loss
A RCS or SG Tube Leakage	None	None	1. An automatic or manual Safety Injection (SI) actuation required by <u>EITHER</u> : <ul style="list-style-type: none">UNISOLABLE RCS leakageSG tube RUPTURE	1. Operation of a second charging pump is required by <u>EITHER</u> : <ul style="list-style-type: none">UNISOLABLE RCS leakageSG tube leakage 2. Integrity-RED path conditions met	1. A leaking or RUPTURED SG is FAULTED outside of CTMT	None
B Inadequate Heat Removal	1. Core Cooling-RED PATH conditions met	1. Core Cooling-ORANGE Path conditions met 2. Heat Sink-RED Path conditions met AND Required feedwater flow cannot be established within 15 min. (Note 1)	None	3. Heat Sink-RED Path conditions met AND Required feedwater flow cannot be established within 15 min. (Note 1)	None	1. Core Cooling-RED PATH conditions met AND Restoration procedures not effective within 15 min. (Note 1)
C CTMT Radiation / RCS Activity	2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column Fuel Clad Loss (Note 14) 3. Coolant activity > 300 µCi/gm dose equivalent I-131 4. Dose rate at 1 ft. from an unpressurized RCS sample ≥ Table F-3 5. Sample line dose rate threshold ≥ Table F-4	None	2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column RCS Loss (Note 14)	None	None	2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column CTMT Potential Loss (Note 14)
D CTMT Integrity or Bypass	None	None	None	None	2. CTMT isolation (CIA) Phase A is required AND EITHER : <ul style="list-style-type: none">CTMT integrity has been lost based on DSEO/ADTS judgmentUNISOLABLE pathway from CTMT atmosphere to the environment exists 3. Indications of RCS leakage outside of CTMT	3. CTMT pressure > 60 psia 4. CTMT hydrogen concentration > 4% 5. CTMT pressure > 25 psia with < one full train of CTMT heat removal systems (Note 11) operating per design for ≥ 15 min. (Note 1)
E DSEO/ADTS Judgment	6. Any condition in the opinion of the DSEO/ADTS that indicates loss of the fuel clad barrier	3. Any condition in the opinion of the DSEO/ADTS that indicates potential loss of the fuel clad barrier	3. Any condition in the opinion of the DSEO/ADTS that indicates loss of the RCS barrier	4. Any condition in the opinion of the DSEO/ADTS that indicates potential loss of the RCS barrier	4. Any condition in the opinion of the DSEO/ADTS that indicates loss of the CTMT barrier	6. Any condition in the opinion of the DSEO/ADTS that indicates potential loss of the CTMT barrier

Barrier: Fuel Clad
Category: A. RCS or SG Tube Leakage
Degradation Threat: Loss
Threshold:

None

Barrier: Fuel Clad

Category: A. RCS or SG Tube Leakage

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Fuel Clad

Category: B. Inadequate Heat Removal

Degradation Threat: Loss

Threshold:

1. Core Cooling-RED Path conditions met

Definition(s):

None

Basis:

This reading condition indicates temperatures within the core are sufficient to cause significant superheating of reactor coolant.

The loss threshold is based on meeting the Core Cooling Red Path criteria for Core Exit Thermocouple readings (> 1,200 °F) (ref. 1, 2).

Reference(s):

1. EOP 35 F-0.2, "CSFST Core Cooling"
2. EOP 35 FR-C.1, "Response to Inadequate Core Cooling"
3. NEI 99-01 Inadequate Heat Removal Fuel Clad Loss 2.A

Barrier: Fuel Clad
Category: B. Inadequate Heat Removal
Degradation Threat: Potential Loss
Threshold:

1. Core Cooling-ORANGE Path conditions met
--

Definition(s):

None

Basis:

This reading condition indicates a reduction in reactor vessel water level sufficient to allow the onset of heat-induced cladding damage.

The potential loss threshold is based on meeting the Core Cooling Orange Path criteria for Core Exit Thermocouple readings ($> 718^{\circ}\text{F}$) or RVLMS plenum level indication ($< 19\%$) with a loss of subcooling margin (ref. 1, 2).

Reference(s):

1. EOP 35 F-0.2, "CSFST Core Cooling"
2. EOP 35 FR-C.1, "Response to Inadequate Core Cooling"
3. NEI 99-01 Inadequate Heat Removal Fuel Clad Potential Loss 2.A

Barrier: Fuel Clad

Category: B. Inadequate Heat Removal

Degradation Threat: Potential Loss

Threshold:

2. Heat Sink-RED Path conditions met

AND

Required feedwater flow **cannot** be established within 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Definition(s):

None

Basis:

The potential loss threshold is based on meeting the Heat Sink Red Path criteria of both of the following conditions existing (ref. 1):

- Narrow Range level in all SGs < 8% (42% adverse CTMT)
- Total feedwater flow to SGs < 530 gpm

This condition indicates an extreme challenge to the ability to remove RCS heat using the steam generators (i.e., loss of an effective secondary-side heat sink). This condition represents a potential loss of the Fuel Clad Barrier. In accordance with EOPs, there may be unusual accident conditions during which operators intentionally reduce the heat removal capability of the steam generators; during these conditions, classification using this threshold is not warranted.

If inventory could not be restored, steam generator dry out could occur within about 30 minutes. If feedwater flow could not be reestablished, FR-H.1 would require establishment of RCS bleed and feed to maintain core cooling and to assure sufficient RCS inventory. If neither of these actions can be accomplished, then fuel clad heatup will begin.

The phrase "Required feedwater flow **cannot** be established within 15 min." precludes the need for classification for conditions that indicate the feed flow requirements have been satisfied in accordance with FR-H.1. For example, FR-H.1 is entered from CSFST Heat Sink-Red. Step 1 tells the operator to determine if heat sink is required by checking that RCS pressure is greater than any non-faulted SG pressure and RCS T_{hot} is greater than 350°F. If these conditions exist, Heat Sink is required. Otherwise, the operator is to either go to the procedure and step in effect or place RHR in service for heat removal. For large LOCA events inside the Containment, the SGs are irrelevant because heat removal through the containment heat removal systems takes place. Therefore, Heat Sink Red is not applicable and should not be assessed for EAL classification because a LOCA event alone should not require higher than an Alert classification (ref. 1, 2).

However, required feed flow may be less than 530 gpm (Heat Sink Red Path entry criteria) and still be acceptable based on the exit criteria from FR-H.1. Per FR-H.1, feed flow can be less than 530 gpm and be considered acceptable if reduced feed flow is due to operator action or if WR level in at least one SG is increasing and Core Exit TCs are stable or decreasing.

The 15 minute allowance provides time for equipment actuation, operator action to restore feedwater flow, and verification that adequate feedwater flow has been established.

Meeting this threshold results in a Site Area Emergency because this threshold is identical to RCS Barrier Potential Loss threshold 2.AB.3; both will be met. This condition warrants a Site Area Emergency declaration because inadequate RCS heat removal may result in fuel heat-up sufficient to damage the cladding and increase RCS pressure to the point where mass will be lost from the system.

Reference(s):

1. EOP 35 F-0.3, "CSFST Heat Sink"
2. EOP 35 FR-H.1, "Response to Loss of Secondary Heat Sink"
3. NEI 99-01 Inadequate Heat Removal Fuel Clad Potential Loss 2.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column Fuel Clad Loss (Note 14)

Note 14: Readings are considered sustained when effects of TIC have dissipated.

Table F-2 CTMT High Range Radiation Monitor Barrier Thresholds RE-04A/05A			
Time > Shutdown (hrs)	Fuel Clad Loss (R/hr)	RCS Loss (R/hr)	CTMT Potential Loss (R/hr)
≤2	140	5	560
> 2 – ≤4	90	5	360
> 4 – ≤8	50	5	200
> 8 – ≤14	25	5	100
> 14	12	5	48

Definition(s):

None

Basis:

Containment radiation monitor readings greater than the Table F-2 Fuel Clad Loss column threshold indicate the release of reactor coolant, with elevated activity indicative of fuel damage, into the containment. The reading is derived assuming the instantaneous release and dispersal of the reactor coolant noble gas and iodine inventory associated with a concentration of 5% clad failure into the containment atmosphere. Reactor coolant concentrations of this magnitude are several times larger than the maximum concentrations (including iodine spiking) allowed within Technical Specifications and are therefore indicative of fuel damage (approximately 5 % clad failure depending on core inventory and RCS volume). The radiation monitor reading corresponds to an instantaneous release of all reactor coolant mass into the containment, assuming that reactor coolant activity equals 300 µCi/gm dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier (ref. 1, 2).

The values specified in Table F-2 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Time periods were chosen to fit monitor response (fast changes in response early following reactor shutdown are broken up into smaller time periods to better approximate expected change). Values were chosen within each time period to minimize error (<50%) to the highest and lowest response within the range.

Time after shutdown values are provided to account for radioactive decay.

The effects of thermally induced currents (TIC) are significant but of relatively short duration. Generally, the effects of TIC will dissipate within 2-5 minutes. Until the TIC affects have dissipated, the RE04A/RE05A readings should not be considered valid.

The radiation monitor reading in this threshold is higher than that specified for RCS Barrier Loss threshold C.4.2 since it indicates a loss of both the Fuel Clad barrier and the RCS barrier. Note that a combination of the two monitor readings appropriately escalates the ECL to a Site Area Emergency.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Damage Computer Program
2. Calculation RA-0075, "Millstone Unit 3 Expected Containment High Range Radiation Monitor Response to a LOCA Based on Fuel Rod Gap Fractions Defined in NUREG 1228"
3. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.A

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

3. Coolant activity > 300 $\mu\text{Ci/gm}$ dose equivalent I-131

Definition(s):

None

Basis:

This threshold indicates that RCS radioactivity concentration is greater than 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

It is recognized that sample collection and analysis of reactor coolant with highly elevated activity levels could require several hours to complete. Nonetheless, a sample-related threshold is included as a backup to other indications.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

4. Dose rate at 1 ft. from an unpressurized RCS sample \geq Table F-3

Table F-3 FC Loss Coolant Activity Dose Rates	
Time > Shutdown (hrs)	mR/hr/ml
≤ 2	15
$> 2 - \leq 8$	8
> 8	3

Definition(s):

None

Basis:

This threshold indicates that RCS radioactivity concentration is greater than 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

It is recognized that sample collection and analysis of reactor coolant with highly elevated activity levels could require several hours to complete. Nonetheless, a sample-related threshold is included as a backup to other indications. This EAL provides the ability to take a dose rate off of an RCS sample to determine fuel clad barrier loss, without the need to analyze the sample before making this determination. This EAL saves significant time by allowing evaluation of contained radioactivity within the RCS by a direct dose rate measurement.

Per Engineering Calculation RA-0059, dose rate is assumed to result from radioactive iodines (I-131 thru I-135) in RCS in concentrations corresponding to the loss of 5% of gap radioactivity of the core. For 5% loss of gap radioactivity ($\sim 300 \mu\text{Ci/gm}$ dose equivalent I-131), 2% of the core inventory of radioactive iodines are assumed to be contained in the gap. The values contained in Table F-3 (FC Loss Coolant Activity Dose Rates) represent expected one foot dose rates per ml of sample based on time since reactor shutdown to the time when the sample is taken. Time after shutdown values are provided to account for radioactive decay. The expected dose rate is a near linear relationship with the volume of the sample, so any volume collected can be determined by dividing the measured dose rate by the sample volume and comparing to the threshold value from Table F-3 for the applicable time frame. These dose rates assume no ECCS injection so there is no dilution credited which would vary coolant

volume. Values in the table have been rounded for ease of use. The > 8 hour threshold is conservative up to 24 hours following reactor shutdown. After 24 hours, the expected response from radioactive iodine levels off. Therefore, the value shown for > 8 hours applies for all samples taken 8 hours or more since reactor shutdown (ref. 1, 2).

The values specified in Table F-3 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Values were chosen to minimize error from the highest to lowest dose rate within each range.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Emergency Plan Procedure MP-26-EPI-FAP11, "Core Damage Assessment"
2. Calculation RA-0059, "Detector Response to an RCS Sample for EAL Classification of Fuel Clad Degradation and Barrier Loss"
3. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

5. Sample line dose rate \geq Table F-4

Table F-4 FC Loss RCS Sample Line Dose Rates	
Time > Shutdown (hrs)	R/hr
≤ 2	4
$> 2 - \leq 8$	2
> 8	1

Definition(s):

None

Basis:

This threshold indicates that RCS radioactivity concentration is greater than 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

Per Engineering Calculation RA-0079, dose rate is assumed to result from radioactive iodines in the RCS in concentrations corresponding to the loss of 5% of gap radioactivity of the core. The values contained in Table F-4 (FC Loss RCS Sample Line Dose Rates) represent fuel clad failure thresholds when measured approximately 2" from the outside of the RCS hot leg sample line. RCS sample line locations have been predetermined for use with this EAL. Other RCS lines could be used if analyzed on a case-by-case basis. Values in the table have been rounded for ease of use. The sample line dose rates have been calculated for various time ranges after shutdown to account for radioactive decay (ref. 1).

The values specified in Table F-4 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Values were chosen to minimize error from the highest to lowest dose rate within each range.

~~It is recognized that sample collection and analysis of reactor coolant with highly elevated activity levels could require several hours to complete. Nonetheless, a sample-related threshold is included as a backup to other indications.~~

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Engineering Calculation RA-0079
2. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Fuel Clad

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

None

Barrier: Fuel Clad
Category: D. CTMT Integrity or Bypass
Degradation Threat: Potential Loss
Threshold:

None

Barrier: Fuel Clad

Category: E. DSEO/ADTS Judgment

Degradation Threat: Loss

Threshold:

6. **Any** condition in the opinion of the DSEO/ADTS that indicates loss of the Fuel Clad barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that are to be used by the Emergency Director DSEO/ADTS in determining whether the Fuel Clad barrier is lost.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Fuel Clad Loss 6.A

Barrier: Fuel Clad

Category: F. DSEO/ADTS Judgment

Degradation Threat: Potential Loss

Threshold:

3. **Any** condition in the opinion of the DSEO/ADTS that indicates potential loss of the Fuel Clad barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that are to be used by the ~~Emergency Director~~DSEO/ADTS in determining whether the Fuel Clad barrier is potentially lost. The ~~Emergency Director~~DSEO/ADTS should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Potential Fuel Clad Loss 6.A

Barrier: Reactor Coolant System

Category: A. RCS or S/G Tube Leakage

Degradation Threat: Loss

Threshold:

1. An automatic or manual Safety Injection (SI) actuation required by **EITHER:**
- UNISOLABLE RCS leakage
 - SG tube RUPTURE

Definition(s):

UNISOLABLE - An open or breached system line that cannot be isolated, remotely or locally.

RUPTURE - The condition of a steam generator in which primary-to-secondary leakage is of sufficient magnitude to require a safety injection.

Basis:

This threshold is based on an UNISOLABLE RCS leak of sufficient size to require an automatic or manual actuation of the Emergency Core Cooling System (ECCS). This condition clearly represents a loss of the RCS Barrier.

This threshold is applicable to unidentified and pressure boundary leakage, as well as identified leakage. It is also applicable to UNISOLABLE RCS leakage through an interfacing system. The mass loss may be into any location – inside containment, to the secondary-side (i.e., steam generator tube leakage) or outside of containment.

A steam generator with primary-to-secondary leakage of sufficient magnitude to require a safety injection is considered to be RUPTURED. If a RUPTURED steam generator is also FAULTED outside of containment, the declaration escalates to a Site Area Emergency since the Containment Barrier Loss threshold 4-AA.1 will also be met.

If EOPs direct operators to open the Pressurizer pressure relief valves to implement a core cooling strategy (i.e., a “feed and bleed” cooldown), then there will exist a reactor coolant flow path from the RCS, past the “pressurizer safety and relief valves” and into the containment that operators cannot isolate without compromising the effectiveness of the strategy (i.e., for the strategy to be effective, the valves must be kept in the open position); therefore, the flow through the pressure relief line is UNISOLABLE. In this case, the ability of the RCS pressure boundary to serve as an effective barrier to a release of fission products has been eliminated and thus this condition constitutes a loss of the RCS barrier.

A SI actuation resulting from a non-RCS leak event (ex.: FAULTED SG) does not meet the intent of this threshold.

Reference(s):

1. EOP 35 E-0, "Reactor Trip or Safety Injection"
2. EOP 35 E-3, "Steam Generator Tube Rupture"
3. NEI 99-01 RCS or SG Tube Leakage Reactor Coolant System Loss 1.A

Barrier: Reactor Coolant System

Category: A. RCS or S/G Tube Leakage

Degradation Threat: Potential Loss

Threshold:

1. Operation of a second charging pump is required by **EITHER**:
- UNISOLABLE RCS leakage
 - SG tube leakage

Definition(s):

UNISOLABLE - An open or breached system line that cannot be isolated, remotely or locally.

Basis:

This threshold is based on an UNISOLABLE RCS leak that results in the inability to maintain pressurizer level within specified limits by operation of a normally used charging (makeup) pump, but an ECCS (SI) actuation has not occurred. The threshold is met when an operating procedure, or operating crew supervision, directs that a ~~standby-second~~ charging (makeup) pump be placed in service to restore and maintain pressurizer level (ref. 1, 2).

This threshold is applicable to unidentified and pressure boundary leakage, as well as identified leakage. It is also applicable to UNISOLABLE RCS leakage through an interfacing system. The mass loss may be into any location – inside containment, to the secondary-side (i.e., steam generator tube leakage) or outside of containment.

If ~~a~~ the leaking steam generator requiring the start of a second charging pump is also FAULTED outside of containment, the declaration escalates to a Site Area Emergency since the Containment Barrier Loss threshold 4-AA.1 will also be met.

When classifying RCS leaks, the classification should be based on actual leakage vice the impact of normal plant response to Operator directed actions. Example: A rapid down power may have the impact of increasing charging flow. The increase in charging should not be considered increased leakage.

Reference(s):

1. AOP 3555, "Reactor Coolant System Leak"
2. AOP 3576, "Steam Generator Tube Leak"
3. NEI 99-01 RCS or SG Tube Leakage Reactor Coolant System Potential Loss 1.A

Barrier: Reactor Coolant System

Category: A. RCS or S/G Tube Leakage

Degradation Threat: Potential Loss

Threshold:

2. Integrity-RED path conditions met

Definition(s):

None

Basis:

This condition indicates an extreme challenge to the integrity of the RCS pressure boundary due to pressurized thermal shock – a transient that causes rapid RCS cooldown while the RCS is in Mode 3 or higher (i.e., hot and pressurized).

The potential loss threshold is defined by the CSFST Integrity - RED path. CSFST Integrity - Red Path plant conditions (> 100°F/hr cold leg cooldown) and associated PTS Limit A Curve indicates an extreme challenge to the safety function when plant parameters are to the right of the limit curve following excessive RCS cooldown under pressure (ref. 1).

Reference(s):

1. EOP 35 F-0.4, "CSFST Integrity"
2. NEI 99-01 RCS or SG Tube Leakage Reactor Coolant System Potential Loss 1.B

Barrier: Reactor Coolant System

Category: B. Inadequate Heat Removal

Degradation Threat: Loss

Threshold:

None

Barrier: Reactor Coolant System

Category: B. Inadequate Heat Removal

Degradation Threat: Potential Loss

Threshold:

3. Heat Sink-RED Path conditions met

AND

Required feedwater flow **cannot** be established within 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Definition(s):

None

Basis:

The potential loss threshold is based on meeting the Heat Sink Red Path criteria of both of the following conditions existing (ref. 1):

- Narrow Range level in all SGs < 8% (42% adverse CTMT)
- Total feedwater flow to SGs < 530 gpm

This condition indicates an extreme challenge to the ability to remove RCS heat using the steam generators (i.e., loss of an effective secondary-side heat sink). This condition represents a potential loss of the RCS Barrier. In accordance with EOPs, there may be unusual accident conditions during which operators intentionally reduce the heat removal capability of the steam generators; during these conditions, classification using this threshold is not warranted.

If inventory could not be restored, steam generator dry out could occur within about 30 minutes. If feedwater flow could not be reestablished, FR-H.1 would require establishment of RCS bleed and feed to maintain core cooling and to assure sufficient RCS inventory. If neither of these actions can be accomplished, then fuel clad heatup will begin.

The phrase "Required feedwater flow **cannot** be established within 15 min." precludes the need for classification for conditions that indicate the feed flow requirements have been satisfied in accordance with FR-H.1. For example, FR-H.1 is entered from CSFST Heat Sink-Red. Step 1 tells the operator to determine if heat sink is required by checking that RCS pressure is greater than any non-faulted SG pressure and RCS T_{hot} is greater than 350°F. If these conditions exist, Heat Sink is required. Otherwise, the operator is to either go to the procedure and step in effect or place RHR in service for heat removal. For large LOCA events inside the Containment, the SGs are irrelevant because heat removal through the containment heat removal systems takes place. Therefore, Heat Sink Red is not applicable and should not be assessed for EAL classification because a LOCA event alone should not require higher than an Alert classification. (ref. 1, 2).

However, required feed flow may be less than 530 gpm (Heat Sink Red Path entry criteria) and still be acceptable based on the exit criteria from FR-H.1. Per FR-H.1, feed flow can be less than 530 gpm and be considered acceptable if reduced feed flow is due to operator action or if WR level in at least one SG is increasing and Core Exit TCs are stable or decreasing.

The 15 minute allowance provides time for equipment actuation, operator action to restore feedwater flow, and verification that adequate feedwater flow has been established.

Meeting this threshold results in a Site Area Emergency because this threshold is identical to Fuel Clad Barrier Potential Loss threshold ~~2-BB.2~~; both will be met. This condition warrants a Site Area Emergency declaration because inadequate RCS heat removal may result in fuel heat-up sufficient to damage the cladding and increase RCS pressure to the point where mass will be lost from the system.

Reference(s):

1. EOP 35 F-0.3, "CSFST Heat Sink"
2. EOP 35 FR-H.1, "Response to Loss of Secondary Heat Sink"
3. NEI 99-01 Inadequate Heat Removal RCS Loss 2.B

Barrier: Reactor Coolant System

Category: C. CTMT Radiation/ RCS Activity

Degradation Threat: Loss

Threshold:

2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column RCS Loss (Note 14)

Note 14: Readings are considered sustained when effects of TIC have dissipated.

Table F-2 CTMT High Range Radiation Monitor Barrier Thresholds RE-04A/05A			
Time > Shutdown (hrs)	Fuel Clad Loss (R/hr)	RCS Loss (R/hr)	CTMT Potential Loss (R/hr)
≤ 2	140	5	560
$> 2 - \leq 4$	90	5	360
$> 4 - \leq 8$	50	5	200
$> 8 - \leq 14$	25	5	100
> 14	12	5	48

Definition(s):

None

Basis:

A reading > 5 R/hr (minimum practical reading) on RE-04A or 05A is indicative of a breach in the RCS barrier (ref. 1, 2).

The radiation monitor reading corresponds to an instantaneous release of all reactor coolant mass into the containment, assuming that reactor coolant activity equals Technical Specification allowable limits. This value is lower than that specified for Fuel Clad barrier loss threshold 3-AC.2 since it indicates a loss of the RCS Barrier only.

Because of the very high fuel clad integrity, only small amounts of noble gases would be dissolved in the primary coolant. Conservative estimates indicated that the readings from release of the normal RCS inventory would be below normal readings on the monitor while the station was operating. Therefore, a value 5 times the normal containment radiation monitor (RE-04A/05A) reading of ~ 1 R/hr is used. The reading is less than that specified for fuel cladding barrier loss because no damage to the fuel cladding is assumed. Only leakage from the RCS is assumed for this barrier loss threshold. The value is high enough to preclude erroneous classification of barrier loss due to normal plant operations and is set at the mid-point of the lowest decade of monitor indication (ref. 2).

The effects of thermally induced currents (TIC) are significant but of relatively short duration. Generally, the effects of TIC will dissipate within 2-5 minutes. Until the TIC affects have dissipated, the RE04A/RE05A readings should not be considered valid.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Damage Computer Program
2. Calculation RA-0075, "Millstone Unit 3 Expected Containment High Range Radiation Monitor Response to a LOCA Based on Fuel Rod Gap Fractions Defined in NUREG 1228"
3. NEI 99-01 CMT Radiation / RCS Activity RCS Loss 3.A

Barrier: Reactor Coolant System

Category: C. CTMT Radiation/ RCS Activity

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Reactor Coolant System

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

None

Barrier: Reactor Coolant System

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Reactor Coolant System

Category: E. DSEO/ADTS Judgment

Degradation Threat: Loss

Threshold:

3. **Any** condition in the opinion of the DSEO/ADTS that indicates loss of the RCS barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the Emergency Director DSEO/ADTS in determining whether the RCS barrier is lost.

Reference(s):

1. NEI 99-01 Emergency Director Judgment RCS Loss 6.A

Barrier: Reactor Coolant System

Category: E. DSEO/ADTS Judgment

Degradation Threat: Potential Loss

Threshold:

4. **Any** condition in the opinion of the DSEO/ADTS that indicates potential loss of the RCS barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the Emergency Director DSEO/ADTS in determining whether the RCS barrier is potentially lost. The Emergency Director DSEO/ADTS should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Reference(s):

1. NEI 99-01 Emergency Director Judgment RCS Potential Loss 6.A

Barrier: Containment

Category: A. RCS or SG Tube Leakage

Degradation Threat: Loss

Threshold:

1. A leaking or RUPTURED SG is FAULTED outside of CTMT

Definition(s):

FAULTED - The term applied to a steam generator that has a steam leak on the secondary side of sufficient size to cause an uncontrolled drop in steam generator pressure or the steam generator to become completely depressurized.

RUPTURED - The condition of a steam generator in which primary-to-secondary leakage is of sufficient magnitude to require a safety injection.

Basis:

This threshold addresses a leaking or RUPTURED Steam Generator (SG) that is also FAULTED outside of containment. The condition of the SG, whether leaking or RUPTURED, is determined in accordance with the thresholds for RCS Barrier Potential Loss 4.A.A.1 and Loss 4.AA.1, respectively. This condition represents a bypass of the containment barrier.

FAULTED is a defined term within the NEI 99-01 methodology; this determination is not necessarily dependent upon entry into, or diagnostic steps within, an EOP. For example, if the pressure in a steam generator is decreasing uncontrollably (part of the FAULTED definition) and the FAULTED steam generator isolation procedure is not entered because EOP user rules are dictating implementation of another procedure to address a higher priority condition, the steam generator is still considered FAULTED for emergency classification purposes.

The FAULTED criterion establishes an appropriate lower bound on the size of a steam release that may require an emergency classification. Steam releases of this size are readily observable with normal Control Room indications. The lower bound for this aspect of the containment barrier is analogous to the lower bound criteria specified in IC SU4-MU4 for the fuel clad barrier (i.e., RCS activity values) and IC SU5-MU5 for the RCS barrier (i.e., RCS leak rate values).

This threshold also applies to prolonged steam releases necessitated by operational considerations such as the forced steaming of a leaking or RUPTURED steam generator directly to atmosphere to cooldown the plant, or to drive an auxiliary (emergency) feed water pump. These types of conditions will result in a significant and sustained release of radioactive steam to the environment (and are thus similar to a FAULTED condition). The inability to isolate the steam flow without an adverse effect on plant cooldown meets the intent of a loss of containment.

Steam releases associated with the expected operation of a SG power operated relief valve or safety relief valve do not meet the intent of this threshold. Such releases may occur

intermittently for a short period of time following a reactor trip as operators process through emergency operating procedures to bring the plant to a stable condition and prepare to initiate a plant cooldown. Steam releases associated with the unexpected operation of a valve (e.g., a stuck-open safety valve) do meet this threshold.

Following an SG tube leak or rupture, there may be minor radiological releases through a secondary-side system component (e.g., air ejectors, gland seal exhausters, valve packing, etc.). These types of releases do not constitute a loss or potential loss of containment but should be evaluated using the Recognition-Category A-R ICs.

The emergency classification levels resulting from primary-to-secondary leakage, with or without a steam release from the FAULTED SG, are summarized below.

P-to-S Leak Rate	Affected SG is FAULTED Outside of Containment?	
	Yes	No
Less than or equal to 25 gpm	No classification	No classification
Greater than 25 gpm	Unusual Event per SU4MU5.1	Unusual Event per SU4MU5.1
Requires operation of a standby charging (makeup) pump (RCS Barrier Potential Loss)	Site Area Emergency per FS1.1	Alert per FA1.1
Requires an automatic or manual ECCS (SIAS) actuation (RCS Barrier Loss)	Site Area Emergency per FS1.1	Alert per FA1.1

There is no Potential Loss threshold associated with RCS or SG Tube Leakage.

Reference(s):

1. NEI 99-01 RCS or SG Tube Leakage Containment Loss 1.A

Barrier: Containment

Category: A. RCS or SG Tube Leakage

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Containment

Category: B. Inadequate Heat Removal

Degradation Threat: Loss

None

Barrier: Containment

Category: B. Inadequate Heat Removal

Degradation Threat: Potential Loss

Threshold:

1. Core Cooling-RED Path conditions met
AND
Restoration procedures **not** effective within 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Definition(s):

IMMINENT: The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

The potential loss threshold is based on meeting the Core Cooling Red Path criteria for Core Exit Thermocouple readings ($> 1,200^{\circ}\text{F}$) and restoration procedures not effective within 15 minutes (ref. 1, 2).

This condition represents an IMMINENT core melt sequence which, if not corrected, could lead to vessel failure and an increased potential for containment failure. For this condition to occur there must already have been a loss of the RCS Barrier and the Fuel Clad Barrier. If implementation of a procedure(s) to restore adequate core cooling is not effective (successful) within 15 minutes, it is assumed that the event trajectory will likely lead to core melting and a subsequent challenge of the Containment Barrier.

The restoration procedure is considered "effective" if core exit thermocouple readings are decreasing and/or if reactor vessel level is increasing. Whether or not the procedure(s) will be effective should be apparent within 15 minutes. The ~~Emergency Director~~ DSEO/ADTS should escalate the emergency classification level to a General Emergency as soon as it is determined that the procedure(s) will not be effective.

Severe accident analyses (e.g., NUREG-1150) have concluded that functional restoration procedures can arrest core degradation in a significant fraction of core damage scenarios, and that the likelihood of containment failure is very small in these events. Given this, it is appropriate to provide 15 minutes beyond the required entry point to determine if procedural actions can reverse the core melt sequence.

Reference(s):

1. EOP 35 F-0.2, "CSFST Core Cooling"
2. EOP 35 FR-C.1, "Response to Inadequate Core Cooling"
3. NEI 99-01 Inadequate Heat Removal Containment Potential Loss 2.A

Barrier: Containment

Category: C. CTMT Radiation/RCS Activity

Degradation Threat: Loss

Threshold:

None

Barrier: Containment

Category: C. CTMT Radiation/RCS Activity

Degradation Threat: Potential Loss

Threshold:

2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column CTMT Potential Loss (Note 14)

Note 14: Readings are considered sustained when effects of TIC have dissipated.

Table F-2 CTMT High Range Radiation Monitor Barrier Thresholds RE-04A/05A			
Time > Shutdown (hrs)	Fuel Clad Loss (R/hr)	RCS Loss (R/hr)	CTMT Potential Loss (R/hr)
≤ 2	140	5	560
$> 2 - \leq 4$	90	5	360
$> 4 - \leq 8$	50	5	200
$> 8 - \leq 14$	25	5	100
> 14	12	5	48

Definition(s):

None

Basis:

The radiation monitor reading corresponds to an instantaneous release of all reactor coolant mass into the containment, assuming that 20% of the fuel cladding has failed. This level of fuel clad failure is well above that used to determine the analogous Fuel Clad Barrier Loss and RCS Barrier Loss thresholds (ref. 1, 2).

Time after shutdown values are provided to account for radioactive decay.

The values specified in Table F-2 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Time periods were chosen to fit monitor response (fast changes in response early following reactor shutdown are broken up into smaller time periods to better approximate expected change). Values were chosen within each time period to minimize error (<50%) to the highest and lowest response within the range.

NUREG-1228, Source Estimations During Incident Response to Severe Nuclear Power Plant Accidents, indicates the fuel clad failure must be greater than approximately 20% in order for there to be a major release of radioactivity requiring offsite protective actions. For this condition to exist, there must already have been a loss of the RCS barrier and the Fuel Clad

barrier. It is therefore prudent to treat this condition as a potential loss of containment which would then escalate the emergency classification level to a General Emergency.

The effects of thermally induced currents (TIC) are significant but of relatively short duration. Generally, the effects of TIC will dissipate within 2-5 minutes. Until the TIC affects have dissipated, the RE04A/RE05A readings should not be considered valid.

Reference(s):

1. Damage Computer Program
2. Calculation RA-0075, "Millstone Unit 3 Expected Containment High Range Radiation Monitor Response to a LOCA Based on Fuel Rod Gap Fractions Defined in NUREG 1228"
3. NEI 99-01 CMT Radiation / RCS Activity Containment Potential Loss 3.A

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

2. CTMT isolation (CIA) Phase A is required

AND EITHER:

- CTMT integrity has been lost based on DSEO/ADTS judgment
- UNISOLABLE pathway from CTMT atmosphere to the environment exists

Definition(s):

UNISOLABLE - An open or breached system line that cannot be isolated, remotely or locally.

Basis:

The status of the containment barrier during an event involving steam generator tube leakage is assessed using Loss Threshold 4.AA.1. Therefore this threshold is not applicable to steam generator tube leakage.

These thresholds address a situation where containment isolation is required and one of two conditions exists as discussed below. Users are reminded that there may be accident and release conditions that simultaneously meet both bulleted thresholds 4.A.1 and 4.A.2.

4.A.1 First Threshold – Containment integrity has been lost, i.e., the actual containment atmospheric leak rate likely exceeds that associated with allowable leakage (or sometimes referred to as design leakage). Following the release of RCS mass into containment, containment pressure will fluctuate based on a variety of factors; a loss of containment integrity condition may (or may not) be accompanied by a noticeable drop in containment pressure. Recognizing the inherent difficulties in determining a containment leak rate during accident conditions, it is expected that the ~~Emergency Director~~ DSEO/ADTS will assess this threshold using judgment, and with due consideration given to current plant conditions, and available operational and radiological data (e.g., containment pressure, readings on radiation monitors outside containment, operating status of containment pressure control equipment, etc.).

Refer to the middle piping run of Figure 9-F-41. Two simplified examples are provided. One is leakage from a penetration and the other is leakage from an in-service system valve. Depending upon radiation monitor locations and sensitivities, the leakage could be detected by any of the four monitors depicted in the figure.

Another example would be a loss or potential loss of the RCS barrier, and the simultaneous occurrence of two FAULTED locations on a steam generator where one fault is located inside containment (e.g., on a steam or feedwater line) and the other outside of containment. In this case, the associated steam line provides a pathway for the containment atmosphere to escape to an area outside the containment.

Following the leakage of RCS mass into containment and a rise in containment pressure, there may be minor radiological releases associated with allowable (design) containment leakage through various penetrations or system components. These releases do not constitute a loss or potential loss of containment but should be evaluated using the ~~Recognition~~-Category A ICs.

4.A.2 Second Threshold – Conditions are such that there is an UNISOLABLE pathway for the migration of radioactive material from the containment atmosphere to the environment. As used here, the term “environment” includes the atmosphere of a room or area, outside the containment, that may, in turn, communicate with the outside-the-plant atmosphere (e.g., through discharge of a ventilation system or atmospheric leakage). Depending upon a variety of factors, this condition may or may not be accompanied by a noticeable drop in containment pressure.

Refer to the top piping run of Figure 9-F-41. In this simplified example, the inboard and outboard isolation valves remained open after a containment isolation was required (i.e., containment isolation was not successful). There is now an UNISOLABLE pathway from the containment to the environment.

The existence of a filter is not considered in the threshold assessment. Filters do not remove fission product noble gases. In addition, a filter could become ineffective due to iodine and/or particulate loading beyond design limits (i.e., retention ability has been exceeded) or water saturation from steam/high humidity in the release stream.

Leakage between two interfacing liquid systems, by itself, does not meet this threshold.

Refer to the bottom piping run of Figure 9-F-41. In this simplified example, leakage in an RCP seal cooler is allowing radioactive material to enter the Auxiliary Building. The radioactivity would be detected by the Process Monitor. If there is no leakage from the closed water cooling system to the Auxiliary Building, then no threshold has been met. If the pump developed a leak that allowed steam/water to enter the Auxiliary Building, then the second threshold 4.B would be met. Depending upon radiation monitor locations and sensitivities, this leakage could be detected by any of the four monitors depicted in the figure and cause the first threshold 4.A.1 to be met as well.

Following the leakage of RCS mass into containment and a rise in containment pressure, there may be minor radiological releases associated with allowable containment leakage through various penetrations or system components. Minor releases may also occur if a containment isolation valve(s) fails to close but the containment atmosphere escapes to an enclosed system. These releases do not constitute a loss or potential loss of containment but should be evaluated using the ~~Recognition~~-Category A-R ICs.

Reference(s):

1. NEI 99-01 CMT Integrity or Bypass Containment Loss 4.A

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

3. Indications of RCS leakage outside of CTMT

Definition(s):

None

Basis:

To ensure proper escalation of the emergency classification, the RCS leakage outside of containment must be related to the mass loss that is causing the RCS Loss and/or Potential Loss threshold ~~4.AA.1~~ to be met.

The status of the containment barrier during an event involving steam generator tube leakage is assessed using Containment Loss Threshold A.1. Therefore this threshold is not applicable to steam generator tube leakage.

This threshold **does not** apply to an UNISOLABLE RSHX tube leak outside containment. Such leaks are properly addressed under the category R radiological release based EALs.

Containment sump, temperature, pressure and/or radiation levels will increase if reactor coolant mass is leaking into the containment. If these parameters have not increased, then the reactor coolant mass may be leaking outside of containment (i.e., a containment bypass sequence). Increases in sump, temperature, pressure, flow and/or radiation level readings outside of the containment may indicate that the RCS mass is being lost outside of containment.

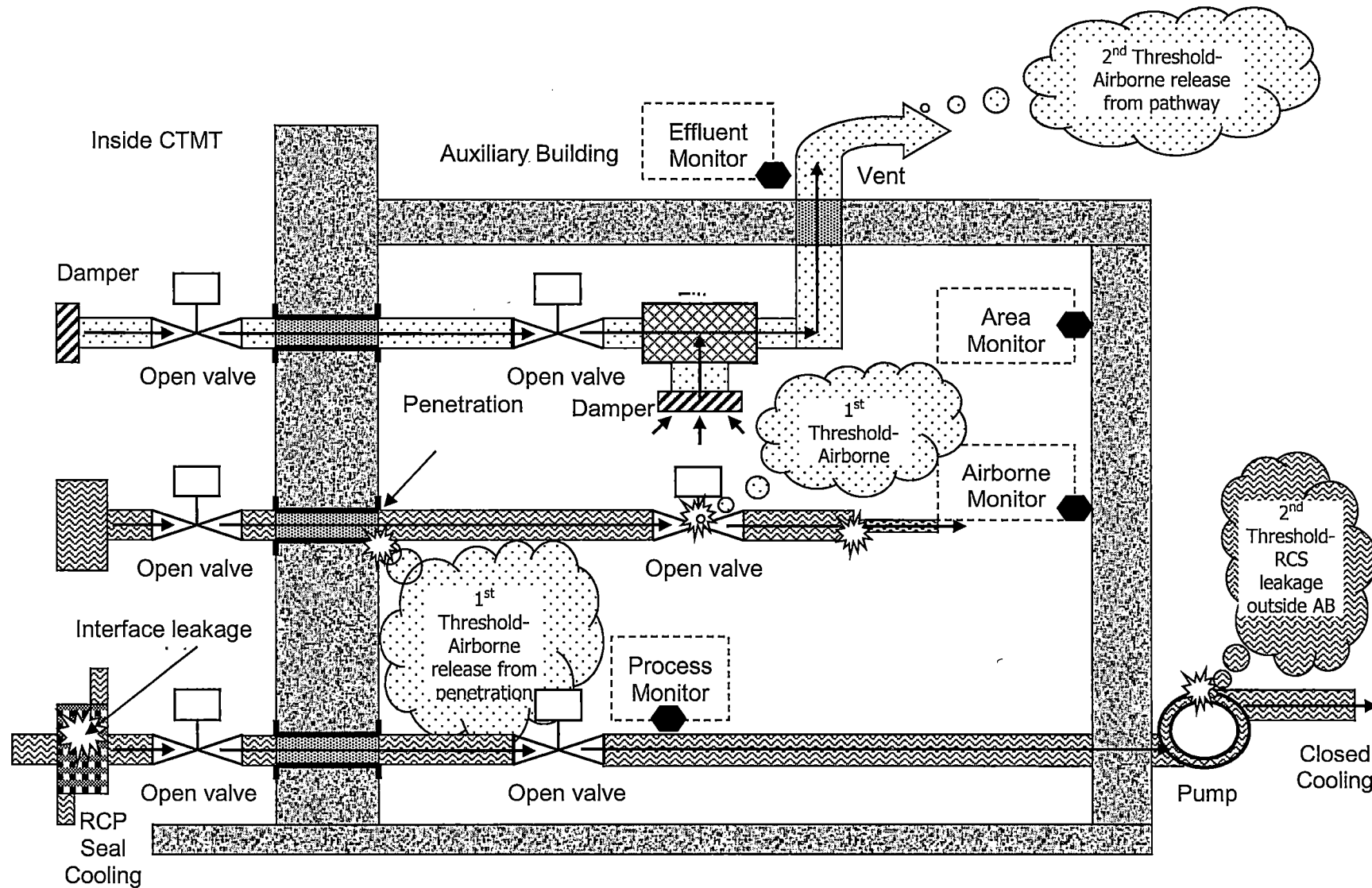
Unexpected elevated readings and alarms on radiation monitors with detectors outside containment should be corroborated with other available indications to confirm that the source is a loss of RCS mass outside of containment. If the fuel clad barrier has not been lost, radiation monitor readings outside of containment may not increase significantly; however, other unexpected changes in sump levels, area temperatures or pressures, flow rates, etc. should be sufficient to determine if RCS mass is being lost outside of the containment.

Refer to the middle piping run of Figure 9-F-41. In this simplified example, a leak has occurred at a reducer on a pipe carrying reactor coolant in the Auxiliary Building. Depending upon radiation monitor locations and sensitivities, the leakage could be detected by any of the four monitors depicted in the figure and cause loss threshold ~~CNB4-D.2~~ to be met as well.

Reference(s):

1. NEI 99-01 CMT Integrity or Bypass Containment Loss 4.B

Figure 1: Containment Integrity or Bypass Examples



Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

3. CTMT pressure > 60 psia

Definition(s):

None

Basis:

If containment pressure exceeds the design pressure of 60 psia (ref. 1), there exists a potential to lose the containment barrier. To reach this level, there must be an inadequate core cooling condition for an extended period of time; therefore, the RCS and Fuel Clad barriers would already be lost. Thus, this threshold is a discriminator between a Site Area Emergency and General Emergency since there is now a potential to lose the third barrier.

Reference(s):

1. MPS3 FSAR Section 6.2.1.1.1, "Containment Structure Design Bases"
2. NEI 99-01 CMT Integrity or Bypass Containment Potential Loss 4.A

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

4. CTMT hydrogen concentration > 4%

Definition(s):

None

Basis:

The existence of an explosive mixture means, at a minimum, that the containment atmospheric hydrogen concentration is sufficient to support a hydrogen burn (i.e., at the lower deflagration limit). A hydrogen burn will raise containment pressure and could result in collateral equipment damage leading to a loss of containment integrity. It therefore represents a potential loss of the containment barrier.

A containment hydrogen concentration of 4% conservatively represents the lowest threshold for flammability in the presence of oxygen (ref. 1).

Reference(s):

1. Calculation PRA98NQA-01555S3, "SAMG – Computational Aids and Setpoints"
2. NEI 99-01 CMT Integrity or Bypass Containment Potential Loss 4.B

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

5. CTMT pressure > 25 psia with < one full train of CTMT heat removal systems (Note 11) operating per design for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 11: One full train of containment heat removal systems consist of one train of Recirculation Spray System (RSS) and one train of Quench Spray System (QSS).

Definition(s):

None

Basis:

This threshold describes a condition where containment pressure is greater than the setpoint (25 psia) at which containment energy (heat) removal systems are designed to automatically actuate, and less than one full train of equipment is capable of operating per design (ref. 1, 2, 3). The 15-minute criterion is included to allow operators time to manually start equipment that may not have automatically started, if possible. This threshold represents a potential loss of containment in that containment heat removal/depressurization systems (e.g., containment sprays, ice condenser fans, etc., but not including containment venting strategies) are either lost or performing in a degraded manner.

The Quench Spray System (QSS) is activated immediately upon receipt of a CDA signal. QSS becomes effective within 70 seconds of initiation.

The containment Recirculation Spray System (RSS) is activated on receipt of the RWST Low-Low level signal coincident with a CDA signal. There is a maximum delay of 5 minutes before recirculation spray becomes effective.

One full train of containment heat removal systems consist of one train of Recirculation Spray System (RSS) and one train of Quench Spray System (QSS) (ref. 1, 2). The combination of required equipment can be obtained from using equipment on either emergency bus in order to meet the "one full train" requirement.

Reference(s):

1. MPS3 FSAR Section 6.2.2, "Containment Heat Removal System"
2. MPS3 Technical Specifications 3.6.2.1, "Containment Quench Spray and Recirculation Spray System"
3. EOP 35 FR-Z.1, "Response to High Containment Pressure"
4. NEI 99-01 CMT Integrity or Bypass Containment Potential Loss 4.C

Barrier: Containment

Category: E. DSEO/ADTS Judgment

Degradation Threat: Loss

Threshold:

4. **Any** condition in the opinion of the DSEO/ADTS that indicates loss of the CTMT barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the Emergency Director DSEO/ADTS in determining whether the containment barrier is lost.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Containment Loss 6.A

Barrier: Containment

Category: E. DSEO/ADTS Judgment

Degradation Threat: Potential Loss

Threshold:

6. **Any** condition in the opinion of the DSEO/ADTS that indicates potential loss of the CTMT barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the ~~Emergency Director~~ DSEO/ADTS in determining whether the containment barrier is potentially lost. The ~~Emergency Director~~ DSEO/ADTS should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Containment Potential Loss 6.A

Category H – Hazards and Other Conditions Affecting Plant Safety

EAL Group: ANY (EALs in this category are applicable to any plant condition, hot or cold.)

Hazards are non-plant, system-related events that can directly or indirectly affect plant operation, reactor plant safety or personnel safety.

1. Security

Unauthorized entry attempts into the PROTECTED AREA, bomb threats, sabotage attempts, and actual security compromises threatening loss of physical control of the plant.

2. Seismic Event

Natural events such as earthquakes have potential to cause plant structure or equipment damage of sufficient magnitude to threaten personnel or plant safety.

3. Natural or Technological Hazard

Other natural and non-naturally occurring events that can cause damage to plant facilities include tornados, FLOODING, hazardous material releases and events restricting site access warranting classification.

4. Fire

FIRES can pose significant hazards to personnel and reactor safety. Appropriate for classification are FIRES within the plant PROTECTED AREA or which may affect operability of equipment needed for safe shutdown

5. Hazardous Gas

Toxic, corrosive, asphyxiant or flammable gas leaks can affect normal plant operations or preclude access to plant areas required to safely shutdown the plant.

6. Control Room Evacuation

Events that are indicative of loss of Control Room habitability. If the Control Room must be evacuated, additional support for monitoring and controlling plant functions is necessary through the emergency response facilities.

7. DSEO/ADTS Judgment

The EALs defined in other categories specify the predetermined symptoms or events that are indicative of emergency or potential emergency conditions and thus warrant classification. While these EALs have been developed to address the full spectrum of possible emergency conditions which may warrant classification and subsequent implementation of the Emergency Plan, a provision for classification of emergencies based on operator/management experience and judgment is still necessary. The EALs of this category provide the DSEO/ADTS the latitude to classify emergency conditions consistent with the established classification criteria based upon DSEO/ADTS judgment.

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 1 – Security

Initiating Condition: Confirmed SECURITY CONDITION or threat

EAL:

HU1.1 Unusual Event

A SECURITY CONDITION that does **not** involve a HOSTILE ACTION as reported by MPS Security Shift Supervision

OR

Notification of a credible security threat directed at the site

OR

A validated notification from the NRC providing information of an aircraft threat

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA (OCA) - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;

- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

SECURITY CONDITION - Any security event as listed in the approved security contingency plan that constitutes a threat/compromise to site security, threat/risk to site personnel, or a potential degradation to the level of safety of the plant. A security condition does **not** involve a HOSTILE ACTION.

Basis:

This IC addresses events that pose a threat to plant personnel or SAFETY SYSTEM equipment, and thus represent a potential degradation in the level of plant safety. Security events which do not meet one of these EALs are adequately addressed by the requirements of 10CFR73.71 or 10CFR50.72. Security events assessed as HOSTILE ACTIONS are classifiable under ICs HA1, ~~and HS1 and HG1~~. Guidance on assessing Security Conditions is included in the Security Contingency Plan Implementing Procedures (SCIP). The SCIPs are implementing procedures for the Station Safeguards Contingency Plan.

Timely and accurate communications between Security Shift Supervision and the Control Room is essential for proper classification of a security-related event. Classification of these events will initiate appropriate threat-related notifications to plant personnel and Offsite Response Organizations.

Security plans and terminology are based on the guidance provided by NEI 03-12, *Template for the Security Plan, Training and Qualification Plan, Safeguards Contingency Plan [and Independent Spent Fuel Storage Installation Security Program]*.

The first threshold EAL #1 references the Security Shift Supervision ~~(site-specific security shift supervision)~~ because these are the individuals trained to confirm that a security event is occurring or has occurred. Training on security event confirmation and classification is controlled due to the nature of Safeguards and 10CFR2.39 information.

The second threshold EAL #2 addresses the receipt of a credible security threat. The credibility of the threat is assessed in accordance with the Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program ~~(site-specific procedure)~~ (ref. 1).

The third threshold EAL #3 addresses the threat from the impact of an aircraft on the plant. The NRC Headquarters Operations Officer (HOO) will communicate to the licensee if the threat involves an aircraft. The status and size of the plane may also be provided by NORAD through the NRC. Validation of the threat is performed in accordance with OP-AA-900 Authentication (ref. 2) ~~(site-specific procedure)~~.

Emergency plans and implementing procedures are public documents; therefore, EALs should not incorporate Security-sensitive information. This includes information that may be advantageous to a potential adversary, such as the particulars concerning a specific threat or threat location. Security-sensitive information should be contained in non-public documents such as the Security Plan for MPS (ref. 1).

Escalation of the emergency classification level would be via IC HA1.

Reference(s):

1. Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program
2. OP-AA-900, "Authentication"
3. NEI 99-01 HU1

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 1 – Security
Initiating Condition: HOSTILE ACTION within the OWNER CONTROLLED AREA or airborne attack threat within 30 minutes

EAL:

HA1.1 Alert

A HOSTILE ACTION is occurring or has occurred within the OWNER CONTROLLED AREA as reported by MPS Security Shift Supervision

OR

A validated notification from NRC of an aircraft attack threat within 30 min. of the site

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

HOSTILE FORCE - One or more individuals who are engaged in a determined assault, overtly or by stealth and deception, equipped with suitable weapons capable of killing, maiming, or causing destruction.

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses the occurrence of a HOSTILE ACTION within the OWNER CONTROLLED AREA or notification of an aircraft attack threat. This event will require rapid response and assistance due to the possibility of the attack progressing to the PROTECTED AREA, or the need to prepare the plant and staff for a potential aircraft impact.

Timely and accurate communications between Security Shift Supervision and the Control Room is essential for proper classification of a security-related event.

Security plans and terminology are based on the guidance provided by NEI 03-12, *Template for the Security Plan, Training and Qualification Plan, Safeguards Contingency Plan [and Independent Spent Fuel Storage Installation Security Program]*.

As time and conditions allow, these events require a heightened state of readiness by the plant staff and implementation of onsite protective measures (e.g., evacuation, dispersal or sheltering). The Alert declaration will also heighten the awareness of Offsite Response Organizations State and local agencies, allowing them to be better prepared should it be necessary to consider further actions.

This ~~IC~~ EAL does not apply to incidents that are accidental events, acts of civil disobedience, or otherwise are not a HOSTILE ACTION perpetrated by a HOSTILE FORCE. Examples include the crash of a small aircraft, shots from hunters, physical disputes between employees, etc. Reporting of these types of events is adequately addressed by other EALs, or the requirements of 10CFR73.71 or 10CFR50.72.

The first threshold EAL #1 is applicable for any HOSTILE ACTION occurring, or that has occurred, in the OWNER CONTROLLED AREA. ~~This includes any action directed against an ISFSI that is located outside the plant PROTECTED AREA.~~

The second threshold EAL #2 addresses the threat from the impact of an aircraft on the plant, and the anticipated arrival time is within 30 minutes. The intent of this EAL is to ensure that threat-related notifications are made in a timely manner so that plant personnel and State and local agencies ~~OROs~~ are in a heightened state of readiness. This EAL is met when the threat-related information has been validated in accordance with OP-AA-900 Authentication ~~(site-specific procedure)~~ (ref. 2).

The NRC Headquarters Operations Officer (HOO) will communicate to the licensee if the threat involves an aircraft. The status and size of the plane may be provided by NORAD through the NRC.

In some cases, it may not be readily apparent if an aircraft impact within the OWNER CONTROLLED AREA was intentional (i.e., a HOSTILE ACTION). It is expected, although not certain, that notification by an appropriate Federal agency to the site would clarify this point. In this case, the appropriate federal agency is intended to be NORAD, FBI, FAA or NRC. The emergency declaration, including one based on other ICs/EALs, should not be unduly delayed while awaiting notification by a Federal agency.

Emergency plans and implementing procedures are public documents; therefore, EALs should not incorporate Security-sensitive information. This includes information that may be advantageous to a potential adversary, such as the particulars concerning a specific threat or threat location. Security-sensitive information should be contained in non-public documents such as the Security Plan for MPS (ref. 1).

Escalation of the emergency classification level would be via IC HS1.

Reference(s):

1. Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program
2. OP-AA-900, "Authentication"
3. NEI 99-01 HA1

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 1 – Security

Initiating Condition: HOSTILE ACTION within the PROTECTED AREA

EAL:

HS1.1 Site Area Emergency

A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by MPS Security Shift Supervision

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

HOSTILE FORCE - One or more individuals who are engaged in a determined assault, overtly or by stealth and deception, equipped with suitable weapons capable of killing, maiming, or causing destruction.

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses the occurrence of a HOSTILE ACTION within the PROTECTED AREA. This event will require rapid response and assistance due to the possibility for damage to plant equipment.

Timely and accurate communications between Security Shift Supervision and the Control Room is essential for proper classification of a security-related event (ref. 1).

Security plans and terminology are based on the guidance provided by NEI 03-12, *Template for the Security Plan, Training and Qualification Plan, Safeguards Contingency Plan [and Independent Spent Fuel Storage Installation Security Program]*.

As time and conditions allow, these events require a heightened state of readiness by the plant staff and implementation of onsite protective measures (e.g., evacuation, dispersal or sheltering). The Site Area Emergency declaration will mobilize State and local agency ~~OR~~ resources and have them available to develop and implement public protective actions in the unlikely event that the attack is successful in impairing multiple safety functions.

This ~~IC EAL~~ does not apply to a ~~HOSTILE ACTION~~ directed at an ~~ISFSI Protected Area~~ located outside the ~~PROTECTED AREA~~; such an attack should be assessed using ~~IC HA1~~. It also does not apply to incidents that are accidental events, acts of civil disobedience, or otherwise are not a ~~HOSTILE ACTION~~ perpetrated by a ~~HOSTILE FORCE~~. Examples include the crash of a small aircraft, shots from hunters, physical disputes between employees, etc. Reporting of these types of events is adequately addressed by other EALs, or the requirements of 10CFR73.71 or 10CFR50.72.

Emergency plans and implementing procedures are public documents; therefore, EALs should not incorporate Security-sensitive information. This includes information that may be advantageous to a potential adversary, such as the particulars concerning a specific threat or threat location. Security-sensitive information should be contained in non-public documents such as the Security Plan for MPS (ref. 1).

~~Escalation of the emergency classification level would be via IC HG1.~~

Reference(s):

1. Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program
2. NEI 99-01 HS1

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 2 – Seismic Event

Initiating Condition: Seismic event greater than OBE levels

EAL:

HU2.1 Unusual Event

Seismic event > OBE as indicated by OBE alarm light on Panel 3ERS-PNL1SM1C

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses a seismic event that results in accelerations at the plant site greater than those specified for an Operating Basis Earthquake (OBE). An earthquake greater than an OBE but less than a Safe Shutdown Earthquake (SSE) should have no significant impact on safety-related systems, structures and components; however, some time may be required for the plant staff to ascertain the actual post-event condition of the plant (e.g., performs walk-downs and post-event inspections). Given the time necessary to perform walk-downs and inspections, and fully understand any impacts, this event represents a potential degradation of the level of safety of the plant.

Event verification with external sources should not be necessary during or following an OBE. Earthquakes of this magnitude should be readily felt by on-site personnel and recognized as a seismic event (e.g., lateral accelerations in excess of 0.08g to 0.09g). The Shift Manager or Emergency Director/DSEO/ADTS may seek external verification if deemed appropriate (e.g., a call to the U.S. Geological Survey (USGS), check internet news sources, etc.); however, the verification action must not preclude a timely emergency declaration.

For both MPS2 and MPS3, the OBE ground acceleration thresholds are > 0.09g horizontal or > 0.06g vertical. The MPS3 Control Room has real time OBE exceedance alarm indications (ref. 1, 2, 3). Therefore classification shall be based upon the receipt of the MPS3 OBE alarm light on MPS3 Panel 3ERS-PNL1SM1C (ref. 2).

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or SA9MA9.

Reference(s):

1. AOP 3570, "Earthquake"
2. MPS3 FSAR Section 2.1.1, "Site Location and Description"
3. NEI 99-01 HU2

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.1 Unusual Event

A tornado strike within the PROTECTED AREA

Mode Applicability:

All

Definition(s):

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

This EAL EAL #1-addresses a tornado striking (touching down) within the PROTECTED AREA.

~~EAL #2-addresses flooding of a building room or area that results in operators isolating power to a SAFETY SYSTEM component due to water level or other wetting concerns. Classification is also required if the water level or related wetting causes an automatic isolation of a SAFETY SYSTEM component from its power source (e.g., a breaker or relay trip). To warrant classification, operability of the affected component must be required by Technical Specifications for the current operating mode.~~

~~EAL #3-addresses a hazardous materials event originating at an offsite location and of sufficient magnitude to impede the movement of personnel within the PROTECTED AREA.~~

~~EAL #4-addresses a hazardous event that causes an on-site impediment to vehicle movement and significant enough to prohibit the plant staff from accessing the site using personal vehicles. Examples of such an event include site flooding caused by a hurricane, heavy rains, up-river water releases, dam failure, etc., or an on-site train derailment blocking the access road.~~

~~This EAL is not intended apply to routine impediments such as fog, snow, ice, or vehicle breakdowns or accidents, but rather to more significant conditions such as the Hurricane Andrew strike on Turkey Point in 1992, the flooding around the Cooper Station during the Midwest floods of 1993, or the flooding around Ft. Calhoun Station in 2011.~~

~~EAL #5-addresses (site-specific description).~~

Escalation of the emergency classification level would be based on ICs in Recognition Categories AR, F, S-M or C.

If damage is confirmed visually or by other in-plant indications, the event may be escalated to an Alert under IC CA6 or MA9.

A tornado striking (touching down) within the PROTECTED AREA warrants declaration of an Unusual Event regardless of the measured wind speed at the meteorological tower. A tornado is defined as a violently rotating column of air in contact with the ground and extending from the base of a thunderstorm (ref. 1).

Reference(s):

1. AOP 3569, "Severe Weather Conditions"
2. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.2 Unusual Event

Internal room or area FLOODING of a magnitude sufficient to require manual or automatic electrical isolation of a SAFETY SYSTEM component required by Technical Specifications for the current operating mode

Mode Applicability:

All

Definition(s):

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

~~EAL #1 addresses a tornado striking (touching down) within the PROTECTED AREA.~~

This EAL addresses FLOODING of a building room or area that results in operators isolating power to a SAFETY SYSTEM component due to water level or other wetting concerns. Classification is also required if the water level or related wetting causes an automatic isolation of a SAFETY SYSTEM component from its power source (e.g., a breaker or relay trip). To warrant classification, operability of the affected component must be required by Technical Specifications for the current operating mode (ref. 1, 2).

~~EAL #3 addresses a hazardous materials event originating at an offsite location and of sufficient magnitude to impede the movement of personnel within the PROTECTED AREA.~~

~~EAL #4 addresses a hazardous event that causes an on-site impediment to vehicle movement and significant enough to prohibit the plant staff from accessing the site using personal vehicles. Examples of such an event include site flooding caused by a hurricane, heavy rains,~~

~~up river water releases, dam failure, etc., or an on-site train derailment blocking the access road.~~

~~This EAL is not intended apply to routine impediments such as fog, snow, ice, or vehicle breakdowns or accidents, but rather to more significant conditions such as the Hurricane Andrew strike on Turkey Point in 1992, the flooding around the Cooper Station during the Midwest floods of 1993, or the flooding around Ft. Calhoun Station in 2011.~~

~~EAL #5 addresses (site specific description).~~

Escalation of the emergency classification level would be based on ICs in Recognition Categories AR, F, S-M or C.

Refer to EAL CA6.1 or MA9.1 for internal flooding affecting more than one SAFETY SYSTEM train.

Reference(s):

1. AOP 3569, "Severe Weather Conditions"
2. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.3 Unusual Event

Movement of personnel within the PROTECTED AREA is IMPEDED due to an event external to the PROTECTED AREA involving hazardous materials (e.g., an offsite chemical spill or toxic gas release)

Mode Applicability:

All

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

~~This EAL #1 addresses a tornado striking (touching down) within the PROTECTED AREA.~~

~~This EAL addresses flooding of a building room or area that results in operators isolating power to a SAFETY SYSTEM component due to water level or other wetting concerns. Classification is also required if the water level or related wetting causes an automatic isolation of a SAFETY SYSTEM component from its power source (e.g., a breaker or relay trip). To warrant classification, operability of the affected component must be required by Technical Specifications for the current operating mode.~~

~~EAL #3 addresses a hazardous materials event originating at an offsite location outside the PROTECTED AREA and of sufficient magnitude to IMPEDE the movement of personnel within the PROTECTED AREA.~~

~~EAL #4 addresses a hazardous event that causes an on-site impediment to vehicle movement and significant enough to prohibit the plant staff from accessing the site using personal vehicles. Examples of such an event include site flooding caused by a hurricane, heavy rains, up-river water releases, dam failure, etc., or an on-site train derailment blocking the access road.~~

~~This EAL is not intended apply to routine impediments such as fog, snow, ice, or vehicle breakdowns or accidents, but rather to more significant conditions such as the Hurricane Andrew strike on Turkey Point in 1992, the flooding around the Cooper Station during the Midwest floods of 1993, or the flooding around Ft. Calhoun Station in 2011.~~

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~~EAL #5 addresses (site-specific description).~~

Escalation of the emergency classification level would be based on ICs in Recognition Categories AR, F, S-M or C.

Reference(s):

1. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.4 Unusual Event

A hazardous event that results in on-site conditions sufficient to prohibit the plant staff from accessing the site via personal vehicles (Note 7)

Note 7: This EAL does **not** apply to routine traffic impediments such as fog, snow, ice, or vehicle breakdowns or accidents.

Mode Applicability:

All

Definition(s):

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant. ~~EAL #1 addresses a tornado striking (touching down) within the PROTECTED AREA.~~

~~This EAL addresses flooding of a building room or area that results in operators isolating power to a SAFETY SYSTEM component due to water level or other wetting concerns. Classification is also required if the water level or related wetting causes an automatic isolation of a SAFETY SYSTEM component from its power source (e.g., a breaker or relay trip). To warrant classification, operability of the affected component must be required by Technical Specifications for the current operating mode.~~

~~EAL #3 addresses a hazardous materials event originating at an offsite location and of sufficient magnitude to impede the movement of personnel within the PROTECTED AREA.~~

This EAL ~~EAL #4~~ addresses a hazardous event that causes an on-site impediment to vehicle movement and significant enough to prohibit the plant staff from accessing the site using personal vehicles. Examples of such an event include site FLOODING caused by a hurricane, heavy rains, up-river water releases, dam failure, etc., or an on-site train derailment blocking the access road.

This EAL is not intended to apply to routine impediments such as fog, snow, ice, or vehicle breakdowns or accidents, but rather to more significant conditions such as the Hurricane Andrew strike on Turkey Point in 1992, the FLOODING around the Cooper Station during the Midwest floods of 1993, or the FLOODING around Ft. Calhoun Station in 2011.

~~EAL #5 addresses (site specific description). Escalation of the emergency classification level would be based on ICs in Recognition Categories AR, F, S-M or C.~~

Reference(s):

1. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.1 Unusual Event

A FIRE is **not** extinguished within 15 min. of **any** of the following fire detection indications (Note 1):

- Report from the field (i.e., visual observation)
- Receipt of multiple (more than 1) fire alarms or indications
- Field verification of a single fire alarm

AND

The FIRE is located within **any** Table H-1 area

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table H-1 MPS3 Fire Areas
<ul style="list-style-type: none">• Containment Building• Auxiliary Building• Control Building• Emergency Generator Enclosure• ESF Building• Main Steam Valve Building• A & B Train Service Water Cubicles• North & South Cable Tunnels• Yard Areas<ul style="list-style-type: none">○ RWST○ DWST

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

EAL #1

The 15 minute requirement begins with a credible notification that a FIRE is occurring, or receipt of multiple VALID fire detection system alarms or field validation of a single fire alarm. The alarm is to be validated using available Control Room indications or alarms to prove that it is not spurious, or by reports from the field.

Table H-1 Fire Areas are those areas that contain equipment necessary for safe operation and shutdown of the plant (ref. 1, 2).

The intent of the 15-minute duration is to size the FIRE and to discriminate against small FIRES that are readily extinguished (e.g., smoldering waste paper basket). In addition to alarms, other indications of a FIRE could be a drop in fire main pressure, automatic activation of a suppression system, etc.

Upon receipt, operators will take prompt actions to confirm the validity of an initial fire alarm, indication, or report. For EAL assessment purposes, the emergency declaration clock starts at the time that the initial alarm, indication, or report was received, and not the time that a subsequent verification action was performed. Similarly, the fire duration clock also starts at the time of receipt of the initial alarm, indication or report. EAL #2

~~This EAL addresses receipt of a single fire alarm, and the existence of a FIRE is not verified (i.e., proved or disproved) within 30 minutes of the alarm. Upon receipt, operators will take prompt actions to confirm the validity of a single fire alarm. For EAL assessment purposes, the 30 minute clock starts at the time that the initial alarm was received, and not the time that a subsequent verification action was performed.~~

~~A single fire alarm, absent other indication(s) of a FIRE, may be indicative of equipment failure or a spurious activation, and not an actual FIRE. For this reason, additional time is allowed to verify the validity of the alarm. The 30 minute period is a reasonable amount of time to determine if an actual FIRE exists; however, after that time, and absent information to the contrary, it is assumed that an actual FIRE is in progress.~~

~~If an actual FIRE is verified by a report from the field, then EAL #1 is immediately applicable, and the emergency must be declared if the FIRE is not extinguished within 15 minutes of the report. If the alarm is verified to be due to an equipment failure or a spurious activation, and this verification occurs within 30 minutes of the receipt of the alarm, then this EAL is not applicable and no emergency declaration is warranted.~~

EAL #3

~~In addition to a FIRE addressed by EAL #1 or EAL #2, a FIRE within the plant PROTECTED AREA not extinguished within 60 minutes may also potentially degrade the level of plant safety. This basis extends to a FIRE occurring within the PROTECTED AREA of an ISFSI located outside the plant PROTECTED AREA. [Sentence for plants with an ISFSI outside the plant Protected Area]~~

EAL #4

~~If a FIRE within the plant or ISFSI [for plants with an ISFSI outside the plant Protected Area] PROTECTED AREA is of sufficient size to require a response by an offsite firefighting agency (e.g., a local town Fire Department), then the level of plant safety is potentially degraded. The dispatch of an offsite firefighting agency to the site requires an emergency declaration only if it is needed to actively support firefighting efforts because the fire is beyond the capability of the Fire Brigade to extinguish. Declaration is not necessary if the agency resources are placed on stand-by, or supporting post-extinguishment recovery or investigation actions.~~

~~Basis-Related Requirements from Appendix R~~

~~Appendix R to 10 CFR 50, states in part:~~

~~Criterion 3 of Appendix A to this part specifies that "Structures, systems, and components important to safety shall be designed and located to minimize, consistent with other safety requirements, the probability and effect of fires and explosions."~~

~~When considering the effects of fire, those systems associated with achieving and maintaining safe shutdown conditions assume major importance to safety because damage to them can lead to core damage resulting from loss of coolant through boil-off.~~

~~Because fire may affect safe shutdown systems and because the loss of function of systems used to mitigate the consequences of design-basis accidents under post-fire conditions does not per se impact public safety, the need to limit fire damage to systems required to achieve and maintain safe shutdown conditions is greater than the need to limit fire damage to those systems required to mitigate the consequences of design basis accidents.~~

~~In addition, Appendix R to 10 CFR 50, requires, among other considerations, the use of 1-hour fire barriers for the enclosure of cable and equipment and associated non-safety circuits of one redundant train (G.2.c). As used in EAL #2, the 30 minutes to verify a single alarm is well within this worst case 1-hour time period.~~

~~Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or SA9MA9.~~

Reference(s):

1. EOP 3509, "Fire Emergency"
2. CM-AA-FPA-102, "MP3 Branch Technical Position 9.5-1 Compliance Report"
3. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.2 Unusual Event

Receipt of a single fire alarm (i.e., **no** other indications of a FIRE)

AND

The fire alarm is indicating a FIRE within **any** Table H-1 area (excluding Containment Building)

AND

The existence of a FIRE is **not** verified within 30 min. of alarm receipt (Notes 1, 15)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 15: A Containment Building fire alarm is considered VALID upon receipt of multiple (more than one) fire zone alarms.

Table H-1 MPS3 Fire Areas

- Containment Building
- Auxiliary Building
- Control Building
- Emergency Generator Enclosure
- ESF Building
- Main Steam Valve Building
- A & B Train Service Water Cubicles
- North & South Cable Tunnels
- Yard Areas
 - RWST
 - DWST

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

EAL #1

~~The intent of the 15-minute duration is to size the FIRE and to discriminate against small FIRES that are readily extinguished (e.g., smoldering waste paper basket). In addition to alarms, other indications of a FIRE could be a drop in fire main pressure, automatic activation of a suppression system, etc.~~

~~Upon receipt, operators will take prompt actions to confirm the validity of an initial fire alarm, indication, or report. For EAL assessment purposes, the emergency declaration clock starts at the time that the initial alarm, indication, or report was received, and not the time that a subsequent verification action was performed. Similarly, the fire duration clock also starts at the time of receipt of the initial alarm, indication or report.~~

EAL #2

The 30 minute requirement begins upon receipt of a single VALID fire detection system alarm. The alarm is to be validated using available Control Room indications or alarms to prove that it is not spurious, or by reports from the field. Actual field reports must be made within the 30 minute time limit or a classification must be made. If a fire is verified to be occurring by field report, classification shall be made based on EAL HU4.1, with the 15 minute requirement beginning with the verification of the fire by field report.

Table H-1 Fire Areas are those areas that contain equipment necessary for safe operation and shutdown of the plant (ref. 1, 2).

This EAL addresses receipt of a single fire alarm, and the existence of a FIRE is not verified (i.e., proved or disproved) within 30-minutes of the alarm. Upon receipt, operators will take prompt actions to confirm the validity of a single fire alarm. For EAL assessment purposes, the 30-minute clock starts at the time that the initial alarm was received, and not the time that a subsequent verification action was performed.

With regard to Containment Building fire alarms, there is constant air movement in the enclosed containment due to the operation of the containment ventilation system. The operating cooling units are drawing air to the units past the smoke detectors. It can be reasonably expected that a fire that burns for 15 minutes would produce sufficient products of combustion to cause fire detectors in multiple zones to alarm. Therefore, a single Containment Building fire alarm is not considered VALID.

A single fire alarm, absent other indication(s) of a FIRE, may be indicative of equipment failure or a spurious activation, and not an actual FIRE. For this reason, additional time is allowed to verify the validity of the alarm. The 30-minute period is a reasonable amount of time to determine if an actual FIRE exists; however, after that time, and absent information to the contrary, it is assumed that an actual FIRE is in progress.

If an actual FIRE is verified by a report from the field, then HU4.1 EAL #1 is immediately applicable, and the emergency must be declared if the FIRE is not extinguished within 15-minutes of the report. If the alarm is verified to be due to an equipment failure or a spurious

activation, and this verification occurs within 30-minutes of the receipt of the alarm, then this EAL is not applicable and no emergency declaration is warranted. EAL #3

~~In addition to a FIRE addressed by EAL #1 or EAL #2, a FIRE within the plant PROTECTED AREA not extinguished within 60 minutes may also potentially degrade the level of plant safety. This basis extends to a FIRE occurring within the PROTECTED AREA of an ISFSI located outside the plant PROTECTED AREA. [Sentence for plants with an ISFSI outside the plant Protected Area]~~

EAL #4

Basis-Related Requirements from Appendix R (Although MPS3 is a BTP 9.5-1 plant (ref. 2), the following is provided as justification for the use of the 30 minute criteria)

Appendix R to 10 CFR 50 Appendix R, states in part:

Criterion 3 of Appendix A to this part specifies that "Structures, systems, and components important to safety shall be designed and located to minimize, consistent with other safety requirements, the probability and effect of fires and explosions."

When considering the effects of fire, those systems associated with achieving and maintaining safe shutdown conditions assume major importance to safety because damage to them can lead to core damage resulting from loss of coolant through boil-off.

Because fire may affect safe shutdown systems and because the loss of function of systems used to mitigate the consequences of design basis accidents under post-fire conditions does not per se impact public safety, the need to limit fire damage to systems required to achieve and maintain safe shutdown conditions is greater than the need to limit fire damage to those systems required to mitigate the consequences of design basis accidents.

In addition, Appendix R to 10 CFR 50, requires, among other considerations, the use of 1-hour fire barriers for the enclosure of cable and equipment and associated non-safety circuits of one redundant train (G.2.c). As used in HU4.2EAL #2, the 30-minutes to verify a single alarm is well within this worst-case 1-hour time period.

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or SA9MA9.

Reference(s):

1. EOP 3509, "Fire Emergency"
2. CM-AA-FPA-102, "MP3 Branch Technical Position 9.5-1 Compliance Report"
4. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.3 Unusual Event

A FIRE within the PROTECTED AREA **not** extinguished within 60 min. of the initial report, alarm or indication (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

PROTECTED AREA - An area encompassed by physical barriers (i.e., the security fence) and to which access is controlled.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

EAL #1

~~The intent of the 15 minute duration is to size the FIRE and to discriminate against small FIRES that are readily extinguished (e.g., smoldering waste paper basket). In addition to alarms, other indications of a FIRE could be a drop in fire main pressure, automatic activation of a suppression system, etc.~~

~~Upon receipt, operators will take prompt actions to confirm the validity of an initial fire alarm, indication, or report. For EAL assessment purposes, the emergency declaration clock starts at the time that the initial alarm, indication, or report was received, and not the time that a subsequent verification action was performed. Similarly, the fire duration clock also starts at the time of receipt of the initial alarm, indication or report.~~

EAL #2

~~This EAL addresses receipt of a single fire alarm, and the existence of a FIRE is not verified (i.e., proved or disproved) within 30 minutes of the alarm. Upon receipt, operators will take prompt actions to confirm the validity of a single fire alarm. For EAL assessment purposes, the 30 minute clock starts at the time that the initial alarm was received, and not the time that a subsequent verification action was performed.~~

~~A single fire alarm, absent other indication(s) of a FIRE, may be indicative of equipment failure or a spurious activation, and not an actual FIRE. For this reason, additional time is allowed to~~

verify the validity of the alarm. The 30-minute period is a reasonable amount of time to determine if an actual FIRE exists; however, after that time, and absent information to the contrary, it is assumed that an actual FIRE is in progress.

If an actual FIRE is verified by a report from the field, then EAL #1 is immediately applicable, and the emergency must be declared if the FIRE is not extinguished within 15 minutes of the report. If the alarm is verified to be due to an equipment failure or a spurious activation, and this verification occurs within 30 minutes of the receipt of the alarm, then this EAL is not applicable and no emergency declaration is warranted.

EAL #3

In addition to a FIRE addressed by EAL HU4.1 #1 or HU4.2EAL #2, a FIRE within the plant PROTECTED AREA not extinguished within 60-minutes may also potentially degrade the level of plant safety. This basis extends to a FIRE occurring within the PROTECTED AREA of an ISFSI located outside the plant PROTECTED AREA. *[Sentence for plants with an ISFSI outside the plant Protected Area]* EAL #4

If a FIRE within the plant or ISFSI *[for plants with an ISFSI outside the plant Protected Area]* PROTECTED AREA is of sufficient size to require a response by an offsite firefighting agency (e.g., a local town Fire Department), then the level of plant safety is potentially degraded. The dispatch of an offsite firefighting agency to the site requires an emergency declaration only if it is needed to actively support firefighting efforts because the fire is beyond the capability of the Fire Brigade to extinguish. Declaration is not necessary if the agency resources are placed on stand-by, or supporting post-extinguishment recovery or investigation actions.

Basis Related Requirements from Appendix R

Appendix R to 10 CFR 50, states in part:

Criterion 3 of Appendix A to this part specifies that "Structures, systems, and components important to safety shall be designed and located to minimize, consistent with other safety requirements, the probability and effect of fires and explosions."

When considering the effects of fire, those systems associated with achieving and maintaining safe shutdown conditions assume major importance to safety because damage to them can lead to core damage resulting from loss of coolant through boil-off.

Because fire may affect safe shutdown systems and because the loss of function of systems used to mitigate the consequences of design basis accidents under post-fire conditions does not per se impact public safety, the need to limit fire damage to systems required to achieve and maintain safe shutdown conditions is greater than the need to limit fire damage to those systems required to mitigate the consequences of design basis accidents.

In addition, Appendix R to 10 CFR 50, requires, among other considerations, the use of 1-hour fire barriers for the enclosure of cable and equipment and associated non-safety circuits of one redundant train (G.2.c). As used in EAL #2, the 30 minutes to verify a single alarm is well within this worst-case 1-hour time period.

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or SA9MA9.

Reference(s):

Millstone Power Station Unit 3
Emergency Action Level Technical Bases Document
Attachment 1 Emergency Action Level Technical Bases

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1. EOP 3509, "Fire Emergency"
2. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.4 Unusual Event

A FIRE within the PROTECTED AREA that requires an offsite fire department to assist with extinguishment

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

PROTECTED AREA - An area encompassed by physical barriers (i.e., the security fence) and to which access is controlled.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

EAL #1

~~The intent of the 15-minute duration is to size the FIRE and to discriminate against small FIRES that are readily extinguished (e.g., smoldering waste paper basket). In addition to alarms, other indications of a FIRE could be a drop in fire main pressure, automatic activation of a suppression system, etc.~~

~~Upon receipt, operators will take prompt actions to confirm the validity of an initial fire alarm, indication, or report. For EAL assessment purposes, the emergency declaration clock starts at the time that the initial alarm, indication, or report was received, and not the time that a subsequent verification action was performed. Similarly, the fire duration clock also starts at the time of receipt of the initial alarm, indication or report.~~

EAL #2

~~This EAL addresses receipt of a single fire alarm, and the existence of a FIRE is not verified (i.e., proved or disproved) within 30 minutes of the alarm. Upon receipt, operators will take prompt actions to confirm the validity of a single fire alarm. For EAL assessment purposes, the 30-minute clock starts at the time that the initial alarm was received, and not the time that a subsequent verification action was performed.~~

~~A single fire alarm, absent other indication(s) of a FIRE, may be indicative of equipment failure or a spurious activation, and not an actual FIRE. For this reason, additional time is allowed to verify the validity of the alarm. The 30-minute period is a reasonable amount of time to~~

determine if an actual FIRE exists; however, after that time, and absent information to the contrary, it is assumed that an actual FIRE is in progress.

If an actual FIRE is verified by a report from the field, then EAL #1 is immediately applicable, and the emergency must be declared if the FIRE is not extinguished within 15 minutes of the report. If the alarm is verified to be due to an equipment failure or a spurious activation, and this verification occurs within 30 minutes of the receipt of the alarm, then this EAL is not applicable and no emergency declaration is warranted.

EAL #3

In addition to a FIRE addressed by EAL #1 or EAL #2, a FIRE within the plant PROTECTED AREA not extinguished within 60 minutes may also potentially degrade the level of plant safety. *This basis extends to a FIRE occurring within the PROTECTED AREA of an ISFSI located outside the plant PROTECTED AREA. [Sentence for plants with an ISFSI outside the plant Protected Area]*

EAL #4

If a FIRE within the plant or ISFSI *[for plants with an ISFSI outside the plant Protected Area]* PROTECTED AREA is of sufficient size to require a response by an offsite firefighting agency (e.g., a local town Fire Department), then the level of plant safety is potentially degraded. The dispatch of an offsite firefighting agency to the site requires an emergency declaration only if it is needed to actively support firefighting efforts because the fire is beyond the capability of the Fire Brigade to extinguish. Declaration is not necessary if the agency resources are placed on stand-by, or supporting post-extinguishment recovery or investigation actions.

The Shift Fire Brigade Advisor or Shift Fire Brigade Leader will assess whether the fire conditions warrant outside assistance (ref. 1).

Basis-Related Requirements from Appendix R

Appendix R to 10 CFR 50, states in part:

Criterion 3 of Appendix A to this part specifies that "Structures, systems, and components important to safety shall be designed and located to minimize, consistent with other safety requirements, the probability and effect of fires and explosions."

When considering the effects of fire, those systems associated with achieving and maintaining safe shutdown conditions assume major importance to safety because damage to them can lead to core damage resulting from loss of coolant through boil-off.

Because fire may affect safe shutdown systems and because the loss of function of systems used to mitigate the consequences of design basis accidents under post fire conditions does not per se impact public safety, the need to limit fire damage to systems required to achieve and maintain safe shutdown conditions is greater than the need to limit fire damage to those systems required to mitigate the consequences of design basis accidents.

In addition, Appendix R to 10 CFR 50, requires, among other considerations, the use of 1-hour fire barriers for the enclosure of cable and equipment and associated non-safety circuits of one

~~redundant train (G.2.c). As used in EAL #2, the 30 minutes to verify a single alarm is well within this worst case 1 hour time period.~~

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or ~~SA9~~MA9.

Reference(s):

1. EOP 3509, "Fire Emergency"
2. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 5 – Hazardous Gases
Initiating Condition: Gaseous release IMPEDING access to equipment necessary for normal plant operations, cooldown or shutdown

EAL:

HA5.1 Alert

Release of a toxic, corrosive, asphyxiant or flammable gas into **any** Table H-2 room or area

AND

Entry into the room or area is prohibited or IMPEDED (Note 5)

Note 5: If the equipment in the listed room or area was already inoperable or out-of-service before the event occurred, then no emergency classification is warranted.

Table R-2 Safe Operation & Shutdown Rooms/Areas	
Room/Area	Mode
Aux. Building EI 43'	1, 2, 3
Aux. Building EI 24' East MCC/RCA EI 24' West MCC/RCA EI 24'	3, 4
Aux. Building EI 3' 8"	3
Containment ESF Building A RHR Cubicle ESF Building B RHR Cubicle ESF Building EI 36' ESF Building EI 24' East Switchgear West Switchgear	4

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

Basis:

This IC addresses an event involving a release of a hazardous gas that precludes or IMPEDES access to equipment necessary to maintain normal plant operation, or required for a normal plant cooldown and shutdown. This condition represents an actual or potential substantial degradation of the level of safety of the plant.

An Alert declaration is warranted if entry into the affected room/area is, or may be, procedurally required during the plant operating mode in effect at the time of the gaseous release. The emergency classification is not contingent upon whether entry is actually necessary at the time of the release.

Evaluation of the IC and EAL do not require atmospheric sampling; it only requires the Emergency Director/DSEO/ADTS's judgment that the gas concentration in the affected room/area is sufficient to preclude or significantly IMPEDE procedurally required access. This judgment may be based on a variety of factors including an existing job hazard analysis, report of ill effects on personnel, advice from a subject matter expert or operating experience with the same or similar hazards. Access should be considered as IMPEDED if extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is not routinely employed).

An emergency declaration is **not** warranted if any of the following conditions apply:

- The plant is in an operating mode different than the mode specified for the affected room/area (i.e., entry is not required during the operating mode in effect at the time of the gaseous release). For example, the plant is in Mode 1 when the gaseous release occurs, and the procedures used for normal operation, cooldown and shutdown do not require entry into the affected room until Mode 4.
- The gas release is a planned activity that includes compensatory measures which address the temporary inaccessibility of a room or area (e.g., fire suppression system testing).
- The action for which room/area entry is required is of an administrative or record keeping nature (e.g., normal rounds or routine inspections).
- The access control measures are of a conservative or precautionary nature, and would not actually prevent or IMPEDE a required action.
- If the equipment in the listed room or area was already inoperable, or out-of-service, before the event occurred, then no emergency should be declared since the event will have no adverse impact beyond that already allowed by Technical Specifications at the time of the event.

An asphyxiant is a gas capable of reducing the level of oxygen in the body to dangerous levels. Most commonly, asphyxiants work by merely displacing air in an enclosed environment. This reduces the concentration of oxygen below the normal level of around 19%, which can lead to breathing difficulties, unconsciousness or even death.

This EAL does not apply to firefighting activities that generate smoke and that automatically or manually activate a fire suppression system in an area, ~~or to intentional inerting of containment (BWR only).~~

Escalation of the emergency classification level would be via Recognition-Category AR, C or F ICs.

Reference(s):

1. Attachment 2, "Safe Operation & Shutdown Areas Tables A-3 & H-2 Bases"
2. NEI 99-01 HA5

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 6 – Control Room Evacuation
Initiating Condition: Control Room evacuation resulting in transfer of plant control to alternate locations

EAL:

HA6.1 Alert

An event has resulted in plant control being transferred from the Control Room to the Auxiliary Shutdown Panel

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses an evacuation of the Control Room that results in transfer of plant control to alternate locations outside the Control Room. The loss of the ability to control the plant from the Control Room is considered to be a potential substantial degradation in the level of plant safety.

Following a Control Room evacuation, control of the plant will be transferred to alternate shutdown locations. The necessity to control a plant shutdown from outside the Control Room, in addition to responding to the event that required the evacuation of the Control Room, will present challenges to plant operators and other on-shift personnel. Activation of the ERO and emergency response facilities will assist in responding to these challenges.

Transfer of plant control begins when the last licensed operator leaves the Control Room.

Control will be established at the Auxiliary Shutdown Panel if the Control Room was evacuated for any reason (ref. 1, 2, 3).

Escalation of the emergency classification level would be via IC HS6.

Reference(s):

1. EOP 3503, "Shutdown Outside the Control Room"
2. EOP 3504, "Cooldown Outside the Control Room"
3. EOP 3509,.1 "Control Room, Cable Spreading Room Area or Instrument Rack Room Fire"
4. NEI 99-01 HA6

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 6 – Control Room Evacuation
Initiating Condition: Inability to control a key safety function from outside the Control Room
EAL:

HS6.1 Site Area Emergency

An event has resulted in plant control being transferred from the Control Room to the Auxiliary Shutdown Panel

AND

Control of **any** of the following key safety functions is **not** re-established within 15 min. of the last licensed operator leaving the Control Room (Note 1):

- Reactivity (modes 1, 2 and 3 **only**)
- Core cooling
- RCS heat removal

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 - Hot Standby, 4 – Hot Shutdown, 5 – Cold Shutdown, 6 – Refueling

Definition(s):

None

Basis:

This IC addresses an evacuation of the Control Room that results in transfer of plant control to alternate locations, and the control of a key safety function cannot be reestablished in a timely manner. The failure to gain control of a key safety function following a transfer of plant control to alternate locations is a precursor to a challenge to one or more fission product barriers within a relatively short period of time.

The determination of whether or not “control” is established at the remote safe shutdown location(s) is based on ~~Emergency Director~~DSEO/ADTS judgment. The ~~Emergency Director~~DSEO/ADTS is expected to make a reasonable, informed judgment within 15 (the site-specific time for transfer) minutes whether or not the operating staff has control of key safety functions from the remote safe shutdown location(s).

Transfer of plant control and the time period to establish control begins when the last licensed operator leaves the Control Room.

Control will be established at the Auxiliary Shutdown Panel if the Control Room was evacuated for any reason (ref. 1, 2, 3).

Establishment of the reactivity safety function is only applicable in Modes 1, 2 and 3. Sufficient shutdown margin has already been established once in modes 4, 5 and 6 (ref.4).

Escalation of the emergency classification level would be via IC FG1 or CG1

Reference(s):

1. EOP 3503, "Shutdown Outside the Control Room"
2. EOP 3504, "Cooldown Outside the Control Room"
3. EOP 3509.1, "Control Room, Cable Spreading Room Area or Instrument Rack Room Fire"
4. NRC EP FAQ 2015-014
5. NEI 99-01 HS6

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions existing that in the judgment of the DSEO warrant declaration of a Unusual Event

EAL:

HU7.1 Unusual Event

Other conditions exist which in the judgment of the DSEO indicate that events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate a security threat to facility protection has been initiated. **No** releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of SAFETY SYSTEMS occurs.

Mode Applicability:

All

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the Emergency Director/DSEO to fall under the emergency classification level description for an Unusual Event ~~NOUE~~.

Reference(s):

1. NEI 99-01 HU7

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions exist that in the judgment of the DSEO/ADTS warrant declaration of an Alert

EAL:

HA7.1 Alert

Other conditions exist which, in the judgment of the DSEO/ADTS, indicate that events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. **Any** releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the Emergency Director DSEO/ADTS to fall under the emergency classification level description for an Alert.

Reference(s):

1. NEI 99-01 HA7

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions existing that in the judgment of the DSEO/ADTS warrant declaration of a Site Area Emergency

EAL:

HS7.1 Site Area Emergency

Other conditions exist which in the judgment of the DSEO/ADTS indicate that events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage or malicious acts, (1) toward site personnel or equipment that could lead to the likely failure of or, (2) that prevent effective access to equipment needed for the protection of the public. **Any** releases are **not** expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels beyond the SITE BOUNDARY

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the Emergency Director/DSEO/ADTS to fall under the emergency classification level description for a SITE AREA EMERGENCY.

Millstone Power Station Unit 3
Emergency Action Level Technical Bases Document
Attachment 1 Emergency Action Level Technical Bases

Serial No. 18-364
Docket Nos. 50-423; 72-47
Enclosure 3; Attachment 2C

Reference(s):

1. NEI 99-01 HS7

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions exist that in the judgment of the DSEO/ADTS warrant declaration of a General Emergency

EAL:

HG7.1 General Emergency

Other conditions exist which in the judgment of the DSEO/ADTS indicate that events are in progress or have occurred which involve actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity or HOSTILE ACTION that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels offsite for more than the immediate site area.

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should not be construed to include acts of civil disobedience or felonious acts that are not part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the Emergency Director/DSEO/ADTS to fall under the emergency classification level description for a GENERAL EMERGENCY.

Millstone Power Station Unit 3
Emergency Action Level Technical Bases Document
Attachment 1 Emergency Action Level Technical Bases

Serial No. 18-364
Docket Nos. 50-423; 72-47
Enclosure 3; Attachment 2C

Reference(s):

1. NEI 99-01 HG7

Category M – System Malfunction

EAL Group: Hot Conditions (RCS temperature > 200°F); EALs in this category are applicable only in one or more hot operating modes.

Numerous system-related equipment failure events that warrant emergency classification have been identified in this category. They may pose actual or potential threats to plant safety.

The events of this category pertain to the following subcategories:

1. Loss of Emergency AC Power

Loss of emergency plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of onsite and offsite power sources for 4.16 KV emergency buses.

2. Loss of Vital DC Power

Loss of emergency plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of power to or degraded voltage on the 125V DC vital buses.

3. Loss of Control Room Indications

Certain events that degrade plant operator ability to effectively assess plant conditions within the plant warrant emergency classification. Losses of indicators are in this subcategory.

4. RCS Activity

During normal operation, reactor coolant fission product activity is very low. Small concentrations of fission products in the coolant are primarily from the fission of tramp uranium in the fuel clad or minor perforations in the clad itself. Any significant rise from these base-line levels (2% - 5% clad failures) is indicative of fuel failures and is covered under the Fission Product Barrier Degradation category. However, lesser amounts of clad damage may result in coolant activity exceeding Technical Specification limits. These fission products will be circulated with the reactor coolant and can be detected by coolant sampling.

5. RCS Leakage

The reactor vessel provides a volume for the coolant that covers the reactor core. The reactor pressure vessel and associated pressure piping (reactor coolant system) together provide a barrier to limit the release of radioactive material should the reactor fuel clad integrity fail. Excessive RCS leakage greater than Technical Specification limits indicates potential pipe cracks that may propagate to an extent threatening fuel clad, RCS and containment integrity.

6. RPS Failure

This subcategory includes events related to failure of the Reactor Protection System (RPS) to initiate and complete reactor trips. In the plant licensing basis, postulated failures of the RPS to complete a reactor trip comprise a specific set of analyzed events referred to as Anticipated Transient Without Scram (ATWS) events. For EAL classification, however, ATWS is intended to mean any scram failure event that does not achieve reactor shutdown. If RPS actuation fails to properly result in reactor shutdown, positive control of reactivity is at risk and could cause a threat to fuel clad, RCS and containment integrity.

7. Loss of Communications

Certain events that degrade plant operator ability to effectively communicate with essential personnel within or external to the plant warrant emergency classification.

8. Containment Failure

Failure of containment isolation capability (under conditions in which the containment is not currently challenged) warrants emergency classification. Failure of containment pressure control capability also warrants emergency classification.

9. Hazardous Event Affecting Safety Systems

Various natural and technological events that result in degraded plant safety system train performance or significant VISIBLE DAMAGE warrant emergency classification under this subcategory.

Category: M – System Malfunction

Subcategory: 1 – Loss of Emergency AC Power

Initiating Condition: Loss of **all** offsite AC power capability to emergency buses for 15 minutes or longer

EAL:

MU1.1 Unusual Event

Loss of **all** offsite AC power capability, Table M-1, to 4.16 kV emergency buses 34C and 34D for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-1 AC Power Sources	
Offsite	
<ul style="list-style-type: none"> • Unit 3 Normal Station Service Transformer (NSST) • Unit 3 Reserve Station Service Transformer (RSST) 	
Onsite	
<ul style="list-style-type: none"> • EDG A • EDG B • SBO Diesel Generator (if already aligned) 	

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

Table M-1 provides a list of offsite AC electrical power sources credited for this EAL.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC addresses a prolonged loss of offsite power. The loss of offsite power sources renders the plant more vulnerable to a complete loss of power to AC emergency buses. This condition represents a potential reduction in the level of safety of the plant.

For emergency classification purposes, “capability” means that an offsite AC power source(s) is available to the emergency buses, whether or not the buses are powered from it.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of offsite power.

Escalation of the emergency classification level would be via IC SA4MA1.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS3 FSAR Section 8.2, "Off Site Power System"
3. MPS3 FSAR Section 8.3, "On Site Power System"
4. AOP 3577, "Loss of Normal and Offsite Power to a 4.16 kV Emergency Bus"
5. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
6. NEI 99-01 SU1

Category: M – System Malfunction
Subcategory: 1 – Loss of Emergency AC Power
Initiating Condition: Loss of **all but one** AC power source to emergency buses for 15 minutes or longer

EAL:

MA1.1 Alert

AC power capability, Table M-1, to 4.16 kV emergency buses 34C and 34D reduced to a single power source for ≥ 15 min. (Note 1)

AND

Any additional single power source failure will result in loss of **all** AC power to SAFETY SYSTEMS

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-1 AC Power Sources	
Offsite	
<ul style="list-style-type: none"> • Unit 3 Normal Station Service Transformer (NSST) • Unit 3 Reserve Station Service Transformer (RSST) 	
Onsite	
<ul style="list-style-type: none"> • EDG A • EDG B • SBO Diesel Generator (if already aligned) 	

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

Table M-1 provides a list of offsite and onsite AC electrical power sources credited for this EAL. The AC power source (SBO Diesel Generator) annotated "(if already aligned)" requires more than 15 minutes to establish and therefore is only credited if the source was already aligned at the time of AC power loss.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC describes a significant degradation of offsite and onsite AC power sources such that any additional single failure would result in a loss of all AC power to SAFETY SYSTEMS. In this condition, the sole AC power source may be powering one, or more than one, train of safety-related equipment. This IC provides an escalation path from IC ~~SU1~~MU1.

An "AC power source" is a source recognized in AOPs and EOPs, and capable of supplying required power to an emergency bus. Some examples of this condition are presented below.

- A loss of all offsite power with a concurrent failure of all but one emergency power source (e.g., an onsite diesel generator).
- A loss of all offsite power and loss of all emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being back-fed from the unit main generator ~~SBO Diesel Generator~~.
- A loss of emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being back-fed from an offsite power source.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of power.

Escalation of the emergency classification level would be via IC ~~SS4~~MS1.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

This hot condition EAL is equivalent to the cold condition EAL CU2.1.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS3 FSAR Section 8.2, "Off Site Power System"
3. MPS3 FSAR Section 8.3, "On Site Power System"
4. AOP 3577, "Loss of Normal and Offsite Power to a 4.16 kV Emergency Bus"
5. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
6. NEI 99-01 SA1

Category: M – System Malfunction

Subcategory: 1 – Loss of Emergency AC Power

Initiating Condition: Loss of **all** offsite power and **all** onsite AC power to emergency buses for 15 minutes or longer

EAL:

MS1.1 Site Area Emergency

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 34C and 34D for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators, provided it can be aligned within the 15 minute classification criteria.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC addresses a total loss of AC power that compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. In addition, fission product barrier monitoring capabilities may be degraded under these conditions. This IC represents a condition that involves actual or likely major failures of plant functions needed for the protection of the public.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs AG4RG1, FG1 or SG4MG1.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

This hot condition EAL is equivalent to the cold condition EAL CA2.1.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS3 FSAR Section 8.2, "Off Site Power System"
3. MPS3 FSAR Section 8.3, "On Site Power System"
4. AOP 3577, "Loss of Normal and Offsite Power to a 4.16 kV Emergency Bus"
5. EOP 35 ECA-0.0, "Loss of All AC Power"
6. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
7. NEI 99-01 SS1

Category: M –System Malfunction
Subcategory: 1 – Loss of Vital AC Power
Initiating Condition: Prolonged loss of **all** offsite and **all** onsite AC power to emergency buses

EAL:

MG1.1 General Emergency

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 34C and 34D

AND

Core Cooling-RED Path conditions met

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators (ref. 1, 2, 3, 4, 5).

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC addresses a prolonged loss of all power sources to AC emergency buses that results in degraded core cooling. A loss of all AC power compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. A prolonged loss of these buses will eventually lead to a loss of one or more fission product barriers. In addition, fission product barrier monitoring capabilities may be degraded under these conditions.

The degraded core cooling threshold is based on meeting the Core Cooling Red Path criteria for Core Exit Thermocouple readings (> 1,200 °F) (ref. 6).

The For extended loss of emergency bus AC power events that do not result in a breach of the RCS barrier, this EAL should require declaration of a General Emergency prior to meeting the thresholds for IC FG1. This will allow additional time for implementation of offsite protective actions.

Escalation of the emergency classification from Site Area Emergency will occur if it is projected that power cannot be restored to at least one AC emergency bus by the end of the analyzed station blackout coping period. Beyond this time, plant responses and event trajectory are subject to greater uncertainty, and there is an increased likelihood of challenges to multiple fission product barriers.

The estimate for restoring at least one emergency bus should be based on a realistic appraisal of the situation. Mitigation actions with a low probability of success should not be used as a basis for delaying a classification upgrade. The goal is to maximize the time available to prepare for, and implement, protective actions for the public.

The EAL will also require a General Emergency declaration if the loss of AC power results in parameters that indicate an inability to adequately remove decay heat from the core.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A. MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS3 FSAR Section 8.2, "Off Site Power System"
3. MPS3 FSAR Section 8.3, "On Site Power System"
4. AOP 3577, "Loss of Normal and Offsite Power to a 4.16 kV Emergency Bus"
5. EOP 35 ECA-0.0, "Loss of All AC Power"
6. EOP 35 F-0.2, "CSFST Core Cooling"
7. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"

8. NEI 99-01 SG1

Category: M – System Malfunction
Subcategory: 2 – Loss of Vital DC Power
Initiating Condition: Loss of **all** vital DC power for 15 minutes or longer
EAL:

MS2.1 Site Area Emergency

Indicated voltage is < 105 VDC on **both** vital 125 VDC battery buses 1 **AND** 2 for ≥15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

The vital (Class 1E) 125 V DC power system consists of four physically and electrically separated channels (buses). Each bus has a 60 cell battery bank with a two-hour rating before reaching the minimum bus voltage of 105 Volts (1.75 volts/cell).

Battery buses 1 and 2 power reactor protection, engineered safety features, vital loads and instrumentation (ref. 1, 2, 3). Battery buses 3 and 4 are not credited for this EAL.

This IC addresses a loss of vital DC power which compromises the ability to monitor and control SAFETY SYSTEMS. In modes above Cold Shutdown, this condition involves a major failure of plant functions needed for the protection of the public.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs AG4RG1, FG1 or MG1SG8.

This hot condition EAL equivalent of the cold condition EAL CU4.1.

Reference(s):

1. MPS3 FSAR Section 8.3.2, "DC Power Systems"
2. AOP 3563, "Loss of DC Bus Power"
3. Technical Specifications Section 3.8.2.1, "D.C. Sources"
4. NEI 99-01 SS8

Category: M –System Malfunction
Subcategory: 2 – Loss of Vital DC Power
Initiating Condition: Loss of **all** emergency AC and vital DC power sources for 15 minutes or longer

EAL:

MG2.1 General Emergency

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 34C and 34D for ≥ 15 min. (Note 1)

AND

Indicated voltage is < 105 VDC on **both** vital 125 VDC battery buses 1 **AND** 2 for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This IC addresses a concurrent and prolonged loss of both emergency AC and vital DC power. A loss of all emergency AC power compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. A loss of vital DC power compromises the ability to monitor and control SAFETY SYSTEMS. A sustained loss of both emergency AC and vital DC power will lead to multiple challenges to fission product barriers.

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators, provided it can be aligned within the 15 minute classification criteria (ref. 1, 2, 3, 4, 5).

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A. MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

The vital (Class 1E) 125 V DC power system consists of four physically and electrically separated channels (buses). Each bus has a 60 cell battery bank with a two-hour rating before reaching the minimum bus voltage of 105 Volts (1.75 volts/cell).

Battery buses 1 and 2 power reactor protection, engineered safety features, vital loads and instrumentation (ref. 6, 7, 8). Battery buses 3 and 4 are not credited for this EAL.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses. The 15-minute emergency declaration clock begins at the point when both EAL thresholds are met.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS3 FSAR Section 8.2, "Off Site Power System"
3. MPS3 FSAR Section 8.3, "On Site Power System"
4. AOP 3577, "Loss of Normal and Offsite Power to a 4.16 kV Emergency Bus"
5. EOP 35 ECA-0.0, "Loss of All AC Power"
6. MPS3 FSAR Section 8.3.2, "DC Power Systems"
7. AOP 3563, "Loss of DC Bus Power"
8. Technical Specifications Section 3.8.2.1, "D.C. Sources"
9. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
10. NEI 99-01 SG8

Category: M – System Malfunction
Subcategory: 3 – Loss of Control Room Indications
Initiating Condition: UNPLANNED loss of Control Room indications for 15 minutes or longer

EAL:

MU3.1 Unusual Event

An UNPLANNED event results in the inability to monitor one or more Table M-2 parameters from within the Control Room for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-2 Safety System Parameters

- Reactor power
- RCS level
- RCS pressure
- CET temperature
- Level in at least one SG
- Auxiliary feedwater flow to at least one SG

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

Applicable safety system parameters are listed in Table M-2.

The Plant Process Computer System and Safety Parameter Display System (SPDS) serve as redundant indicators which may be utilized as compensatory measures in lieu of the Control Room indicators associated with safety functions (ref. 1, 2, 3).

This IC addresses the difficulty associated with monitoring normal plant conditions without the ability to obtain SAFETY SYSTEM parameters from within the Control Room. This condition is a precursor to a more significant event and represents a potential degradation in the level of safety of the plant.

As used in this EAL, an "inability to monitor" means that values for one or more of the listed parameters cannot be determined from within the Control Room. This situation would require a loss of all of the Control Room sources for the given parameter(s). For example, the reactor power level cannot be determined from any analog, digital and recorder source within the Control Room.

An event involving a loss of plant indications, annunciators and/or display systems is evaluated in accordance with 10 CFR 50.72 (and associated guidance in NUREG-1022) to determine if an NRC event report is required. The event would be reported if it significantly impaired the capability to perform emergency assessments. In particular, emergency assessments necessary to implement abnormal operating procedures, emergency operating procedures, and emergency plan implementing procedures addressing emergency classification, accident assessment, or protective action decision-making.

This EAL is focused on a selected subset of plant parameters associated with the key safety functions of reactivity control, core cooling [~~PWR~~] / RPV level [~~BWR~~] and RCS heat removal. The loss of the ability to determine one or more of these parameters from within the Control Room is considered to be more significant than simply a reportable condition. In addition, if all indication sources for one or more of the listed parameters are lost, then the ability to determine the values of other SAFETY SYSTEM parameters may be impacted as well. For example, if the value for reactor vessel level [~~PWR~~] / RPV/RCS water level [~~BWR~~] cannot be determined from the indications and recorders on a main control board, the SPDS or the plant computer, the availability of other parameter values may be compromised as well.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation of the emergency classification level would be via IC MA3SA2.

Reference(s):

1. MPS3 FSAR Section 7.5, "Safety Related Display Instrumentation"
2. AOP 3571, "Instrument Failure Response"
3. AOP 3574, "Loss of Main Board Annunciation"
4. NEI 99-01 SU2

Category: M – System Malfunction
Subcategory: 3 – Loss of Control Room Indications
Initiating Condition: UNPLANNED loss of Control Room indications for 15 minutes or longer with a significant transient in progress

EAL:

MA3.1 Alert

An UNPLANNED event results in the inability to monitor one or more Table M-2 parameters from within the Control Room for ≥ 15 min. (Note 1)

AND

Any significant transient is in progress, Table M-3

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-2 Safety System Parameters

- Reactor power
- RCS level
- RCS pressure
- CET temperature
- Level in at least one SG
- Auxiliary feedwater flow to at least one SG

Table M-3 Significant Transients

- Automatic turbine runback > 25% reactor power
- Electrical load rejection > 25% full electrical load
- Reactor Trip
- SI Actuation

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

Applicable safety system parameters are listed in Table M-2.

Significant transients are listed in Table M-3.

The Plant Process Computer System and Safety Parameter Display System (SPDS) serve as redundant indicators which may be utilized as compensatory measures in lieu of the Control Room indicators associated with safety functions (ref. 1, 2, 3).

This IC addresses the difficulty associated with monitoring rapidly changing plant conditions during a transient without the ability to obtain SAFETY SYSTEM parameters from within the Control Room. During this condition, the margin to a potential fission product barrier challenge is reduced. It thus represents a potential substantial degradation in the level of safety of the plant.

As used in this EAL, an "inability to monitor" means that values for one or more of the listed parameters cannot be determined from within the Control Room. This situation would require a loss of all of the Control Room sources for the given parameter(s). For example, the reactor power level cannot be determined from any analog, digital and recorder source within the Control Room.

An event involving a loss of plant indications, annunciators and/or display systems is evaluated in accordance with 10 CFR 50.72 (and associated guidance in NUREG-1022) to determine if an NRC event report is required. The event would be reported if it significantly impaired the capability to perform emergency assessments. In particular, emergency assessments necessary to implement abnormal operating procedures, emergency operating procedures, and emergency plan implementing procedures addressing emergency classification, accident assessment, or protective action decision-making.

This EAL is focused on a selected subset of plant parameters associated with the key safety functions of reactivity control, core cooling [~~PWR~~] / RPV level [~~BWR~~] and RCS heat removal. The loss of the ability to determine one or more of these parameters from within the Control Room is considered to be more significant than simply a reportable condition. In addition, if all

indication sources for one or more of the listed parameters are lost, then the ability to determine the values of other SAFETY SYSTEM parameters may be impacted as well. For example, if the value for reactor vessel level [~~PWR~~] / RPV RCS water level [~~BWR~~] cannot be determined from the indications and recorders on a main control board, the SPDS or the plant computer, the availability of other parameter values may be compromised as well.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation of the emergency classification level would be via ICs FS1 or ~~IC-AS1~~ RS1

Reference(s):

1. MPS3 FSAR Section 7.5, "Safety Related Display Instrumentation"
2. AOP 3571, "Instrument Failure Response"
3. AOP 3574, "Loss of Main Board Annunciation"
4. NEI 99-01 SA2

Category: M – System Malfunction

Subcategory: 4 – RCS Activity

Initiating Condition: RCS activity greater than Technical Specification allowable limits

EAL:

MU4.1 Unusual Event

Dose rate at 1 ft. from an unpressurized RCS sample \geq Table M-4

Table M-4 Tech. Spec. Coolant Activity Dose Rates	
Time > Shutdown (hrs)	mR/hr/ml
≤ 2	0.7
$> 2 - \leq 8$	0.5
> 8	0.3

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

This IC addresses a reactor coolant activity value that exceeds an allowable limit specified in Technical Specifications. This condition is a precursor to a more significant event and represents a potential degradation of the level of safety of the plant.

Per Engineering Calculation RA-0059 (ref. 1), dose rate is assumed to result from radioactive iodines (I-131 thru I-135) in RCS in concentrations corresponding to 60 μ Ci/gm DEI-131. This value corresponds to the Technical Specification coolant activity limit for iodine spike at full power operations (ref. 2). The values contained in Table M-4 (Tech. Spec. Coolant Activity Dose Rates) represent expected one foot dose rates per ml of sample based on time since reactor shutdown to the time when the sample is taken (ref. 3). Time after shutdown values are provided to account for radioactive decay. The expected dose rate is a near linear relationship with the volume of the sample, so any volume collected can be determined by dividing the measured dose rate by the sample volume and comparing to the threshold value from Table M-4 for the applicable time frame. These dose rates assume no emergency core cooling system (ECCS) injection so there is no dilution credited which would vary coolant volume. Values in the table have been rounded for ease of use. The > 8 hour threshold is conservative up to 24 hours following reactor shutdown. After 24 hours, the expected response from radioactive iodine levels off. Therefore, the value shown for > 8 hours applies for all samples taken 8 hours or more since reactor shutdown.

The values specified in Table M-4 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Values were chosen to minimize error from the highest to lowest dose rate within each range.

It should be noted that this EALs is primarily directed toward mechanical damage to the clad not involving inadequate core cooling (ICC) sequences. Clad damage due to ICC sequences is addressed by the fuel clad and CTMT fission product barrier thresholds (Category F).

Escalation of the emergency classification level would be via ICs FA1 or the Recognition Category A-R ICs.

Reference(s):

1. RA-0059, "Detector Response to an RCS Sample for EAL Classification of Fuel Clad Degradation and Barrier Loss"
2. MPS3 Technical Specification 3.4.8, "RCS Specific Activity"
3. AOP 3553, "High RCS Activity"
4. NEI 99-01 SU3

Category: M – System Malfunction

Subcategory: 4 – RCS Activity

Initiating Condition: Reactor coolant activity greater than Technical Specification allowable limits

EAL:

MU4.2 Unusual Event

Sample analysis indicates that a reactor coolant activity value is > an allowable limit specified in Technical Specification 3.4.8

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

This IC addresses a reactor coolant activity value that exceeds an allowable limit specified in Technical Specifications. This condition is a precursor to a more significant event and represents a potential degradation of the level of safety of the plant.

Escalation of the emergency classification level would be via ICs FA1 or the Recognition Category A-R ICs.

Reference(s):

1. Technical Specification Section 3.4.8, "RCS Specific Activity"
2. AOP 3553, "High RCS Activity"
3. NEI 99-01 SU3

Category: M – System Malfunction
Subcategory: 5 – RCS Leakage
Initiating Condition: RCS leakage for 15 minutes or longer
EAL:

MU5.1 Unusual Event

RCS unidentified or pressure boundary leakage > 10 gpm for ≥ 15 min.

OR

RCS identified leakage > 25 gpm for ≥ 15 min.

OR

Leakage from the RCS to a location outside containment > 25 gpm for ≥ 15 min.

(Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

Basis:

The 15-minute threshold duration allows sufficient time for prompt operator actions to isolate the leakage, if possible.

Once the RCS leak rate has been quantified to be greater than the specified value, failure to isolate the leak within 15 minutes, or if known that the leak cannot be isolated within 15 minutes, from the time of leak rate quantification, requires immediate classification.

This IC addresses RCS leakage which may be a precursor to a more significant event. In this case, RCS leakage has been detected and operators, following applicable procedures, have been unable to promptly isolate the leak. This condition is considered to be a potential degradation of the level of safety of the plant.

The first and second EAL conditions EAL #1 and EAL #2 are focused on a loss of mass from the RCS due to "unidentified leakage", "pressure boundary leakage" or "identified leakage" (as these leakage types are defined in the plant Technical Specifications) (ref. 1, 2). The third condition EAL #3 addresses an RCS mass loss caused by an UNISOLABLE leak through an interfacing system. These conditions EALs thus apply to leakage into the containment, a secondary-side system (e.g., steam generator tube leakage in a PWR) or a location outside of containment.

The leak rate values for each condition EAL were selected because they are usually observable with normal Control Room indications. Lesser values typically require time-consuming calculations to determine (e.g., a mass balance calculation). The first condition

EAL #1 uses a lower value that reflects the greater significance of unidentified or pressure boundary leakage.

The release of mass from the RCS due to the as-designed/expected operation of a relief valve does not warrant an emergency classification. ~~For PWRs, a~~ An emergency classification would be required if a mass loss is caused by a relief valve that is not functioning as designed/expected (e.g., a relief valve sticks open and the line flow cannot be isolated). ~~For BWRs, a stuck open Safety Relief Valve (SRV) or SRV leakage is not considered either identified or unidentified leakage by Technical Specifications and, therefore, is not applicable to this EAL.~~

Escalation of the emergency classification level would be via ICs of Recognition Category A-R or F.

Reference(s):

1. Technical Specification Section 1.0, "Definitions"
2. Technical Specification Section 3.4.6, "RCS Leakage"
3. AOP 3555, "RCS Leak"
4. NEI 99-01 SU4

Category: M – System Malfunction

Subcategory: 6 – RPS Failure

Initiating Condition: Automatic or manual trip fails to shut down the reactor

EAL:

MU6.1 Unusual Event

An automatic trip did **not** shut down the reactor as indicated by reactor power > 5% after any RPS setpoint is exceeded

AND

A subsequent automatic trip or **EITHER** manual trip (Rx Trip Switches **OR** opening breakers 32B and 32N) are successful in shutting down the reactor as indicated by reactor power $\leq 5\%$ (Note 8)

Note 8: A manual trip action is **any** operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does **not** include manually driving in control rods or implementation of boron injection strategies.

Mode Applicability:

1 - Power Operation

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

This ~~IC-EAL~~ addresses a failure of the RPS to initiate or complete an automatic or manual reactor (trip [PWR] / scram [BWR]) that results in a reactor shutdown, and either a subsequent operator manual action taken at the reactor control consoles or an automatic (trip [PWR] / scram [BWR]) (i.e., any subsequent RPS setpoint trip) is successful in shutting down the reactor. This event is a precursor to a more significant condition and thus represents a potential degradation of the level of safety of the plant.

This EAL is applicable to failure to trip events initiated in Mode 1 only, since the power level specified for a successful shutdown (5%) aligns with the Mode 1 transition power (5%) (ref. 1, 5).

Following the failure on an automatic reactor (trip [PWR] / scram [BWR]), operators will promptly initiate manual actions at the reactor control consoles to shutdown the reactor (e.g., initiate a manual reactor (trip [PWR] / scram [BWR]) using Rx Trip Switches or opening breakers 32B and 32N). If these manual actions are successful in shutting down the reactor, core heat generation will quickly fall to a level within the capabilities of the plant's decay heat removal systems ($\leq 5\%$).

If an initial manual reactor (trip [PWR] / scram [BWR]) is unsuccessful, operators will promptly take manual action at another location(s) on the reactor control consoles to shutdown the reactor (e.g., initiate a manual reactor (trip [PWR] / scram [BWR]) using a different switch). Depending upon several factors, the initial or subsequent effort to manually (trip [PWR] / scram [BWR]) the reactor, or a concurrent plant condition, may lead to the generation of an automatic

~~reactor (trip [PWR] / scram [BWR]) signal. If a subsequent manual or automatic (trip [PWR] / scram [BWR]) is successful in shutting down the reactor, core heat generation will quickly fall to a level within the capabilities of the plant's decay heat removal systems.~~

A manual action at the reactor control consoles is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor (trip using Rx Trip Switches or opening breakers 32B and 32N [PWR] / scram [BWR])). This action does not include manually driving in control rods or implementation of boron injection strategies. Actions taken at back panels or other locations within the Control Room, or any location outside the Control Room, are not considered to be "at the reactor control consoles". Taking the Reactor Mode Switch to SHUTDOWN is considered to be a manual scram action. [BWR]

The plant response to the failure of an automatic or manual reactor (trip [PWR] / scram [BWR]) will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other concurrent plant conditions, etc. If subsequent operator manual actions taken at the reactor control consoles are also unsuccessful in shutting down the reactor, then the emergency classification level will escalate to an Alert via IC SA5MA6. Depending upon the plant response, escalation is also possible via IC FA1. Absent the plant conditions needed to meet either IC SA5-MA6 or FA1, an Unusual Event declaration is appropriate for this event.

A reactor shutdown ($\leq 5\%$) is determined in accordance with EOP 35 F-0.1 Subcriticality Red Path applicable Emergency Operating Procedure criteria (ref. 3, 4, 5).

Should a reactor (trip [PWR] / scram [BWR]) signal be generated as a result of plant work (e.g., RPS setpoint testing), the following classification guidance should be applied.

- If the signal causes a plant transient that should have included an automatic reactor (trip [PWR] / scram [BWR]) and the RPS fails to automatically shutdown the reactor, then this IC and the EALs are applicable, and should be evaluated.
- If the signal does not cause a plant transient and the (trip [PWR] / scram [BWR]) failure is determined through other means (e.g., assessment of test results), then this IC and the EALs are not applicable and no classification is warranted.

In the event that the operator identifies a reactor trip is IMMINENT and initiates a successful manual reactor trip before the automatic RPS trip setpoint is reached, no declaration is required. The successful manual trip of the reactor before it reaches its automatic trip setpoint or reactor trip signals caused by instrumentation channel failures do not lead to a potential fission product barrier loss. However, if subsequent manual reactor trip actions fail to shutdown the reactor, the event escalates to the Alert under EAL MA6.1.

Reference(s):

1. Technical Specification Table 1.2, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Trip System Instrumentation"
3. EOP 35, E-0 "Reactor Trip or Safety Injection"
4. EOP 35, F-0.1 "Subcriticality"
5. EOP FR-S.1, "Response to Nuclear Power Generation/ATWS"
6. NEI 99-01 SU5

Category: M – System Malfunction

Subcategory: 6 – RPS Failure

Initiating Condition: Automatic or manual trip fails to shut down the reactor

EAL:

MU6.2 Unusual Event

A manual trip (Rx Trip Switches OR opening breakers 32B and 32N) did **not** shut down the reactor as indicated by reactor power > 5%

AND

A subsequent manual trip OR automatic trip is successful in shutting down the reactor as indicated by reactor power $\leq 5\%$ (Note 8)

Note 8: A manual trip action is **any** operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does **not** include manually driving in control rods or implementation of boron injection strategies.

Mode Applicability:

1 - Power Operation

Definition(s):

None

Basis:

This ~~IC-EAL~~ addresses a failure of the RPS to initiate or complete an automatic or a manual reactor (trip [PWR] / scram [BWR]) that results in a reactor shutdown, and either a subsequent operator manual action taken at the reactor control consoles or an automatic (trip [PWR] / scram [BWR]) is successful in shutting down the reactor. This event is a precursor to a more significant condition and thus represents a potential degradation of the level of safety of the plant.

~~Following the failure on an automatic reactor (trip [PWR] / scram [BWR]), operators will promptly initiate manual actions at the reactor control consoles to shutdown the reactor (e.g., initiate a manual reactor (trip [PWR] / scram [BWR])). If these manual actions are successful in shutting down the reactor, core heat generation will quickly fall to a level within the capabilities of the plant's decay heat removal systems.~~

This EAL is applicable to failure to trip events initiated in Mode 1 only, since the power level specified for a successful shutdown (5%) aligns with the Mode 1 transition power (5%) (ref. 1, 5).

If an initial manual reactor (trip [PWR] / scram [BWR]) is unsuccessful, operators will promptly take manual actions at another location(s) on the reactor control consoles to shutdown the reactor (e.g., initiate a manual reactor (trip [PWR] / scram [BWR])) using a different switch or breaker).

Depending upon several factors, the initial or subsequent effort to manually (trip [PWR] / scram [BWR]) the reactor, or a concurrent plant condition, may lead to the generation of an automatic reactor (trip [PWR] / scram [BWR]) signal. If a subsequent manual or automatic (trip [PWR] /

~~scram [BWR]~~ is successful in shutting down the reactor, core heat generation will quickly fall to a level within the capabilities of the plant's decay heat removal systems ($\leq 5\%$) (ref. 4, 5).

A manual action at the reactor control consoles is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor (trip using Rx Trip Switches or opening breakers 32B and 32N ~~[PWR] / scram [BWR]~~)). This action does not include locally tripping reactor trip and bypass breakers or tripping the turbine, manually driving in control rods or implementation of boron injection strategies. Actions taken at ~~back panels or other locations within the Control Room~~, or any location outside the Control Room, are not considered to be "at the reactor control consoles".

~~Taking the Reactor Mode Switch to SHUTDOWN is considered to be a manual scram action.~~
~~[BWR]~~

The plant response to the failure of an ~~automatic or manual reactor (trip [PWR] / scram [BWR])~~ will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other concurrent plant conditions, etc. If subsequent operator manual actions taken at the reactor control consoles are also unsuccessful in shutting down the reactor, then the emergency classification level will escalate to an Alert via IC SA5MA6. Depending upon the plant response, escalation is also possible via IC FA1. Absent the plant conditions needed to meet either IC SA5-MA6 or FA1, an Unusual Event declaration is appropriate for this event.

A reactor shutdown ($\leq 5\%$) is determined in accordance with EOP 35 F-0.1 Subcriticality Red Path criteria (ref. 3, 4, 5).

Should a reactor (trip ~~[PWR] / scram [BWR]~~) signal be generated as a result of plant work (e.g., RPS setpoint testing), the following classification guidance should be applied.

- If the signal causes a plant transient that should have included an automatic reactor (trip ~~[PWR] / scram [BWR]~~) and the RPS fails to automatically shutdown the reactor, then this IC and the EALs are applicable, and should be evaluated.
- If the signal does not cause a plant transient and the (trip ~~[PWR] / scram [BWR]~~) failure is determined through other means (e.g., assessment of test results), then this IC and the EALs are not applicable and no classification is warranted.

Reference(s):

1. Technical Specification Table 1.2, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Trip System Instrumentation"
3. EOP 35 E-0, "Reactor Trip or Safety Injection"
4. EOP 35 F-0.1, "Subcriticality"
5. EOP FR-S.1, "Response to Nuclear Power Generation/ATWS"
6. NEI 99-01 SU5

Category: M – System Malfunction

Subcategory: 2 – RPS Failure

Initiating Condition: Automatic or manual trip fails to shut down the reactor and subsequent manual actions taken at the reactor control consoles are **not** successful in shutting down the reactor

EAL:

MA6.1 Alert

An automatic or manual trip (Rx Trip Switches **OR** opening breakers 32B and 32N) did **not** shut down the reactor as indicated by reactor power > 5%

AND

Subsequent automatic or manual trip actions (Rx Trip Switches **AND** opening breakers 32B and 32N) are **not** successful in shutting down the reactor as indicated by reactor power > 5% (Note 8)

Note 8: A manual trip action is **any** operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does **not** include manually driving in control rods or implementation of boron injection strategies.

Mode Applicability:

1 - Power Operation

Definition(s):

None

Basis:

This IC addresses a failure of the RPS to initiate or complete an automatic reactor trip or failure of a manual reactor (trip [~~PWR~~] / scram [~~BWR~~]) that results in a reactor shutdown, and subsequent operator manual actions taken at the reactor control consoles to shutdown the reactor are also unsuccessful. This condition represents an actual or potential substantial degradation of the level of safety of the plant. An emergency declaration is required even if the reactor is subsequently shutdown by an action taken away from the reactor control consoles since this event entails a significant failure of the RPS.

This EAL is applicable to failure to trip events initiated in Mode 1 only, since the power level specified for a successful shutdown (5%) aligns with the Mode 1 transition power (5%) (ref. 1, 5).

A manual action at the reactor control consoles is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor (trip[~~PWR~~] / scram [~~BWR~~]) using Rx Trip Switches or opening breakers 32B and 32N). This action does not include locally tripping reactor trip and bypass breakers or tripping the turbine, manually driving in control rods or implementation of boron injection strategies. If this action(s) is unsuccessful, operators would immediately pursue additional manual actions at locations away from the reactor control consoles (e.g., locally opening breakers). Actions taken at back panels or other locations within the Control Room, or any location outside the Control Room, are not considered to be "at the reactor control consoles".

~~Taking the Reactor Mode Switch to SHUTDOWN is considered to be a manual scram action.~~
~~[BWR]~~

The plant response to the failure of an automatic or manual reactor (trip ~~[PWR]~~ / scram ~~[BWR]~~) will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other concurrent plant conditions, etc. If the failure to shut down the reactor is prolonged enough to cause a challenge to the core cooling ~~[PWR]~~ / RPV water level ~~[BWR]~~ or RCS heat removal safety functions, the emergency classification level will escalate to a Site Area Emergency via IC ~~SMS65~~. Depending upon plant responses and symptoms, escalation is also possible via IC FS1. Absent the plant conditions needed to meet either IC ~~SMS65~~ or FS1, an Alert declaration is appropriate for this event.

It is recognized that plant responses or symptoms may also require an Alert declaration in accordance with the ~~Recognition~~-Category F ICs; however, this IC and EAL are included to ensure a timely emergency declaration.

A reactor shutdown ($\leq 5\%$) is determined in accordance with applicable EOP 35 F-0.1 SUBCRITICALITY Red Path criteria (ref. 3, 4, 5).

Reference(s):

1. Technical Specification Table 1.2, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Trip System Instrumentation"
3. EOP 35 E-0, "Reactor Trip or Safety Injection"
4. EOP 35 F-0.1, "Subcriticality"
5. EOP FR-S.1, "Response to Nuclear Power Generation/ATWS"
6. NEI 99-01 SA5

Category: M – System Malfunction

Subcategory: 2 – RPS Failure

Initiating Condition: Inability to shut down the reactor causing a challenge to core cooling or RCS heat removal

EAL:

MS6.1 Site Area Emergency

An automatic or manual trip did **not** shut down the reactor as indicated by reactor power > 5%

AND

All actions taken to shut down the reactor are **not** successful as indicated by reactor power > 5%

AND EITHER:

- Core Cooling-RED Path conditions met
- Heat Sink-RED Path conditions met

Mode Applicability:

1 - Power Operation

Definition(s):

None

Basis:

This ~~IC-EAL~~ addresses a failure of the RPS to initiate or complete an automatic or manual reactor (trip [~~PWR~~] / scram [~~BWR~~]) that results in a reactor shutdown, all subsequent operator actions to manually shutdown the reactor are unsuccessful, and continued power generation is challenging the capability to adequately remove heat from the core and/or the RCS. This condition will lead to fuel damage if additional mitigation actions are unsuccessful and thus warrants the declaration of a Site Area Emergency.

This EAL is applicable to failure to trip events initiated in Mode 1 only, since the power level specified for a successful shutdown (5%) aligns with the Mode 1 transition power (5%) (ref. 1, 5).

Reactor shutdown achieved by use of other trip actions such as opening locally supply breakers, emergency boration, or manually driving control rods are also credited as a successful manual trip if reactor power is $\leq 5\%$ before indications of an extreme challenge to either core cooling or heat removal exist (ref. 1, 2, 3, 4).

In some instances, the emergency classification resulting from this IC/EAL may be higher than that resulting from an assessment of the plant responses and symptoms against the Recognition-Category F ICs/EALs. This is appropriate in that the Recognition-Category F ICs/EALs do not address the additional threat posed by a failure to shut down the reactor. The

inclusion of this IC and EAL ensures the timely declaration of a Site Area Emergency in response to prolonged failure to shut down the reactor.

A reactor shutdown ($\leq 5\%$) is determined in accordance with applicable EOP 35 F-0.1 SUBCRITICALITY Red Path Emergency Operating Procedure criteria (ref. 4).

A severe challenge to adequate core cooling is based on meeting the Core Cooling Red Path criteria for Core Exit Thermocouple readings ($> 1,200$ °F) (ref. 5, 6).

The severe challenge to RCS heat removal is based on meeting the Heat Sink Red Path criteria of both of the following conditions existing (ref. 7, 8):

- Narrow Range level in all SGs $< 8\%$ (42% adverse CTMT)
- Total feedwater flow to SGs < 530 gpm

Escalation of the emergency classification level would be via IC AG1-RG1 or FG1.

Reference(s):

1. Technical Specification Table 1.2, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Trip System Instrumentation"
3. EOP 35 E-0, "Reactor Trip or Safety Injection"
4. EOP 35 F-0.1, "Subcriticality"
5. EOP 35 F-0.2, "CSFST Core Cooling"
6. EOP 35 FR-C.1, "Response to Inadequate Core Cooling"
7. EOP 35 F-0.3, "CSFST Heat Sink"
8. EOP 35 FR-H.1, "Response to Loss of Secondary Heat Sink"
9. NEI 99-01 SS5

Category: M – System Malfunction
Subcategory: 7 – Loss of Communications
Initiating Condition: Loss of **all** onsite or offsite communications capabilities
EAL:

MU7.1 Unusual Event

Loss of **all** Table M-5 onsite communication methods

OR

Loss of **all** Table M-5 State and local agency communication methods

OR

Loss of **all** Table M-5 NRC communication methods

Table M-5 Communication Methods			
System	Onsite	State/ Local	NRC
ENRS / ARCOS		X	
Station Radio System	X	X	
Plant Phone System	X	X	
Public Address System	X		
Gaitronics / Maintenance Jacks	X		
Federal Telephone System (ENS)			X
Commercial Telephone System		X	X
Satellite Phones		X	X
Dedicated Hotlines		X	

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

This IC addresses a significant loss of on-site or offsite communications capabilities. While not a direct challenge to plant or personnel safety, this event warrants prompt notifications to ~~OROs~~ State and local agencies and the NRC.

This IC should be assessed only when extraordinary means are being utilized to make communications possible (e.g., use of non-plant, privately owned equipment, relaying of on-site information via individuals or multiple radio transmission points, individuals being sent to offsite locations, etc.).

The first EAL condition EAL #1 addresses a total loss of the communications methods used in support of routine plant operations.

The second EAL condition EAL #2 addresses a total loss of the communications methods used to notify all ~~OROs~~ State and local agencies of an emergency declaration. The ~~OROs~~ State and local agencies referred to here are the State of Connecticut and local communities. (see Developer Notes)

The third EAL EAL #3 addresses a total loss of the communications methods used to notify the NRC of an emergency declaration.

This hot condition EAL is equivalent to the cold condition EAL CU5.1.

Reference(s):

1. MPS Emergency Plan Section 7.9, "Communication Systems"
2. MP-26-EPI-FAP07, "Notifications and Communications"
3. NEI 99-01 SU6

Category: M – System Malfunction

Subcategory: 8 – Containment Failure

Initiating Condition: Failure to isolate containment or loss of containment pressure control

EAL:

MU8.1 Unusual Event

Any penetration is **not** closed within 15 min. of a VALID Phase A or B isolation signal

OR

CTMT pressure > 25 psia with < one full train of CTMT heat removal systems
(Note 11) operating per design for ≥15 min.

(Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 11: One full train of containment heat removal systems consist of one train of Recirculation Spray System (RSS) and one train of Quench Spray System (QSS).

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This ~~IC~~ EAL addresses a failure of one or more containment penetrations to automatically isolate (close) when required by an actuation signal. It also addresses an event that results in high containment pressure with a concurrent failure of containment pressure control systems. Absent challenges to another fission product barrier, either condition represents potential degradation of the level of safety of the plant.

For ~~EAL #1~~ the first condition, the containment isolation signal (Phase A or B) must be generated as the result on an off-normal/accident condition (e.g., a safety injection or high containment pressure); a failure resulting from testing or maintenance does not warrant classification. The determination of containment and penetration status – isolated or not isolated – should be made in accordance with the appropriate criteria contained in the plant AOPs and EOPs. The 15-minute criterion is included to allow operators time to manually isolate the required penetrations, if possible (ref. 4).

~~EAL #2~~ The second condition addresses a condition where containment pressure is greater than the setpoint (25 psia) at which containment energy (heat) removal systems are designed to automatically actuate, and less than one full train of equipment is capable of operating per design. One full train of containment heat removal systems consist of one train of Recirculation

Spray System (RSS) and one train of Quench Spray System (QSS) (ref. 1, 2). The combination of required equipment can be obtained from using equipment on either emergency busses in order to meet the "one full train" requirement.

The Quench Spray System (QSS) is activated immediately upon receipt of a CDA signal. QSS becomes effective within 70 seconds of initiation.

The containment Recirculation Spray System (RSS) is activated on receipt of the RWST Low-Low level signal coincident with a CDA signal

The 15-minute criterion is included to allow operators time to manually start equipment that may not have automatically started, if possible. The inability to start the required equipment indicates that containment heat removal/depressurization systems (e.g., containment sprays ~~or ice condenser fans~~) are either lost or performing in a degraded manner.

This event would escalate to a Site Area Emergency in accordance with IC FS1 if there were a concurrent loss or potential loss of either the Fuel Clad or RCS fission product barriers.

Reference(s):

1. MPS3 FSAR Section 6.2.2, "Containment Heat Removal System"
2. MPS3 Technical Specifications 3.6.2.1, "Containment Quench Spray and Recirculation Spray System"
3. EOP 35 FR-Z.1, "Response to High Containment Pressure"
4. MPS3 FSAR Section 6.2.4, "Containment Isolation System"
5. NEI 99-01 SU7

Category: M – System Malfunction
Subcategory: 9 – Hazardous Event Affecting Safety Systems
Initiating Condition: Hazardous event affecting SAFETY SYSTEMS needed for the current operating mode

EAL:

MA9.1 Alert

The occurrence of **any** Table M-6 hazardous event

AND

Event damage has caused indications of degraded performance on one train of a SAFETY SYSTEM needed for the current operating mode

AND EITHER:

- Event damage has caused indications of degraded performance to the second train of the SAFETY SYSTEM needed for the current operating mode
- Event damage has resulted in **VISIBLE DAMAGE** to the second train of the SAFETY SYSTEM needed for the current operating mode

(Notes 9, 10)

Note 9: If the affected SAFETY SYSTEM train was already inoperable or out of service before the hazardous event occurred, then emergency classification is **not** warranted.

Note 10: If the hazardous event **only** resulted in **VISIBLE DAMAGE**, with **no** indications of degraded performance to at least one train of a SAFETY SYSTEM, then this emergency classification is **not** warranted.

Table M-6 Hazardous Events

- Seismic event (earthquake)
- Internal or external FLOODING event
- High winds or tornado strike
- FIRE
- EXPLOSION
- Other events with similar hazard characteristics as determined by the DSEO/ADTS

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

EXPLOSION - A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding,

arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

VISIBLE DAMAGE - Damage to a SAFETY SYSTEM train that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected SAFETY SYSTEM train.

Basis:

This IC addresses a hazardous event that causes damage to SAFETY SYSTEMS needed for the current operating mode. In order to provide the appropriate context for consideration of an ALERT classification, the hazardous event must have caused indications of degraded SAFETY SYSTEM performance in one train, and there must be either indications of performance issues with the second SAFETY SYSTEM train or VISIBLE DAMAGE to the second train such that the potential exists for this second SAFETY SYSTEM train to have performance issues. In other words, in order for this EAL to be classified, the hazardous event must occur, at least one SAFETY SYSTEM train must have indications of degraded performance, and the second SAFETY SYSTEM train must have indications of degraded performance or VISIBLE DAMAGE such that the potential exists for performance issues. Note that this second SAFETY SYSTEM train is from the same SAFETY SYSTEM that has indications of degraded performance; commercial nuclear power plants are designed to be able to support single system issues without compromising public health and safety from radiological events.

Indications of degraded performance addresses damage to a SAFETY SYSTEM train that is in service/operation since indications for it will be readily available. The indications of degraded performance should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

VISIBLE DAMAGE addresses damage to a SAFETY SYSTEM train that is not in service/operation and that potentially could cause performance issues. Operators will make this determination based on the totality of available event and damage report information. This is intended to be a brief assessment not requiring lengthy analysis or quantification of the

damage. This VISIBLE DAMAGE should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

~~This IC addresses a hazardous event that causes damage to a SAFETY SYSTEM, or a structure containing SAFETY SYSTEM components, needed for the current operating mode. This condition significantly reduces the margin to a loss or potential loss of a fission product barrier, and therefore represents an actual or potential substantial degradation of the level of safety of the plant.~~

~~EAL 1.b.1 addresses damage to a SAFETY SYSTEM train that is in service/operation since indications for it will be readily available. The indications of degraded performance should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.~~

~~EAL 1.b.2 addresses damage to a SAFETY SYSTEM component that is not in service/operation or readily apparent through indications alone, or to a structure containing SAFETY SYSTEM components. Operators will make this determination based on the totality of available event and damage report information. This is intended to be a brief assessment not requiring lengthy analysis or quantification of the damage.~~

An event affecting equipment common to two or more trains of a safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the common equipment) should be classified as an Alert under this EAL, as appropriate to the plant mode. By affecting the functionality of multiple trains of a safety system, the loss of the common equipment effectively meets the two-train impact criteria that underlie the EALs and bases.

An event affecting a single-train safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the one train) would not be classified under this EAL because the two-train impact criteria that underlie the EALs and bases would not be met. If an event affects a single-train safety system, then the emergency classification should be made based on plant parameters/symptoms meeting the EALs for another IC. Depending upon the circumstances, classification may also occur based on Shift Manager/DSEO/ADTS judgement.

An event that affects two trains of a safety system (e.g., one train has indications of degraded performance and the other VISIBLE DAMAGE) that also has one or more additional trains should be classified as an Alert under this EAL, as appropriate to the plant mode. This approach maintains consistency with the two-train impact criteria that underlie the EALs and bases, and is warranted because the event was severe enough to affect the functionality of two trains of a safety system despite plant design criteria associated with system and system train separation and protection. Such an event may have caused other plant impacts that are not immediately apparent.

Escalation of the emergency classification level would be via IC FS1 or AS4RS1.

This hot condition EAL is equivalent of the cold condition EAL CA6.1.

Reference(s):

1. AOP 3569, "Severe Weather Conditions"
2. AOP 3570, "Earthquake"
3. EP FAQ 2016-002

4. NEI 99-01 SA9

Attachment 2 Safe Operation & Shutdown Rooms/Areas Tables R-2 & H-2 Bases

Background

NEI 99-01 Revision 6 ICs AA3 and HA5 prescribe declaration of an Alert based on IMPEDED access to rooms or areas (due to either area radiation levels or hazardous gas concentrations) where equipment necessary for normal plant operations, cooldown or shutdown is located. These areas are intended to be plant operating mode dependent. Specifically the Developers Notes for AA3 and HA5 states:

The "site-specific list of plant rooms or areas with entry-related mode applicability identified" should specify those rooms or areas that contain equipment which require a manual/local action as specified in operating procedures used for normal plant operation, cooldown and shutdown. Do not include rooms or areas in which actions of a contingent or emergency nature would be performed (e.g., an action to address an off-normal or emergency condition such as emergency repairs, corrective measures or emergency operations). In addition, the list should specify the plant mode(s) during which entry would be required for each room or area.

The list should not include rooms or areas for which entry is required solely to perform actions of an administrative or record keeping nature (e.g., normal rounds or routine inspections).

Further, as specified in IC HA5:

The list need not include the Control Room if adequate engineered safety/design features are in place to preclude a Control Room evacuation due to the release of a hazardous gas. Such features may include, but are not limited to, capability to draw air from multiple air intakes at different and separate locations, inner and outer atmospheric boundaries, or the capability to acquire and maintain positive pressure within the Control Room envelope.

Attachment 2 Safe Operation & Shutdown Rooms/Areas Tables R-2 & H-2 Bases

MPS3 Table R-2 and H-2 Bases

A review of station operating procedures identified the following mode dependent in-plant actions and associated areas that are required for normal plant operation, cooldown or shutdown:

In-Plant Actions (MPS3)	Safe Shutdown Area	Modes
Chemistry sample RCS for boron concentration	AB EI 43'	1
Chemistry sample RCS for boron concentration	AB EI 43'	2
Chemistry sample RCS for boron concentration	AB EI 43'	3
Throttle seal injection flows	AB 3'8"	3
Isolate SI Accumulators	AB 24' East MCC/RCA 24' West MCC/RCA 24'	3
Mode 4 gas accumulation testing for all trains for TS 3.4.1.3	Containment ESF Bldg A RHR Cubicle ESF Bldg B RHR Cubicle	4
Establish mass addition protection	East Switchgear West Switchgear	4
Align RHR A for Cooldown	ESF 24' ESF 36' AB 24' MCC/RCA 24'	4
Warmup RHR A Train	ESF 36'	4
Align RHR B for Cooldown	ESF 24' ESF 36' AB 24' MCC/RCA 24'	4
Warmup RHR B Train	ESF 36'	4

Control Room ventilation systems have adequate engineered safety/design features in place to preclude a Control Room evacuation due to the external release of a hazardous gas (FSAR Section 9.4.1). Therefore, the Control Room is not included in this assessment or in Table H-2.

Ref: OP 3204, "At Power Operation"
OP 3206, "Plant Shutdown"
OP 3208, "Reactor Shutdown"

Attachment 2 Safe Operation & Shutdown Rooms/Areas Tables R-2 & H-2 Bases

Table R-2 & H-2 Results

Table R-2/H-2 Safe Operation & Shutdown Rooms/Areas	
Room/Area	Mode
Aux. Building EI 43'	1, 2, 3
Aux. Building EI 24' East MCC/RCA EI 24' West MCC/RCA EI 24'	3, 4
Aux. Building EI 3' 8"	3
Containment ESF Building A RHR Cubicle ESF Building B RHR Cubicle ESF Building EI 36' ESF Building EI 24' East Switchgear West Switchgear	4

ATTACHMENT 3A

MPS1 EAL TECHNICAL BASES DOCUMENT (Final)

**Dominion Energy Nuclear Connecticut, Inc. (DENC)
Millstone Power Station Unit 1**

**Emergency Action Level Technical Bases Document
Millstone Power Station – Unit 1**

(Final)

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1.0 INTRODUCTION

This document provides an explanation and rationale for each Emergency Action Level (EAL) included in the NEI 99-01, Revision 6, EAL Upgrade Project for Millstone Power Station Unit 1 (MPS1). It should be used to facilitate review of the MPS1 EALs and provide historical documentation for future reference. Decision-makers responsible for implementation of MP-26-EPI-FAP06, Classification and PARs, may use this document as a technical reference in support of EAL interpretation. This information may assist the Director of Station Emergency Operations (DSEO)/Assistant Director Technical Support (ADTS) in making classifications, particularly those involving judgment or multiple events. The basis information may also be useful in training and for explaining event classifications to off-site officials.

The expectation is that emergency classifications are to be made as soon as conditions are present and recognizable for the classification, but within 15 minutes or less in all cases of conditions present. Use of this document for assistance is not intended to delay the emergency classification.

Since the information in a basis document can affect emergency classification decision-making (e.g., the DSEO/ADTS refers to it during an event), the NRC staff expects that changes to the basis document will be evaluated in accordance with the provisions of 10 CFR 50.54(q). For Dominion Energy sites, a 10 CFR 50.54(q)(3) screening/evaluation will be performed to evaluate changes to this document.

Dominion Energy fleet procedure CM-AA-400, "10 CFR 50.59 and 10 CFR 72.48 – Changes, Tests and Experiments," provides a method to determine the impacts to licensing basis documents when changes are proposed to procedures, including changes to Abnormal Operating Procedures (AOPs) and Emergency Operating Procedures (EOPs). The 50.59/72.48 applicability review specifically requires that the effect of a proposed procedure change on the Emergency Plan (and associated EALs) be reviewed/assessed. When impacts to the Emergency Plan are identified, a separate review in accordance to 10 CFR 50.54(q) will be performed to determine the acceptability of the proposed procedure change.

2.0 DISCUSSION

2.1 Background

EALs are the plant-specific indications, conditions or instrument readings that are utilized to classify emergency conditions defined in the Millstone Power Station (MPS) Emergency Plan.

In 2012, the NRC endorsed NEI 99-01, Revision 6, "Methodology for the Development of Emergency Action Levels for Non-Passive Reactors." This methodology includes guidance for development of EAL schemes for a Permanently Defueled station. This is a station that has generated spent fuel under a 10 CFR 50 license, has permanently ceased operations and stores the spent fuel onsite for an extended period of time. The emergency classification levels applicable to this type of station are consistent with the requirements of 10 CFR 50 and the guidance in NUREG 0654/FEMA-REP-1.

NEI 99-01, Revision 6, Recognition Category PD (Permanently Defueled) provides a stand-alone set of Initiating Conditions (ICs)/EALs for a Permanently Defueled station to consider for use in developing a site-specific emergency classification scheme. For development, it was

assumed that the station had operated under a 10 CFR 50 license and that the licensee has permanently ceased power operations. Further, the licensee intends to store the spent fuel within the plant for some period of time.

When in a permanently defueled condition, a licensee receives approval from the NRC for exemption from specific emergency planning requirements. These exemptions reflect the lowered radiological source term and risks associated with spent fuel pool storage relative to reactor at-power operation. Source terms and accident analyses associated with plausible accidents are documented in the station's Defueled Safety Analysis Report (DSAR), as updated. As a result, a licensee with a permanently defueled station must develop a site-specific emergency classification scheme using the NRC-approved exemptions, revised source terms, and revised accident analyses as documented in the station's DSAR.

Recognition Category PD uses the same Emergency Classification Levels (ECLs) as operating reactors; however, the source term and accident analyses typically limit the ECLs to an Unusual Event and Alert. The Unusual Event ICs provide for an increased awareness of abnormal conditions while the Alert ICs are specific to actual or potential impacts to spent fuel. The source terms and release motive forces associated with a permanently defueled station would not be sufficient to require declaration of a Site Area Emergency or General Emergency.

A permanently defueled station is essentially a spent fuel storage facility with the spent fuel stored in a pool of water that serves as both a cooling medium (i.e., removal of decay heat) and shield from direct radiation. These primary functions of the spent fuel storage pool are the focus of the Recognition Category PD ICs and EALs. Radiological effluent IC and EALs were included to provide a basis for classifying events that cannot be readily classified based on an observable event or plant conditions alone.

Appropriate ICs and EALs were included in Recognition Category PD to address a spectrum of events that may affect a spent fuel pool. The Recognition Category PD ICs and EALs reflect the relevant guidance in Section 3.0 of this document (e.g., the importance of avoiding both over-classification and under-classification). MPS1 has developed an emergency classification scheme using the NRC-approved exemptions and the source terms and accident analyses specific to the MPS1. While security-related events are also considered in the generic guidance, classification of security-related events impacting MPS1 are classified under the operating units (Millstone Power Station Units 2 and 3) security EALs.

2.2 EAL Organization and Technical Bases Information

EAL technical bases are provided in Attachment 1, grouped by EAL. A summary is given at the beginning of each group, which provides a brief description of the Category.

For each EAL, the following information is provided:

EAL Category designator and Title

See Table 2.2-1 below for the Category designators and associated titles that are used at MPS1.

EAL Subcategory designator and Title

See Table 2.2-1 below for the Subcategory designators and associated titles that are used at MPS1.

Initiating Condition (IC) wording

Site-specific description of the generic IC given in NEI 99-01, Rev. 6 and the EAL Classification Matrix

EAL identifier (enclosed in rectangle)

Each EAL is assigned a unique identifier to support accurate communication of the emergency classification to onsite and offsite personnel. Four characters define each EAL identifier as indicated below:

1. First character (letter): Corresponds to the EAL category as described above
2. Second character (letter): The emergency classification (A or U)

A = Alert

U = Unusual Event

3. Third character (number): Subcategory number within the given category. Subcategories are sequentially numbered beginning with the number one (1). If a category does not have a subcategory, this character is assigned the number one (1). Subcategories are used in the MPS1 scheme as necessary to further divide the EALs of a category into logical sets of possible emergency classification thresholds. The MPS1 EAL categories and subcategories are listed below.
4. Fourth character (number): The numerical sequence of the EAL within the EAL subcategory. If the subcategory has only one EAL, it is given the number one (1).

Classification (enclosed in rectangle):

Alert (A) or Unusual Event (U)

EAL wording (enclosed in rectangle)

Exact wording of the EAL as it appears in the EAL Classification Matrix

Notes (as applicable)

Definition(s):

If the EAL wording contains a defined term, the definition of the term is included in this section. These definitions can also be found in Section 5.1.

Basis:

An EAL basis section that provides MPS1-relevant information concerning the EAL as well as a description of the rationale for the EAL as provided in NEI 99-01, Rev. 6.

Reference(s):

Source documentation from which the EAL is derived.

Table 2.2-1: EAL Categories and Subcategories

EAL Category	EAL Subcategory
R – Abnormal Rad Levels / Rad Effluent	1 – Radiological Effluent 2 – Area Radiation Levels
H – Hazards	1 – Hazardous Events Affecting Plant Safety 2 – DSEO/ADTS Judgment
M – System Malfunction	1 – Loss of SFP Cooling

3.0 GUIDANCE ON MAKING EMERGENCY CLASSIFICATIONS

3.1 General Considerations

When making an emergency classification, the DSEO/ADTS must consider all information having a bearing on the proper assessment of an Initiating Condition (IC). This includes the EAL plus any associated Notes and the informing basis information.

3.1.1 Classification Timeliness

NRC regulations require the licensee to establish and maintain the capability to assess, classify, and declare an emergency condition within 15 minutes after the availability of indications to plant operators that an emergency action level has been exceeded and to promptly declare the emergency condition as soon as possible following identification of the appropriate emergency classification level. The NRC staff has provided guidance on implementing this requirement in NSIR/DPR-ISG-01, "Interim Staff Guidance, Emergency Planning for Nuclear Power Plants" (ref. 4.1.7).

3.1.2 Valid Indications

All emergency classification assessments shall be based upon valid indications, reports or conditions. A valid indication, report, or condition, is one that has been verified through appropriate means such that there is no doubt regarding the indicator's operability, the condition's existence, or the report's accuracy.

An indication, report, or condition is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt is removed related to the indicator's operability, the condition's existence, or the report's accuracy. Implicit in this definition is the need for timely assessment.

3.1.3 IMMINENT Conditions

For ICs and EALs that have a stipulated time duration (e.g., 15 minutes, 60 minutes, etc.), the DSEO/ADTS should not wait until the applicable time has elapsed, but should declare the event as soon as it is determined that the condition has exceeded, or will likely exceed, the applicable time. If an ongoing radiological release is detected and the release start time is unknown, it should be assumed that the release duration specified in the IC/EAL has been exceeded, absent data to the contrary.

3.1.4 Planned vs. Unplanned Events

A planned work activity that results in an expected event or condition which meets or exceeds an EAL does not warrant an emergency declaration provided that: 1) the activity proceeds as planned, and 2) the plant remains within the limits imposed by the operating license. Such activities include planned work to test, manipulate, repair, maintain or modify a system or component. In these cases, the controls associated with the planning, preparation and execution of the work will ensure that compliance is maintained with all aspects of the operating license provided that the activity proceeds and concludes as expected. Events or conditions of this type may be subject to the reporting requirements of 10 CFR 50.72 (ref. 4.1.4).

3.1.5 Classification Based on Analysis

The assessment of some EALs is based on the results of analyses that are necessary to ascertain whether a specific EAL threshold has been exceeded (e.g., dose assessments, chemistry sampling, etc.). For these EALs, the wording of the EAL or associated basis discussion will identify the necessary analysis. In these cases, the 15-minute declaration period starts with the availability of the analysis results that show the threshold to be exceeded (i.e., this is the time that the EAL information is first available). The NRC expects licensees to establish the capability to initiate and complete EAL-related analyses within a reasonable period of time (e.g., maintain the necessary expertise on-shift).

3.1.6 DSEO/ADTS Judgment

While the EALs have been developed to address a full spectrum of possible events and conditions which may warrant emergency classification, a provision for classification based on operator/management experience and judgment is still necessary. The NEI 99-01 EAL scheme provides the DSEO/ADTS with the ability to classify events and conditions based upon judgment using EALs that are consistent with the Emergency Classification Level (ECL) definitions (refer to Category H). The DSEO/ADTS will need to determine if the effects or consequences of the event or condition reasonably meet or exceed a particular ECL definition.

3.2 Classification Methodology

To make an emergency classification, the user will compare an event or condition (i.e., the relevant plant indications and reports) to an EAL(s) and determine if the EAL has been met or exceeded. The evaluation of an EAL must be consistent with the related Notes. If an EAL has been met or exceeded, the associated IC is likewise met, the emergency classification process “clock” starts, and the ECL must be declared in accordance with plant procedures no later than 15 minutes after the process “clock” started.

When assessing an EAL that specifies a time duration for the potentially classifiable condition, the “clock” for the EAL time duration runs concurrently with the emergency classification process “clock.” For a full discussion of this timing requirement, refer to NSIR/DPR-ISG-01 (ref. 4.1.7).

3.2.1 Classification of Multiple Events and Conditions

When multiple emergency events or conditions are present, the user will identify all met or exceeded EALs. The highest applicable ECL identified during this review is declared.

There is no “additive” effect from multiple EALs meeting the same ECL.

Related guidance concerning classification of rapidly escalating events or conditions is provided in Regulatory Issue Summary (RIS) 2007-02, *Clarification of NRC Guidance for Emergency Notifications During Quickly Changing Events* (ref. 4.1.2).

3.2.2 Classification of IMMINENT Conditions

Although EALs provide specific thresholds, the DSEO/ADTS must remain alert to events or conditions that could lead to meeting or exceeding an EAL within a relatively short period of time (i.e., a change in the ECL is IMMINENT). If, in the judgment of the DSEO/ADTS, meeting an EAL is IMMINENT, the emergency classification should be made as if the EAL has been met. While applicable to all emergency classification levels, this approach is particularly

important at the higher emergency classification levels since it provides additional time for implementation of protective measures.

3.2.3 Emergency Classification Level Upgrading and Downgrading

An ECL may be downgraded when the event or condition that meets the highest IC and EAL no longer exists, and other site-specific downgrading requirements are met. If downgrading the ECL is deemed appropriate, the new ECL would then be based on a lower applicable IC(s) and EAL(s). The ECL may also simply be terminated.

As noted above, guidance concerning classification of rapidly escalating events or conditions is provided in RIS 2007-02 (ref. 4.1.2).

3.2.4 Classification of Short-Lived Events

Event-based ICs and EALs define a variety of specific occurrences that have potential or actual safety significance. By their nature, some of these events may be short-lived and, thus, over before the emergency classification assessment can be completed. If an event occurs that meets or exceeds an EAL, the associated ECL must be declared regardless of its continued presence at the time of declaration. Examples of such events include an earthquake or security event.

3.2.5 Classification of Transient Conditions

Some radiological effluent based EALs employ time-based criteria. These criteria will require that the IC/EAL conditions be present for a defined period of time before an emergency declaration is warranted. In cases where no time-based criterion is specified, it is recognized that some transient conditions may cause an EAL to be met for a brief period of time (e.g., a few seconds to a few minutes). The following guidance should be applied to the classification of these conditions.

EAL momentarily met during expected plant response - In instances in which an EAL is briefly met during an expected (normal) plant response, an emergency declaration is not warranted provided that associated systems and components are operating as expected, and operator actions are performed in accordance with procedures.

EAL momentarily met but the condition is corrected prior to an emergency declaration – If an operator takes prompt manual action to address a condition, and the action is successful in correcting the condition prior to the emergency declaration, then the applicable EAL is not considered met and the associated emergency declaration is not required.

It is important to stress that the 15-minute emergency classification assessment period (process clock) is not a “grace period” during which a classification may be delayed to allow the performance of a corrective action that would obviate the need to classify the event. Emergency classification assessments must be deliberate and timely, with no undue delays. The provision discussed above addresses only those rapidly evolving situations when an operator is able to take a successful corrective action prior to the DSEO/ADTS completing the review and steps necessary to make the emergency declaration. This provision is included to ensure that any public protective actions resulting from the emergency classification are truly warranted by the plant conditions.

3.2.6 After-the-Fact Discovery of an Emergency Event or Condition

In some cases, an EAL may be met but the emergency classification was not made at the time of the event or condition. This situation can occur when personnel discover that an event or condition existed which met an EAL, but no emergency was declared, and the event or condition no longer exists at the time of discovery. This may be due to the event or condition not being recognized at the time or an error that was made in the emergency classification process.

In these cases, no emergency declaration is warranted; however, the guidance contained in NUREG-1022 (ref. 4.1.3) is applicable. Specifically, the event should be reported to the NRC in accordance with 10 CFR 50.72 (ref. 4.1.4) within one hour of the discovery of the undeclared event or condition. The licensee should also notify appropriate State and local agencies in accordance with the agreed upon arrangements.

3.2.7 Retraction of an Emergency Declaration

Guidance on the retraction of an emergency declaration reported to the NRC is discussed in NUREG-1022 (ref. 4.1.3).

4.0 REFERENCES

4.1 Developmental

- 4.1.1 NEI 99-01, Revision 6, "Methodology for the Development of Emergency Action Levels for Non-Passive Reactors" (ADAMS Accession No. ML12326A805)
- 4.1.2 RIS 2007-02, "Clarification of NRC Guidance for Emergency Notifications During Quickly Changing Events," February 2, 2007.
- 4.1.3 NUREG-1022, "Event Reporting Guidelines: 10CFR50.72 and 50.73"
- 4.1.4 10 CFR 50.72, "Immediate Notification Requirements for Operating Nuclear Power Reactors"
- 4.1.5 10 CFR 50.73, "License Event Report System"
- 4.1.6 MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
- 4.1.7 NSIR/DPR-ISG-01, "Interim Staff Guidance, Emergency Planning for Nuclear Power Plants"
- 4.1.8 MPS Emergency Plan

4.2 Implementing

- 4.2.1 MP-26-EPI-FAP06, "Classification and PARs"
- 4.2.2 MPS1 EAL Comparison Matrix
- 4.2.3 MPS1 EAL Matrix

5.0 DEFINITIONS, ACRONYMS & ABBREVIATIONS

5.1 Definitions (ref. 4.1.1 except as noted)

Selected terms used in Initiating Condition, EAL statements and EAL bases are set in all capital letters (e.g., ALL CAPS). These are defined terms that have specific meanings as used in this document. The definitions of these terms are provided below.

ALERT

Events are in progress, or have occurred, which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. **Any** releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.

EMERGENCY ACTION LEVEL (EAL)

A pre-determined, site-specific, observable threshold for an INITIATING CONDITION that, when met or exceeded, places the plant in a given emergency classification level.

EMERGENCY CLASSIFICATION LEVEL (ECL)

One of a set of names or titles established by the US Nuclear Regulatory Commission (NRC) for grouping off-normal events or conditions according to (1) potential or actual effects or consequences, and (2) resulting onsite and offsite response actions. The emergency classification levels, in ascending order of severity, are:

- Unusual Event (UE)
- Alert
- Site Area Emergency (SAE)
- General Emergency (GE)

EXPLOSION

A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding, arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FIRE

Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FLOODING

A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

HOSTILE ACTION

An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take hostages, and/or intimidate the licensee to achieve an end. This includes attack by air,

land, or water using guns, explosives, projectiles, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

IMMINENT

The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

IMPEDE(D)

Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

INITIATING CONDITION (IC)

An event or condition that aligns with the definition of one of the four emergency classification levels by virtue of the potential or actual effects or consequences.

NORMAL LEVELS

As applied to radiological IC/EALs, the highest reading in the past twenty-four hours excluding the current peak value.

OWNER CONTROLLED AREA (OCA)

The area within the SITE BOUNDARY including the PROTECTED AREA (ref. 4.1.8).

PROTECTED AREA

The area within the Millstone Power Station security fence (ref. 4.1.8).

SAFETY SYSTEM

A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

SITE BOUNDARY

That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA (ref. 4.1.8).

UNPLANNED

A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

UNUSUAL EVENT

Events are in progress or have occurred which indicate a potential degradation in the level of safety of the plant or indicate a security threat to facility protection has been initiated. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs.

VALID

An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

5.2 Abbreviations/Acronyms

°F	Degrees Fahrenheit
°	Degrees
ADTS	Assistant Director Technical Support
ARM	Area Radiation Monitor
CDE	Committed Dose Equivalent
CFR	Code of Federal Regulations
CPM	Counts Per Minute
CTMT	Containment
DBA	Design Basis Accident
DSAR	Defueled Safety Analysis Report
DSEO	Director of Station Emergency Operations
EAL	Emergency Action Level
ECL	Emergency Classification Level
EOF	Emergency Operations Facility
EOP	Emergency Operating Procedure
EPA	Environmental Protection Agency
EPI	Emergency Plan Implementing
FEMA	Federal Emergency Management Agency
GPM	Gallons Per Minute
Hr.	Hour
IC	Initiating Condition
ISFSI	Independent Spent Fuel Storage Installation
LCO	Limiting Condition of Operation
LRW	Liquid Radwaste
LWR	Light Water Reactor
MCB	Main Control Board
Min.	Minute
MPH	Miles Per Hour
MPS	Millstone Power Station
mR, mRem, mrem, mREM	milli-Roentgen Equivalent Man
NEI	Nuclear Energy Institute
NPP	Nuclear Power Plant

NRC	Nuclear Regulatory Commission
NORAD	North American Aerospace Defense Command
UE	Unusual Event
OBE	Operating Basis Earthquake
OCA	Owner Controlled Area
ONP	Off Normal Procedure
PA	Protected Area
PAG	Protective Action Guideline
PLC	Programmable Logic Controller
PSIG	Pounds per Square Inch Gauge
R	Roentgen
RCS	Reactor Coolant System
Rem, rem, REM	Roentgen Equivalent Man
REMDCM ...	Radiological Effluent Monitoring Manual/Off-site Dose Calculation Manual
SCBA	Self-Contained Breathing Apparatus
SFP	Spent Fuel Pool (Pit)
SM	Shift Manager
SRO	Senior Reactor Operator
TEDE	Total Effective Dose Equivalent
TS	Technical Specifications
TSC	Technical Support Center

6.0 MPS1-TO-NEI 99-01, Rev. 6 EAL CROSS-REFERENCE

This cross-reference is provided to facilitate association and location of a MPS1 EAL within the NEI 99-01 IC/EAL example identification scheme. Further information regarding the development of the MPS1 EALs based on the NEI 99-01, Rev. 6 guidance can be found in the MPS1 EAL Comparison Matrix.

MPS1	NEI 99-01, Rev. 6	
EAL	IC	EAL Example
RU1.1	PD-AU1	1
RU1.2	PD-AU1	1
RU1.3	PD-AU1	2
RU2.1	PD-AU2	1
RU2.2	PD-AU2	2
RA1.1	PD-AA1	1
RA1.2	PD-AA1	2
RA1.3	PD-AA1	3
RA1.4	PD-AA1	4
RA2.1	PD-AA2	2
HU1.1	PD-HU2	1
HU2.1	PD-HU3	1
HA2.1	PD-HA3	1
MU1.1	PD-SU1	1

7.0 ATTACHMENTS

7.1 Attachment 1, Emergency Action Level Technical Bases

Category R – Abnormal Rad Levels / Rad Effluent

Direct indication of elevated radiological effluents or area radiation levels are appropriate symptoms for emergency classification.

Events of this category pertain to the following subcategories:

1. Radiological Effluent

Direct indication of effluent radiation monitoring systems provides a rapid assessment mechanism to determine releases in excess of classifiable limits. Projected offsite doses, actual offsite field measurements or measured release rates via sampling indicate doses or dose rates above classifiable limits.

2. Area Radiation Levels

Sustained general area radiation levels which may preclude access to areas needed to maintain spent fuel integrity also warrant emergency classification.

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.1 Unusual Event

Reading on RM-SFPI-02 Unit 1 Spent Fuel Pool Island Monitor > 1.1E-2 $\mu\text{Ci/cc}$ for ≥ 60 min. (Notes 1, 2, 3)

- Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.
- Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.
- Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This EAL addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous or liquid radiological release, monitored or un-monitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.

This EAL addresses radioactivity releases that cause effluent radiation monitor readings to exceed 2 times the station REMODCM limit for 60 minutes or longer. This EAL addresses a continuous gaseous monitored pathway (ref. 1, 2).

The basis for the MPS 1 UE value corresponds to the unplanned release of gaseous radioactivity greater than two times the site-specific effluent release limit. This UE gaseous release criterion is consistent with NEI 99-01, Rev. 6 guidance but different from the alternative criterion used for Dominion Energy's other operating nuclear facilities. The operating facilities use a dose-based criterion based on exceeding 1 mrem TEDE for 60 minutes or longer. For a shutdown nuclear facility like MPS 1 which has essentially only Kr-85 as a source term, dose-based criteria would be inappropriate. This is due to fact that the damage to fuel to produce sufficient source to create 1 mrem TEDE would be inconsistent with the Unusual Event definition and basis (represents a loss of plant control and degradation in the level of safety). To create a release that would result in 1 mrem TEDE, nearly 600 irradiated fuel assemblies decayed for over 22 years would need to fail. Setting the UE threshold based on a release greater than two times the site-specific release limit would correlate to the failure of approximately 1.5 irradiated fuel assemblies. This number of fuel assemblies failures would appropriately represent the Unusual Event definition and basis (ref. 2).

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMODOCM)"
2. Calculation RP-18-08, "Millstone Unit 1 Abnormal Rad Release Gaseous EAL Threshold based on NEI 99-01, Rev. 6"
3. NEI 99-01 IC PD-AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.2 Unusual Event

Reading on RE-M6-110 Liquid Waste Effluent Discharge Monitor > 2 x the “alarm” setpoint established by a current radioactivity discharge permit for ≥ 60 min. (Notes 1, 2, 3)

- Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.
- Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.
- Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator’s operability, the condition’s existence, or the report’s accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This EAL addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous or liquid radiological release, monitored or un-monitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.

This EAL addresses radioactivity releases that cause effluent radiation monitor readings to exceed 2 times the limit established by a radioactivity discharge permit. This EAL will typically be associated with planned batch releases from non-continuous release pathways (e.g., radwaste) (ref. 1, 2, 3).

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Millstone Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMODCM)"
2. Calculation RP-17-24, "Millstone Abnormal Rad Release Liquid EAL Thresholds based on NEI 99-01, Rev. 6"
3. NEI 99-01 IC PD-AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer.

EAL:

RU1.3 Unusual Event

Sample analysis for a liquid or gaseous release indicates a concentration or release rate $> 2 \times$ the allocated REMODCM limits for ≥ 60 min. (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Definition(s):

None

Basis:

This EAL addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous or liquid radiological release, monitored or un-monitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

This EAL addresses uncontrolled gaseous or liquid releases that are detected by sample analyses or environmental surveys, particularly on unmonitored pathways (e.g., spills of radioactive liquids into storm drains, leakage into ocean water systems, etc.).

Escalation of the emergency classification level would be via IC RA2.

Reference(s):

1. MP-22-REC-BAP01, "Millstone Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMODCM)"
2. NEI 99-01 IC PD-AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.1 Alert

Reading on RM-SFPI-02 Unit 1 Spent Fuel Pool Island Monitor > 4.0E+01 $\mu\text{Ci/cc}$ for ≥ 15 min. (Notes 1, 2, 3, 4)

- Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.
- Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.
- Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.
- Note 4: The pre-calculated effluent monitor values presented in EAL RA1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This EAL addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem .

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have

stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.

The basis for the MPS1 ALERT value corresponds to the unplanned release of gaseous radioactivity over 15 minutes to produce 10 mrem TEDE. The damage of spent fuel to produce enough source to create 10 mrem TEDE would require approximately 50% of the irradiated fuel assemblies in the spent fuel pool to fail.

Reference(s):

1. Calculation RP-18-08, "Millstone Unit 1 Abnormal Rad Release Gaseous EAL Threshold based on NEI 99-01, Rev. 6"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incidents"
3. NEI 99-01 IC PD-AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.2 Alert

Dose assessment using actual meteorology indicates doses > 10 mrem TEDE at or beyond the SITE BOUNDARY (Note 4)

Note 4: The pre-calculated effluent monitor values presented in EAL RA1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This EAL addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem .

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 IC PD-AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.3 Alert

Analysis of a liquid effluent sample indicates a concentration or release rate that would result in doses > 10 mrem TEDE at or beyond the SITE BOUNDARY for 60 min. of exposure (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This EAL addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

This EAL is assessed per the REMODCM (ref. 2). REMODCM software can be used to produce a dose to the maximum individual.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem .

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMODCM)"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 ICPD-AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.4 Alert

Field survey results indicate closed window dose rates > 10 mR/hr expected to continue for ≥ 60 min. at or beyond the SITE BOUNDARY (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This EAL addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem .

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-26-EPI-FAP04, "Emergency Operations Facility Activation and Operation"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 IC PD-AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 2 – Area Radiation Levels
Initiating Condition: UNPLANNED rise in plant radiation levels
EAL:

RU2.1 Unusual Event

UNPLANNED water level drop in the SFP as indicated by **any** of the following:

- SPENT FUEL POOL LEVEL LOW alarm PLC-AL101
- SPENT FUEL POOL LEVEL LOW-LOW alarm PLC-AL100
- Report of dropping level in SFP
- Loss of SFP Cooling suction flow

AND

UNPLANNED rise in area radiation levels as indicated by **any** of the following radiation monitors:

- RM-SFPI-01 Channel 1 Refuel Floor West (Low Range)
- RM-SFPI-01 Channel 2 Refuel Floor East
- RM-SFPI-01 Channel 3 Refuel Floor West (High Range)

Definition(s):

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This EAL addresses elevated plant radiation levels caused by a decrease in water level above irradiated (spent) fuel or other UNPLANNED events. The increased radiation levels are indicative of a minor loss in the ability to control radiation levels within the plant or radioactive materials. Either condition is a potential degradation in the level of safety of the plant.

A water level decrease will be primarily determined by indications from available level instrumentation. Other sources of level indications may include reports from plant personnel or video camera observations (if available). A significant drop in the water level may also cause an increase in the radiation levels of adjacent areas that can be detected by monitors in those locations.

The effects of planned evolutions should be considered. Note that this EAL is applicable only in cases where the elevated reading is due to an UNPLANNED water level drop.

Escalation of the emergency classification level would be via IC RA2.

Reference(s):

1. ARP 230.44, "PLC Alarm Response"
2. ONP 532, "Loss of Spent Fuel Pool Cooling"

3. ONP 509, "Excessive Radioactive Levels"
4. NEI 99-01 IC PD-AU2

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 2 – Area Radiation Levels
Initiating Condition: UNPLANNED rise in plant radiation levels
EAL:

RU2.2 Unusual Event

Area radiation monitor reading or survey result indicates an UNPLANNED rise of 25 mR/hr over NORMAL LEVELS

Definition(s):

NORMAL LEVELS - As applied to radiological IC/EALs, the highest reading in the past twenty-four hours excluding the current peak value.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This EAL addresses elevated plant radiation levels caused by a decrease in water level above irradiated (spent) fuel or other UNPLANNED events. The increased radiation levels are indicative of a minor loss in the ability to control radiation levels within the plant or radioactive materials. Either condition is a potential degradation in the level of safety of the plant.

A water level decrease will be primarily determined by indications from available level instrumentation. Other sources of level indications may include reports from plant personnel or video camera observations (if available). A significant drop in the water level may also cause an increase in the radiation levels of adjacent areas that can be detected by monitors in those locations.

The effects of planned evolutions should be considered. This EAL excludes radiation level increases that result from planned activities such as use of radiographic sources and movement of radioactive waste materials.

Escalation of the emergency classification level would be via IC RA2.

Reference(s):

1. ONP 509, "Excessive Radioactive Levels"
2. NEI 99-01 IC PD-AU2

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 2 – Area Radiation Levels
Initiating Condition: UNPLANNED rise in plant radiation levels that IMPEDES plant access required to maintain spent fuel integrity

EAL:

RA2.1 Alert

UNPLANNED Area Radiation Monitor readings or survey results indicate a rise by 100 mR/hr over NORMAL LEVELS that IMPEDES access to areas of the Reactor Building needed to maintain control of radioactive material or operation of systems needed to maintain spent fuel integrity

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

NORMAL LEVELS - As applied to radiological IC/EALs, the highest reading in the past twenty-four hours excluding the current peak value.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This EAL addresses increased radiation levels that impede necessary access to areas containing equipment that must be operated manually or that requires local monitoring, in order to maintain systems needed to maintain spent fuel integrity. As used here, 'IMPEDE' includes hindering or interfering, provided that the interference or delay is sufficient to significantly threaten necessary plant access. It is this impaired access that results in the actual or potential substantial degradation of the level of safety of the plant.

This EAL does not apply to anticipated temporary increases due to planned events.

Reference(s):

1. ONP 509, "Excessive Radioactive Levels"
2. NEI 99-01 IC PD-AA2

Category H – Hazards

Hazards are non-plant, system-related events that can directly or indirectly affect spent fuel integrity.

1. Hazardous Events Affecting Plant Safety

Natural events such as earthquakes have potential to cause plant structure or equipment damage of sufficient magnitude to threaten personnel or spent fuel storage safety.

2 DSEO/ADTS Judgment

The EALs defined in other categories specify the predetermined symptoms or events that are indicative of emergency or potential emergency conditions and thus warrant classification. While these EALs have been developed to address the full spectrum of possible emergency conditions which may warrant classification and subsequent implementation of the Emergency Plan, a provision for classification of emergencies based on operator/management experience and judgment is still necessary. The EALs of this category provide the DSEO/ADTS the latitude to classify emergency conditions consistent with the established classification criteria based upon DSEO/ADTS judgment.

Security events affecting MPS1 are classified per either the MPS2 or MPS3 EALs.

Category: H – Hazards
Subcategory: 1 – Hazardous Event Affecting Safety Systems
Initiating Condition: Hazardous event affecting SAFETY SYSTEM equipment necessary for spent fuel cooling

EAL:

HU1.1 Unusual Event

The occurrence of **any** Table H-1 hazardous event

AND

Event damage (excluding loss of offsite AC power) has caused indications of degraded performance of **EITHER**:

- Spent Fuel Pool Cooling System
- Decay Heat Removal System

Table H-1 Hazardous Events

- Seismic event (earthquake)
- Internal or external FLOODING event
- High winds or tornado strike
- FIRE (refer to Unit 2/3 EALs for a fire within the Protected Area)
- EXPLOSION
- Other events with similar hazard characteristics as determined by the Shift Manager

Definition(s):

EXPLOSION - A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding, arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10 CFR 50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This EAL addresses a hazardous event that causes damage to SAFETY SYSTEMS needed for spent fuel cooling. In order to provide the appropriate context for consideration of an UNUSUAL EVENT classification, the hazardous event must have caused indications of degraded SAFETY SYSTEM performance.

Indications of degraded performance addresses damage to a SAFETY SYSTEM that is in service/operation since indications for it will be readily available. The indications of degraded performance should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM.

A loss of offsite AC power is excluded as the loss of power does not result in actual damage to the specified SAFETY SYSTEM.

If the hazardous event only resulted in visible damage, with no indications of degraded performance of either SAFETY SYSTEM, then this emergency classification is not warranted.

Escalation of the emergency classification level could, depending upon the event, be based on any of the Alert ICs; AA1, AA2, or HA2.

Reference(s):

1. EP FAQ 2016-002 Emergency Preparedness Program Frequently Asked Questions
2. NEI 99-01 IC PD-HU2

Category: H – Hazards

Subcategory: 2 – DSEO/ADTS Judgment

Initiating Condition: Other conditions existing that in the judgment of the DSEO warrant declaration of an Unusual Event

EAL:

HU2.1 Unusual Event

Other conditions exist which in the judgment of the DSEO indicate that events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate a security threat to facility protection has been initiated. **No** releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of SAFETY SYSTEMS occurs.

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This EAL addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the DSEO to fall under the emergency classification level description for an Unusual Event.

Reference(s):

1. NEI 99-01 IC PD-HU3

Category: H – Hazards
Subcategory: 2 – DSEO/ADTS Judgment
Initiating Condition: Other conditions existing that in the judgment of the DSEO/ADTS warrant declaration of an Alert

EAL:

HA2.1 Alert

Other conditions exist which in the judgment of the DSEO/ADTS indicate that events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. **Any** releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels

Definition(s):

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take hostages, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, projectiles, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

Basis:

This EAL addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the DSEO/ADTS to fall under the emergency classification level description for an Alert.

Reference(s):

1. NEI 99-01 IC PD-HA3

Category M – System Malfunctions

A single EAL is provided for permanently defueled reactors.

1. Loss of Spent Fuel Cooling

Uncontrolled or inadvertent temperature rises in the Spent Fuel Pool are indicative of a potential loss of safety functions. If uncorrected, boiling in the pool will occur, and result in a loss of pool level and increased radiation levels.

Category: M – System Malfunctions
Subcategory: 1 – Loss of SFP Cooling
Initiating Condition: UNPLANNED spent fuel pool temperature rise
EAL:

MU1.1 Unusual Event

UNPLANNED spent fuel pool temperature rise to > 140 °F

Definition(s):

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This EAL addresses a condition that is a precursor to a more serious event and represents a potential degradation in the level of safety of the plant. If uncorrected, boiling in the pool will occur, and result in a loss of pool level and increased radiation levels.

Annunciator alarm PLC-AL106, "SPENT FUEL POOL BULK TEMPERATURE HIGH," alarms at 125°F (ref. 1).

The maximum allowed bulk SFP water temperature is 140°F. The requirement to maintain the spent fuel pool bulk water temperature $\leq 140^{\circ}\text{F}$ ensures that high water temperature will not degrade the resin in the spent fuel pool demineralizer, the fuel pool structure, pool liner, fuel racks, or external cooling system (ref. 3).

Spent Fuel Pool temperature is normally maintained well below this point thus allowing time to correct the cooling system malfunction prior to classification.

With a complete loss of external cooling and a closed airspace above the pool, it would take approximately 7.5 days to rise to 212°F if starting from the TRM upper temperature limit of 140°F (ref. 4).

Escalation of the emergency classification level would be via IC RA2.

Reference(s):

1. ARP 230.44, "PLC Alarm Response"
2. ONP 532, "Loss of Spent Fuel Pool Cooling"
3. Unit 1 Defueled Technical Requirements Manual
4. MPS-1 DSAR, Section 3.2.1.3, "Spent Fuel Pool Cooling System"
5. NEI 99-01 IC PD-SU1

ATTACHMENT 3B

MPS2 EAL TECHNICAL BASES DOCUMENT (Final)

**Dominion Energy Nuclear Connecticut, Inc. (DENC)
Millstone Power Station Unit 2 and ISFSI**

**Emergency Action Level Technical Bases Document
Millstone Power Station – Unit 2**

(Final)

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1.0 INTRODUCTION

This document provides an explanation and rationale for each Emergency Action Level (EAL) included in the NEI 99-01, Revision 6, EAL Upgrade Project for Millstone Power Station Unit 2 (MPS2). It should be used to facilitate review of the MPS2 EALs and provide historical documentation for future reference. Decision-makers responsible for implementation of MP-26-EPI-FAP06, Classification and PARs, may use this document as a technical reference in support of EAL interpretation. This information may assist the Director of Station Emergency Operations/Assistant Director Technical Support (DSEO/ADTS) in making classifications, particularly those involving judgment or multiple events. The basis information may also be useful in training and for explaining event classifications to off-site officials.

The expectation is that emergency classifications are to be made as soon as conditions are present and recognizable for the classification, but within 15 minutes or less in all cases of conditions present. Use of this document for assistance is not intended to delay the emergency classification.

Since the information in a basis document can affect emergency classification decision-making (e.g., the DSEO/ADTS refers to it during an event), the NRC staff expects that changes to the basis document will be evaluated in accordance with the provisions of 10 CFR 50.54(q). For Dominion Energy sites, a 10 CFR 50.54(q)(3) screening/evaluation will be performed to evaluate changes to this document.

Dominion Energy fleet procedure CM-AA-400, "10 CFR 50.59 and 10 CFR 72.48 – Changes, Tests and Experiments" provides a method to determine the impacts to licensing basis documents when changes are proposed to procedures, including changes to Abnormal Operating Procedures (AOPs) and Emergency Operating Procedures (EOPs). The 50.59/72.48 applicability review specifically requires that the effect of a proposed procedure change on the Emergency Plan (and associated EALs) be reviewed/assessed. When impacts to the Emergency Plan are identified, a separate review in accordance to 10 CFR 50.54(q) will be performed to determine the acceptability of the proposed procedure change.

2.0 DISCUSSION

2.1 Background

EALs are the plant-specific indications, conditions or instrument readings that are utilized to classify emergency conditions defined in the Millstone Power Station (MPS) Emergency Plan.

In 1992, the NRC endorsed NUMARC/NESP-007, "Methodology for Development of Emergency Action Levels" as an alternative guidance to the original Standard Review Plan and NUREG-0654 EAL schemes.

NEI 99-01 (NUMARC/NESP-007), Revisions 4 and 5 were subsequently issued for industry implementation. Enhancements over earlier revisions included:

- Consolidating the system malfunction initiating conditions and example emergency action levels which address conditions that may be postulated to occur during plant shutdown conditions.

- Initiating conditions and example emergency action levels that fully address conditions that may be postulated to occur at permanently Defueled Stations and Independent Spent Fuel Storage Installations (ISFSIs).
- Simplifying the fission product barrier EAL threshold for a Site Area Emergency.

Subsequently, Revision 6 of NEI 99-01 has been issued which incorporates resolutions to numerous implementation issues including the NRC EAL Frequently Asked Questions (FAQs). Using NEI 99-01, Revision 6, "Methodology for the Development of Emergency Action Levels for Non-Passive Reactors," November 2012 (ref. 4.1.1), MPS2 conducted an EAL implementation upgrade project that produced the EALs discussed herein.

2.2 Fission Product Barriers

Fission product barrier thresholds represent threats to the defense in depth design concept that precludes the release of radioactive fission products to the environment. This concept relies on multiple physical barriers, any one of which, if maintained intact, precludes the release of significant amounts of radioactive fission products to the environment.

Many of the EALs derived from the NEI methodology are fission product barrier threshold based. That is, the conditions that define the EALs are based upon thresholds that represent the loss or potential loss of one or more of the three fission product barriers. "Loss" and "Potential Loss" signify the relative damage and threat of damage to the barrier. A "Loss" threshold means the barrier no longer assures containment of radioactive materials. A "Potential Loss" threshold implies a greater probability of barrier loss and reduced certainty of maintaining the barrier.

The primary fission product barriers are:

- A. Fuel Clad Barrier (FC): The Fuel Clad Barrier consists of the cladding material that contains the fuel pellets.
- B. Reactor Coolant System Barrier (RCS): The RCS Barrier includes the RCS primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary isolation valves.
- C. Containment Barrier (CTMT): The Containment Barrier includes the containment building and connections up to and including the outermost containment isolation valves. This barrier also includes the main steam, feedwater, and blowdown line extensions outside the containment building up to and including the outermost secondary side isolation valve. Containment Barrier thresholds are used as criteria for escalation of the Emergency Classification Level (ECL) from an Alert to a Site Area Emergency or a General Emergency.

2.3 Fission Product Barrier Classification Criteria

The following criteria are the bases for event classification related to fission product barrier loss or potential loss:

Alert:

Any loss or any potential loss of either Fuel Clad or RCS Barrier

Site Area Emergency:

Loss or potential loss of any two barriers

General Emergency:

Loss of any two barriers and loss or potential loss of the third barrier

2.4 EAL Organization

The MPS2 EAL scheme includes the following features:

- Division of the EAL set into three broad groups:
 - EALs applicable under any plant operational modes – This group would be reviewed by the EAL-user any time emergency classification is considered.
 - EALs applicable only under hot operational modes – This group would only be reviewed by the EAL-user when the plant is in Hot Shutdown, Hot Standby, Startup, or Power Operation mode.
 - EALs applicable only under cold operating modes – This group would only be reviewed by the EAL-user when the plant is in Cold Shutdown, Refueling or Defueled mode.

The purpose of the groups is to avoid review of hot condition EALs when the plant is in a cold condition and avoid review of cold condition EALs when the plant is in a hot condition. This approach significantly minimizes the total number of EALs that must be reviewed by the EAL-user for a given plant condition, reduces EAL-user reading burden and, thereby, speeds identification of the EAL that applies to the emergency.

- Within each group, assignment of EALs to categories and subcategories:

Category and subcategory titles are selected to represent conditions that are operationally significant to the EAL-user. The MPS2 EAL categories are aligned to and represent the NEI 99-01 "Recognition Categories." Subcategories are used in the MPS2 scheme as necessary to further divide the EALs of a category into logical sets of possible emergency classification thresholds. The MPS2 EAL categories and subcategories are listed below.

The EALs are pre-determined, site-specific, observable thresholds for determining whether an Initiating Condition (IC) has occurred and that an EAL threshold was met or exceeded. Thus failure to evaluate the IC and EAL together could result in an incorrect declaration.

The primary tool for determining the emergency classification level is the EAL Classification Matrix. The user of the EAL Classification Matrix may (but is not required to) consult the EAL technical bases in order to obtain additional information concerning the EALs under classification consideration. The user should consult Section 3.0 and Attachment 1 of this document for such information.

EAL Groups, Categories and Subcategories

EAL Group/Category	EAL Subcategory
<u>Any Operating Mode:</u>	
R – Abnormal Rad Levels / Rad Effluent	1 – Radiological Effluent 2 – Irradiated Fuel Event 3 – Area Radiation Levels
H – Hazards and Other Conditions Affecting Plant Safety	1 – Security 2 – Seismic Event 3 – Natural or Technological Hazard 4 – Fire 5 – Hazardous Gas 6 – Control Room Evacuation 7 – DSEO/ADTS Judgment
E – Independent Spent Fuel Storage Installation (ISFSI)	1 – Confinement Boundary
<u>Hot Conditions:</u>	
M – System Malfunction	1 – Loss of Emergency AC Power 2 – Loss of Vital DC Power 3 – Loss of Control Room Indications 4 – RCS Activity 5 – RCS Leakage 6 – RPS Failure 7 – Loss of Communications 8 – Containment Failure 9 – Hazardous Event Affecting Safety Systems
F – Fission Product Barrier Degradation	None
<u>Cold Conditions:</u>	
C – Cold Shutdown / Refueling System Malfunction	1 – RCS Level 2 – Loss of Emergency AC Power 3 – RCS Temperature 4 – Loss of Vital DC Power 5 – Loss of Communications 6 – Hazardous Event Affecting Safety Systems

2.5 Technical Bases Information

EAL technical bases are provided in Attachment 1 for each EAL according to EAL group (Any, Hot, Cold), EAL category (R, C, E, F, H and M) and EAL subcategory. A summary explanation of each category and subcategory is given at the beginning of the technical bases discussions of the EALs included in the category. For each EAL, the following information is provided:

Category Letter & Title

Subcategory Number & Title

Initiating Condition (IC)

Site-specific description of the generic IC given in NEI 99-01 Rev. 6.

EAL Identifier (enclosed in rectangle)

Each EAL is assigned a unique identifier to support accurate communication of the emergency classification to onsite and offsite personnel. Four characters define each EAL identifier:

1. First character (letter): Corresponds to the EAL category as described above (R, C, E, F, H or M)
2. Second character (letter): The emergency classification (G, S, A or U)
 - G = General Emergency
 - S = Site Area Emergency
 - A = Alert
 - U = Unusual Event
3. Third character (number): Subcategory number within the given category. Subcategories are sequentially numbered beginning with the number one (1). If a category does not have a subcategory, this character is assigned the number one (1).
4. Fourth character (number): The numerical sequence of the EAL within the EAL subcategory. If the subcategory has only one EAL, it is given the number one (1).

Classification (enclosed in rectangle):

General Emergency (G), Site Area Emergency (S), Alert (A) or Unusual Event (U).

EAL (enclosed in rectangle)

Exact wording of the EAL as it appears in the EAL Classification Matrix.

Notes (as applicable)

Mode Applicability

One or more of the following plant operating conditions comprise the mode to which each EAL is applicable: 1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown, 5 - Cold Shutdown, 6 - Refueling, DEF - Defueled, or All. (See Section 2.6 for operating mode definitions)

Definition(s):

If the EAL wording contains a defined term, the definition of the term is included in this section. These definitions can also be found in Section 5.1.

Basis:

An EAL basis section that provides MPS2-relevant information concerning the EAL as well as a description of the rationale for the EAL as provided in NEI 99-01, Rev. 6.

Reference(s):

Source documentation from which the EAL is derived

2.6 Operational Mode Applicability

MODE	K_{eff}	THERMAL POWER *	T_{AVG}
1. Power Operation	≥ 0.99	$> 5\%$	$\geq 300^{\circ}\text{F}$
2. Startup	≥ 0.99	$\leq 5\%$	$\geq 300^{\circ}\text{F}$
3. Hot Standby	< 0.99	0	$\geq 300^{\circ}\text{F}$
4. Hot Shutdown	< 0.99	0	$> 200^{\circ}\text{F} \text{ \& } < 300^{\circ}\text{F}$
5. Cold Shutdown	< 0.98	0	$\leq 200^{\circ}\text{F}$
6. Refueling **	≤ 0.95	0	$\leq 140^{\circ}\text{F}$
Defueled	NA	NA	NA – no fuel in reactor vessel

* Excluding Decay Heat

** Fuel in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed

The plant operating mode that exists at the time that the event occurs (prior to any protective system or operator action being initiated in response to the condition) should be compared to the mode applicability of the EALs. If a lower or higher plant operating mode is reached before the emergency classification is made, the declaration shall be based on the mode that existed at the time the event occurred.

3.0 GUIDANCE ON MAKING EMERGENCY CLASSIFICATIONS

3.1 General Considerations

When making an emergency classification, the DSEO/ADTS must consider all information having a bearing on the proper assessment of an Initiating Condition (IC). This includes the EAL plus any associated Operational Mode Applicability, Notes, and the informing basis information. In the Category F matrices, EALs are based on loss or potential loss of Fission Product Barrier thresholds.

3.1.1 Classification Timeliness

NRC regulations require the licensee to establish and maintain the capability to assess, classify, and declare an emergency condition within 15 minutes after the availability of indications to plant operators that an emergency action level has been exceeded and to promptly declare the emergency condition as soon as possible following identification of the appropriate emergency classification level. The NRC staff has provided guidance on implementing this requirement in NSIR/DPR-ISG-01, "Interim Staff Guidance, Emergency Planning for Nuclear Power Plants" (ref. 4.1.8).

3.1.2 Valid Indications

All emergency classification assessments shall be based upon valid indications, reports or conditions. A valid indication, report, or condition, is one that has been verified through appropriate means such that there is no doubt regarding the indicator's operability, the condition's existence, or the report's accuracy.

An indication, report, or condition is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

3.1.3 Imminent Conditions

For ICs and EALs that have a stipulated time duration (e.g., 15 minutes, 30 minutes, etc.), the DSEO/ADTS should not wait until the applicable time has elapsed, but should declare the event as soon as it is determined that the condition has exceeded, or will likely exceed, the applicable time. If an ongoing radiological release is detected and the release start time is unknown, it should be assumed that the release duration specified in the IC/EAL has been exceeded, absent data to the contrary.

3.1.4 Planned vs. Unplanned Events

A planned work activity that results in an expected event or condition which meets or exceeds an EAL does not warrant an emergency declaration provided that: 1) the activity proceeds as planned, and 2) the plant remains within the limits imposed by the operating license. Such activities include planned work to test, manipulate, repair, maintain or modify a system or component. In these cases, the controls associated with the planning, preparation and execution of the work will ensure that compliance is maintained with all aspects of the operating license provided that the activity proceeds and concludes as expected. Events or

conditions of this type may be subject to the reporting requirements of 10CFR 50.72 (ref. 4.1.4).

3.1.5 Classification Based on Analysis

The assessment of some EALs is based on the results of analyses that are necessary to ascertain whether a specific EAL threshold has been exceeded (e.g., dose assessments, chemistry sampling, RCS leak rate calculation, etc.). For these EALs, the wording of the EAL or associated basis discussion will identify the necessary analysis. In these cases, the 15-minute declaration period starts with the availability of the analysis results that show the threshold to be exceeded (i.e., this is the time that the EAL information is first available). The NRC expects licensees to establish the capability to initiate and complete EAL-related analyses within a reasonable period of time (e.g., maintain the necessary expertise on-shift).

3.1.6 DSEO/ADTS Judgment

While the EALs have been developed to address a full spectrum of possible events and conditions which may warrant emergency classification, a provision for classification based on operator/management experience and judgment is still necessary. The NEI 99-01 EAL scheme provides the DSEO/ADTS with the ability to classify events and conditions based upon judgment using EALs that are consistent with the Emergency Classification Level (ECL) definitions (refer to Category H). The DSEO/ADTS will need to determine if the effects or consequences of the event or condition reasonably meet or exceed a particular ECL definition. A similar provision is incorporated in the Fission Product Barrier Tables; judgment may be used to determine the status of a fission product barrier.

3.2 Classification Methodology

To make an emergency classification, the user will compare an event or condition (i.e., the relevant plant indications and reports) to an EAL(s) and determine if the EAL has been met or exceeded. The evaluation of an EAL must be consistent with the related Operating Mode Applicability and Notes. If an EAL has been met or exceeded, the associated IC is likewise met, the emergency classification process “clock” starts, and the ECL must be declared in accordance with plant procedures no later than 15 minutes after the process “clock” started.

When assessing an EAL that specifies a time duration for the potentially classifiable condition, the “clock” for the EAL time duration runs concurrently with the emergency classification process “clock.” For a full discussion of this timing requirement, refer to NSIR/DPR-ISG-01 (ref. 4.1.8).

3.2.1 Classification of Multiple Events and Conditions

When multiple emergency events or conditions are present, the user will identify all met or exceeded EALs. The highest applicable ECL identified during this review is declared. For example:

- If an Alert EAL and a Site Area Emergency EAL are met, whether at one unit or at two units, a Site Area Emergency should be declared.

There is no “additive” effect from multiple EALs meeting the same ECL. For example:

- If two Alert EALs are met, whether at one unit or at two units, an Alert should be declared.

Related guidance concerning classification of rapidly escalating events or conditions is provided in Regulatory Issue Summary (RIS) 2007-02, *Clarification of NRC Guidance for Emergency Notifications During Quickly Changing Events* (ref. 4.1.2).

3.2.2 Consideration of Mode Changes During Classification

The mode in effect at the time that an event or condition occurred, and prior to any plant or operator response, is the mode that determines whether or not an IC is applicable. If an event or condition occurs, and results in a mode change before the emergency is declared, the emergency classification level is still based on the mode that existed at the time that the event or condition was initiated (and not when it was declared). Once a different mode is reached, any new event or condition, not related to the original event or condition, requiring emergency classification should be evaluated against the ICs and EALs applicable to the operating mode at the time of the new event or condition.

For events that occur in Cold Shutdown or Refueling, escalation is via EALs that are applicable in the Cold Shutdown or Refueling modes, even if Hot Shutdown (or a higher mode) is entered during the subsequent plant response. In particular, the fission product barrier EALs are applicable only to events that initiate in the Hot Shutdown mode or higher.

3.2.3 Classification of Imminent Conditions

Although EALs provide specific thresholds, the DSEO/ADTS must remain alert to events or conditions that could lead to meeting or exceeding an EAL within a relatively short period of time (i.e., a change in the ECL is IMMINENT). If, in the judgment of the DSEO/ADTS, meeting an EAL is IMMINENT, the emergency classification should be made as if the EAL has been met. While applicable to all emergency classification levels, this approach is particularly important at the higher emergency classification levels since it provides additional time for implementation of protective measures.

3.2.4 Emergency Classification Level Upgrading and Downgrading

An ECL may be downgraded when the event or condition that meets the highest IC and EAL no longer exists, and other site-specific downgrading requirements are met. If downgrading the ECL is deemed appropriate, the new ECL would then be based on a lower applicable IC(s) and EAL(s). The ECL may also simply be terminated.

As noted above, guidance concerning classification of rapidly escalating events or conditions is provided in RIS 2007-02 (ref. 4.1.2).

3.2.5 Classification of Short-Lived Events

Event-based ICs and EALs define a variety of specific occurrences that have potential or actual safety significance. By their nature, some of these events may be short-lived and, thus, over before the emergency classification assessment can be completed. If an event occurs that meets or exceeds an EAL, the associated ECL must be declared regardless of its continued presence at the time of declaration. Examples of such events include an earthquake or a failure of the reactor protection system to automatically scram the reactor followed by a successful manual scram.

3.2.6 Classification of Transient Conditions

Many of the ICs and/or EALs employ time-based criteria. These criteria will require that the IC/EAL conditions be present for a defined period of time before an emergency declaration is

warranted. In cases where no time-based criterion is specified, it is recognized that some transient conditions may cause an EAL to be met for a brief period of time (e.g., a few seconds to a few minutes). The following guidance should be applied to the classification of these conditions.

EAL momentarily met during expected plant response - In instances in which an EAL is briefly met during an expected (normal) plant response, an emergency declaration is not warranted provided that associated systems and components are operating as expected, and operator actions are performed in accordance with procedures.

EAL momentarily met but the condition is corrected prior to an emergency declaration – If an operator takes prompt manual action to address a condition, and the action is successful in correcting the condition prior to the emergency declaration, then the applicable EAL is not considered met and the associated emergency declaration is not required. For illustrative purposes, consider the following example:

An ATWS occurs and the high pressure ECCS systems fail to automatically start. The plant enters an inadequate core cooling condition (a potential loss of both the Fuel Clad and RCS Barriers). If an operator manually starts a high pressure ECCS system in accordance with an EOP step and clears the inadequate core cooling condition prior to an emergency declaration, then the classification should be based on the ATWS only.

It is important to stress that the 15-minute emergency classification assessment period (process clock) is not a “grace period” during which a classification may be delayed to allow the performance of a corrective action that would obviate the need to classify the event. Emergency classification assessments must be deliberate and timely, with no undue delays. The provision discussed above addresses only those rapidly evolving situations when an operator is able to take a successful corrective action prior to the DSEO/ADTS completing the review and steps necessary to make the emergency declaration. This provision is included to ensure that any public protective actions resulting from the emergency classification are truly warranted by the plant conditions.

3.2.7 After-the-Fact Discovery of an Emergency Event or Condition

In some cases, an EAL may be met but the emergency classification was not made at the time of the event or condition. This situation can occur when personnel discover that an event or condition existed which met an EAL, but no emergency was declared, and the event or condition no longer exists at the time of discovery. This may be due to the event or condition not being recognized at the time or an error that was made in the emergency classification process.

In these cases, no emergency declaration is warranted; however, the guidance contained in NUREG-1022 (ref. 4.1.3) is applicable. Specifically, the event should be reported to the NRC in accordance with 10CFR 50.72 (ref. 4.1.4) within one hour of the discovery of the undeclared event or condition. The licensee should also notify appropriate State and local agencies in accordance with the agreed upon arrangements.

3.2.8 Retraction of an Emergency Declaration

Guidance on the retraction of an emergency declaration reported to the NRC is discussed in NUREG-1022 (ref. 4.1.3).

4.0 REFERENCES

4.1 Developmental

- 4.1.1 NEI 99-01, Revision 6, "Methodology for the Development of Emergency Action Levels for Non-Passive Reactors" (ADAMS Accession No. ML12326A805)
- 4.1.2 RIS 2007-02, "Clarification of NRC Guidance for Emergency Notifications During Quickly Changing Events," February 2, 2007.
- 4.1.3 NUREG-1022, "Event Reporting Guidelines: 10CFR50.72 and 50.73"
- 4.1.4 10 CFR 50.72, "Immediate Notification Requirements for Operating Nuclear Power Reactors"
- 4.1.5 10 CFR 50.73, "License Event Report System"
- 4.1.6 Technical Specifications Table 1.1-1, "Operational Modes"
- 4.1.7 MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
- 4.1.8 NSIR/DPR-ISG-01, "Interim Staff Guidance, Emergency Planning for Nuclear Power Plants"
- 4.1.9 MPS Emergency Plan
- 4.1.10 NUH-003, "UFSAR for the Horizontal Modular Storage System for Irradiated Fuel"
- 4.1.11 MPS2 ETE-NAF-2010-0004, "MPS Unit 2 ISFSI 10 CFR72.212 Evaluation Report"
- 4.1.12 UFSAR Chapter 9.8, "Fuel and Reactor Component Handling Equipment"
- 4.1.13 AOP 2577, "Fuel Handling Accident"

4.2 Implementing

- 4.2.1 MP-26-EPI-FAP06, "Classification and PARs"
- 4.2.2 NEI 99-01 Rev. 6 to MPS2 EAL Comparison Matrix
- 4.2.3 MPS2 EAL Matrix

5.0 DEFINITIONS, ACRONYMS & ABBREVIATIONS

5.1 Definitions (ref. 4.1.1 except as noted)

Selected terms used in Initiating Condition, EAL statements and EAL bases are set in all capital letters (e.g., ALL CAPS). These are defined terms that have specific meanings as used in this document. The definitions of these terms are provided below.

ALERT

Events are in progress, or have occurred, which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. **Any** releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.

CONFINEMENT BOUNDARY

The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielding Canister (DSC) (ref. 4.1.10).

CONTAINMENT CLOSURE

The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist (ref. 4.1.13):

- The equipment door is closed and held in place by a minimum of four bolts or the outage equipment door is installed.
- A minimum of one door in each airlock is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere is closed by an isolation valve, blind flange, manual valve, or special device.

EMERGENCY ACTION LEVEL (EAL)

A pre-determined, site-specific, observable threshold for an INITIATING CONDITION that, when met or exceeded, places the plant in a given emergency classification level.

EMERGENCY CLASSIFICATION LEVEL (ECL)

One of a set of names or titles established by the US Nuclear Regulatory Commission (NRC) for grouping off-normal events or conditions according to (1) potential or actual effects or consequences, and (2) resulting onsite and offsite response actions. The emergency classification levels, in ascending order of severity, are:

- Unusual Event (UE)
- Alert
- Site Area Emergency (SAE)
- General Emergency (GE)

EXPLOSION

A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding, arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FAULTED

The term applied to a steam generator that has a steam leak on the secondary side of sufficient size to cause an uncontrolled drop in steam generator pressure or the steam generator to become completely depressurized.

FIRE

Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FISSION PRODUCT BARRIER THRESHOLD

A pre-determined, site-specific, observable threshold indicating the loss or potential loss of a fission product barrier.

FLOODING

A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

GENERAL EMERGENCY

Events are in progress or have occurred which involve actual or **IMMINENT** substantial core degradation or melting with potential for loss of containment integrity or **HOSTILE ACTION** that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA PAG exposure levels offsite for more than the immediate site area.

HOSTAGE

A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION

An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take **HOSTAGES**, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, **PROJECTILES**, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the **OWNER CONTROLLED AREA**).

HOSTILE FORCE

One or more individuals who are engaged in a determined assault, overtly or by stealth and deception, equipped with suitable weapons capable of killing, maiming, or causing destruction.

IMMINENT

The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

IMPEDE(D)

Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

INDEPENDENT SPENT FUEL STORAGE INSTALLATION (ISFSI)

A complex that is designed and constructed for the interim storage of spent nuclear fuel and other radioactive materials associated with spent fuel storage.

INITIATING CONDITION (IC)

An event or condition that aligns with the definition of one of the four emergency classification levels by virtue of the potential or actual effects or consequences.

OWNER CONTROLLED AREA (OCA)

The area within the SITE BOUNDARY including the PROTECTED AREA (ref. 4.1.9).

PROJECTILE

An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA

The area within the Millstone Power Station security fence (ref. 4.1.9).

REFUELING PATHWAY

Refueling pool (RFP), fuel transfer canal, and spent fuel pool (SFP), but **not** including the reactor vessel, comprise the refueling pathway (ref. 4.1.12).

RUPTURED

The condition of a steam generator in which primary-to-secondary leakage is of sufficient magnitude to require a safety injection.

SAFETY SYSTEM

A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

SECURITY CONDITION

Any security event as listed in the approved security contingency plan that constitutes a threat/compromise to site security, threat/risk to site personnel, or a potential degradation to the level of safety of the plant. A Security Condition does **not** involve a HOSTILE ACTION.

SITE AREA EMERGENCY

Events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage or malicious acts; (1) toward site personnel or equipment that could lead to the likely failure of or; (2) that prevent effective access to equipment needed for the protection of the public. **Any** releases are **not** expected to result in exposure levels which exceed EPA PAG exposure levels beyond the SITE BOUNDARY.

SITE BOUNDARY

That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA (ref. 4.1.7)

UNISOLABLE

An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED

A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

UNUSUAL EVENT

Events are in progress or have occurred which indicate a potential degradation in the level of safety of the plant or indicate a security threat to facility protection has been initiated. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs.

VALID

An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

VISIBLE DAMAGE

Damage to a SAFETY SYSTEM train that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected SAFETY SYSTEM train.

5.2 Abbreviations/Acronyms

°F	Degrees Fahrenheit
°	Degrees
μCi.....	Micro Curie
AC	Alternating Current
ADTS	Assistant Director Technical Support
AFW	Auxiliary Feedwater
AOP	Abnormal Operating Procedure
ARM	Area Radiation Monitor
ATWS.....	Anticipated Transient Without Scram
CDE	Committed Dose Equivalent
CET	Core Exit Thermocouple
CFR.....	Code of Federal Regulations
CIAS.....	Containment Isolation Actuation Signal
CPM	Counts Per Minute
CTMT	Containment
DEF	Defueled
DBA.....	Design Basis Accident
DC.....	Direct Current
D/G.....	Diesel Generator
DSEO	Director of Station Emergency Operations
EAL	Emergency Action Level
ECCS	Emergency Core Cooling System
ECL	Emergency Classification Level
EOF.....	Emergency Operations Facility
EOP	Emergency Operating Procedure
EPA.....	Environmental Protection Agency
EPI	Emergency Plan Implementing
FAA	Federal Aviation Administration
FBI	Federal Bureau of Investigation
FC	Fuel Clad Barrier
FEMA	Federal Emergency Management Agency
GE.....	General Emergency
GPM.....	Gallons Per Minute
HR.....	Heat Removal
Hr.	Hour
HSM	Horizontal Storage Module
IC	Initiating Condition

ICC	Inadequate Core Cooling
ISFSI	Independent Spent Fuel Storage Installation
K_{eff}	Effective Neutron Multiplication Factor
LCO	Limiting Condition of Operation
LER	Licensee Event Report
LOCA	Loss of Coolant Accident
LPSI	Low Pressure Safety Injection
LRW	Liquid Radwaste
LWR	Light Water Reactor
MCB	Main Control Board
Min.	Minute
MPH	Miles Per Hour
MPS	Millstone Power Station
mR, mRem, mrem, mREM	milli-Roentgen Equivalent Man
MW	Megawatt
NEI	Nuclear Energy Institute
NEIC	National Earthquake Information Center
NPP	Nuclear Power Plant
NRC	Nuclear Regulatory Commission
NSSS	Nuclear Steam Supply System
NORAD	North American Aerospace Defense Command
OBE	Operating Basis Earthquake
OCA	Owner Controlled Area
PAG	Protective Action Guideline
PORV	Power Operated Relief Valve
PSIG	Pounds per Square Inch Gauge
PTS	Pressurized Thermal Shock
R	Roentgen
RBCCW	Reactor Building Closed Coolant Water
RCS	Reactor Coolant System
Rem, rem, REM	Roentgen Equivalent Man
REMODCM	Radiological Effluent Monitoring Manual Off-site Dose Calculation Manual
RFP	Refueling Pool
RPS	Reactor Protection System
RVLIS	Reactor Vessel Level Instrumentation System
RVLMS	Reactor Vessel Level Monitoring System
SBO	Station Blackout
SCBA	Self-Contained Breathing Apparatus
SCIP	Security Contingency Plan Implementing Procedures

SFP	Spent Fuel Pool (Pit)
SFSC	Safety Function Status Check
SG	Steam Generator
SI	Safety Injection
SIAS	Safety Injection Actuation System
SM	Shift Manager
SPDS	Safety Parameter Display System
SRO	Senior Reactor Operator
SRV	Safety Relief Valve
TC (T/C)	Thermocouple
TEDE	Total Effective Dose Equivalent
TAF	Top of Active Fuel
TCBS	Trip Circuit Breakers
TS	Technical Specifications
TSC	Technical Support Center
UE	Unusual Event
UFSAR	Updated Final Safety Analysis Report
USGS	United States Geological Survey

6.0 MPS2-TO-NEI 99-01 Rev. 6 EAL CROSS-REFERENCE

This cross-reference is provided to facilitate association and location of a MPS2 EAL within the NEI 99-01 IC/EAL identification scheme. Further information regarding the development of the MPS2 EALs based on the NEI guidance can be found in the EAL Comparison Matrix.

MPS2	NEI 99-01, Rev. 6	
	IC	Example EAL
RU1.1	AU1	1
RU1.2	AU1	2
RU1.3	AU1	3
RU1.4	AU1	1
RU1.5	AU1	3
RU2.1	AU2	1
RA1.1	AA1	1
RA1.2	AA1	2
RA1.3	AA1	3
RA1.4	AA1	4
RA2.1	AA2	1
RA2.2	AA2	2
RA2.3	AA2	3
RA3.1	AA3	1
RA3.2	AA3	2
RS1.1	AS1	1
RS1.2	AS1	2
RS1.3	AS1	3
RS2.1	AS2	1
RG1.1	AG1	1
RG1.2	AG1	2

MPS2	NEI 99-01, Rev. 6	
EAL	IC	Example EAL
RG1.3	AG1	3
RG2.1	AG2	1
CU1.1	CU1	1
CU1.2	CU1	2
CU2.1	CU2	1
CU3.1	CU3	1
CU3.2	CU3	2
CU4.1	CU4	1
CU5.1	CU5	1, 2, 3
CA1.1	CA1	1
CA1.2	CA1	2
CA2.1	CA2	1
CA3.1	CA3	1, 2
CA6.1	CA6	1
CS1.1	CS1	1
CS1.2	CS1	2
CS1.3	CS1	3
CG1.1	CG1	1
CG1.2	CG1	2
EU1.1	EU1	1
FA1.1	FA1	1
FS1.1	FS1	1
FG1.1	FG1	1
HU1.1	HU1	1, 2, 3

**Millstone Power Station Unit 2
Emergency Action Level Technical Bases Document**

Serial No. 18-364
Docket Nos. 50-366; 72-47
Enclosure 3; Attachment 3B

MPS2	NEI 99-01, Rev. 6	
EAL	IC	Example EAL
HU2.1	HU2	1
HU3.1	HU3	1
HU3.2	HU3	2
HU3.3	HU3	3
HU3.4	HU3	4
HU4.1	HU4	1
HU4.2	HU4	2
HU4.3	HU4	3
HU4.4	HU4	4
HU7.1	HU7	1
HA1.1	HA1	1, 2
HA5.1	HA5	1
HA6.1	HA6	1
HA7.1	HA7	1
HS1.1	HS1	1
HS6.1	HS6	1
HS7.1	HS7	1
HG7.1	HG7	1
MU1.1	SU1	1
MU3.1	SU2	1
MU4.1	SU3	1
MU4.2	SU3	2
MU5.1	SU4	1, 2, 3
MU6.1	SU5	1

**Millstone Power Station Unit 2
Emergency Action Level Technical Bases Document**

Serial No. 18-364
Docket Nos. 50-366; 72-47
Enclosure 3; Attachment 3B

MPS2	NEI 99-01, Rev. 6	
EAL	IC	Example EAL
MU6.2	SU5	2
MU7.1	SU6	1, 2, 3
MU8.1	SU7	1, 2
MA1.1	SA1	1
MA3.1	SA2	1
MA6.1	SA5	1
MA9.1	SA9	1
MS1.1	SS1	1
MS2.1	SS8	1
MS6.1	SS5	1
MG1.1	SG1	1
MG2.1	SG8	1

7.0 ATTACHMENTS

7.1 Attachment 1, Emergency Action Level Technical Bases

7.2 Attachment 2, Safe Operation & Shutdown Areas Tables R-2 & H-2 Bases

Category R – Abnormal Rad Release / Rad Effluent

EAL Group: ANY (EALs in this category are applicable to any plant condition, hot or cold.)

Many EALs are based on actual or potential degradation of fission product barriers because of the elevated potential for offsite radioactivity release. Degradation of fission product barriers though is not always apparent via non-radiological symptoms. Therefore, direct indication of elevated radiological effluents or area radiation levels are appropriate symptoms for emergency classification.

At lower levels, abnormal radioactivity releases may be indicative of a failure of containment systems or precursors to more significant releases. At higher release rates, offsite radiological conditions may result which require offsite protective actions. Elevated area radiation levels in plant may also be indicative of the failure of containment systems or preclude access to plant vital equipment necessary to ensure plant safety.

Events of this category pertain to the following subcategories:

1. Radiological Effluent

Direct indication of effluent radiation monitoring systems provides a rapid assessment mechanism to determine releases in excess of classifiable limits. Projected offsite doses, actual offsite field measurements or measured release rates via sampling indicate doses or dose rates above classifiable limits.

2. Irradiated Fuel Events

Conditions indicative of a loss of adequate shielding or damage to irradiated fuel may preclude access to vital plant areas or result in radiological releases that warrant emergency classification.

3. Area Radiation Levels

Sustained general area radiation levels which may preclude access to areas required to safely operate and shutdown the plant also warrant emergency classification.

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1a – Radiological Effluent (Liquid)
Initiating Condition: Release of liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.1 Unusual Event

Reading on RM4262 SG Blowdown radiation monitor > 2x the “alarm” setpoint for ≥60 min. (Notes 1, 2, 3)

- Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.
- Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.
- Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator’s operability, the condition’s existence, or the report’s accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any liquid radiological release, monitored or un-monitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

This EAL addresses normally occurring continuous radioactivity releases from monitored liquid effluent pathways (ref. 1).

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1a – Radiological Effluent (Liquid)
Initiating Condition: Release of liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.2 Unusual Event

Reading on **any** of the following effluent radiation monitors > 2 x the “alarm” setpoint established by a current radioactivity discharge permit for ≥ 60 min.

- RM9049 Clean Liquid Radwaste Effluent
- RM9116 Aerated Liquid Radwaste Effluent
- CND245 CPF Waste Neut Sump Effluent

(Notes 1, 2, 3)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any liquid radiological release, monitored or unmonitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions

alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

This EAL addresses radioactivity releases that cause effluent radiation monitor readings to exceed 2 times the limit established by a radioactivity discharge permit. Such releases are typically associated with planned batch releases from non-continuous release pathways (e.g., radwaste) (ref. 1).

CND245 CPF Waste Neut Sump Effluent monitor is local indication only.

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1a – Radiological Effluent (Liquid)
Initiating Condition: Release of liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.3 Unusual Event

Sample analysis for a liquid release indicates a concentration or release rate
> 2 x the allocated REMODCM limits for ≥ 60 min. (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous or liquid radiological release, monitored or un-monitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

This EAL addresses uncontrolled liquid releases that are detected by sample analyses or environmental surveys, particularly on unmonitored pathways (e.g., spills of radioactive liquids into storm drains, heat exchanger leakage in ocean water systems, etc.).

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1b – Radiological Effluent (Gaseous)

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1 mrem TEDE

EAL:

RU1.4 Unusual Event

Reading on **any** Table R-1 effluent radiation monitor > column "UE" for ≥ 60 min.
(Notes 1, 2, 3)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no longer VALID** for classification purposes.

Table R-1 Unit 2 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Unit 2 Stack Gaseous				
Normal Range RM-8132B	N/A	N/A	N/A	3.7E+05 cpm
Mid/High Range RM-8168	1.6E+01 $\mu\text{Ci/cc}$	1.6E+00 $\mu\text{Ci/cc}$	1.6E-01 $\mu\text{Ci/cc}$	1.6E-02 $\mu\text{Ci/cc}$
Millstone Stack (WRGM)				
RM-8169	3.6E+02 $\mu\text{Ci/cc}$	3.6E+01 $\mu\text{Ci/cc}$	3.6E+00 $\mu\text{Ci/cc}$	3.6E-01 $\mu\text{Ci/cc}$
Main Steam Line Rad				
RM-4299 A/C	3.5E+01 R/hr	3.5E+00 R/hr	3.5E-01 R/hr	N/A
RM-4299 B	1.7E+01 R/hr	1.7E+00 R/hr	1.7E-01 R/hr	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous radiological release, monitored or un-monitored.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

This EAL addresses normally occurring continuous radioactivity releases from monitored gaseous effluent pathways (ref. 1, 2).

The basis for the gaseous Unusual Event IC and associated thresholds has been revised to correspond to any unplanned release of gaseous effluent radioactivity to the environment that will result in greater than 1 mrem TEDE. This Unusual Event gaseous release criterion is being used consistently across all operating nuclear units at Dominion Energy nuclear stations at Millstone, North Anna and Surry. The reason this alternative criterion is required is due to the fact that for some effluent gaseous release pathways, the resulting calculated Unusual Event threshold following the NEI 99-01 guidance of two times the site-specific effluent release limit would result in a Unusual Event threshold value greater than the corresponding calculated ALERT threshold based on exceeding 10 mrem TEDE. For the other gaseous release pathways that did not show an incongruent relationship when compared to the ALERT threshold, many showed Unusual Event values essentially equivalent to 1 mrem TEDE when applying the guidance in NEI 99-01 of a value set at two times the site-specific effluent release limit. The fact that, (1) many of the gaseous release pathway Unusual Event values following NEI 99-01 guidance were essentially equivalent to 1 mrem TEDE, (2) application of an alternative definition set at a value of 1 mrem TEDE results in a more limiting value for those release paths that showed incongruent comparison to the corresponding ALERT threshold, and (3) Unusual Event criterion set at a value ten (10) times lower than the ALERT threshold provides a logical and consistent escalation between each classification level, provides justification for the Unusual Event criterion of 1 mrem TEDE. This single Initiating Condition (IC) definition for gaseous releases at the Unusual Event level is being applied to maintain consistency across the Dominion Energy nuclear fleet and to reduce confusion and human error potential if two different (IC) definitions were applied.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for

each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Due to the fact that there are no REMODCM limits on main steam line exhausts and the limited ability for these respective radiation monitors to detect low level activity in these steam line configurations, the UE classification threshold for the steam line exhaust is being labeled N/A (not applicable).

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. RP 18-02, "MP2 Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01, Rev. 6"
3. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1b – Radiological Effluent (Gaseous)
Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1 mrem TEDE

EAL:

RU1.5 Unusual Event

Sample analysis for a gaseous release indicates a concentration or release rate $> 2 \times$ the allocated REMODCM limits for ≥ 60 min. (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous radiological release, monitored or unmonitored.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

Calculation RP 18-02 (ref. 2) demonstrates how a release rate limit based on $2 \times$ the allocated REMODCM limit will produce essentially 1 mrem TEDE assuming most prevalent meteorological dispersion.

Most prevalent meteorology represents conditions that would most likely to exist (based on most prevalent stability class and average wind speed within that stability class). Dispersion based on most prevalent meteorology differs from that assumed in the REMODCM which uses annual average meteorology. Dispersion based on actual meteorological conditions at the time

of the emergency (most prevalent) can be 10 – 20 times higher than the annual average dispersion prescribed for use in an ODCM.

This EAL addresses uncontrolled gaseous releases that are detected by sample analyses or environmental surveys.

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. RP 18-02, "MP2 Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
3. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.1 Alert

Reading on **any** Table R-1 effluent radiation monitor > column "ALERT" for ≥15 min.
(Notes 1, 2, 3, 4)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Table R-1 Unit 2 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Unit 2 Stack Gaseous				
Normal Range RM-8132B	N/A	N/A	N/A	3.7E+05 cpm
Mid/High Range RM-8168	1.6E+01 µCi/cc	1.6E+00 µCi/cc	1.6E-01 µCi/cc	1.6E-02 µCi/cc
Millstone Stack (WRGM)				
RM-8169	3.6E+02 µCi/cc	3.6E+01 µCi/cc	3.6E+00 µCi/cc	3.6E-01 µCi/cc
Main Steam Line Rad				
RM-4299 A/C	3.5E+01 R/hr	3.5E+00 R/hr	3.5E-01 R/hr	N/A
RM-4299 B	1.7E+01 R/hr	1.7E+00 R/hr	1.7E-01 R/hr	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an

actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem TEDE .

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Escalation of the emergency classification level would be via IC RS1.

Reference(s):

1. RP 18-02, "MP2 Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.2 Alert

Dose assessment using actual meteorology indicates doses > 10 mrem TEDE at or beyond the SITE BOUNDARY (Note 4)

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Actual meteorology is specifically identified since it gives the most accurate dose assessment. Actual meteorology (including forecasts) should be used whenever possible.

Escalation of the emergency classification level would be via IC RS1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.3 Alert

Analysis of a liquid effluent sample indicates a concentration or release rate that would result in doses > 10 mrem TEDE at or beyond the SITE BOUNDARY for 60 min. of exposure (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem for TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

This EAL is assessed per the REMODCM (ref. 2). REMODCM software can be used to produce a dose to the maximum individual.

Escalation of the emergency classification level would be via IC RS1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.4 Alert

Field survey results indicate closed window dose rates > 10 mR/hr expected to continue for ≥60 min. at or beyond the SITE BOUNDARY (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem for TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

Escalation of the emergency classification level would be via IC RS1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-26-EPI-FAP04, "Emergency Operations Facility Activation and Operation"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 100 mrem TEDE

EAL:

RS1.1 Site Area Emergency

Reading on **any** Table R-1 effluent radiation monitor > column "SAE" for ≥ 15 min.
(Notes 1, 2, 3, 4)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Table R-1 Unit 2 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Unit 2 Stack Gaseous				
Normal Range RM-8132B	N/A	N/A	N/A	3.7E+05 cpm
Mid/High Range RM-8168	1.6E+01 $\mu\text{Ci/cc}$	1.6E+00 $\mu\text{Ci/cc}$	1.6E-01 $\mu\text{Ci/cc}$	1.6E-02 $\mu\text{Ci/cc}$
Millstone Stack (WRGM)				
RM-8169	3.6E+02 $\mu\text{Ci/cc}$	3.6E+01 $\mu\text{Ci/cc}$	3.6E+00 $\mu\text{Ci/cc}$	3.6E-01 $\mu\text{Ci/cc}$
Main Steam Line Rad				
RM-4299 A/C	3.5E+01 R/hr	3.5E+00 R/hr	3.5E-01 R/hr	N/A
RM-4299 B	1.7E+01 R/hr	1.7E+00 R/hr	1.7E-01 R/hr	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to 10% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude are associated with

the failure of plant systems needed for the protection of the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 10% of the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Escalation of the emergency classification level would be via IC RG1.

Reference(s):

1. RP 18-02 MP2, "Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AS1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 100 mrem TEDE

EAL:

RS1.2 Site Area Emergency

Dose assessment using actual meteorology indicates doses > 100 mrem TEDE at or beyond the SITE BOUNDARY (Note 4)

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to 10% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude are associated with the failure of plant systems needed for the protection of the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 10% of the EPA PAG of 1,000 mrem for TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Actual meteorology is specifically identified since it gives the most accurate dose assessment. Actual meteorology (including forecasts) should be used whenever possible.

Escalation of the emergency classification level would be via IC RG1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AS1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 100 mrem TEDE

EAL:

RS1.3 Site Area Emergency

Field survey results indicate closed window dose rates > 100 mR/hr expected to continue for ≥60 min. at or beyond the SITE BOUNDARY (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to 10% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude are associated with the failure of plant systems needed for the protection of the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 10% of the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

Escalation of the emergency classification level would be via IC RG1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-26-EPI-FAP04, "Emergency Operations Facility Activation and Operation"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AS1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE

EAL:

RG1.1 General Emergency

Reading on **any** Table R-1 effluent radiation monitor > column "GE" for ≥ 15 min.
(Notes 1, 2, 3, 4)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Table R-1 Unit 2 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Unit 2 Stack Gaseous				
Normal Range RM-8132B	N/A	N/A	N/A	3.7E+05 cpm
Mid/High Range RM-8168	1.6E+01 $\mu\text{Ci/cc}$	1.6E+00 $\mu\text{Ci/cc}$	1.6E-01 $\mu\text{Ci/cc}$	1.6E-02 $\mu\text{Ci/cc}$
Millstone Stack (WRGM)				
RM-8169	3.6E+02 $\mu\text{Ci/cc}$	3.6E+01 $\mu\text{Ci/cc}$	3.6E+00 $\mu\text{Ci/cc}$	3.6E-01 $\mu\text{Ci/cc}$
Main Steam Line Rad				
RM-4299 A/C	3.5E+01 R/hr	3.5E+00 R/hr	3.5E-01 R/hr	N/A
RM-4299 B	1.7E+01 R/hr	1.7E+00 R/hr	1.7E-01 R/hr	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude will require implementation

of protective actions for the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Reference(s):

1. RP 18-02 MP2, "Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
2. EPA-400, PAG, "Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AG1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE

EAL:

RG1.2 General Emergency

Dose assessment using actual meteorology indicates doses > 1,000 mrem TEDE at or beyond the SITE BOUNDARY (Note 4)

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude will require implementation of protective actions for the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Actual meteorology is specifically identified since it gives the most accurate dose assessment. Actual meteorology (including forecasts) should be used whenever possible.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AG1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE

EAL:

RG1.3 General Emergency

Field survey results indicate closed window dose rates > 1,000 mR/hr expected to continue for ≥60 min. at or beyond the SITE BOUNDARY (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude will require implementation of protective actions for the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-26-EPI-FAP04, "Emergency Operations Facility Activation and Operation"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AG1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: UNPLANNED loss of water level above irradiated fuel

EAL:

RU2.1 Unusual Event

UNPLANNED water level drop in the REFUELING PATHWAY as indicated by **any** of the following:

- SFP LEVEL LO alarm C-06/7 B-14
- CTMT NORM SUMP LEVEL HI/LO alarm C-06/7 BA-21
- Report of dropping level in RFP or SFP
- SFP CLG PUMP SUCTION FLOW LO alarm C-06/07 D-13

AND

UNPLANNED rise in corresponding area radiation levels as indicated by **any** of the following radiation monitors:

- RM-7890 Personnel Access Area
- RM-7891 Containment Refuel Floor
- RM-8139 SFP SW Area
- RM-8142 SFP NW Area
- RM-8156 SFP NE Area
- RM-8157 SFP SE Area

Mode Applicability:

All

Definition(s):

UNPLANNED- A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

REFUELING PATHWAY- Refueling pool (RFP), fuel transfer canal, and spent fuel pool (SFP), but **not** including the reactor vessel, comprise the refueling pathway.

Basis:

This IC addresses a decrease in water level above irradiated fuel sufficient to cause elevated radiation levels. This condition could be a precursor to a more serious event and is also indicative of a minor loss in the ability to control radiation levels within the plant. It is therefore a potential degradation in the level of safety of the plant.

A water level decrease will be primarily determined by indications from available level instrumentation. Other sources of level indications may include reports from plant personnel (e.g., from a refueling crew) or video camera observations (if available). A significant drop in the water level may also cause a loss of SFP Cooling suction flow and an increase in the radiation levels of adjacent areas that can be detected by monitors in those locations.

The specified radiation monitors are those expected to see increase area radiation levels as a result of a loss of REFUELING PATHWAY inventory (ref. 1, 2, 3). Increasing radiation indications on these monitors in the absence of indications of decreasing REFUELING PATHWAY level are not classifiable under this EAL.

The effects of planned evolutions should be considered. For example, a refueling bridge area radiation monitor reading may increase due to planned evolutions such as lifting of the reactor vessel head or movement of a fuel assembly. Note that this EAL is applicable only in cases where the elevated reading is due to an unplanned loss of water level.

A drop in water level above irradiated fuel within the reactor vessel may be classified in accordance Category C during the Cold Shutdown and Refueling modes.

Escalation of the emergency classification level would be via IC RA2.

Reference(s):

1. AOP 2578, "Loss of Refuel Pool and Spent Fuel Pool Level"
2. AOP 2577, "Fuel Handling Accident"
3. OP 2383B, "Area Radiation Monitors"
4. NEI 99-01 AU2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Significant lowering of water level above, or damage to, irradiated fuel

EAL:

RA2.1 Alert

IMMINENT uncovering of irradiated fuel in the REFUELING PATHWAY

Mode Applicability:

All

Definition(s):

CONFINEMENT BOUNDARY- The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielding Canister (DSC).

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

REFUELING PATHWAY- Refueling pool (RFP), fuel transfer canal, and spent fuel pool (SFP), but **not** including the reactor vessel, comprise the refueling pathway.

Basis:

This IC addresses events that have caused IMMINENT or actual damage to an irradiated fuel assembly, or a significant lowering of water level within the REFUELING PATHWAY. These events present radiological safety challenges to plant personnel and are precursors to a release of radioactivity to the environment. As such, they represent an actual or potential substantial degradation of the level of safety of the plant.

For irradiated fuel that is licensed for dry storage, this EAL applies up to the point that the loaded storage cask is sealed. Once sealed, damage to a loaded cask causing loss of the CONFINEMENT BOUNDARY is classified in accordance with IC EU1.

Escalation of the emergency would be based on either Category R or C ICs.

This EAL escalates from RU2.1 in that the loss of level, in the affected portion of the REFUELING PATHWAY, is of sufficient magnitude to have resulted in uncovering of irradiated fuel. Indications of irradiated fuel uncovering may include direct or indirect visual observation (e.g., reports from personnel or camera images), as well as significant changes in water and radiation levels, or other plant parameters. Computational aids may also be used (e.g., a boil-off curve). Classification of an event using this EAL should be based on the totality of available indications, reports and observations.

While an area radiation monitor could detect an increase in a dose rate due to a lowering of water level in some portion of the REFUELING PATHWAY, the reading may not be a reliable indication of whether or not the fuel is actually uncovered. To the degree possible, readings should be considered in combination with other available indications of inventory loss.

A drop in water level above irradiated fuel within the reactor vessel may be classified in

accordance with Category C during the Cold Shutdown and Refueling modes.

Reference(s):

1. NEI 99-01 AA2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Significant lowering of water level above, or damage to, irradiated fuel
EAL:

RA2.2 Alert

Damage to irradiated fuel resulting in a release of radioactivity

AND

VALID alarm on **any** of the following radiation monitors:

- RM-7890 Personnel Access Area
- RM-7891 Containment Refuel Floor
- RM-8123B or RM-8262B Containment Gaseous
- RM-8139 SFP SW Area
- RM-8142 SFP NW Area
- RM-8156 SFP NE Area
- RM-8157 SFP SE Area
- RM-8145B Spent Fuel Pool Exhaust Gaseous

Mode Applicability:

All

CONFINEMENT BOUNDARY- The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielding Canister (DSC).

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

The specified radiation monitors are those expected to see increased area radiation levels as a result of damage to irradiated fuel (ref. 1, 2, 3, 4).

This EAL addresses events that have caused actual damage to an irradiated fuel assembly. These events present radiological safety challenges to plant personnel and are precursors to a release of radioactivity to the environment. As such, they represent an actual or potential substantial degradation of the level of safety of the plant.

For irradiated fuel that is licensed for dry storage, this EAL applies up to the point that the

loaded storage cask is sealed. Once sealed, damage to a loaded cask causing loss of the CONFINEMENT BOUNDARY is classified in accordance with IC EU1.

This EAL addresses a release of radioactive material caused by mechanical damage to irradiated fuel. Damaging events may include the dropping, bumping or binding of an assembly, or dropping a heavy load onto an assembly. A rise in readings on radiation monitors should be considered in conjunction with in-plant reports or observations of a potential fuel damaging event (e.g., a fuel handling accident).

Escalation of the emergency would be based on either Category R or C ICs.

Reference(s):

1. AOP 2577, "Fuel Handling Accident"
2. AOP 2578, "Loss of Refuel Pool and Spent Fuel Pool Level"
3. OP 2383B, "Area Radiation Monitors"
4. NEI 99-01 AA2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Significant lowering of water level above, or damage to, irradiated fuel

EAL:

RA2.3 Alert

Lowering of spent fuel pool level to 10 ft. (Level 2) on LI-7013 or LI-7014

Mode Applicability:

All

Definition(s):

None

Basis:

This EAL addresses events that have caused a significant lowering of water level within the spent fuel pool. These events present radiological safety challenges to plant personnel and are precursors to a release of radioactivity to the environment. As such, they represent an actual or potential substantial degradation of the level of safety of the plant.

Spent fuel pool water level at this value is within the lower end of the level range necessary to prevent significant dose consequences from direct gamma radiation to personnel performing operations in the vicinity of the spent fuel pool. This condition reflects a significant loss of spent fuel pool water inventory and thus it is also a precursor to a loss of the ability to adequately cool the irradiated fuel assemblies stored in the pool.

Escalation of the emergency classification level would be via ICs RS1 or RS2.

Post-Fukushima order EA-12-051 required the installation of reliable SFP level indication (LI-7013 and LI-7014) capable of identifying normal level (Level 1 – EL 36 ft. 3 in.), SFP level 10 ft. above the top of the fuel racks (Level 2 – EL 23 ft. 3 in.) and SFP level at 1 ft. above the top of the fuel racks (Level 3 – EL 14 ft. 3 in.) (ref. 1, 2).

Primary level indicator LI-7013 and display cabinet C547 are located on the 25 ft. 6 in. elevation of the Auxiliary Building in the Cable Vault Room near column lines K.6-15.9. The backup channel level indicator LI-7014 and display cabinet C548 are located on the 36 ft. 6 in. elevation of the Auxiliary Building in the East 480V Switchgear Room near column line F.8-15.9.

Reference(s):

1. ETE-CPR-2012-0009, "Millstone Unit 2 – Beyond Design Basis FLEX Strategy Basis Document and Final Integrated Plan"
2. DC MP2-13-01011 "Beyond Design Basis Spent Fuel Pool Level Instrument Installation, Millstone Unit 2"
3. NEI 99-01 AA2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Spent fuel pool level at the top of the fuel racks

EAL:

RS2.1 Site Area Emergency

Lowering of spent fuel pool level to 1 ft. (Level 3) on LI-7013 or LI-7014

Mode Applicability:

All

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

This EAL addresses a significant loss of spent fuel pool inventory control and makeup capability leading to IMMINENT fuel damage. This condition entails major failures of plant functions needed for protection of the public and thus warrant a Site Area Emergency declaration.

It is recognized that this IC would likely not be met until well after another Site Area Emergency IC was met; however, it is included to provide classification diversity.

Escalation of the emergency classification level would be via IC RG1 or RG2.

Post-Fukushima order EA-12-051 required the installation of reliable SFP level indication (LI-7013 and LI-7014) capable of identifying normal level (Level 1 – EL 36 ft. 3 in.), SFP level 10 ft. above the top of the fuel racks (Level 2 – EL 23 ft. 3 in.) and SFP level at 1 ft. above the top of the fuel racks (Level 3 – EL 14 ft. 3 in.) (ref. 1, 2).

Primary level indicator LI-7013 and display cabinet C547 are located on the 25 ft. 6 in. elevation of the Auxiliary Building in the Cable Vault Room near column lines K.6-15.9. The backup channel level indicator LI-7014 and display cabinet C548 are located on the 36 ft. 6 in. elevation of the Auxiliary Building in the East 480V Switchgear Room near column line F.8-15.9.

Reference(s):

1. ETE-CPR-2012-0009, "Millstone Unit 2 – Beyond Design Basis FLEX Strategy Basis Document and Final Integrated Plan"
2. DC MP2-13-01011 "Beyond Design Basis Spent Fuel Pool Level Instrument Installation, Millstone Unit 2"
3. NEI 99-01 AS2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Spent fuel pool level **cannot** be restored to at least the top of the fuel racks for 60 minutes or longer

EAL:

RG2.1 General Emergency

Spent fuel pool level **cannot** be restored to at least 1 ft. (Level 3) on LI-7013 or LI-7014 for ≥ 60 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

All

Definition(s):

None

Basis:

This EAL addresses a significant loss of spent fuel pool inventory control and makeup capability leading to a prolonged uncover of spent fuel. This condition will lead to fuel damage and a radiological release to the environment.

It is recognized that this EAL would likely not be met until well after another General Emergency EAL was met; however, it is included to provide classification diversity.

Post-Fukushima order EA-12-051 required the installation of reliable SFP level indication (LI-7013 and LI-7014) capable of identifying normal level (Level 1 – EL 36 ft. 3 in.), SFP level 10 ft. above the top of the fuel racks (Level 2 – EL 23 ft. 3 in.) and SFP level at 1 ft. above the top of the fuel racks (Level 3 – EL 14 ft. 3 in.) (ref. 1, 2).

Primary level indicator LI-7013 and display cabinet C547 are located on the 25 ft. 6 in. elevation of the Auxiliary Building in the Cable Vault Room near column lines K.6-15.9. The backup channel level indicator LI-7014 and display cabinet C548 are located on the 36 ft. 6 in. elevation of the Auxiliary Building in the East 480V Switchgear Room near column line F.8-15.9.

Reference(s):

1. ETE-CPR-2012-0009, "Millstone Unit 2 – Beyond Design Basis FLEX Strategy Basis Document and Final Integrated Plan"
2. DC MP2-13-01011 "Beyond Design Basis Spent Fuel Pool Level Instrument Installation, Millstone Unit 2"
3. NEI 99-01 AG2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 3 – Area Radiation Levels

Initiating Condition: Radiation levels that IMPEDE access to equipment necessary for normal plant operations, cooldown or shutdown

EAL:

RA3.1 Alert

Dose rate > 15 mR/hr in **EITHER** of the following areas:

- Control Room
- Central Alarm Station

Mode Applicability:

All

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

Basis:

This IC addresses elevated radiation levels in certain plant rooms/areas sufficient to preclude or IMPEDE personnel from performing actions necessary to maintain normal plant operation, or to perform a normal plant cooldown and shutdown. As such, it represents an actual or potential substantial degradation of the level of safety of the plant. The DSEO/ADTS should consider the cause of the increased radiation levels and determine if another IC may be applicable.

Areas that meet this threshold include the Control Room (CR) and the Central Alarm Station (CAS). The Control Room is monitored for excessive radiation by one detector, RM-7899 (ref. 1, 2). The CAS is included in this EAL because of its importance to permitting access to areas required to assure safe plant operations.

Reference(s):

1. OP 2383B, "Area Radiation Monitors"
2. NEI 99-01 AA3

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 3 – Area Radiation Levels

Initiating Condition: Radiation levels that IMPEDE access to equipment necessary for normal plant operations, cooldown or shutdown

EAL:

RA3.2 Alert

An UNPLANNED event results in radiation levels that prohibit or IMPEDE access to **any** Table R-2 room or area (Note 5)

Note 5: If the equipment in the listed room or area was already inoperable or out-of-service before the event occurred, then no emergency classification is warranted.

Table R-2 Safe Operation & Shutdown Rooms/Areas		
Room/Area		Mode
Aux. Building EI -5'6"	West Area	3
Aux. Building EI -5'6"	East Near SFP Cooling	
Aux. Building EI 14'6"	B51 & B61 Enclosures	
Aux. Building EI 14'6"	Boric Acid Batching Tank	
Aux. Building EI -25'6"	RB Hx Area	
Enc. Building EI -5'6"	East Pipe Penetration	
Aux. Building EI 14'6"	By B61	3, 4
Enc. Building EI -45'6"	"A" & "B" ESF Rooms	
Aux. Building EI -45'6"	General Area	
Enc. Building EI -5'6"	West Pipe Penetration	3, 5
Aux. Building EI 14'6"	SE Across From MCC B51	4
Aux. Building EI -5'6"	VCT Block Wall Area	5
Enc. Building EI -45'6"	"A" ESF Room	

Mode Applicability:

3 – Hot Standby, 4 – Hot Shutdown, 5 – Cold Shutdown

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

UNPLANNED- A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses elevated radiation levels in certain plant rooms/areas sufficient to preclude or IMPEDE personnel from performing actions necessary to maintain normal plant operation, or to perform a normal plant cooldown and shutdown. As such, it represents an actual or potential substantial degradation of the level of safety of the plant. The DSEO/ADTS should consider the cause of the increased radiation levels and determine if another IC may be applicable.

For RA3.2, an Alert declaration is warranted if entry into the affected room/area is, or may be, procedurally required during the plant operating mode in effect at the time of the elevated radiation levels. The emergency classification is not contingent upon whether entry is actually necessary at the time of the increased radiation levels. Access should be considered as IMPEDED if extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., installing temporary shielding, requiring use of non-routine protective equipment, requesting an extension in dose limits beyond normal administrative limits).

If the equipment in the listed room or area was already inoperable, or out-of-service, before the event occurred, then no emergency should be declared since the event will have no adverse impact beyond that already allowed by Technical Specifications at the time of the event.

The list of plant rooms or areas with entry-related mode applicability identified specify those rooms or areas that contain equipment which require a manual/local action as specified in operating procedures used for normal plant operation, cooldown and shutdown. Rooms or areas in which actions of a contingent or emergency nature would be performed (e.g., an action to address an off-normal or emergency condition such as emergency repairs, corrective measures or emergency operations) are not included. In addition, the list specifies the plant mode(s) during which entry would be required for each room or area (ref. 1).

An emergency declaration is **not** warranted if any of the following conditions apply:

- The plant is in an operating mode different than the mode specified for the affected room/area (i.e., entry is not required during the operating mode in effect at the time of the elevated radiation levels). For example, the plant is in Mode 1 when the radiation increase occurs, and the procedures used for normal operation, cooldown and shutdown do not require entry into the affected room until Mode 4.
- The increased radiation levels are a result of a planned activity that includes compensatory measures which address the temporary inaccessibility of a room or area (e.g., radiography, spent filter or resin transfer, etc.).
- The action for which room/area entry is required is of an administrative or record keeping nature (e.g., normal rounds or routine inspections).
- The access control measures are of a conservative or precautionary nature, and would not actually prevent or IMPEDE a required action.

Reference(s):

1. Attachment 2, "Safe Operation & Shutdown Areas Tables R-2 & H-2 Bases"
2. NEI 99-01 AA3

Category C – Cold Shutdown / Refueling System Malfunction

EAL Group: Cold Conditions (RCS temperature $\leq 200^{\circ}\text{F}$); EALs in this category are applicable only in one or more cold operating modes.

Category C EALs are directly associated with cold shutdown or refueling system safety functions. Given the variability of plant configurations (e.g., systems out-of-service for maintenance, containment open, reduced AC power redundancy, time since shutdown) during these periods, the consequences of any given initiating event can vary greatly. For example, a loss of decay heat removal capability that occurs at the end of an extended outage has less significance than a similar loss occurring during the first week after shutdown. Compounding these events is the likelihood that instrumentation necessary for assessment may also be inoperable. The cold shutdown and refueling system malfunction EALs are based on performance capability to the extent possible with consideration given to RCS integrity, CONTAINMENT CLOSURE, and fuel clad integrity for the applicable operating modes (5 - Cold Shutdown, 6 - Refueling, DEF – Defueled).

The events of this category pertain to the following subcategories:

1. RCS Level

RCS water level is directly related to the status of adequate core cooling and, therefore, fuel clad integrity.

2. Loss of Emergency AC Power

Loss of vital plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of onsite and offsite power sources for 4.16 KV emergency buses.

3. RCS Temperature

Uncontrolled or inadvertent temperature or pressure rises are indicative of a potential loss of safety functions.

4. Loss of Vital DC Power

Loss of emergency plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of power to or degraded voltage on the 125V DC vital buses.

5. Loss of Communications

Certain events that degrade plant operator ability to effectively communicate with essential personnel within or external to the plant warrant emergency classification.

6. Hazardous Event Affecting Safety Systems

Certain hazardous natural and technological events may result in **VISIBLE DAMAGE** to or degraded performance of safety systems warranting classification.

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: UNPLANNED loss of RCS inventory

EAL:

CU1.1 Unusual Event

UNPLANNED loss of reactor coolant results in RCS water level < a required lower limit for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

UNPLANNED- A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

RCS water level less than a required lower limit is meant to be less than the lower end of the level control band being procedurally maintained for the current condition or evolution.

With the plant in Cold Shutdown, RCS water level is normally maintained within a pressurizer level control band (ref. 1). However, if RCS level is being controlled below the normal pressurizer level control band, or if level is being maintained in a designated band in the reactor vessel it is the inability to maintain level above the low end of the designated control band due to a loss of inventory resulting from a leak in the RCS that is the concern.

With the plant in Refueling mode, RCS water level is normally maintained at or above the reactor vessel flange (ref. 2, 3).

This IC addresses the inability to restore and maintain water level to a required minimum level (or the lower limit of a level band), or a loss of the ability to monitor RCS level concurrent with indications of coolant leakage. Either of these conditions is considered to be a potential degradation of the level of safety of the plant.

Refueling evolutions that decrease RCS water inventory are carefully planned and controlled. An UNPLANNED event that results in water level decreasing below a procedurally required limit warrants the declaration of an Unusual Event due to the reduced water inventory that is available to keep the core covered.

This EAL recognizes that the minimum required RCS level can change several times during the course of a refueling outage as different plant configurations and system lineups are implemented. This EAL is met if the minimum level, specified for the current plant conditions, cannot be maintained for 15 minutes or longer. The minimum level is typically specified in the applicable operating procedure but may be specified in another controlling document.

The 15-minute threshold duration allows sufficient time for prompt operator actions to restore and maintain the expected water level. This criterion excludes transient conditions causing a brief lowering of water level.

Continued loss of RCS inventory may result in escalation to the Alert emergency classification level via either IC CA1 or CA3.

Reference(s):

1. OP 2207, "Plant Cooldown"
2. OP 2209B, "RCS Inventory Tracking"
3. OP 2209A, "Refueling Operations"
4. NEI 99-01 CU1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: UNPLANNED loss of RCS inventory

EAL:

CU1.2 Unusual Event

RCS water level **cannot** be monitored

AND EITHER:

- UNPLANNED rise in **any** Table C-1 sump or tank level due to a loss of RCS inventory
- Visual observation of UNISOLABLE RCS leakage

Table C-1 Sumps/Tanks
<ul style="list-style-type: none"> • Containment Sump • RWST • PDT • Quench Tank • EDST (L9736) • 'A' CWRT (letdown line CH-345 or SDC line SI-468) (Panel C-63) • SIT 1, 2 3, 4

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED-. A parameter changes or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses the inability to restore and maintain water level to a required minimum level (or the lower limit of a level band), or a loss of the ability to monitor RCS level concurrent with indications of coolant leakage. Either of these conditions is considered to be a potential degradation of the level of safety of the plant.

Refueling evolutions that decrease RCS water inventory are carefully planned and controlled. An UNPLANNED event that results in water level decreasing below a procedurally required limit warrants the declaration of an Unusual Event due to the reduced water inventory that is available to keep the core covered.

This EAL addresses a condition where all means to determine RCS level have been lost. In this condition, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels (Table C-1) (ref. 1, 2). Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the RCS (ref. 2, 3, 4).

For indications of increasing water level in the "A" CWRT, sources of RCS inventory is via either letdown line CH-345 or shutdown cooling line SI-468 on Panel (C-63).

Continued loss of RCS inventory may result in escalation to the Alert emergency classification level via either IC CA1 or CA3.

Reference(s):

1. OP 2207, "Plant Cooldown"
2. OP 2209B, "RCS Inventory Tracking"
3. AOP 2572, "Loss of Shutdown Cooling"
4. AOP 2568A, "RCS Leak, Mode 4, 5, 6 and Defueled"
5. NEI 99-01 CU1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Significant Loss of RCS inventory

EAL:

CA1.1 Alert

Loss of RCS inventory as indicated by RCS water level < - 4 in. on LI-112 (CCTV) or L-122 (PPC)

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

None

Basis:

This IC addresses conditions that are precursors to a loss of the ability to adequately cool irradiated fuel (i.e., a precursor to a challenge to the fuel clad barrier). This condition represents a potential substantial reduction in the level of plant safety.

For this EAL, a lowering of RCS water level below – 4 in. on LI-112 or L-122 indicates that operator actions have not been successful in restoring and maintaining RCS water level. The heat-up rate of the coolant will increase as the available water inventory is reduced. A continuing decrease in water level will lead to core uncover. RCS hot leg level of 4 in. below centerline is the lowest RCS level that supports continued decay heat removal pump operations (SDC) (ref. 1, 2, 3).

Although related, this EAL is concerned with the loss of RCS inventory and not the potential concurrent effects on systems needed for decay heat removal (e.g., loss of a Decay Heat Removal suction point). An increase in RCS temperature caused by a loss of decay heat removal capability is evaluated under IC CA3.

A decreasing RCS level can be indicated by (ref. 1):

- L-112 RCS mid-loop wide range RCS level transmitter (PPC)
- L-122 No. 2 hot leg narrow range level transmitter (PPC)
- LI-112 No. 1 hot leg RCS mid-loop level indicator (CCTV)
- ICC Reactor Vessel Level Monitoring System (RVLMS)

If RCS water level continues to lower, then escalation to Site Area Emergency would be via IC CS1.

Reference(s):

1. AOP 2572, "Loss of Shutdown Cooling"
2. AOP 2568A, "RCS Leak, Mode 4, 5, 6 and Defueled"
3. OP 2301E, "Draining the RCS (ICCE)"
4. NEI 99-01 CA1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Significant Loss of RCS inventory

EAL:

CA1.2 Alert

RCS water level **cannot** be monitored for ≥ 15 min. (Note 1)

AND EITHER

- UNPLANNED rise in **any** Table C-1 sump or tank level due to a loss of RCS inventory
- Visual observation of UNISOLABLE RCS leakage

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table C-1 Sumps/Tanks
<ul style="list-style-type: none"> • Containment Sump • RWST • PDT • Quench Tank • EDST (L9736) • 'A' CWRT (letdown line CH-345 or SDC line SI-468) (Panel C-63) • SIT 1, 2 3, 4

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses conditions that are precursors to a loss of the ability to adequately cool irradiated fuel (i.e., a precursor to a challenge to the fuel clad barrier). This condition represents a potential substantial reduction in the level of plant safety.

For this EAL, the inability to monitor RCS level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation. If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels (ref. 1, 2). Sump and/or tank level changes

must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the RCS (ref 2, 3, 4).

The 15-minute duration for the loss of level indication was chosen because it is half of the EAL duration specified in IC CS1.

If the RCS inventory level continues to lower, then escalation to Site Area Emergency would be via IC CS1.

Reference(s):

1. OP 2207, "Plant Cooldown"
2. OP 2209B, "RCS Inventory Tracking"
3. AOP 2572, "Loss of Shutdown Cooling"
4. AOP 2568A, "RCS Leak, Mode 4, 5, 6 and Defueled"
5. NEI 99-01 CA1

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 1 – RCS Level
Initiating Condition: Loss of RCS inventory affecting core decay heat removal capability
EAL:

CS1.1 Site Area Emergency RVLMS reading 0% (#8) (Note 12)

Note 12: This EAL is only applicable if a RVLMS channel #8 string is operable.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

This IC addresses a significant and prolonged loss of RCS inventory control and makeup capability leading to IMMINENT fuel damage. The lost inventory may be due to a RCS component failure, a loss of configuration control or prolonged boiling of reactor coolant. These conditions entail major failures of plant functions needed for protection of the public and thus warrant a Site Area Emergency declaration.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in RCS level. If RCS level cannot be restored, fuel damage is probable.

Millstone 2 includes in its Inadequate Core Cooling instrumentation a reactor vessel level monitoring system (RVLMS) that is displayed to the operators and can measure discrete reactor vessel water levels from the top of the reactor vessel head to the top of the core plate (string #8 - approximately 10 inches above the top of the active fuel). The bottom of this instrument's span in the reactor vessel plenum is the lowest available reactor vessel level indicator and is used in this EAL to represent approximately the top of active fuel (ref. 1).

RVLMS is only required to be operable in modes 1, 2 and 3 (ref. 2). For plant conditions in which RVLMS is disconnected or otherwise inoperable, such as in the Refueling mode, classification should be made based on CS1.2 when RCS water level cannot be monitored.

This EAL addresses concerns raised by Generic Letter 88-17, Loss of Decay Heat Removal; SECY 91-283, Evaluation of Shutdown and Low Power Risk Issues; NUREG-1449, Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States; and NUMARC 91-06, Guidelines for Industry Actions to Assess Shutdown Management.

Escalation of the emergency classification level would be via ICs CG1 or RG1.

Reference(s):

1. Millstone 2 UFSAR Section 7.5.4.4, "In-Core Instrumentation"
2. Millstone 2 Technical Specifications Section 3.3.3.8, "Instrumentation Accident Monitoring"
3. NEI 99-01 CS1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Loss of RCS inventory affecting core decay heat removal capability

EAL:

CS1.2 Site Area Emergency

RCS level **cannot** be monitored for ≥ 30 min. (Note 1)

AND

Core uncover is indicated by **any** of the following:

- UNPLANNED rise in **any** Table C-1 sump or tank level of sufficient magnitude to indicate core uncover
- Visual observation of UNISOLABLE RCS leakage of sufficient magnitude to indicate core uncover
- Erratic source range monitor indications
- **Any** containment area radiation monitor reading > 4 R/hr (Refueling mode)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table C-1 Sumps/Tanks
<ul style="list-style-type: none"> • Containment Sump • RWST • PDT • Quench Tank • EDST (L9736) • 'A' CWRT (letdown line CH-345 or SDC line SI-468) (Panel C-63) • SIT 1, 2 3, 4

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses a significant and prolonged loss of RCS inventory control and makeup capability leading to IMMINENT fuel damage. The lost inventory may be due to a RCS component failure, a loss of configuration control or prolonged boiling of reactor coolant. These conditions entail major failures of plant functions needed for protection of the public and thus warrant a Site Area Emergency declaration.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in reactor vessel level. If RCS level cannot be restored, fuel damage is probable.

In this EAL, the 30-minute criterion is tied to a readily recognizable event start time (i.e., the total loss of ability to monitor level), and allows sufficient time to monitor, assess and correlate reactor and plant conditions to determine if core uncover has actually occurred (i.e., to account for various accident progression and instrumentation uncertainties). It also allows sufficient time for performance of actions to terminate leakage, recover inventory control/makeup equipment and/or restore level monitoring.

The inability to monitor RCS level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation (ref. 1, 2). If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels. Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the RCS (ref. 2, 3, 4).

If the make-up rate to the RCS unexplainably rises above the pre-established rate, a loss of RCS inventory may be occurring even if the source of the leakage cannot be immediately identified. Visual observation of leakage from systems connected to the RCS that cannot be isolated could also be indicative of a loss of RCS inventory.

Post-TMI accident studies indicated that the installed PWR nuclear instrumentation will operate erratically when the core is uncovered and that this should be used as a tool for making such determinations.

In the Refueling mode, as water level in the reactor vessel lowers, the dose rate above the core will rise. The dose rate due to this core shine should result in on-scale indications of > 4 R/hr on containment area radiation monitors (ref. 5).

This EAL addresses concerns raised by Generic Letter 88-17, Loss of Decay Heat Removal; SECY 91-283, Evaluation of Shutdown and Low Power Risk Issues; NUREG-1449, Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States; and NUMARC 91-06, Guidelines for Industry Actions to Assess Shutdown Management.

Escalation of the emergency classification level would be via IC CG1 or AG1

Reference(s):

1. OP 2207, "Plant Cooldown"
2. OP 2209B, "RCS Inventory Tracking"
3. AOP 2572, "Loss of Shutdown Cooling"
4. AOP 2568A, "RCS Leak, Mode 4, 5, 6 and Defueled"
5. Calculation M2EP-04164R2, "MP2 Rad Monitor Response to Core Uncovery"

Millstone Power Station Unit 2
Emergency Action Level Technical Bases Document
Attachment 1 Emergency Action Level Technical Bases

Serial No. 18-364
Docket Nos. 50-336; 72-47
Enclosure 3; Attachment 3B

6. Millstone Unit 2 Radiation Monitor Manual
7. NEI 99-01 CS1

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 1 – RCS Level
Initiating Condition: Loss of RCS inventory affecting fuel clad integrity with containment challenged

EAL:

CG1.1 General Emergency

RVLMS reading 0% (#8) for ≥ 30 min. (Notes 1, 12)

AND

Any Containment Challenge indication, Table C-2

- Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.
- Note 6: If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, declaration of a General Emergency is **not** required.
- Note 12: This EAL is only applicable if a RVLMS channel #8 string is operable.

Table C-2 Containment Challenge Indications

- CONTAINMENT CLOSURE **not** established (Note 6)
- CTMT hydrogen concentration > 4%
- UNPLANNED increase in CTMT pressure

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The actions taken to secure containment (Primary or Secondary) and their associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment door is closed and held in place by a minimum of four bolts or the outage equipment door is installed.
- A minimum of one door in each airlock is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere is closed by an isolation valve, blind flange, manual valve, or special device.

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

Millstone 2 includes in its Inadequate Core Cooling instrumentation a reactor vessel level monitoring system (RVLMS) that is displayed to the operators and can measure discrete reactor vessel water levels from the top of the reactor vessel head to the top of the core plate (string #8 - approximately 10 inches above the top of the active fuel). The bottom of this instrument's span in the reactor vessel plenum is the lowest available reactor vessel level indicator and is used in this EAL to represent approximately the top of active fuel (ref. 1).

RVLMS is only required to be operable in modes 1, 2 and 3 (ref. 2). For plant conditions in which RVLMS is disconnected or otherwise inoperable, such as in the Refueling mode, classification should be made based on CG1.2 when RCS water level cannot be monitored.

This IC addresses the inability to restore and maintain RCS level above the top of active fuel with containment challenged. This condition represents actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity. Releases can be reasonably expected to exceed EPA PAG exposure levels offsite for more than the immediate site area.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in reactor vessel level. If RCS level cannot be restored, fuel damage is probable.

Three conditions are associated with a challenge to containment's capability to serve as an effective barrier to fission product release:

1. With CONTAINMENT CLOSURE not established, there is a high potential for a direct and unmonitored release of radioactivity to the environment. If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, then declaration of a General Emergency is not required.
2. The existence of an explosive mixture means, at a minimum, that the containment atmospheric hydrogen concentration is sufficient to support a hydrogen burn (i.e., at the lower deflagration limit of 4%). A hydrogen burn will raise containment pressure and could result in collateral equipment damage leading to a loss of containment integrity. It therefore represents a challenge to containment integrity.

In the early stages of a core uncover event, it is unlikely that hydrogen buildup due to a core uncover could result in an explosive gas mixture in containment. However, containment monitoring and/or sampling should be performed to verify this assumption and a General Emergency declared if it is determined that hydrogen concentration has exceeded the minimum necessary to support a hydrogen burn (4%) (ref. 3, 4). If all installed hydrogen gas monitors are out-of-service during an event leading to fuel cladding damage, it may not be possible to obtain a containment hydrogen gas concentration reading as ambient conditions within the containment will preclude personnel access. During periods when installed containment hydrogen gas monitors are out-of-service, operators may use the other listed indications to assess whether or not containment is challenged.

3. Any UNPLANNED rise in containment pressure in the Cold Shutdown or Refueling mode indicates a potential challenge of CONTAINMENT CLOSURE capability due to likely reduced pressure capability of the containment (temporary barriers installed for CONTAINMENT CLOSURE). UNPLANNED containment pressure rise indicates CONTAINMENT CLOSURE cannot be assured and the containment cannot be relied upon as a barrier to fission product release.

This EAL addresses concerns raised by Generic Letter 88-17, *Loss of Decay Heat Removal*; SECY 91-283, *Evaluation of Shutdown and Low Power Risk Issues*; NUREG-1449, *Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States*; and NUMARC 91-06, *Guidelines for Industry Actions to Assess Shutdown Management*.

Reference(s):

1. Millstone 2 UFSAR Section 7.5.4.4, "In-Core Instrumentation"
2. Millstone 2 Technical Specifications Section 3.3.3.8, "Instrumentation Accident Monitoring"
3. EOP 2540, "Functional Recovery"
4. SAMG 4215 Attachment 1, "Calculation Aids, Calculational Aid #7 Containment Challenge Due to Hydrogen Combustion"
5. NEI 99-01 CG1

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 1 – RCS Level
Initiating Condition: Loss of RCS inventory affecting fuel clad integrity with containment challenged

EAL:

CG1.2 General Emergency

RCS level **cannot** be monitored for ≥ 30 min. (Note 1)

AND

Core uncover is indicated by **any** of the following:

- UNPLANNED rise in **any** Table C-1 sump or tank level of sufficient magnitude to indicate core uncover
- Visual observation of UNISOLABLE RCS leakage of sufficient magnitude to indicate core uncover
- Erratic source range monitor indications
- **Any** containment area radiation monitor reading > 4 R/hr (Refueling mode)

AND

Any Containment Challenge indication, Table C-2

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 6: If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, declaration of a General Emergency is **not** required.

Table C-1 Sumps/Tanks
<ul style="list-style-type: none">• Containment Sump• RWST• PDT• Quench Tank• EDST (L9736)• 'A' CWRT (letdown line CH-345 or SDC line SI-468) (Panel C-63)• SIT 1, 2 3, 4

Table C-2 Containment Challenge Indications

- CONTAINMENT CLOSURE **not** established (Note 6)
- CTMT hydrogen concentration > 4%
- UNPLANNED increase in CTMT pressure

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The actions taken to secure containment (Primary or Secondary) and their associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment door is closed and held in place by a minimum of four bolts or the outage equipment door is installed.
- A minimum of one door in each airlock is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere is closed by an isolation valve, blind flange, manual valve, or special device.

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses the inability to restore and maintain RCS level above the top of active fuel with containment challenged. This condition represents actual or **IMMINENT** substantial core degradation or melting with potential for loss of containment integrity. Releases can be reasonably expected to exceed EPA PAG exposure levels offsite for more than the immediate site area.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in reactor vessel level. If RCS level cannot be restored, fuel damage is probable.

The inability to monitor RCS level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation (ref. 1, 2). If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels. Sump and/or tank level changes must be evaluated

against other potential sources of water flow to ensure they are indicative of leakage from the RCS (ref. 2, 3, 4).

If the make-up rate to the RCS unexplainably rises above the pre-established rate, a loss of RCS inventory may be occurring even if the source of the leakage cannot be immediately identified. Visual observation of leakage from systems connected to the RCS that cannot be isolated could also be indicative of a loss of RCS inventory.

Post-TMI accident studies indicated that the installed PWR nuclear instrumentation will operate erratically when the core is uncovered and that this should be used as a tool for making such

In the Refueling mode, as water level in the reactor vessel lowers, the dose rate above the core will rise. The dose rate due to this core shine should result in on-scale indications of > 4 R/hr on containment area radiation monitors (ref. 5).

The 30-minute criterion is tied to a readily recognizable event start time (i.e., the total loss of ability to monitor level), and allows sufficient time to monitor, assess and correlate reactor and plant conditions to determine if core uncover has actually occurred (i.e., to account for various accident progression and instrumentation uncertainties). It also allows sufficient time for performance of actions to terminate leakage, recover inventory control/makeup equipment and/or restore level monitoring.

Three conditions are associated with a challenge to containment's capability to serve as an effective barrier to fission product release:

1. With CONTAINMENT CLOSURE not established, there is a high potential for a direct and unmonitored release of radioactivity to the environment. If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, then declaration of a General Emergency is not required.
2. The existence of an explosive mixture means, at a minimum, that the containment atmospheric hydrogen concentration is sufficient to support a hydrogen burn (i.e., at the lower deflagration limit of 4%). A hydrogen burn will raise containment pressure and could result in collateral equipment damage leading to a loss of containment integrity. It therefore represents a challenge to containment integrity.

In the early stages of a core uncover event, it is unlikely that hydrogen buildup due to a core uncover could result in an explosive gas mixture in containment. However, containment monitoring and/or sampling should be performed to verify this assumption and a General Emergency declared if it is determined that hydrogen concentration has exceeded the minimum necessary to support a hydrogen burn (4%) (ref. 7). If all installed hydrogen gas monitors are out-of-service during an event leading to fuel cladding damage, it may not be possible to obtain a containment hydrogen gas concentration reading as ambient conditions within the containment will preclude personnel access. During periods when installed containment hydrogen gas monitors are out-of-service, operators may use the other listed indications to assess whether or not containment is challenged.

3. Any UNPLANNED rise in containment pressure in the Cold Shutdown or Refueling mode indicates a potential challenge of CONTAINMENT CLOSURE capability due to likely reduced pressure capability of the containment (temporary barriers installed for CONTAINMENT CLOSURE). UNPLANNED containment pressure rise indicates

CONTAINMENT CLOSURE cannot be assured and the containment cannot be relied upon as a barrier to fission product release.

This EAL addresses concerns raised by Generic Letter 88-17, *Loss of Decay Heat Removal*; SECY 91-283, *Evaluation of Shutdown and Low Power Risk Issues*; NUREG-1449, *Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States*; and NUMARC 91-06, *Guidelines for Industry Actions to Assess Shutdown Management*.

Reference(s):

1. OP 2207, "Plant Cooldown"
2. OP 2209B, "RCS Inventory Tracking"
3. AOP 2572, "Loss of Shutdown Cooling"
4. AOP 2568A, "RCS Leak, Mode 4, 5, 6 and Defueled"
5. Calculation M2EP-04164R2, "MP2 Rad Monitor Response to Core Uncovery"
6. Millstone Unit 2 Radiation Monitor Manual
7. EOP 2540, "Functional Recovery"
8. NEI 99-01 CG1

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 2 – Loss of Emergency AC Power
Initiating Condition: Loss of **all** but one AC power source to emergency buses for 15 minutes or longer

EAL:

CU2.1 Unusual Event

AC power capability, Table C-3, to 4.16 kV emergency buses 24C and 24D reduced to a single power source for ≥ 15 min. (Note 1)

AND

Any additional single power source failure will result in loss of **all** AC power to SAFETY SYSTEMS

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table C-3 AC Power Sources

Offsite

- Unit 2 Reserve Station Service Transformer (RSST)
- Unit 2 Normal Station Service Transformer (NSST) back-fed via the Main Transformer (if already aligned)
- Unit 3 Normal Station Service Transformer (NSST) via Buses 34A/B to Unit 2 emergency bus 24E (if already aligned)
- Unit 3 Reserve Station Service Transformer (RSST) via Buses 34A/B to Unit 2 emergency bus 24E (if already aligned)

Onsite

- Diesel Generator 15G-12U
- Diesel Generator 15G-13U
- SBO Diesel Generator (if already aligned)

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling, DEF - Defueled

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;

- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

Table C-3 provides a list of offsite and onsite AC electrical power sources credited for this EAL. The AC power sources annotated "(if already aligned)" require more than 15 minutes to establish and therefore are only credited if the source was already aligned at the time of AC power loss.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC describes a significant degradation of offsite and onsite AC power sources such that any additional single failure would result in a loss of all AC power to SAFETY SYSTEMS. In this condition, the sole AC power source may be powering one, or more than one, train of safety-related equipment.

When in the cold shutdown, refueling, or defueled mode, this condition is not classified as an Alert because of the increased time available to restore another power source to service. Additional time is available due to the reduced core decay heat load, and the lower temperatures and pressures in various plant systems. Thus, when in these modes, this condition is considered to be a potential degradation of the level of safety of the plant.

An "AC power source" is a source recognized in AOPs and EOPs, and capable of supplying required power to an emergency bus. Some examples of this condition are presented below.

- A loss of all offsite power with a concurrent failure of all but one emergency power source (e.g., an onsite diesel generator).
- A loss of all offsite power and loss of all emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being back-fed from Unit 3.
- A loss of emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being fed or back-fed from an offsite power source.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of power.

The subsequent loss of the remaining single power source would escalate the event to an Alert in accordance with IC CA2.

This cold condition EAL is equivalent to the hot condition EAL MA1.1.

Under normal conditions, four 345 kV lines will be in service to connect the Millstone Station to the main electric system. The emergency buses powering equipment for safe shutdown are 4.16 kV emergency buses 24C (A3), 24D (A4), and 24E (A5). 4.16 kV emergency bus 24E (A5) may be fed from either 24C or 24D. The associated 480 V emergency buses are 22E (fed from 24C) and 22F (fed from 24D). MPS2 has the following methods available to provide power to the 4.16 kV emergency buses (ref. 1, 2, 3):

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 24A (A1) and 24B (A2). Bus ties connect to 4.16 kV emergency

buses 24C (A3) and 24D (A4). Additionally, the NSST may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer by removing links for the main generator and providing power through the 15G-8T-2 or 15G-9T-2 breakers.

- The Unit 2 Reserve Station Service Transformer (RSST). This is the connection to the utility system for the preferred power supply. During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the Unit 2 RSST directly to 4.16 kV emergency buses 24C (A3) and 24D (A4).
- Diesel Generator 15G-12U(H7A) supplying 4.16 kV emergency bus 24C(A3).
- Diesel Generator 15G-13U(H7B) supplying 4.16 kV emergency bus 24D(A4).
- Unit 3 supplying power through bus 34A/34B to emergency bus 24E. This is controlled by procedure. However, there are power limitations when using this method to supply MPS2 buses when Millstone 3 bus 34A/34B is being used.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "Electrical Power Systems Shutdown"
2. UFSAR Section 8.2, "4160-Volt and 6900-Volt Systems"
3. UFSAR Section 8.3, "Emergency Generators"
4. AOP 2580, "Degraded Voltage"
5. AOP 2583, "Loss of All AC Power During Shutdown Conditions"
6. Dwg. No. 25203-30004, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Buses 24A & 24B (A1 & A2)"
7. Dwg. No. 25203-30005, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Emergency Buses 24C, 24D (A3, A4)"
8. NEI 99-01 CU2

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 2 – Loss of Emergency AC Power
Initiating Condition: Loss of **all** offsite and **all** onsite AC power to emergency buses for 15 minutes or longer

EAL:

CA2.1 Alert

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 24C and 24D for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling, DEF - Defueled

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators, provided it can be aligned within the 15 minute classification criteria.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC addresses a total loss of AC power that compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink.

When in the cold shutdown, refueling, or defueled mode, this condition is not classified as a Site Area Emergency because of the increased time available to restore an emergency bus to service. Additional time is available due to the reduced core decay heat load, and the lower temperatures and pressures in various plant systems. Thus, when in these modes, this condition represents an actual or potential substantial degradation of the level of safety of the plant.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs CS1 or RS1.

This cold condition EAL is equivalent to the hot condition EAL MS1.1.

Under normal conditions, four 345 kV lines will be in service to connect the Millstone Station to the main electric system. The emergency buses powering equipment for safe shutdown are 4.16 kV emergency buses 24C (A3), 24D (A4), and 24E (A5). 4.16 kV emergency bus 24E (A5) may be fed from either 24C or 24D. The associated 480 V emergency buses are 22E (fed from 24C) and 22F (fed from 24D). MPS2 has the following methods available to provide power to the 4.16 kV emergency buses (ref. 1, 2, 3):

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 24A (A1) and 24B (A2). Bus ties connect to 4.16 kV emergency buses 24C (A3) and 24D (A4). Additionally, the NSST may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer by removing links for the main generator and providing power through the 15G-8T-2 or 15G-9T-2 breakers.
- The Unit 2 Reserve Station Service Transformer (RSST). This is the connection to the utility system for the preferred power supply. During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the Unit 2 RSST directly to 4.16 kV emergency buses 24C (A3) and 24D (A4).
- Diesel Generator 15G-12U(H7A) supplying 4.16 kV emergency bus 24C(A3).
- Diesel Generator 15G-13U(H7B) supplying 4.16 kV emergency bus 24D(A4).
- Unit 3 supplying power through bus 34A/34B to emergency bus 24E. This is controlled by procedure. However, there are power limitations when using this method to supply MPS2 buses when Millstone 3 bus 34A/34B is being used.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "Electrical Power Systems Shutdown"
2. UFSAR Section 8.2, "4160-Volt and 6900-Volt Systems"
3. UFSAR Section 8.3, "Emergency Generators"
4. AOP 2580, "Degraded Voltage"
5. AOP 2583, "Loss of All AC Power During Shutdown Conditions"
6. Dwg. No. 25203-30004, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Buses 24A & 24B (A1 & A2)"
7. Dwg. No. 25203-30005, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Emergency Buses 24C, 24D (A3, A4)"
8. NEI 99-01 CU2

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 3 – RCS Temperature

Initiating Condition: UNPLANNED increase in RCS temperature

EAL:

CU3.1 Unusual Event

UNPLANNED rise in RCS temperature to > 200°F

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The actions taken to secure containment (Primary or Secondary) and their associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment door is closed and held in place by a minimum of four bolts or the outage equipment door is installed.
- A minimum of one door in each airlock is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere is closed by an isolation valve, blind flange, manual valve, or special device.

UNPLANNED-. A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

In the absence of reliable RCS temperature indication caused by the loss of decay heat removal capability, classification should be based on time to boil data when in Mode 6 or the RCS is not intact in Mode 5 (ref. 1). If the RCS is intact, classification should be based on the RCS pressure rise criteria of CA3.1. RCS time to 200°F is provided on the Shutdown Safety Assessment Sheet (ref. 2).

This EAL addresses an UNPLANNED increase in RCS temperature above the Technical Specification cold shutdown temperature limit and represents a potential degradation of the level of safety of the plant (ref. 1). If the RCS is not intact and CONTAINMENT CLOSURE is not established during this event, the DSEO/ADTS should also refer to EAL CA3.1.

A momentary UNPLANNED excursion above the Technical Specification cold shutdown temperature limit when the heat removal function is available does not warrant a classification.

This EAL involves a loss of decay heat removal capability, or an addition of heat to the RCS in excess of that which can currently be removed, such that reactor coolant temperature cannot be maintained below the cold shutdown temperature limit specified in Technical Specifications. During this condition, there is no immediate threat of fuel damage because the core decay heat load has been reduced since the cessation of power operation.

During an outage, the level in the reactor vessel will normally be maintained at or above the reactor vessel flange. Refueling evolutions that lower water level below the reactor vessel flange are carefully planned and controlled. A loss of forced decay heat removal at reduced inventory may result in a rapid increase in reactor coolant temperature depending on the time after shutdown (ref. 3).

Escalation to Alert would be via IC CA1 based on an inventory loss or IC CA3 based on exceeding plant configuration-specific time criteria.

Reference(s):

1. Technical Specifications Table 1.1-1
2. AOP 2572, "Loss of Shutdown Cooling"
3. OP 2301E, "Draining the RCS (ICCE)"
4. NEI 99-01 CU3

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 3 – RCS Temperature

Initiating Condition: UNPLANNED rise in RCS temperature

EAL:

CU3.2 Unusual Event

Loss of all RCS temperature and RCS water level indication for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6- Refueling

Definition(s):

CONTAINMENT CLOSURE - The actions taken to secure containment (Primary or Secondary) and their associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment door is closed and held in place by a minimum of four bolts or the outage equipment door is installed.
- A minimum of one door in each airlock is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere is closed by an isolation valve, blind flange, manual valve, or special device.

Basis:

This EAL addresses the inability to determine RCS temperature and level, and represents a potential degradation of the level of safety of the plant. If the RCS is not intact and CONTAINMENT CLOSURE is not established during this event, the DSEO/ADTS should also refer to EAL CA3.1.

This EAL reflects a condition where there has been a significant loss of instrumentation capability necessary to monitor RCS conditions and operators would be unable to monitor key parameters necessary to assure core decay heat removal. During this condition, there is no immediate threat of fuel damage because the core decay heat load has been reduced since the cessation of power operation.

RCS temperature indications include (ref. 1):

- RCS loop RTDs
- Core inlet and outlet SDC to RCS T351Y & T351X (require SDC flow)
- CETs (when operable)
- UJTEM8-A/B
- UJTEM7-A/B

RCS level indications include (ref. 1):

- L-112 RCS mid-loop wide range RCS level transmitter
- L-122 No. 2 hot leg narrow range level transmitter
- LI-112 No. 1 hot leg RCS mid-loop level indicator (CCTV)
- ICC Reactor Vessel Level Monitoring System (RVLMS)

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation to Alert would be via IC CA1 based on an inventory loss or IC CA3 based on exceeding plant configuration-specific time criteria.

Reference(s):

1. AOP 2572, "Loss of Shutdown Cooling"
2. NEI 99-01 CU3

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 3 – RCS Temperature

Initiating Condition: Inability to maintain plant in cold shutdown

EAL:

CA3.1 Alert

UNPLANNED rise in RCS temperature to > 200°F for > Table C-4 duration

(Notes 1, 13)

OR

UNPLANNED RCS pressure rise > 10 psig

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded.

Note 13: If an RCS heat removal system is in operation within the applicable Table C-4 heat-up duration and RCS temperature is being reduced, the EAL is **not** applicable.

Table C-4 RCS Heat-up Duration Thresholds		
RCS Status	CONTAINMENT CLOSURE Status	Heat-up Duration
Intact <u>AND</u> not reduced inventory		60 min.
Not intact <u>OR</u> reduced inventory	Established	20 min.
	Not established	0 min.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The actions taken to secure containment (Primary or Secondary) and their associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment door is closed and held in place by a minimum of four bolts or the outage equipment door is installed.
- A minimum of one door in each airlock is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere is closed by an isolation valve, blind flange, manual valve, or special device.

UNPLANNED- A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

In the absence of reliable RCS temperature indication caused by the loss of decay heat removal capability, classification should be based on the RCS pressure rise criteria when the RCS is intact in Mode 5 or based on time to boil data when in Mode 6 or the RCS is not intact in Mode 5 (ref. 2, 3). RCS time to 200°F is provided on the Shutdown Safety Assessment Sheet (ref. 2).

The RCS is considered to be at reduced inventory when RCS level is less than 43 in. above RCS hot leg centerline (greater than 3 ft. below the reactor vessel flange) (ref. 3).

This EAL addresses conditions involving a loss of decay heat removal capability or an addition of heat to the RCS in excess of that which can currently be removed. Either condition represents an actual or potential substantial degradation of the level of safety of the plant.

A momentary UNPLANNED excursion above the Technical Specification cold shutdown temperature limit when the heat removal function is available does not warrant a classification.

The RCS Heat-up Duration Thresholds table addresses an increase in RCS temperature when CONTAINMENT CLOSURE is established but the RCS is not intact, or RCS inventory is reduced (e.g., mid-loop operation in PWRs). The 20-minute criterion was included to allow time for operator action to address the temperature increase.

The RCS Heat-up Duration Thresholds table also addresses an increase in RCS temperature with the RCS intact. The status of CONTAINMENT CLOSURE is not crucial in this condition since the intact RCS is providing a high pressure barrier to a fission product release. The 60-minute time frame should allow sufficient time to address the temperature increase without a substantial degradation in plant safety.

Finally, in the case where there is an increase in RCS temperature, the RCS is not intact or is at reduced inventory, and CONTAINMENT CLOSURE is not established, no heat-up duration is allowed (i.e., 0 minutes). This is because 1) the evaporated reactor coolant may be released directly into the containment atmosphere and subsequently to the environment, and 2) there is reduced reactor coolant inventory above the top of irradiated fuel.

The RCS pressure rise threshold provides a pressure-based indication of RCS heat-up in the absence of RCS temperature monitoring capability. RCS pressure indicators PI-103 and PI-103-1 are capable of reading a 10 psi RCS pressure rise.

Escalation of the emergency classification level would be via IC CS1 or RS1.

Reference(s):

1. Technical Specifications Table 1.1-1
2. AOP 2572, "Loss of Shutdown Cooling"
3. OP 2301E, "Draining the RCS (ICCE)"
4. NEI 99-01 CA3

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 4 – Loss of Vital DC Power

Initiating Condition: Loss of vital DC power for 15 minutes or longer

EAL:

CU4.1 Unusual Event

Indicated voltage is < 105 VDC on **required** vital 125 VDC buses 201A OR 201B for ≥15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis

The vital (Class 1E) 125 V DC power system consists of two physically and electrically separated redundant buses - Bus 201A and Bus 201B. Each bus has a 60 cell battery bank with an eight-hour rating of 2300 amp-hours, with an operating range of 105 Volts to 140 Volts. In the event that either battery system is out of service, it is possible through interlocked circuit breakers to feed both battery buses from one battery. Each of the vital batteries supplies control logic, field flashing and breaker control for one diesel generator. They also supply power to a backup DC motor driven starting air compressor for each diesel generator, emergency lighting, turbine driven auxiliary feedwater pump (Terry Turbine) controls (bus 201B), and 125 VDC/120 VAC inverters for vital instrumentation.

The four vital 120 VAC instrumentation panels power reactor protection, engineered safety features and vital instrumentation which is normally supplied by four physically isolated and electrically independent inverters, two of which are supplied by each of the two redundant batteries 201A and 201B (ref. 1). Each of the four vital instrumentation panels has an alternate power supply via a "zero break" static transfer switch. Vital 120 VAC instrumentation channels 1 and 2 can be fed from separate DC/AC inverters whose power source is the turbine battery (i.e. 201D). Vital 120 VAC instrumentation channels 3 and 4 can be fed from one of the two regulated AC instrument power panels.

This IC addresses a loss of vital DC power which compromises the ability to monitor and control operable SAFETY SYSTEMS when the plant is in the cold shutdown or refueling mode. In these modes, the core decay heat load has been significantly reduced, and coolant system temperatures and pressures are lower; these conditions increase the time available to restore a vital DC bus to service. Thus, this condition is considered to be a potential degradation of the level of safety of the plant.

As used in this EAL, "required" means the vital DC buses necessary to support operation of the in-service, or operable, train or trains of SAFETY SYSTEM equipment. For example, if Train A is out-of-service (inoperable) for scheduled outage maintenance work and Train B is in-service (operable), then a loss of vital DC power affecting Train B would require the declaration of an Unusual Event. A loss of vital DC power to Train A would not warrant an emergency classification.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Depending upon the event, escalation of the emergency classification level would be via IC CA1 or CA3, or an IC in Category R.

This cold condition EAL is equivalent to the hot condition EAL MS2.1.

Reference(s):

1. UFSAR Section 8.5, "Battery System"
2. NEI 99-01 CU4

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 5 – Loss of Communications
Initiating Condition: Loss of all onsite or offsite communications capabilities
EAL:

CU5.1 Unusual Event

Loss of all Table C-5 onsite communication methods

OR

Loss of all Table C-5 State and local agency communication methods

OR

Loss of all Table C-5 NRC communication methods

Table C-5 Communication Methods			
System	Onsite	State/ Local	NRC
ENRS / ARCOS		X	
Station Radio System	X	X	
Plant Phone System	X	X	
Public Address System	X		
Gaitronics / Maintenance Jacks	X		
Federal Telephone System (ENS)			X
Commercial Telephone System		X	X
Satellite Phones		X	X
Dedicated Hotlines		X	

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling, DEF – Defueled

Definition(s):

None

Basis:

This IC addresses a significant loss of on-site or offsite communications capabilities. While not a direct challenge to plant or personnel safety, this event warrants prompt notifications to State and local agencies and the NRC.

This IC should be assessed only when extraordinary means are being utilized to make communications possible (e.g., use of non-plant, privately owned equipment, relaying of on-site information via individuals or multiple radio transmission points, individuals being sent to offsite locations, etc.).

The first EAL condition addresses a total loss of the communications methods used in support of routine plant operations.

The second EAL condition addresses a total loss of the communications methods used to notify all State and local agencies of an emergency declaration. The State and local agencies referred to here are the State of Connecticut and affected local communities.

The third EAL condition addresses a total loss of the communications methods used to notify the NRC of an emergency declaration.

This cold condition EAL is equivalent to the hot condition EAL MU7.1.

Reference(s):

1. MPS Emergency Plan Section 7.9, "Communication Systems"
2. MP-26-EPI-FAP07, "Notifications and Communications"
3. NEI 99-01 CU5

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 6 – Hazardous Event Affecting Safety Systems
Initiating Condition: Hazardous event affecting SAFETY SYSTEMS needed for the current operating mode

EAL:

CA6.1 Alert

The occurrence of **any** Table C-6 hazardous event

AND

Event damage has caused indications of degraded performance on one train of a SAFETY SYSTEM needed for the current operating mode

AND EITHER:

- Event damage has caused indications of degraded performance to the second train of the SAFETY SYSTEM needed for the current operating mode
- Event damage has resulted in **VISIBLE DAMAGE** to the second train of the SAFETY SYSTEM needed for the current operating mode

(Notes 9, 10)

Note 9: If the affected SAFETY SYSTEM train was already inoperable or out of service before the hazardous event occurred, then emergency classification is **not** warranted.

Note 10: If the hazardous event **only** resulted in **VISIBLE DAMAGE**, with **no** indications of degraded performance to at least one train of a SAFETY SYSTEM, then this emergency classification is **not** warranted.

Table C-6 Hazardous Events

- Seismic event (earthquake)
- Internal or external FLOODING event
- High winds or tornado strike
- FIRE
- EXPLOSION
- Other events with similar hazard characteristics as determined by the DSEO/ADTS

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

EXPLOSION - A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding, arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

VISIBLE DAMAGE - Damage to a SAFETY SYSTEM train that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected SAFETY SYSTEM train.

Basis:

This IC addresses a hazardous event that causes damage to SAFETY SYSTEMS needed for the current operating mode. In order to provide the appropriate context for consideration of an ALERT classification, the hazardous event must have caused indications of degraded SAFETY SYSTEM performance in one train, and there must be either indications of performance issues with the second SAFETY SYSTEM train or VISIBLE DAMAGE to the second train such that the potential exists for this second SAFETY SYSTEM train to have performance issues. In other words, in order for this EAL to be classified, the hazardous event must occur, at least one SAFETY SYSTEM train must have indications of degraded performance, and the second SAFETY SYSTEM train must have indications of degraded performance or VISIBLE DAMAGE such that the potential exists for performance issues. Note that this second SAFETY SYSTEM train is from the same SAFETY SYSTEM that has indications of degraded performance; commercial nuclear power plants are designed to be able to support single system issues without compromising public health and safety from radiological events.

Indications of degraded performance addresses damage to a SAFETY SYSTEM train that is in service/operation since indications for it will be readily available. The indications of degraded performance should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

VISIBLE DAMAGE addresses damage to a SAFETY SYSTEM train that is not in service/operation and that potentially could cause performance issues. Operators will make this determination based on the totality of available event and damage report information. This is intended to be a brief assessment not requiring lengthy analysis or quantification of the damage. This VISIBLE DAMAGE should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

An event affecting equipment common to two or more trains of a safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the common equipment) should be classified as an Alert under this EAL, as appropriate to the plant mode. By affecting the functionality of multiple trains of a safety system, the loss of the common equipment effectively meets the two-train impact criteria that underlie the EALs and bases.

An event affecting a single-train safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the one train) would not be classified under this EAL because the two-train impact criteria that underlie the EALs and bases would not be met. If an event affects a single-train safety system, then the emergency classification should be made based on plant parameters/symptoms meeting the EALs for another IC. Depending upon the circumstances, classification may also occur based on Shift Manager/DSEO/ADTS judgement.

An event that affects two trains of a safety system (e.g., one train has indications of degraded performance and the other VISIBLE DAMAGE) that also has one or more additional trains should be classified as an Alert under this EAL, as appropriate to the plant mode. This approach maintains consistency with the two-train impact criteria that underlie the EALs and bases, and is warranted because the event was severe enough to affect the functionality of two trains of a safety system despite plant design criteria associated with system and system train separation and protection. Such an event may have caused other plant impacts that are not immediately apparent.

Escalation of the emergency classification level would be via IC CS1 or RS1.

This cold condition EAL is equivalent to the hot condition EAL MA8.1.

Reference(s):

1. AOP 2560, "Storms, High Winds and High Tides"
2. AOP 2562, "Earthquake"
3. EP FAQ 2016-002
4. NEI 99-01 CA6

Category E – Independent Spent Fuel Storage Installation (ISFSI)

EAL Group: ANY (EALs in this category are applicable to any plant condition, hot or cold.)

An independent spent fuel storage installation (ISFSI) is a complex that is designed and constructed for the interim storage of spent nuclear fuel and other radioactive materials associated with spent fuel storage. A significant amount of the radioactive material contained within a canister must escape its packaging and enter the biosphere for there to be a significant environmental effect resulting from an accident involving the dry storage of spent nuclear fuel.

A Notification of Unusual Event is declared on the basis of the occurrence of an event of sufficient magnitude that a loaded cask CONFINEMENT BOUNDARY is damaged or violated.

The MPS ISFSI is located wholly within the MPS plant PROTECTED AREA. Therefore any security event related to the ISFSI is classified under Category H1 security event related EALs.

Category: ISFSI

Subcategory: Confinement Boundary

Initiating Condition: Damage to a loaded cask CONFINEMENT BOUNDARY

EAL:

EU1.1 Unusual Event

Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by an on-contact radiation reading on the surface of a loaded spent fuel cask (HSM) > **any** of the following:

- 1,700 mrem/hr on the HSM front bird screen
- 400 mrem/hr on the outside HSM door
- 12 mrem/hr on the HSM end shield wall

Mode Applicability:

All

Definition(s):

CONFINEMENT BOUNDARY- The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielded Canister (DSC).

INDEPENDENT SPENT FUEL STORAGE INSTALLATION (ISFSI): A complex that is designed and constructed for the interim storage of spent nuclear fuel and other radioactive materials associated with spent fuel storage.

Basis:

This IC addresses an event that results in damage to the CONFINEMENT BOUNDARY of a storage cask containing spent fuel. It applies to irradiated fuel that is licensed for dry storage beginning at the point that the loaded storage cask is sealed. The issues of concern are the creation of a potential or actual release path to the environment, degradation of one or more fuel assemblies due to environmental factors, and configuration changes which could cause challenges in removing the cask or fuel from storage.

The existence of "damage" is determined by radiological survey. The specified EAL threshold values correspond to 2 times the NUHOMS Horizontal Storage Module (HSM) technical specification external surface dose rate limits (ref. 2). The technical specification multiple of "2 times", which is also used in Category R IC RU1, is used here to distinguish between non-emergency and emergency conditions. The emphasis for this classification is the degradation in the level of safety of the spent fuel cask and not the magnitude of the associated dose or dose rate. It is recognized that in the case of extreme damage to a loaded cask, the fact that the "on-contact" dose rate limit is exceeded may be determined based on measurement of a dose rate at some distance from the cask (ref. 1, 2.3).

Security-related events for ISFSIs are covered under Category H ICs.

Reference(s):

1. NUH-003, "UFSAR for the Horizontal Modular Storage System for Irradiated Fuel"
2. MPS2 ETE-NAF-2010-0004, "MPS Unit 2 ISFSI 10 CFR72.212 Evaluation Report Attachment 1 Section 5.4 HSM or HSM-H Dose Rate Evaluation Program"
3. RPM 2.5.9, "Dry Shielded Canisters (DSC) Surveys (ISFSI)"
4. NEI 99-01 E-HU1

Category F – Fission Product Barrier Degradation

EAL Group: Hot Conditions (RCS temperature > 200°F); EALs in this category are applicable only in one or more hot operating modes.

EALs in this category represent threats to the defense-in depth design concept that precludes the release of highly radioactive fission products to the environment. This concept relies on multiple physical barriers any one of which, if maintained intact, precludes the release of significant amounts of radioactive fission products to the environment. The primary fission product barriers are:

- A. Fuel Clad Barrier (FC): The Fuel Clad Barrier consists of the cladding material that contains the fuel pellets.
- B. Reactor Coolant System Barrier (RCS): The RCS Barrier includes the RCS primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary isolation valves.
- C. Containment Barrier (CTMT): The Containment Barrier includes the containment building and connections up to and including the outermost containment isolation valves. This barrier also includes the main steam, feedwater, and blowdown line extensions outside the containment building up to and including the outermost secondary side isolation valve. Containment Barrier thresholds are used as criteria for escalation of the Emergency Classification Level (ECL) from an Alert to a Site Area Emergency or a General Emergency.

The EALs in this category require evaluation of the loss and potential loss thresholds listed in the fission product barrier matrix of Table F-1. "Loss" and "Potential Loss" signify the relative damage and threat of damage to the barrier. "Loss" means the barrier no longer assures containment of radioactive materials. "Potential Loss" means integrity of the barrier is threatened and could be lost if conditions continue to degrade. The number of barriers that are lost or potentially lost and the following criteria determine the appropriate emergency classification level:

Alert:

Any loss or any potential loss of either Fuel Clad or RCS Barrier

Site Area Emergency:

Loss or potential loss of any two barriers

General Emergency:

Loss of any two barriers and loss or potential loss of third barrier

The logic used for emergency classification based on fission product barrier monitoring should reflect the following considerations:

- The Fuel Clad Barrier and the RCS Barrier are weighted more heavily than the Containment Barrier.
- Unusual Event ICs associated with RCS and Fuel Clad Barriers are addressed under System Malfunction ICs.

- For accident conditions involving a radiological release, evaluation of the fission product barrier thresholds will need to be performed in conjunction with dose assessments to ensure correct and timely escalation of the emergency classification. For example, an evaluation of the fission product barrier thresholds may result in a Site Area Emergency classification while a dose assessment may indicate that an EAL for General Emergency IC RG1 has been exceeded.
- The fission product barrier thresholds specified within a scheme reflect plant-specific MPS2 design and operating characteristics.
- As used in this category, the term RCS leakage encompasses not just those types defined in Technical Specifications but also includes the loss of RCS mass to any location— inside the containment, an interfacing system, or outside of the containment. The release of liquid or steam mass from the RCS due to the as-designed/expected operation of a relief valve is not considered to be RCS leakage.
- At the Site Area Emergency level, EAL users should maintain cognizance of how far present conditions are from meeting a threshold that would require a General Emergency declaration. For example, if the Fuel Clad and RCS fission product barriers were both lost, then there should be frequent assessments of containment radioactive inventory and integrity. Alternatively, if both the Fuel Clad and RCS fission product barriers were potentially lost, the DSEO/ADTS would have more assurance that there was no immediate need to escalate to a General Emergency.

Category: Fission Product Barrier Degradation

Subcategory: N/A

Initiating Condition: Any loss or any potential loss of either Fuel Clad or RCS

EAL:

FA1.1 Alert

Any loss or any potential loss of **EITHER** Fuel Clad or RCS barrier (Table F-1)

Mode Applicability:

1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown

Definition(s):

None

Basis:

Fuel Clad, RCS and Containment comprise the fission product barriers. Table F-1 lists the fission product barrier thresholds, bases and references.

At the Alert classification level, Fuel Clad and RCS barriers are weighted more heavily than the Containment barrier. Unlike the Containment barrier, loss or potential loss of either the Fuel Clad or RCS barrier may result in the relocation of radioactive materials or degradation of core cooling capability. Note that the loss or potential loss of Containment barrier in combination with loss or potential loss of either Fuel Clad or RCS barrier results in declaration of a Site Area Emergency under EAL FS1.1

Reference(s):

1. NEI 99-01 FA1

Category: Fission Product Barrier Degradation
Subcategory: N/A
Initiating Condition: Loss or potential loss of **any** two barriers
EAL:

FS1.1 Site Area Emergency

Loss or potential loss of any two barriers (Table F-1)

Mode Applicability:

1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

Fuel Clad, RCS and Containment comprise the fission product barriers. Table F-1 lists the fission product barrier thresholds, bases and references.

At the Site Area Emergency classification level, each barrier is weighted equally. A Site Area Emergency is therefore appropriate for any combination of the following conditions:

- One barrier loss and a second barrier loss (i.e., loss - loss)
- One barrier loss and a second barrier potential loss (i.e., loss - potential loss)
- One barrier potential loss and a second barrier potential loss (i.e., potential loss - potential loss)

At the Site Area Emergency classification level, the ability to dynamically assess the proximity of present conditions with respect to the threshold for a General Emergency is important. For example, the existence of Fuel Clad and RCS Barrier loss thresholds in addition to offsite dose assessments would require continual assessments of radioactive inventory and Containment integrity in anticipation of reaching a General Emergency classification. Alternatively, if both Fuel Clad and RCS potential loss thresholds existed, they would have greater assurance that escalation to a General Emergency is less IMMINENT.

Reference(s):

1. NEI 99-01 FS1

Category: Fission Product Barrier Degradation

Subcategory: N/A

Initiating Condition: Loss of **any** two barriers and loss or potential loss of the third barrier

EAL:

FG1.1 General Emergency

Loss of **any** two barriers

AND

Loss or potential loss of the third barrier (Table F-1)

Mode Applicability:

1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown

Definition(s):

None

Basis:

Fuel Clad, RCS and Containment comprise the fission product barriers. Table F-1 lists the fission product barrier thresholds, bases and references.

At the General Emergency classification level each barrier is weighted equally. A General Emergency is therefore appropriate for any combination of the following conditions:

- Loss of Fuel Clad, RCS and Containment Barriers
- Loss of Fuel Clad and RCS Barriers with potential loss of Containment Barrier
- Loss of RCS and Containment Barriers with potential loss of Fuel Clad Barrier
- Loss of Fuel Clad and Containment Barriers with potential loss of RCS Barrier

Reference(s):

1. NEI 99-01 FG1

Table F-1 Fission Product Barrier Threshold Matrix & Bases

Table F-1 lists the threshold conditions that define the Loss and Potential Loss of the three fission product barriers (Fuel Clad, Reactor Coolant System, and Containment). The table is structured so that each of the three barriers occupies adjacent columns. Each fission product barrier column is further divided into two columns; one for Loss thresholds and one for Potential Loss thresholds.

The first column of the table (to the left of the Fuel Clad Loss column) lists the categories (types) of fission product barrier thresholds. The fission product barrier categories are:

- A. RCS or SG Tube Leakage
- B. Inadequate Heat removal
- C. CTMT Radiation / RCS Activity
- D. CTMT Integrity or Bypass
- E. DSEO/ADTS Judgment

Each category occupies a row in Table F-1 thus forming a matrix defined by the categories. The intersection of each row with each Loss/Potential Loss column forms a cell in which one or more fission product barrier thresholds appear. If NEI 99-01 does not define a threshold for a barrier Loss/Potential Loss, the word "None" is entered in the cell.

Thresholds are assigned sequential numbers within each barrier column beginning with number one.

If a cell in Table F-1 contains more than one numbered threshold, each of the numbered thresholds, if exceeded, signifies a Loss or Potential Loss of the barrier. It is not necessary to exceed all of the thresholds in a category before declaring a barrier Loss/Potential Loss.

Subdivision of Table F-1 by category facilitates association of plant conditions to the applicable fission product barrier Loss and Potential Loss thresholds. This structure promotes a systematic approach to assessing the classification status of the fission product barriers.

When equipped with knowledge of plant conditions related to the fission product barriers, the EAL-user first scans down the category column of Table F-1, locates the likely category and then reads across the fission product barrier Loss and Potential Loss thresholds in that category to determine if a threshold has been exceeded. If a threshold has not been exceeded, the EAL-user proceeds to the next likely category and continues review of the thresholds in the new category.

If the EAL-user determines that any threshold has been exceeded, by definition, the barrier is lost or potentially lost – even if multiple thresholds in the same barrier column are exceeded, only that one barrier is lost or potentially lost. The EAL-user must examine each of the three fission product barriers to determine if other barrier thresholds in the category are lost or potentially lost. For example, if containment radiation is sufficiently high, a Loss of the Fuel Clad and RCS Barriers and a Potential Loss of the Containment Barrier can occur. Barrier Losses and Potential Losses are then applied to the algorithms given in EALs FG1.1, FS1.1, and FA1.1 to determine the appropriate emergency classification.

Table F-1 Fission Product Barrier Threshold Matrix

	Fuel Clad Barrier (FC)		Reactor Coolant System Barrier (RCS)		Containment Barrier (CTMT)	
Category	Loss	Potential Loss	Loss	Potential Loss	Loss	Potential Loss
A RCS or SG Tube Leakage	None	1. RVLMS reading 0% (#8) (Note 12)	1. An automatic or manual SIAS actuation required by <u>EITHER</u> : <ul style="list-style-type: none"> UNISOLABLE RCS leakage SG tube RUPTURE 	1. UNISOLABLE RCS or SG tube leakage > 50 gpm excluding normal reductions in RCS inventory (e.g. letdown, RCP seal leakage) 2. Uncontrolled RCS cooldown > 100°F/hr and RCS pressure and temperature are to the left of the 200°F Subcooling (PTS) Curve (EOP Figure 1)	1. A leaking or RUPTURED SG is FAULTED outside of CTMT	None
B Inadequate Heat Removal	1. CETs > 1200°F	2. CETs > 700°F 3. Applicable RCS and Core Heat Removal (HR) Safety Function Status Check acceptance criteria <u>not</u> met	None	3. Applicable RCS and Core Heat Removal (HR) Safety Function Status Check acceptance criteria <u>not</u> met	None	1. CETs > 1200°F <u>AND</u> Restoration procedures <u>not</u> effective within 15 min. (Note 1)
C CTMT Radiation / RCS Activity	2. CTMT high range radiation monitor RE-8240/8241 reading > Table F-2 column Fuel Clad Loss 3. Coolant activity > 300 µCi/gm dose equivalent I-131 4. Dose rate at 1 ft. from an unpressurized RCS sample ≥ Table F-3 5. Sample line dose rate ≥ Table F-4 6. Core damage estimate > 5% clad damage	None	2. CTMT high range radiation monitor RE-8240/8241 reading > Table F-2 column RCS Loss	None	None	2. CTMT high range radiation monitor RE-8240/8241 reading > Table F-2 column CTMT Potential Loss
D CTMT Integrity or Bypass	None	None	None	None	2. CTMT isolation is required <u>AND EITHER</u> : <ul style="list-style-type: none"> CTMT integrity has been lost based on DSEO/ADTS judgment UNISOLABLE pathway from CTMT atmosphere to the environment exists 3. Indications of RCS leakage outside of CTMT	3. CTMT pressure > 54 psig 4. CTMT hydrogen concentration > 4% 5. CTMT pressure > 10 psig with < one full train of containment heat removal systems (Note 11) operating per design for ≥ 15 min. (Note 1)
E DSEO/ADTS Judgment	7. <u>Any</u> condition in the opinion of the DSEO/ADTS that indicates loss of the fuel clad barrier	4. <u>Any</u> condition in the opinion of the DSEO/ADTS that indicates potential loss of the fuel clad barrier	3. <u>Any</u> condition in the opinion of the DSEO/ADTS that indicates loss of the RCS barrier	4. <u>Any</u> condition in the opinion of the DSEO/ADTS that indicates potential loss of the RCS barrier	4. <u>Any</u> condition in the opinion of the DSEO/ADTS that indicates loss of the CTMT barrier	6. <u>Any</u> condition in the opinion of the DSEO/ADTS that indicates potential loss of the CTMT barrier

Millstone Power Station Unit 2
Emergency Action Level Technical Bases Document
Attachment 1 Emergency Action Level Technical Bases

Serial No. 18-364
Docket Nos. 50-336; 72-47
Enclosure 3; Attachment 3B

Barrier: Fuel Clad

Category: A. RCS or SG Tube Leakage

Degradation Threat: Loss

Threshold:

None

Barrier: Fuel Clad

Category: A. RCS or SG Tube Leakage

Degradation Threat: Potential Loss

Threshold:

1. RVLMS reading 0% (#8) (Note 12)

Note 12: This EAL is only applicable if a RVLMS channel #8 string is operable.

Definition(s):

None

Basis:

This reading indicates a reduction in reactor vessel water level sufficient to allow the onset of heat-induced cladding damage.

Millstone 2 includes in its Inadequate Core Cooling instrumentation a reactor vessel level monitoring system (RVLMS) that is displayed to the operators and can measure discrete reactor vessel water levels from the top of the reactor vessel head to the top of the core plate (string #8 - approximately 10 inches above the top of the active fuel). The bottom of this instrument's span in the reactor vessel plenum is the lowest available reactor vessel level indicator and is used in this EAL to represent approximately the top of active fuel (ref. 1).

Reference(s):

1. Millstone 2 UFSAR Section 7.5.4.4, "In-Core Instrumentation"
2. NEI 99-01 RCS or SG Tube Leakage Potential Loss 1.A

Barrier: Fuel Clad

Category: B. Inadequate Heat Removal

Degradation Threat: Loss

Threshold:

- | |
|------------------|
| 1. CETs > 1200°F |
|------------------|

Definition(s):

None

Basis:

This reading indicates temperatures within the core are sufficient to cause significant superheating of reactor coolant.

The SAMGs use a CET temperature of 1200 °F as an entry condition and the temperature indicative of a badly damaged core (ref. 1, 2).

Reference(s):

1. SAMG 4211, "Phase 1 Initial Diagnosis"
2. MP-26-EPI-FAP11-001, "Core Damage Assessment: Core Exit Temperatures"
3. NEI 99-01 Inadequate Heat Removal Loss 2.A

Barrier: Fuel Clad

Category: B. Inadequate Heat Removal

Degradation Threat: Potential Loss

Threshold:

2. CETs > 700°F

Definition(s):

None

Basis:

This reading indicates a reduction in reactor vessel water level sufficient to allow the onset of heat-induced cladding damage.

The potential loss threshold is based on not meeting the acceptance criteria for Core Exit Thermocouple readings less than 700 °F for ECCS Heat Removal. 700 °F is the threshold for the threshold indicating a loss of subcooling and further temperature increase will potentially damage fuel cladding (ref. 1, 2, 3).

Reference(s):

1. EOP 2540, "Functional Recovery"
2. EOP 2540D, "Functional Recovery of Heat Removal"
3. MP-26-EPI-FAP11-001, "Core Damage Assessment: Core Exit Temperatures"
4. NEI 99-01 Inadequate Heat Removal Loss 2.A

Barrier: Fuel Clad

Category: B. Inadequate Heat Removal

Degradation Threat: Potential Loss

Threshold:

3. Applicable RCS and Core Heat Removal (HR) Safety Function Status Check acceptance criteria **not** met

Definition(s):

None

Basis:

There are three defined and potentially applicable RCS and Core Heat Removal (HR) Safety Function Status Check acceptance criteria conditions (ref. 1, 2):

- HR-1 – SG Heat Sink With No SI Operating
- HR-2 – SG Heat Sink With SI Operating
- HR-3 – Once-Through-Cooling

Failure to meet the applicable HR acceptance criteria indicates an extreme challenge to the ability to remove RCS heat using the steam generators (i.e., loss of an effective secondary-side heat sink). This condition represents a potential loss of the Fuel Clad Barrier. In accordance with EOPs, there may be unusual accident conditions during which operators intentionally reduce the heat removal capability of the steam generators; during these conditions, classification using threshold is not warranted.

Meeting this threshold results in a Site Area Emergency because this threshold is identical to RCS Barrier Potential Loss threshold B.3; both will be met. This condition warrants a Site Area Emergency declaration because inadequate RCS heat removal may result in fuel heat-up sufficient to damage the cladding and increase RCS pressure to the point where mass will be lost from the system.

Reference(s):

1. EOP 2540, "Functional Recovery"
2. EOP 2540D, "Functional Recovery of Heat Removal"
3. NEI 99-01 Inadequate Heat Removal Fuel Clad Potential Loss 2.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

2. CTMT high range radiation monitor RE-8240/8241 reading > Table F-2 column Fuel Clad Loss

Table F-2 CTMT High Range Radiation Monitor Barrier Thresholds RE-8240/8241			
Time > Shutdown (hrs)	Fuel Clad Loss (R/hr)	RCS Loss (R/hr)	CTMT Potential Loss (R/hr)
≤ 2	65	5	260
$> 2 - \leq 4$	45	5	180
$> 4 - \leq 8$	25	5	100
$> 8 - \leq 14$	10	5	40
> 14	6	5	24

Definition(s):

None

Basis:

Containment radiation monitor readings greater than the Table F-2 Fuel Clad Loss column threshold indicate the release of reactor coolant, with elevated activity indicative of fuel damage, into the containment. The reading is derived assuming the instantaneous release and dispersal of the reactor coolant noble gas and iodine inventory associated with a concentration of 5% clad failure into the containment atmosphere (readings have been rounded for readability). Reactor coolant concentrations of this magnitude are several times larger than the maximum concentrations (including iodine spiking) allowed within Technical Specifications and are therefore indicative of fuel damage (approximately 5 % clad failure depending on core inventory and RCS volume)(ref. 1, 2).

Time after shutdown values are provided to account for radioactive decay.

The values specified in Table F-2 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Time periods were chosen to fit monitor response (fast changes in response early following reactor shutdown are broken up into smaller time periods to better approximate expected change). Values were chosen within each time period to minimize error (<50%) to the highest and lowest response within the range.

The radiation monitor reading in this threshold is higher than that specified for RCS Barrier Loss threshold C.2 since it indicates a loss of both the Fuel Clad barrier and the RCS barrier. Note that a combination of the two monitor readings appropriately escalates the ECL to a Site Area Emergency.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Damage Computer Program
2. Calculation RA-0074," Millstone Unit 2 Expected Containment High Range Radiation Monitor Response to a LOCA Based on Fuel Gap Fractions Defined in NUREG 1228"
3. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.A

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

3. Coolant activity > 300 $\mu\text{Ci/gm}$ dose equivalent I-131

Definition(s):

None

Basis:

This threshold indicates that RCS radioactivity concentration is greater than 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

It is recognized that sample collection and analysis of reactor coolant with highly elevated activity levels could require several hours to complete. Nonetheless, a sample-related threshold is included as a backup to other indications.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

4. Dose rate at 1 ft. from an unpressurized RCS sample \geq Table F-3

Table F-3 FC Loss Coolant Activity Dose Rates	
Time > Shutdown (hrs)	mR/hr/ml
≤ 2	15
$> 2 - \leq 8$	8
> 8	3

Definition(s):

None

Basis:

This threshold indicates that RCS radioactivity concentration is greater than 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

It is recognized that sample collection and analysis of reactor coolant with highly elevated activity levels could require several hours to complete. Nonetheless, a sample-related threshold is included as a backup to other indications. This EAL provides the ability to take a dose rate off of an RCS sample to determine fuel clad barrier loss, without the need to analyze the sample before making this determination. This EAL saves significant time by allowing evaluation of contained radioactivity within the RCS by a direct dose rate measurement.

Per Engineering Calculation RA-0059, dose rate is assumed to result from radioactive iodines (I-131 thru I-135) in RCS in concentrations corresponding to the loss of 5% of gap radioactivity of the core. For 5% loss of gap radioactivity ($\sim 300 \mu\text{Ci/gm}$ dose equivalent I-131), 2% of the core inventory of radioactive iodines are assumed to be contained in the gap. The values contained in Table F-3 (FC Loss Coolant Activity Dose Rates) represent expected one foot dose rates per ml of sample based on time since reactor shutdown to the time when the sample is taken. Time after shutdown values are provided to account for radioactive decay. The expected dose rate is a near linear relationship with the volume of the sample, so any volume collected can be determined by dividing the measured dose rate by the sample volume and comparing to the threshold value from Table F-3 for the applicable time frame. These dose rates assume no ECCS injection so there is no dilution credited which would vary coolant volume. Values in the table have been rounded for ease of use. The > 8 hour threshold is

conservative up to 24 hours following reactor shutdown. After 24 hours, the expected response from radioactive iodine levels off. Therefore, the value shown for > 8 hours applies for all samples taken 8 hours or more since reactor shutdown (ref. 1, 2).

The values specified in Table F-3 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Values were chosen to minimize error from the highest to lowest dose rate within each range.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. MP-26-EPI-FAP11, "Core Damage Assessment"
2. RA-0059, "Detector Response to an RCS Sample for EAL Classification of Fuel Clad Degradation and Barrier Loss"
3. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

5. Sample line dose rate \geq Table F-4

Table F-4 FC Loss RCS Sample Line Dose Rates	
Time > Shutdown (hrs)	R/hr
≤ 2	4
$> 2 - \leq 8$	2
> 8	1

Definition(s):

None

Basis:

This threshold indicates that RCS radioactivity concentration is greater than 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

Per Engineering Calculation RA-0079, dose rate is assumed to result from radioactive iodines in the RCS in concentrations corresponding to the loss of 5% of gap radioactivity of the core. The values contained in Table F-4 (FC Loss RCS Sample Line Dose Rates) represent fuel clad failure thresholds when measured approximately 2" from the outside of the RCS hot leg sample line. RCS sample line locations have been predetermined for use with this EAL. Other RCS lines could be used if analyzed on a case-by-case basis. Values in the table have been rounded for ease of use. The sample line dose rates have been calculated for various time ranges after shutdown to account for radioactive decay (ref. 1).

The values specified in Table F-4 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Values were chosen to minimize error from the highest to lowest dose rate within each range.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Engineering Calculation RA-0079
2. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.B

Barrier: Fuel Clad
Category: C. CTMT Radiation / RCS Activity
Degradation Threat: Loss
Threshold:

6. Core damage estimate > 5% clad damage
--

Definition(s):

None

Basis:

This threshold can be determined through use of various plant parameters that indicate 5% or greater fuel clad damage. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

Technical support personnel can use MP-26-PI-FAP11 "Core Damage Assessment" to estimate clad damage (ref. 1). Provisions are provided to estimate fuel clad damage for MPS2 using core damage assessments in the event that RCS sampling options are not available due to design limitations with the RCS sample cooling system.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. MP-26-EPI-FAP11, "Core Damage Assessment"
2. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.B

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Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Potential Loss

Threshold:

None

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Barrier: Fuel Clad

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

None

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Barrier: Fuel Clad

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Fuel Clad

Category: E. DSEO/ADTS Judgment

Degradation Threat: Loss

Threshold:

7. **Any** condition in the opinion of the DSEO/ADTS that indicates loss of the Fuel Clad barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that are to be used by the DSEO/ADTS in determining whether the Fuel Clad barrier is lost.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Fuel Clad Loss 6.A

Barrier: Fuel Clad

Category: F. DSEO/ADTS Judgment

Degradation Threat: Potential Loss

Threshold:

4. **Any** condition in the opinion of the DSEO/ADTS that indicates potential loss of the Fuel Clad barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that are to be used by the DSEO/ADTS in determining whether the Fuel Clad barrier is potentially lost. The DSEO/ADTS should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Potential Fuel Clad Loss 6.A

Barrier: Reactor Coolant System

Category: A. RCS or S/G Tube Leakage

Degradation Threat: Loss

Threshold:

1. An automatic or manual SIAS actuation required by **EITHER**:

- UNISOLABLE RCS leakage
- SG tube RUPTURE

Definition(s):

UNISOLABLE - An open or breached system line that cannot be isolated, remotely or locally.

RUPTURE - The condition of a steam generator in which primary-to-secondary leakage is of sufficient magnitude to require a safety injection.

Basis:

This threshold is based on an UNISOLABLE RCS leak of sufficient size to require an automatic or manual actuation of the Emergency Core Cooling System (ECCS). This condition clearly represents a loss of the RCS Barrier.

A SIAS actuation resulting from a non-RCS leak event (ex.: FAULTED SG) does not meet the intent of this threshold.

This threshold is applicable to unidentified and pressure boundary leakage, as well as identified leakage. It is also applicable to UNISOLABLE RCS leakage through an interfacing system. The mass loss may be into any location – inside containment, to the secondary-side (i.e., steam generator tube leakage) or outside of containment.

A steam generator with primary-to-secondary leakage of sufficient magnitude to require a safety injection is considered to be RUPTURED. If a RUPTURED steam generator is also FAULTED outside of containment, the declaration escalates to a Site Area Emergency since the Containment Barrier Loss threshold A.1 will also be met.

If EOPs direct operators to open the Pressurizer pressure relief valves to implement a core cooling strategy (i.e., a “feed and bleed” cooldown), then there will exist a reactor coolant flow path from the RCS, past the “pressurizer safety and relief valves” and into the containment that operators cannot isolate without compromising the effectiveness of the strategy (i.e., for the strategy to be effective, the valves must be kept in the open position); therefore, the flow through the pressure relief line is UNISOLABLE. In this case, the ability of the RCS pressure boundary to serve as an effective barrier to a release of fission products has been eliminated and thus this condition constitutes a loss of the RCS barrier.

Reference(s):

1. EOP 2532, “Loss of Coolant Accident”
2. NEI 99-01 RCS or SG Tube Leakage Reactor Coolant System Loss 1.A

Barrier: Reactor Coolant System

Category: A. RCS or S/G Tube Leakage

Degradation Threat: Potential Loss

Threshold:

1. UNISOLABLE RCS or SG tube leakage > 50 gpm excluding normal reductions in RCS inventory (e.g. letdown, RCP seal leakage)

Definition(s):

UNISOLABLE - An open or breached system line that cannot be isolated, remotely or locally.

Basis:

This threshold is based on an UNISOLABLE RCS leak that results in the inability to maintain pressurizer level within specified limits by operation of a normally used charging (makeup) pump, but a SIAS actuation has not occurred. The threshold is met when RCS leakage, excluding normal reductions in RCS inventory (e.g. letdown, RCP seal leakage), exceeds 50 gpm (ref. 1, 2).

When assessing RCS leaks, the classification should be based on actual leakage vice the impact of normal plant response to Operator directed actions. Example: A rapid down power may have the impact of increasing charging flow. The increase in charging should not be considered increased leakage.

This threshold is applicable to unidentified and pressure boundary leakage, as well as identified leakage. It is also applicable to UNISOLABLE RCS leakage through an interfacing system. The mass loss may be into any location – inside containment, to the secondary-side (i.e., steam generator tube leakage) or outside of containment.

If the leaking steam generator is also FAULTED outside of containment, the declaration escalates to a Site Area Emergency since the Containment Barrier Loss threshold A.1 will also be met.

Reference(s):

1. AOP 2568, "Reactor Coolant System Leak"
2. AOP 2569, "Steam Generator Tube Leak"
3. NEI 99-01 RCS or SG Tube Leakage Reactor Coolant System Potential Loss 1.A

Barrier: Reactor Coolant System

Category: A. RCS or S/G Tube Leakage

Degradation Threat: Potential Loss

Threshold:

2. Uncontrolled RCS cooldown > 100°F/hr and RCS pressure and temperature are to the left of the 200°F Subcooling (PTS) Curve (EOP Figure 1)

Definition(s):

None

Basis:

This condition indicates an extreme challenge to the integrity of the RCS pressure boundary due to pressurized thermal shock – a transient that causes rapid RCS cooldown while the RCS is in Mode 3 or higher (i.e., hot and pressurized).

The maximum allowed RCS cooldown rate per Technical Specifications is 100°F/hr with cold leg temperature > 220°F (ref. 1).

A RCS cooldown greater than 100°F/hr below 500°F is the temperature that requires implementation of Pressurized Thermal Shock (PTS) guidance (ref. 2, 3, 4).

The 200°F maximum subcooling limit ensures PTS and brittle fracture of the reactor vessel will not occur following an RCS overcooling transient (defined as an uncontrolled cooldown to less than 500°F) (ref. 3).

Reference(s):

1. Technical Specifications Section 3/4.4.9, "Pressure/Temperature Limits"
2. EOP 2536, "Excess Steam Demand Event"
3. Calc. S-01228-S2 MPS2, "EOP Setpoint Documentation RCS Pressure – Temperature Limits T.06 Figure T.06-1"
4. EOP 2541 Appendix 2 Figures, "Figure 1 – RCS P/T Curve"
5. NEI 99-01 RCS or SG Tube Leakage Reactor Coolant System Potential Loss 1.B

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Barrier: Reactor Coolant System

Category: B. Inadequate Heat Removal

Degradation Threat: Loss

Threshold:

None

Barrier: Reactor Coolant System

Category: B. Inadequate Heat Removal

Degradation Threat: Potential Loss

Threshold:

3. Applicable RCS and Core Heat Removal (HR) Safety Function Status Check acceptance criteria **not** met

Definition(s):

None

Basis:

There are three defined and potentially applicable RCS and Core Heat Removal (HR) Safety Function Status Check acceptance criteria conditions (ref. 1, 2):

- HR-1 – SG Heat Sink With No SI Operating
- HR-2 – SG Heat Sink With SI Operating
- HR-3 – Once-Through-Cooling

Failure to meet the applicable HR acceptance criteria indicates an extreme challenge to the ability to remove RCS heat using the steam generators (i.e., loss of an effective secondary-side heat sink). This condition represents a potential loss of the RCS Barrier. In accordance with EOPs, there may be unusual accident conditions during which operators intentionally reduce the heat removal capability of the steam generators; during these conditions, classification using threshold is not warranted.

Meeting this threshold results in a Site Area Emergency because this threshold is identical to Fuel Clad Barrier Potential Loss threshold B.3; both will be met. This condition warrants a Site Area Emergency declaration because inadequate RCS heat removal may result in fuel heat-up sufficient to damage the cladding and increase RCS pressure to the point where mass will be lost from the system.

Reference(s):

1. EOP 2540, "Functional Recovery"
2. EOP 2540D, "Functional Recovery of Heat Removal"
3. NEI 99-01 Inadequate Heat Removal RCS Loss 2.B

Barrier: Reactor Coolant System

Category: C. CTMT Radiation/ RCS Activity

Degradation Threat: Loss

Threshold:

2. CTMT high range radiation monitor RE-8240/8241 reading > Table F-2 column RCS Loss

Table F-2 CTMT High Range Radiation Monitor Barrier Thresholds RE-8240/8241			
Time > Shutdown (hrs)	Fuel Clad Loss (R/hr)	RCS Loss (R/hr)	CTMT Potential Loss (R/hr)
≤2	56	5	224
> 2 – 4	35	5	140
> 4 – 8	16	5	65
> 8 – 12	10	5	40
> 12	7	5	20

Definition(s):

None

Basis:

A reading > 5 R/hr (minimum practical reading) on RM-8240 or 8241 is indicative of a breach in the RCS barrier (ref. 1, 2).

The radiation monitor reading corresponds to an instantaneous release of all reactor coolant mass into the containment, assuming that reactor coolant activity equals Technical Specification allowable limits. This value is lower than that specified for Fuel Clad barrier loss threshold C.2 since it indicates a loss of the RCS Barrier only.

Because of the very high fuel clad integrity, only small amounts of noble gases would be dissolved in the primary coolant. Conservative estimates indicated that the readings from release of the normal RCS inventory would be below normal readings on the monitor while the station was operating. Therefore, a value 5 times the normal containment radiation monitor (RE-8240/8241) reading of ~ 1 R/hr is used. The reading is less than that specified for fuel cladding barrier loss because no damage to the fuel cladding is assumed. Only leakage from the RCS is assumed for this barrier loss threshold. The value is high enough to preclude erroneous classification of barrier loss due to normal plant operations and is set at the mid-point of the lowest decade of monitor indication (ref. 2).

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Damage Computer Program
2. Calculation RA-0074, "Millstone Unit 2 Expected Containment High Range Radiation Monitor Response to a LOCA Based on Fuel Gap Fractions Defined in NUREG 1228"
3. NEI 99-01 CMT Radiation / RCS Activity RCS Loss 3.A

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Barrier: Reactor Coolant System

Category: C. CTMT Radiation/ RCS Activity

Degradation Threat: Potential Loss

Threshold:

None

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Barrier: Reactor Coolant System

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

None

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Barrier: Reactor Coolant System

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Reactor Coolant System

Category: E. DSEO/ADTS Judgment

Degradation Threat: Loss

Threshold:

3. Any condition in the opinion of the DSEO/ADTS that indicates loss of the RCS barrier
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Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the DSEO/ADTS in determining whether the RCS barrier is lost.

Reference(s):

1. NEI 99-01 Emergency Director Judgment RCS Loss 6.A

Barrier: Reactor Coolant System

Category: E. DSEO/ADTS Judgment

Degradation Threat: Potential Loss

Threshold:

4. **Any** condition in the opinion of the DSEO/ADTS that indicates potential loss of the RCS barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the DSEO/ADTS in determining whether the RCS barrier is potentially lost. The DSEO/ADTS should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Reference(s):

1. NEI 99-01 Emergency Director Judgment RCS Potential Loss 6.A

Barrier: Containment

Category: A. RCS or SG Tube Leakage

Degradation Threat: Loss

Threshold:

1. A leaking or RUPTURED SG is FAULTED outside of CTMT

Definition(s):

FAULTED - The term applied to a steam generator that has a steam leak on the secondary side of sufficient size to cause an uncontrolled drop in steam generator pressure or the steam generator to become completely depressurized.

RUPTURED - The condition of a steam generator in which primary-to-secondary leakage is of sufficient magnitude to require a safety injection.

Basis:

This threshold addresses a leaking or RUPTURED Steam Generator (SG) that is also FAULTED outside of containment. The condition of the SG, whether leaking or RUPTURED, is determined in accordance with the thresholds for RCS Barrier Potential Loss A.1 and Loss A.1, respectively. This condition represents a bypass of the containment barrier.

FAULTED is a defined term within the NEI 99-01 methodology; this determination is not necessarily dependent upon entry into, or diagnostic steps within, an EOP. For example, if the pressure in a steam generator is decreasing uncontrollably (part of the FAULTED definition) and the FAULTED steam generator isolation procedure is not entered because EOP user rules are dictating implementation of another procedure to address a higher priority condition, the steam generator is still considered FAULTED for emergency classification purposes.

The FAULTED criterion establishes an appropriate lower bound on the size of a steam release that may require an emergency classification. Steam releases of this size are readily observable with normal Control Room indications. The lower bound for this aspect of the containment barrier is analogous to the lower bound criteria specified in IC MU4 for the fuel clad barrier (i.e., RCS activity values) and IC MU5 for the RCS barrier (i.e., RCS leak rate values).

This threshold also applies to prolonged steam releases necessitated by operational considerations such as the forced steaming of a leaking or RUPTURED steam generator directly to atmosphere to cooldown the plant, or to drive an auxiliary (emergency) feed water pump. These types of conditions will result in a significant and sustained release of radioactive steam to the environment (and are thus similar to a FAULTED condition). The inability to isolate the steam flow without an adverse effect on plant cooldown meets the intent of a loss of containment.

Steam releases associated with the expected operation of a SG power operated relief valve or safety relief valve do not meet the intent of this threshold. Such releases may occur

intermittently for a short period of time following a reactor trip as operators process through emergency operating procedures to bring the plant to a stable condition and prepare to initiate a plant cooldown. Steam releases associated with the unexpected operation of a valve (e.g., a stuck-open safety valve) do meet this threshold.

Following an SG tube leak or rupture, there may be minor radiological releases through a secondary-side system component (e.g., air ejectors, gland seal exhausters, valve packing, etc.). These types of releases do not constitute a loss or potential loss of containment but should be evaluated using the Category R ICs.

The emergency classification levels resulting from primary-to-secondary leakage, with or without a steam release from the FAULTED SG, are summarized below.

P-to-S Leak Rate	Affected SG is FAULTED Outside of Containment?	
	Yes	No
Less than or equal to 25 gpm	No classification	No classification
Greater than 25 gpm	Unusual Event per MU5.1	Unusual Event per MU5.1
Greater than 50 gpm (<i>RCS Barrier Potential Loss</i>)	Site Area Emergency per FS1.1	Alert per FA1.1
Requires an automatic or manual ECCS (SIAS) actuation (<i>RCS Barrier Loss</i>)	Site Area Emergency per FS1.1	Alert per FA1.1

There is no Potential Loss threshold associated with RCS or SG Tube Leakage.

Reference(s):

1. NEI 99-01 RCS or SG Tube Leakage Containment Loss 1.A

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Barrier: Containment
Category: A. RCS or SG Tube Leakage
Degradation Threat: Potential Loss
Threshold:

None

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Barrier: Containment

Category: B. Inadequate Heat Removal

Degradation Threat: Loss

None

Barrier: Containment

Category: B. Inadequate Heat Removal

Degradation Threat: Potential Loss

Threshold:

1. CETs > 1200°F

AND

Restoration procedures **not** effective within 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Definition(s):

IMMINENT: The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

The SAMGs use a CET temperature of 1200°F as an entry condition and the temperature indicative of a badly damaged core.

This condition represents an IMMINENT core melt sequence which, if not corrected, could lead to vessel failure and an increased potential for containment failure. For this condition to occur there must already have been a loss of the RCS Barrier and the Fuel Clad Barrier. If implementation of a procedure(s) to restore adequate core cooling is not effective (successful) within 15 minutes, it is assumed that the event trajectory will likely lead to core melting and a subsequent challenge of the Containment Barrier.

The restoration procedure is considered "effective" if core exit thermocouple readings are decreasing and/or if reactor vessel level is increasing. Whether or not the procedure(s) will be effective should be apparent within 15 minutes. The DSEO/ADTS should escalate the emergency classification level to a General Emergency as soon as it is determined that the procedure(s) will not be effective.

Severe accident analyses (e.g., NUREG-1150) have concluded that functional restoration procedures can arrest core degradation in a significant fraction of core damage scenarios, and that the likelihood of containment failure is very small in these events. Given this, it is appropriate to provide 15 minutes beyond the required entry point to determine if procedural actions can reverse the core melt sequence.

Reference(s):

1. SAMG 4211, "Phase 1 Initial Diagnosis"
2. MP-26-EPI-FAP11-001, "Core Damage Assessment: Core Exit Temperatures"
3. NEI 99-01 Inadequate Heat Removal Containment Potential Loss 2.A

Millstone Power Station Unit 2
Emergency Action Level Technical Bases Document
Attachment 1 Emergency Action Level Technical Bases

Serial No. 18-364
Docket Nos. 50-336; 72-47
Enclosure 3; Attachment 3B

Barrier: Containment

Category: C. CTMT Radiation/RCS Activity

Degradation Threat: Loss

Threshold:

None

Barrier: Containment

Category: C. CTMT Radiation/RCS Activity

Degradation Threat: Potential Loss

Threshold:

2. CTMT high range radiation monitor RE-8240/8241 reading > Table F-2 column CTMT Potential Loss

Table F-2 CTMT High Range Radiation Monitor Barrier Thresholds RE-8240/8241			
Time > Shutdown (hrs)	Fuel Clad Loss (R/hr)	RCS Loss (R/hr)	CTMT Potential Loss (R/hr)
≤2	56	5	224
> 2 – 4	35	5	140
> 4 – 8	16	5	65
> 8 – 12	10	5	40
> 12	7	5	20

Definition(s):

None

Basis:

The radiation monitor reading corresponds to an instantaneous release of all reactor coolant mass into the containment, assuming that 20% of the fuel cladding has failed (readings have been rounded for readability). This level of fuel clad failure is well above that used to determine the analogous Fuel Clad Barrier Loss and RCS Barrier Loss thresholds (ref. 1, 2).

Time after shutdown values are provided to account for radioactive decay.

NUREG-1228, Source Estimations During Incident Response to Severe Nuclear Power Plant Accidents, indicates the fuel clad failure must be greater than approximately 20% in order for there to be a major release of radioactivity requiring offsite protective actions. For this condition to exist, there must already have been a loss of the RCS barrier and the Fuel Clad barrier. It is therefore prudent to treat this condition as a potential loss of containment which would then escalate the emergency classification level to a General Emergency.

The values specified in Table F-2 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Time periods were chosen to fit monitor response (fast changes in response early following reactor shutdown are broken up into smaller time periods to better approximate expected change). Values were chosen within each time period to minimize error (<50%) to the highest and lowest response within the range.

Reference(s):

1. Damage Computer Program
2. Calculation RA-0074, "Millstone Unit 2 Expected Containment High Range Radiation Monitor Response to a LOCA Based on Fuel Gap Fractions Defined in NUREG 1228"
3. NEI 99-01 CMT Radiation / RCS Activity Containment Potential Loss 3.A

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

2. CTMT isolation is required

AND EITHER:

- CTMT integrity has been lost based on DSEO/ADTS judgment
- UNISOLABLE pathway from CTMT atmosphere to the environment exists

Definition(s):

UNISOLABLE - An open or breached system line that cannot be isolated, remotely or locally.

Basis:

The status of the containment barrier during an event involving steam generator tube leakage is assessed using Loss Threshold A.1. Therefore this threshold is not applicable to steam generator tube leakage.

These thresholds address a situation where containment isolation is required and one of two conditions exists as discussed below. Users are reminded that there may be accident and release conditions that simultaneously meet both bulleted thresholds.

First Threshold – Containment integrity has been lost, i.e., the actual containment atmospheric leak rate likely exceeds that associated with allowable leakage (or sometimes referred to as design leakage). Following the release of RCS mass into containment, containment pressure will fluctuate based on a variety of factors; a loss of containment integrity condition may (or may not) be accompanied by a noticeable drop in containment pressure. Recognizing the inherent difficulties in determining a containment leak rate during accident conditions, it is expected that the DSEOADTS will assess this threshold using judgment, and with due consideration given to current plant conditions, and available operational and radiological data (e.g., containment pressure, readings on radiation monitors outside containment, operating status of containment pressure control equipment, etc.).

Refer to the middle piping run of Figure 1. Two simplified examples are provided. One is leakage from a penetration and the other is leakage from an in-service system valve. Depending upon radiation monitor locations and sensitivities, the leakage could be detected by any of the four monitors depicted in the figure.

Another example would be a loss or potential loss of the RCS barrier, and the simultaneous occurrence of two FAULTED locations on a steam generator where one fault is located inside containment (e.g., on a steam or feedwater line) and the other outside of containment. In this case, the associated steam line provides a pathway for the containment atmosphere to escape to an area outside the containment.

Following the leakage of RCS mass into containment and a rise in containment pressure, there may be minor radiological releases associated with allowable (design) containment leakage through various penetrations or system components. These releases do not constitute a loss or potential loss of containment but should be evaluated using the Category A ICs.

Second Threshold – Conditions are such that there is an UNISOLABLE pathway for the migration of radioactive material from the containment atmosphere to the environment. As used here, the term “environment” includes the atmosphere of a room or area, outside the containment, that may, in turn, communicate with the outside-the-plant atmosphere (e.g., through discharge of a ventilation system or atmospheric leakage). Depending upon a variety of factors, this condition may or may not be accompanied by a noticeable drop in containment pressure.

Refer to the top piping run of Figure 1. In this simplified example, the inboard and outboard isolation valves remained open after a containment isolation was required (i.e., containment isolation was not successful). There is now an UNISOLABLE pathway from the containment to the environment.

The existence of a filter is not considered in the threshold assessment. Filters do not remove fission product noble gases. In addition, a filter could become ineffective due to iodine and/or particulate loading beyond design limits (i.e., retention ability has been exceeded) or water saturation from steam/high humidity in the release stream.

Leakage between two interfacing liquid systems, by itself, does not meet this threshold.

Refer to the bottom piping run of Figure 1. In this simplified example, leakage in an RCP seal cooler is allowing radioactive material to enter the Auxiliary Building. The radioactivity would be detected by the Process Monitor. If there is no leakage from the closed water cooling system to the Auxiliary Building, then no threshold has been met. If the pump developed a leak that allowed steam/water to enter the Auxiliary Building, then the second threshold would be met. Depending upon radiation monitor locations and sensitivities, this leakage could be detected by any of the four monitors depicted in the figure and cause the first threshold to be met as well.

Following the leakage of RCS mass into containment and a rise in containment pressure, there may be minor radiological releases associated with allowable containment leakage through various penetrations or system components. Minor releases may also occur if a containment isolation valve(s) fails to close but the containment atmosphere escapes to an enclosed system. These releases do not constitute a loss or potential loss of containment but should be evaluated using the Category R ICs.

Reference(s):

1. NEI 99-01 CMT Integrity or Bypass Containment Loss 4.A

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

3. Indications of RCS leakage outside of CTMT

Definition(s):

None

Basis:

To ensure proper escalation of the emergency classification, the RCS leakage outside of containment must be related to the mass loss that is causing the RCS Loss and/or Potential Loss threshold A.1 to be met.

The status of the containment barrier during an event involving steam generator tube leakage is assessed using Containment Loss Threshold A.1. Therefore this threshold is not applicable to steam generator tube leakage.

Containment sump, temperature, pressure and/or radiation levels will increase if reactor coolant mass is leaking into the containment. If these parameters have not increased, then the reactor coolant mass may be leaking outside of containment (i.e., a containment bypass sequence). Increases in sump, temperature, pressure, flow and/or radiation level readings outside of the containment may indicate that the RCS mass is being lost outside of containment.

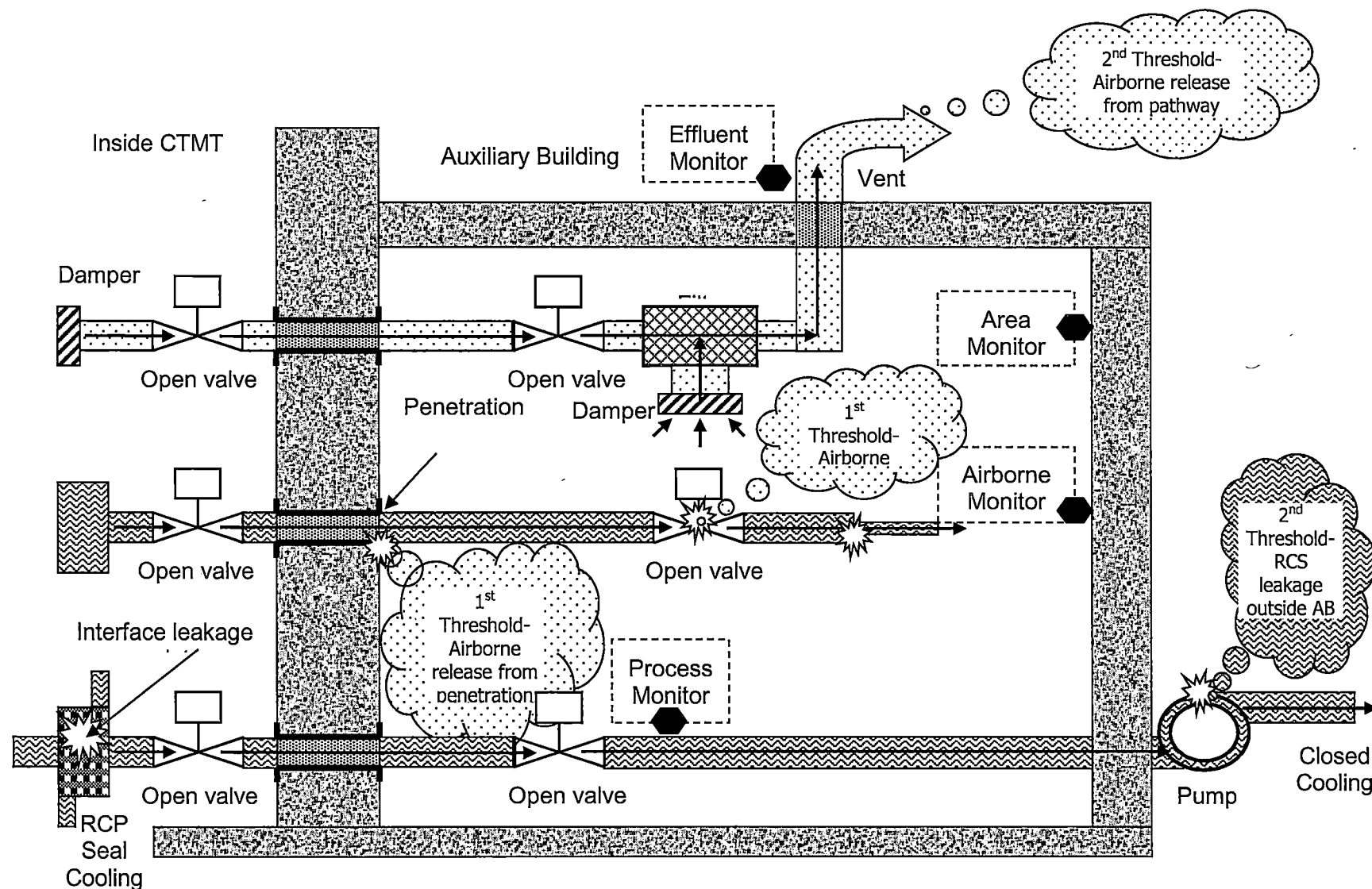
Unexpected elevated readings and alarms on radiation monitors with detectors outside containment should be corroborated with other available indications to confirm that the source is a loss of RCS mass outside of containment. If the fuel clad barrier has not been lost, radiation monitor readings outside of containment may not increase significantly; however, other unexpected changes in sump levels, area temperatures or pressures, flow rates, etc. should be sufficient to determine if RCS mass is being lost outside of the containment.

Refer to the middle piping run of Figure 1. In this simplified example, a leak has occurred at a reducer on a pipe carrying reactor coolant in the Auxiliary Building. Depending upon radiation monitor locations and sensitivities, the leakage could be detected by any of the four monitors depicted in the figure and cause loss threshold D.2 to be met as well.

Reference(s):

1. NEI 99-01 CMT Integrity or Bypass Containment Loss 4.B

Figure 1: Containment Integrity or Bypass Examples



Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

3. CTMT pressure > 54 psig

Definition(s):

None

Basis:

If containment pressure exceeds the design pressure of 54 psig (ref. 1), there exists a potential to lose the containment barrier. To reach this level, there must be an inadequate core cooling condition for an extended period of time; therefore, the RCS and Fuel Clad barriers would already be lost. Thus, this threshold is a discriminator between a Site Area Emergency and General Emergency since there is now a potential to lose the third barrier.

Reference(s):

1. Unit 2 UFSAR Section 5.2.2.1.3, "Loss-of-Coolant Accident Loads"
2. NEI 99-01 CMT Integrity or Bypass Containment Potential Loss 4.A

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

4. CTMT hydrogen concentration > 4%

Definition(s):

None

Basis:

The existence of an explosive mixture means, at a minimum, that the containment atmospheric hydrogen concentration is sufficient to support a hydrogen burn (i.e., at the lower deflagration limit). A hydrogen burn will raise containment pressure and could result in collateral equipment damage leading to a loss of containment integrity. It therefore represents a potential loss of the containment barrier.

A containment hydrogen concentration of 4% conservatively represents the lowest threshold for flammability in the presence of oxygen (ref. 1).

Reference(s):

1. SAMG 4215 Attachment 1, "Calculation Aids, Calculational Aid #7 Containment Challenge Due to Hydrogen Combustion"
2. NEI 99-01 CMT Integrity or Bypass Containment Potential Loss 4.B

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

5. CTMT pressure > 10 psig with < one full train of CTMT heat removal systems
(Note 11) operating per design for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 11: One full train of containment heat removal systems consist of one Containment Spray pump and two containment air recirculation units in the Containment Air Recirculation and Cooling System.

Definition(s):

None

Basis:

This threshold describes a condition where containment pressure is greater than the setpoint (9.48 psig rounded to 10 psig for readability) at which containment energy (heat) removal systems are designed to automatically actuate, and less than one full train of equipment is capable of operating per design (ref. 1, 2). The 15-minute criterion is included to allow operators time to manually start equipment that may not have automatically started, if possible. This threshold represents a potential loss of containment in that containment heat removal/depressurization systems (e.g., containment sprays but not including containment venting strategies) are either lost or performing in a degraded manner.

One full train of containment heat removal systems consist of one Containment Spray pump and two containment air recirculation units. The combination of required equipment can be obtained from using equipment on either emergency bus in order to meet the "one full train" requirement (ref. 3).

Reference(s):

1. UFSAR Section 6.4, "Containment Spray System"
2. UFSAR Section 7.3, "Engineered Safety Features Actuation System"
3. UFSAR Section 6.5, "Containment Air Recirculation and Cooling System"
4. NEI 99-01 CMT Integrity or Bypass Containment Potential Loss 4.C

Barrier: Containment

Category: E. DSEO/ADTS Judgment

Degradation Threat: Loss

Threshold:

4. **Any** condition in the opinion of the DSEO/ADTS that indicates loss of the CTMT barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the DSEO/ADTS in determining whether the containment barrier is lost.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Containment Loss 6.A

Barrier: Containment

Category: E. DSEO/ADTS Judgment

Degradation Threat: Potential Loss

Threshold:

6. **Any** condition in the opinion of the DSEO/ADTS that indicates potential loss of the CTMT barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the DSEO/ADTS in determining whether the containment barrier is potentially lost. The DSEO/ADTS should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Containment Potential Loss 6.A

Category H – Hazards and Other Conditions Affecting Plant Safety

EAL Group: ANY (EALs in this category are applicable to any plant condition, hot or cold.)

Hazards are non-plant, system-related events that can directly or indirectly affect plant operation, reactor plant safety or personnel safety.

1. Security

Unauthorized entry attempts into the PROTECTED AREA, bomb threats, sabotage attempts, and actual security compromises threatening loss of physical control of the plant.

2. Seismic Event

Natural events such as earthquakes have potential to cause plant structure or equipment damage of sufficient magnitude to threaten personnel or plant safety.

3. Natural or Technological Hazard

Other natural and non-naturally occurring events that can cause damage to plant facilities include tornados, FLOODING, hazardous material releases and events restricting site access warranting classification.

4. Fire

FIRES can pose significant hazards to personnel and reactor safety. Appropriate for classification are FIRES within the plant PROTECTED AREA or which may affect operability of equipment needed for safe shutdown

5. Hazardous Gas

Toxic, corrosive, asphyxiant or flammable gas leaks can affect normal plant operations or preclude access to plant areas required to safely shutdown the plant.

6. Control Room Evacuation

Events that are indicative of loss of Control Room habitability. If the Control Room must be evacuated, additional support for monitoring and controlling plant functions is necessary through the emergency response facilities.

7. DSEO/ADTS Judgment

The EALs defined in other categories specify the predetermined symptoms or events that are indicative of emergency or potential emergency conditions and thus warrant classification. While these EALs have been developed to address the full spectrum of possible emergency conditions which may warrant classification and subsequent implementation of the Emergency Plan, a provision for classification of emergencies based on operator/management experience and judgment is still necessary. The EALs of this category provide the DSEO/ADTS the latitude to classify emergency conditions consistent with the established classification criteria based upon DSEO/ADTS judgment.

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 1 – Security

Initiating Condition: Confirmed SECURITY CONDITION or threat

EAL:

HU1.1 Unusual Event

A SECURITY CONDITION that does **not** involve a HOSTILE ACTION as reported by MPS Security Shift Supervision

OR

Notification of a credible security threat directed at the site

OR

A validated notification from the NRC providing information of an aircraft threat

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA (OCA) - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;

- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

SECURITY CONDITION - Any security event as listed in the approved security contingency plan that constitutes a threat/compromise to site security, threat/risk to site personnel, or a potential degradation to the level of safety of the plant. A security condition does **not** involve a HOSTILE ACTION.

Basis:

This IC addresses events that pose a threat to plant personnel or SAFETY SYSTEM equipment, and thus represent a potential degradation in the level of plant safety. Security events which do not meet one of these EALs are adequately addressed by the requirements of 10 CFR 73.71 or 10 CFR 50.72. Security events assessed as HOSTILE ACTIONS are classifiable under ICs HA1 and HS1. Guidance on assessing Security Conditions is included in the Security Contingency Plan Implementing Procedures (SCIP). The SCIPs are implementing procedures for the Station Safeguards Contingency Plan.

Timely and accurate communications between Security Shift Supervision and the Control Room is essential for proper classification of a security-related event. Classification of these events will initiate appropriate threat-related notifications to plant personnel and Offsite Response Organizations.

Security plans and terminology are based on the guidance provided by NEI 03-12, *Template for the Security Plan, Training and Qualification Plan, Safeguards Contingency Plan [and Independent Spent Fuel Storage Installation Security Program]*.

The first threshold references the Security Shift Supervision because these are the individuals trained to confirm that a security event is occurring or has occurred. Training on security event confirmation and classification is controlled due to the nature of Safeguards and 10 CFR 2.39 information.

The second threshold addresses the receipt of a credible security threat. The credibility of the threat is assessed in accordance with the Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program (ref. 1).

The third threshold addresses the threat from the impact of an aircraft on the plant. The NRC Headquarters Operations Officer (HOO) will communicate to the licensee if the threat involves an aircraft. The status and size of the plane may also be provided by NORAD through the NRC. Validation of the threat is performed in accordance with OP-AA-900 Authentication (ref. 2).

Emergency plans and implementing procedures are public documents; therefore, EALs should not incorporate Security-sensitive information. This includes information that may be advantageous to a potential adversary, such as the particulars concerning a specific threat or threat location. Security-sensitive information should be contained in non-public documents such as the Security Plan for MPS (ref. 1).

Escalation of the emergency classification level would be via IC HA1.

Reference(s):

1. Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program
2. OP-AA-900, "Authentication"
3. NEI 99-01 HU1

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 1 – Security
Initiating Condition: HOSTILE ACTION within the OWNER CONTROLLED AREA or airborne attack threat within 30 minutes

EAL:

HA1.1 Alert

A HOSTILE ACTION is occurring or has occurred within the OWNER CONTROLLED AREA as reported by MPS Security Shift Supervision

OR

A validated notification from NRC of an aircraft attack threat within 30 min. of the site

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

HOSTILE FORCE - One or more individuals who are engaged in a determined assault, overtly or by stealth and deception, equipped with suitable weapons capable of killing, maiming, or causing destruction.

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses the occurrence of a HOSTILE ACTION within the OWNER CONTROLLED AREA or notification of an aircraft attack threat. This event will require rapid response and assistance due to the possibility of the attack progressing to the PROTECTED AREA, or the need to prepare the plant and staff for a potential aircraft impact.

Timely and accurate communications between Security Shift Supervision and the Control Room is essential for proper classification of a security-related event.

Security plans and terminology are based on the guidance provided by NEI 03-12, *Template for the Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program*.

As time and conditions allow, these events require a heightened state of readiness by the plant staff and implementation of onsite protective measures (e.g., evacuation, dispersal or sheltering). The Alert declaration will also heighten the awareness of State and local agencies, allowing them to be better prepared should it be necessary to consider further actions.

This EAL does not apply to incidents that are accidental events, acts of civil disobedience, or otherwise are not a HOSTILE ACTION perpetrated by a HOSTILE FORCE. Examples include the crash of a small aircraft, shots from hunters, physical disputes between employees, etc. Reporting of these types of events is adequately addressed by other EALs, or the requirements of 10 CFR 73.71 or 10 CFR 50.72.

The first threshold is applicable for any HOSTILE ACTION occurring, or that has occurred, in the OWNER CONTROLLED AREA.

The second threshold addresses the threat from the impact of an aircraft on the plant, and the anticipated arrival time is within 30 minutes. The intent of this EAL is to ensure that threat-related notifications are made in a timely manner so that plant personnel and State and local agencies are in a heightened state of readiness. This EAL is met when the threat-related information has been validated in accordance with OP-AA-900 Authentication (ref. 2).

The NRC Headquarters Operations Officer (HOO) will communicate to the licensee if the threat involves an aircraft. The status and size of the plane may be provided by NORAD through the NRC.

In some cases, it may not be readily apparent if an aircraft impact within the OWNER CONTROLLED AREA was intentional (i.e., a HOSTILE ACTION). It is expected, although not certain, that notification by an appropriate Federal agency to the site would clarify this point. In this case, the appropriate federal agency is intended to be NORAD, FBI, FAA or NRC. The emergency declaration, including one based on other ICs/EALs, should not be unduly delayed while awaiting notification by a Federal agency.

Emergency plans and implementing procedures are public documents; therefore, EALs should not incorporate Security-sensitive information. This includes information that may be advantageous to a potential adversary, such as the particulars concerning a specific threat or threat location. Security-sensitive information should be contained in non-public documents such as the Security Plan for MPS (ref. 1).

Escalation of the emergency classification level would be via IC HS1.

Reference(s):

1. Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program
2. OP-AA-900, "Authentication"
3. NEI 99-01 HA1

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 1 – Security

Initiating Condition: HOSTILE ACTION within the PROTECTED AREA

EAL:

HS1.1 Site Area Emergency

A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by MPS Security Shift Supervision

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

HOSTILE FORCE - One or more individuals who are engaged in a determined assault, overtly or by stealth and deception, equipped with suitable weapons capable of killing, maiming, or causing destruction.

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses the occurrence of a HOSTILE ACTION within the PROTECTED AREA. This event will require rapid response and assistance due to the possibility for damage to plant equipment.

Timely and accurate communications between Security Shift Supervision and the Control Room is essential for proper classification of a security-related event (ref. 1).

Security plans and terminology are based on the guidance provided by NEI 03-12, *Template for the Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program*.

As time and conditions allow, these events require a heightened state of readiness by the plant staff and implementation of onsite protective measures (e.g., evacuation, dispersal or sheltering). The Site Area Emergency declaration will mobilize State and local agency resources and have them available to develop and implement public protective actions in the unlikely event that the attack is successful in impairing multiple safety functions.

This EAL does not apply to incidents that are accidental events, acts of civil disobedience, or otherwise are not a HOSTILE ACTION perpetrated by a HOSTILE FORCE. Examples include the crash of a small aircraft, shots from hunters, physical disputes between employees, etc. Reporting of these types of events is adequately addressed by other EALs, or the requirements of 10 CFR 73.71 or 10 CFR 50.72.

Emergency plans and implementing procedures are public documents; therefore, EALs should not incorporate Security-sensitive information. This includes information that may be advantageous to a potential adversary, such as the particulars concerning a specific threat or threat location. Security-sensitive information should be contained in non-public documents such as the Security Plan for MPS (ref. 1).

Reference(s):

1. Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program
2. NEI 99-01 HS1

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 2 – Seismic Event

Initiating Condition: Seismic event greater than OBE levels

EAL:

HU2.1 Unusual Event

Notification by the MPS3 Control Room that a seismic event > OBE has occurred

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses a seismic event that results in accelerations at the plant site greater than those specified for an Operating Basis Earthquake (OBE). An earthquake greater than an OBE but less than a Safe Shutdown Earthquake (SSE) should have no significant impact on safety-related systems, structures and components; however, some time may be required for the plant staff to ascertain the actual post-event condition of the plant (e.g., performs walk-downs and post-event inspections). Given the time necessary to perform walk-downs and inspections, and fully understand any impacts, this event represents a potential degradation of the level of safety of the plant.

Event verification with external sources should not be necessary during or following an OBE. Earthquakes of this magnitude should be readily felt by on-site personnel and recognized as a seismic event (e.g., lateral accelerations in excess of 0.09g). The Shift Manager or DSEO/ADTS may seek external verification if deemed appropriate (e.g., a call to the U.S. Geological Survey (USGS), check internet news sources, etc.); however, the verification action must not preclude a timely emergency declaration.

For both MPS2 and MPS3, the OBE ground acceleration thresholds are > 0.09g horizontal or > 0.06g vertical. The MPS3 Control Room has real time OBE exceedance alarm indications (ref. 1, 2, 3). Therefore classification shall be based upon the receipt of the MPS3 OBE alarm light on MPS3 Panel 3ERS-PNLSM1C (ref. 2). The MPS3 Control Room will notify MPS2 if the seismic event exceeded the OBE threshold.

Additionally, the Unit 2 seismic instrumentation provides a trigger alarm (C06 DA-22) at 0.01g ground acceleration (ref. 4).

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or MA9.

Reference(s):

1. AOP 2562, "Earthquake"
2. MPS3 AOP 3570, "Earthquake"
3. UFSAR 2.1.1, "Site Location and Description"
4. ARP 2590E-119, "SEISMIC INST TRIGGERED"
5. NEI 99-01 HU2

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.1 Unusual Event

A tornado strike within the PROTECTED AREA

Mode Applicability:

All

Definition(s):

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

This EAL addresses a tornado striking (touching down) within the PROTECTED AREA.

Escalation of the emergency classification level would be based on ICs in Categories R, F, M or C.

If damage is confirmed visually or by other in-plant indications, the event may be escalated to an Alert under IC CA6 or MA9.

A tornado striking (touching down) within the PROTECTED AREA warrants declaration of an Unusual Event regardless of the measured wind speed at the meteorological tower. A tornado is defined as a violently rotating column of air in contact with the ground and extending from the base of a thunderstorm.

Reference(s):

1. AOP 2560, "Storms, High Winds and High Tides"
2. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.2 Unusual Event

Internal room or area FLOODING of a magnitude sufficient to require manual or automatic electrical isolation of a SAFETY SYSTEM component required by Technical Specifications for the current operating mode

Mode Applicability:

All

Definition(s):

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

This EAL addresses FLOODING of a building room or area that results in operators isolating power to a SAFETY SYSTEM component due to water level or other wetting concerns. Classification is also required if the water level or related wetting causes an automatic isolation of a SAFETY SYSTEM component from its power source (e.g., a breaker or relay trip). To warrant classification, operability of the affected component must be required by Technical Specifications for the current operating mode (ref. 1, 2).

Escalation of the emergency classification level would be based on ICs in Categories R, F, M or C.

Refer to EAL CA6.1 or MA9.1 for internal flooding affecting more than one SAFETY SYSTEM train.

Reference(s):

1. AOP 2560, "Storms, High Winds and High Tides"
2. SP 2615, "Flood Level Determination"
3. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.3 Unusual Event

Movement of personnel within the PROTECTED AREA is IMPEDED due to an event external to the PROTECTED AREA involving hazardous materials (e.g., an offsite chemical spill or toxic gas release)

Mode Applicability:

All

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

This EAL addresses a hazardous materials event originating at a location outside the PROTECTED AREA and of sufficient magnitude to IMPEDE the movement of personnel within the PROTECTED AREA.

Escalation of the emergency classification level would be based on ICs in Categories R, F, M or C.

Reference(s):

1. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.4 Unusual Event

A hazardous event that results in on-site conditions sufficient to prohibit the plant staff from accessing the site via personal vehicles (Note 7)

Note 7: This EAL does **not** apply to routine traffic impediments such as fog, snow, ice, or vehicle breakdowns or accidents.

Mode Applicability:

All

Definition(s):

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

This EAL addresses a hazardous event that causes an on-site impediment to vehicle movement and significant enough to prohibit the plant staff from accessing the site using personal vehicles. Examples of such an event include site FLOODING caused by a hurricane, heavy rains, up-river water releases, dam failure, etc., or an on-site train derailment blocking the access road.

This EAL is not intended to apply to routine impediments such as fog, snow, ice, or vehicle breakdowns or accidents, but rather to more significant conditions such as the Hurricane Andrew strike on Turkey Point in 1992, the FLOODING around the Cooper Station during the Midwest floods of 1993, or the FLOODING around Ft. Calhoun Station in 2011.

Escalation of the emergency classification level would be based on ICs in Categories R, F, M or C.

Reference(s):

1. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.1 Unusual Event

A FIRE is **not** extinguished within 15 min. of **any** of the following fire detection indications (Note 1):

- Report from the field (i.e., visual observation)
- Receipt of multiple (more than 1) fire alarms or indications
- Field verification of a single fire alarm

AND

The FIRE is located within **any** Table H-1 area

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table H-1 MPS2 Fire Areas
<ul style="list-style-type: none"> • Containment Building • Control Room • Auxiliary Building Areas: <ul style="list-style-type: none"> ○ Penetration Areas ○ RBCCW Pump Rooms ○ Diesel Generator Rooms ○ Diesel Generator Day Tank Rooms ○ Charging Pump Cubicles ○ SI Pump Rooms ○ DC Equipment and Battery Rooms ○ East 480 VAC Switchgear Room • Intake Structure • Turbine Building Areas: <ul style="list-style-type: none"> ○ Cable Vaults ○ West 480 VAC Switchgear Room ○ 4.16 KV Switchgear Rooms ○ Steam Driven Aux Feedwater Room ○ Motor Driven Aux Feedwater Room • Yard Areas: <ul style="list-style-type: none"> ○ RWST ○ CST ○ Unit 3 SBO DG

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

The 15 minute requirement begins with a credible notification that a FIRE is occurring, or receipt of multiple VALID fire detection system alarms or field validation of a single fire alarm. The alarm is to be validated using available Control Room indications or alarms to prove that it is not spurious, or by reports from the field.

Table H-1 Fire Areas are those areas that contain equipment necessary for safe operation and shutdown of the plant (ref. 1, 2).

The intent of the 15-minute duration is to size the FIRE and to discriminate against small FIRES that are readily extinguished (e.g., smoldering waste paper basket). In addition to alarms, other indications of a FIRE could be a drop in fire main pressure, automatic activation of a suppression system, etc.

Upon receipt, operators will take prompt actions to confirm the validity of an initial fire alarm, indication, or report. For EAL assessment purposes, the emergency declaration clock starts at the time that the initial alarm, indication, or report was received, and not the time that a subsequent verification action was performed. Similarly, the fire duration clock also starts at the time of receipt of the initial alarm, indication or report.

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or MA9.

Reference(s):

1. AOP 2559, "Fire"
2. MP2 Appendix R Compliance Report
3. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.2 Unusual Event

Receipt of a single fire alarm (i.e., **no** other indications of a FIRE)

AND

The fire alarm is indicating a FIRE within **any** Table H-1 area (excluding Containment Building)

AND

The existence of a FIRE is **not** verified within 30 min. of alarm receipt (Notes 1, 14)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 14: A Containment Building fire alarm is considered VALID upon receipt of multiple (more than one) fire zone alarms.

Table H-1 MPS2 Fire Areas
<ul style="list-style-type: none"> • Containment Building • Control Room • Auxiliary Building Areas: <ul style="list-style-type: none"> ○ Penetration Areas ○ RBCCW Pump Rooms ○ Diesel Generator Rooms ○ Diesel Generator Day Tank Rooms ○ Charging Pump Cubicles ○ SI Pump Rooms ○ DC Equipment and Battery Rooms ○ East 480 VAC Switchgear Room • Intake Structure • Turbine Building Areas: <ul style="list-style-type: none"> ○ Cable Vaults ○ West 480 VAC Switchgear Room ○ 4.16 KV Switchgear Rooms ○ Steam Driven Aux Feedwater Room ○ Motor Driven Aux Feedwater Room • Yard Areas: <ul style="list-style-type: none"> ○ RWST ○ CST ○ Unit 3 SBO DG

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

The 30 minute requirement begins upon receipt of a single VALID fire detection system alarm. The alarm is to be validated using available Control Room indications or alarms to prove that it is not spurious, or by reports from the field. Actual field reports must be made within the 30 minute time limit or a classification must be made. If a fire is verified to be occurring by field report, classification shall be made based on EAL HU4.1, with the 15 minute requirement beginning with the verification of the fire by field report.

Table H-1 Fire Areas are those areas that contain equipment necessary for safe operation and shutdown of the plant (ref. 1, 2).

This EAL addresses receipt of a single fire alarm, and the existence of a FIRE is not verified (i.e., proved or disproved) within 30-minutes of the alarm. Upon receipt, operators will take prompt actions to confirm the validity of a single fire alarm. For EAL assessment purposes, the 30-minute clock starts at the time that the initial alarm was received, and not the time that a subsequent verification action was performed.

With regard to Containment Building fire alarms, there is constant air movement in the enclosed containment due to the operation of the containment ventilation system. The operating cooling units are drawing air to the units past the smoke detectors. It can be reasonably expected that a fire that burns for 15 minutes would produce sufficient products of combustion to cause fire detectors in multiple zones to alarm. Therefore, a single Containment Building fire alarm is not considered VALID.

A single fire alarm, absent other indication(s) of a FIRE, may be indicative of equipment failure or a spurious activation, and not an actual FIRE. For this reason, additional time is allowed to verify the validity of the alarm. The 30-minute period is a reasonable amount of time to determine if an actual FIRE exists; however, after that time, and absent information to the contrary, it is assumed that an actual FIRE is in progress.

If an actual FIRE is verified by a report from the field, then HU4.1 is immediately applicable, and the emergency must be declared if the FIRE is not extinguished within 15-minutes of the report. If the alarm is verified to be due to an equipment failure or a spurious activation, and this verification occurs within 30-minutes of the receipt of the alarm, then this EAL is not applicable and no emergency declaration is warranted.

Basis-Related Requirements from Appendix R (justification for the use of 30 minute criteria)

10 CFR 50 Appendix R, states in part:

Criterion 3 of Appendix A to this part specifies that "Structures, systems, and components important to safety shall be designed and located to minimize, consistent with other safety requirements, the probability and effect of fires and explosions."

When considering the effects of fire, those systems associated with achieving and maintaining safe shutdown conditions assume major importance to safety because damage to them can lead to core damage resulting from loss of coolant through boil-off.

Because fire may affect safe shutdown systems and because the loss of function of systems used to mitigate the consequences of design basis accidents under post-fire conditions does not per se impact public safety, the need to limit fire damage to systems required to achieve and maintain safe shutdown conditions is greater than the need to limit fire damage to those systems required to mitigate the consequences of design basis accidents.

In addition, Appendix R to 10 CFR 50, requires, among other considerations, the use of 1-hour fire barriers for the enclosure of cable and equipment and associated non-safety circuits of one redundant train (G.2.c). As used in HU4.2, the 30-minutes to verify a single alarm is well within this worst-case 1-hour time period.

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or MA9.

Reference(s):

1. AOP, "2559 Fire"
2. MP2 Appendix R Compliance Report
3. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.3 Unusual Event

A FIRE within the PROTECTED AREA **not** extinguished within 60 min. of the initial report, alarm or indication (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

PROTECTED AREA - An area encompassed by physical barriers (i.e., the security fence) and to which access is controlled.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

In addition to a FIRE addressed by EAL HU4.1 or HU4.2, a FIRE within the plant PROTECTED AREA not extinguished within 60-minutes may also potentially degrade the level of plant safety.

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or MA9.

Reference(s):

1. AOP 2559, "Fire"
2. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.4 Unusual Event

A FIRE within the PROTECTED AREA that requires an offsite fire department to assist with extinguishment

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

PROTECTED AREA - An area encompassed by physical barriers (i.e., the security fence) and to which access is controlled.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

If a FIRE within the plant PROTECTED AREA is of sufficient size to require a response by an offsite firefighting agency (e.g., a local town Fire Department), then the level of plant safety is potentially degraded. The dispatch of an offsite firefighting agency to the site requires an emergency declaration only if it is needed to actively support firefighting efforts because the fire is beyond the capability of the Fire Brigade to extinguish. Declaration is not necessary if the agency resources are placed on stand-by, or supporting post-extinguishment recovery or investigation actions.

The Shift Fire Brigade Advisor or Shift Fire Brigade Leader will assess whether the fire conditions warrant outside assistance (ref. 1).

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or MA9.

Reference(s):

1. AOP 2559, "Fire"
2. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 5 – Hazardous Gases
Initiating Condition: Gaseous release IMPEDING access to equipment necessary for normal plant operations, cooldown or shutdown

EAL:

HA5.1 Alert

Release of a toxic, corrosive, asphyxiant or flammable gas into **any** Table H-2 room or area

AND

Entry into the room or area is prohibited or IMPEDED (Note 5)

Note 5: If the equipment in the listed room or area was already inoperable or out-of-service before the event occurred, then no emergency classification is warranted.

Table H-2 Safe Operation & Shutdown Rooms/Areas		
Room/Area		Mode
Aux. Building EI -5'6"	West Area	3
Aux. Building EI -5'6"	East Near SFP Cooling	
Aux. Building EI 14'6"	B51 & B61 Enclosures	
Aux. Building EI 14'6"	Boric Acid Batching Tank	
Aux. Building EI -25'6"	RB Hx Area	
Enc. Building EI -5'6"	East Pipe Penetration	
Aux. Building EI 14'6"	By B61	3, 4
Enc. Building EI -45'6"	"A" & "B" ESF Rooms	
Aux. Building EI -45'6"	General Area	
Enc. Building EI -5'6"	West Pipe Penetration	3, 5
Aux. Building EI 14'6"	SE Across From MCC B51	4
Aux. Building EI -5'6"	VCT Block Wall Area	5
Enc. Building EI -45'6"	"A" ESF Room	

Mode Applicability:

3 – Hot Standby, 4 – Hot Shutdown, 5 – Cold Shutdown

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

Basis:

This IC addresses an event involving a release of a hazardous gas that precludes or IMPEDES access to equipment necessary to maintain normal plant operation, or required for a normal plant cooldown and shutdown. This condition represents an actual or potential substantial degradation of the level of safety of the plant.

An Alert declaration is warranted if entry into the affected room/area is, or may be, procedurally required during the plant operating mode in effect at the time of the gaseous release. The emergency classification is not contingent upon whether entry is actually necessary at the time of the release.

Evaluation of the IC and EAL do not require atmospheric sampling; it only requires the DSEO/ADTS's judgment that the gas concentration in the affected room/area is sufficient to preclude or significantly IMPEDE procedurally required access. This judgment may be based on a variety of factors including an existing job hazard analysis, report of ill effects on personnel, advice from a subject matter expert or operating experience with the same or similar hazards. Access should be considered as IMPEDED if extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is not routinely employed).

An emergency declaration is **not** warranted if any of the following conditions apply:

- The plant is in an operating mode different than the mode specified for the affected room/area (i.e., entry is not required during the operating mode in effect at the time of the gaseous release). For example, the plant is in Mode 1 when the gaseous release occurs, and the procedures used for normal operation, cooldown and shutdown do not require entry into the affected room until Mode 4.
- The gas release is a planned activity that includes compensatory measures which address the temporary inaccessibility of a room or area (e.g., fire suppression system testing).
- The action for which room/area entry is required is of an administrative or record keeping nature (e.g., normal rounds or routine inspections).
- The access control measures are of a conservative or precautionary nature, and would not actually prevent or IMPEDE a required action.
- If the equipment in the listed room or area was already inoperable, or out-of-service, before the event occurred, then no emergency should be declared since the event will have no adverse impact beyond that already allowed by Technical Specifications at the time of the event.

An asphyxiant is a gas capable of reducing the level of oxygen in the body to dangerous levels. Most commonly, asphyxiants work by merely displacing air in an enclosed environment. This reduces the concentration of oxygen below the normal level of around 19%, which can lead to breathing difficulties, unconsciousness or even death.

This EAL does not apply to firefighting activities that generate smoke and that automatically or manually activate a fire suppression system in an area.

Escalation of the emergency classification level would be via Category R, M, C or F ICs.

Reference(s):

1. Attachment 2, "Safe Operation & Shutdown Areas Tables A-3 & H-2 Bases"
2. NEI 99-01 HA5

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 6 – Control Room Evacuation
Initiating Condition: Control Room evacuation resulting in transfer of plant control to alternate locations

EAL:

HA6.1 Alert

An event has resulted in plant control being transferred from the Control Room to the Hot Shutdown Panel (C-21) or Fire Shutdown Panel (C-10)

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses an evacuation of the Control Room that results in transfer of plant control to alternate locations outside the Control Room. The loss of the ability to control the plant from the Control Room is considered to be a potential substantial degradation in the level of plant safety.

Following a Control Room evacuation, control of the plant will be transferred to alternate shutdown locations. The necessity to control a plant shutdown from outside the Control Room, in addition to responding to the event that required the evacuation of the Control Room, will present challenges to plant operators and other on-shift personnel. Activation of the ERO and emergency response facilities will assist in responding to these challenges.

Transfer of plant control begins when the last licensed operator leaves the Control Room.

Control will be established at Hot Shutdown Panel C-21 if the Control Room was evacuated for reasons other than a fire or smoke. Control will be established at Fire Shutdown Panel C-10 if the Control Room was evacuated due to a fire or smoke (ref. 1, 2).

Escalation of the emergency classification level would be via IC HS6.

Reference(s):

1. AOP 2551, "Shutdown from Outside the Control Room"
2. AOP 2579A, "Fire Procedure for Hot Standby Appendix R Fire Area R-1"
3. NEI 99-01 HA6

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 6 – Control Room Evacuation
Initiating Condition: Inability to control a key safety function from outside the Control Room
EAL:

HS6.1 Site Area Emergency

An event has resulted in plant control being transferred from the Control Room to the Hot Shutdown Panel (C-21) or Fire Shutdown Panel (C-10)

AND

Control of **any** of the following key safety functions is **not** re-established within 15 min. of the last licensed operator leaving the Control Room (Note 1):

- Reactivity (modes 1, 2 and 3 **only**)
- Core cooling
- RCS heat removal

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 - Hot Standby, 4 – Hot Shutdown, 5 – Cold Shutdown, 6 – Refueling

Definition(s):

None

Basis:

This IC addresses an evacuation of the Control Room that results in transfer of plant control to alternate locations, and the control of a key safety function cannot be reestablished in a timely manner. The failure to gain control of a key safety function following a transfer of plant control to alternate locations is a precursor to a challenge to one or more fission product barriers within a relatively short period of time.

The determination of whether or not “control” is established at the remote safe shutdown location(s) is based on DSEO/ADTS judgment. The DSEO/ADTS is expected to make a reasonable, informed judgment within 15 minutes whether or not the operating staff has control of key safety functions from the remote safe shutdown location(s).

Transfer of plant control and the time period to establish control begins when the last licensed operator leaves the Control Room.

Control will be established at Hot Shutdown Panel C-21 if the Control Room was evacuated for reasons other than a fire or smoke. Control will be established at Fire Shutdown Panel C-10 if the Control Room was evacuated due to a fire or smoke (ref. 1, 2).

Establishment of the reactivity safety function is only applicable in Modes 1, 2 and 3. Sufficient shutdown margin has already been established once in modes 4, 5 and 6 (ref. 3).

Escalation of the emergency classification level would be via IC FG1 or CG1

Reference(s):

1. AOP 2551, "Shutdown from Outside the Control Room"
2. AOP 2579A, "Fire Procedure for Hot Standby Appendix R Fire Area R-1"
3. NRC EP FAQ 2015-014
4. NEI 99-01 HS6

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions existing that in the judgment of the DSEO warrant declaration of an Unusual Event

EAL:

HU7.1 Unusual Event

Other conditions exist which, in the judgment of the DSEO, indicate that events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate a security threat to facility protection has been initiated. **No** releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of SAFETY SYSTEMS occurs.

Mode Applicability:

All

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the DSEO to fall under the emergency classification level description for an Unusual Event.

Reference(s):

1. NEI 99-01 HU7

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions exist that in the judgment of the DSEO/ADTS warrant declaration of an Alert

EAL:

HA7.1 Alert

Other conditions exist which, in the judgment of the DSEO/ADTS, indicate that events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. **Any** releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the DSEO/ADTS to fall under the emergency classification level description for an Alert.

Reference(s):

1. NEI 99-01 HA7

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions existing that in the judgment of the DSEO/ADTS warrant declaration of a Site Area Emergency

EAL:

HS7.1 Site Area Emergency

Other conditions exist which in the judgment of the DSEO/ADTS indicate that events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage or malicious acts, (1) toward site personnel or equipment that could lead to the likely failure of or, (2) that prevent effective access to equipment needed for the protection of the public. **Any** releases are **not** expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels beyond the SITE BOUNDARY

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the DSEO/ADTS to fall under the emergency classification level description for a SITE AREA EMERGENCY.

Reference(s):

1. NEI 99-01 HS7

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions exist that in the judgment of the DSEO/ADTS warrant declaration of a General Emergency

EAL:

HG7.1 General Emergency

Other conditions exist which in the judgment of the DSEO/ADTS indicate that events are in progress or have occurred which involve actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity or HOSTILE ACTION that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels offsite for more than the immediate site area.

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should not be construed to include acts of civil disobedience or felonious acts that are not part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the DSEO/ADTS to fall under the emergency classification level description for a GENERAL EMERGENCY.

Millstone Power Station Unit 2
Emergency Action Level Technical Bases Document
Attachment 1 Emergency Action Level Technical Bases

Serial No. 18-364
Docket Nos. 50-336; 72-47
Enclosure 3; Attachment 3B

Reference(s):

1. NEI 99-01 HG7

Category M – System Malfunction

EAL Group: Hot Conditions (RCS temperature > 200°F); EALs in this category are applicable only in one or more hot operating modes.

Numerous system-related equipment failure events that warrant emergency classification have been identified in this category. They may pose actual or potential threats to plant safety.

The events of this category pertain to the following subcategories:

1. Loss of Emergency AC Power

Loss of emergency plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of onsite and offsite power sources for 4.16 KV emergency buses.

2. Loss of Vital DC Power

Loss of emergency plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of power to or degraded voltage on the 125V DC vital buses.

3. Loss of Control Room Indications

Certain events that degrade plant operator ability to effectively assess plant conditions within the plant warrant emergency classification. Losses of indicators are in this subcategory.

4. RCS Activity

During normal operation, reactor coolant fission product activity is very low. Small concentrations of fission products in the coolant are primarily from the fission of tramp uranium in the fuel clad or minor perforations in the clad itself. Any significant rise from these base-line levels (2% - 5% clad failures) is indicative of fuel failures and is covered under the Fission Product Barrier Degradation category. However, lesser amounts of clad damage may result in coolant activity exceeding Technical Specification limits. These fission products will be circulated with the reactor coolant and can be detected by coolant sampling.

5. RCS Leakage

The reactor vessel provides a volume for the coolant that covers the reactor core. The reactor pressure vessel and associated pressure piping (reactor coolant system) together provide a barrier to limit the release of radioactive material should the reactor fuel clad integrity fail. Excessive RCS leakage greater than Technical Specification limits indicates potential pipe cracks that may propagate to an extent threatening fuel clad, RCS and containment integrity.

6. RPS Failure

This subcategory includes events related to failure of the Reactor Protection System (RPS) to initiate and complete reactor trips. In the plant licensing basis, postulated failures of the RPS to complete a reactor trip comprise a specific set of analyzed events referred to as Anticipated Transient Without Scram (ATWS) events. For EAL classification, however, ATWS is intended to mean any scram failure event that does not achieve reactor shutdown. If RPS actuation fails to properly result in reactor shutdown, positive control of reactivity is at risk and could cause a threat to fuel clad, RCS and containment integrity.

7. Loss of Communications

Certain events that degrade plant operator ability to effectively communicate with essential personnel within or external to the plant warrant emergency classification.

8. Containment Failure

Failure of containment isolation capability (under conditions in which the containment is not currently challenged) warrants emergency classification. Failure of containment pressure control capability also warrants emergency classification.

9. Hazardous Event Affecting Safety Systems

Various natural and technological events that result in degraded plant safety system train performance or significant VISIBLE DAMAGE warrant emergency classification under this subcategory.

Category: M – System Malfunction
Subcategory: 1 – Loss of Emergency AC Power
Initiating Condition: Loss of **all** offsite AC power capability to emergency buses for 15 minutes or longer

EAL:

MU1.1 Unusual Event

Loss of **all** offsite AC power capability, Table M-1, to 4.16 kV emergency buses 24C and 24D for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded. SU3.1

Table M-1 AC Power Sources	
Offsite	
<ul style="list-style-type: none"> • Unit 2 Reserve Station Service Transformer (RSST) • Unit 2 Normal Station Service Transformer (NSST) • Unit 3 Normal Station Service Transformer (NSST) via Buses 34A/B to Unit 2 emergency bus 24E (if already aligned) • Unit 3 Reserve Station Service Transformer (RSST) via Buses 34A/B to Unit 2 emergency bus 24E (if already aligned) 	
Onsite	
<ul style="list-style-type: none"> • Diesel Generator 15G-12U • Diesel Generator 15G-13U • SBO Diesel Generator (if already aligned) 	

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

Table M-1 provides a list of offsite AC electrical power sources credited for this EAL. The offsite AC power sources annotated “(if already aligned)” require more than 15 minutes to establish and therefore are only credited if the source was already aligned at the time of AC power loss.

This IC addresses a prolonged loss of offsite power. The loss of offsite power sources renders the plant more vulnerable to a complete loss of power to AC emergency buses. This condition represents a potential reduction in the level of safety of the plant.

For emergency classification purposes, “capability” means that an offsite AC power source(s) is available to the emergency buses, whether or not the buses are powered from it.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of offsite power.

Escalation of the emergency classification level would be via IC MA1.

Under normal conditions, four 345 kV lines will be in service to connect the Millstone Station to the main electric system. The emergency buses powering equipment for safe shutdown are 4.16 kV emergency buses 24C (A3), 24D (A4), and 24E (A5). 4.16 kV emergency bus 24E (A5) may be fed from either 24C or 24D. The associated 480 V emergency buses are 22E (fed from 24C) and 22F (fed from 24D). MPS2 has the following methods available to provide power to the 4.16 kV emergency buses (ref. 1, 2, 3):

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 24A (A1) and 24B (A2). Bus ties connect to 4.16 kV emergency buses 24C (A3) and 24D (A4). Additionally, the NSST may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer by removing links for the main generator and providing power through the 15G-8T-2 or 15G-9T-2 breakers.
- The Unit 2 Reserve Station Service Transformer (RSST). This is the connection to the utility system for the preferred power supply. During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the Unit 2 RSST directly to 4.16 kV emergency buses 24C (A3) and 24D (A4).
- Diesel Generator 15G-12U(H7A) supplying 4.16 kV emergency bus 24C(A3).
- Diesel Generator 15G-13U(H7B) supplying 4.16 kV emergency bus 24D(A4).
- Unit 3 supplying power through bus 34A/34B to emergency bus 24E. This is controlled by procedure. However, there are power limitations when using this method to supply MPS2 buses when Millstone 3 bus 34A/34B is being used.

Reference(s):

1. Technical Specifications Section 3.8.1.2, “Electrical Power Systems Shutdown”
2. UFSAR Section 8.2, “4160-Volt and 6900-Volt Systems”
3. UFSAR Section 8.3, “Emergency Generators”
4. AOP 2580, “Degraded Voltage”
5. AOP 2583, “Loss of All AC Power During Shutdown Conditions”
6. Dwg. No. 25203-30004, “Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Buses 24A & 24B (A1 & A2)”
7. Dwg. No. 25203-30005, “Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Emergency Buses 24C, 24D (A3, A4)”
8. NEI 99-01 SU1

Category: M – System Malfunction
Subcategory: 1 – Loss of Emergency AC Power
Initiating Condition: Loss of **all but one** AC power source to emergency buses for 15 minutes or longer

EAL:

MA1.1 Alert

AC power capability, Table M-1, to 4.16 kV emergency buses 24C and 24D reduced to a single power source for ≥ 15 min. (Note 1)

AND

Any additional single power source failure will result in loss of **all** AC power to SAFETY SYSTEMS

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-1 AC Power Sources

Offsite

- Unit 2 Reserve Station Service Transformer (RSST)
- Unit 2 Normal Station Service Transformer (NSST)
- Unit 3 Normal Station Service Transformer (NSST) via Buses 34A/B to Unit 2 emergency bus 24E (if already aligned)
- Unit 3 Reserve Station Service Transformer (RSST) via Buses 34A/B to Unit 2 emergency bus 24E (if already aligned)

Onsite

- Diesel Generator 15G-12U
- Diesel Generator 15G-13U
- SBO Diesel Generator (if already aligned)

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

Table M-1 provides a list of offsite and onsite AC electrical power sources credited for this EAL. AC power sources annotated "(if already aligned)" require more than 15 minutes to establish and therefore are only credited if the source was already aligned at the time of AC power loss.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC describes a significant degradation of offsite and onsite AC power sources such that any additional single failure would result in a loss of all AC power to SAFETY SYSTEMS. In this condition, the sole AC power source may be powering one, or more than one, train of safety-related equipment. This IC provides an escalation path from IC MU1.

An "AC power source" is a source recognized in AOPs and EOPs, and capable of supplying required power to an emergency bus. Some examples of this condition are presented below.

- A loss of all offsite power with a concurrent failure of all but one emergency power source (e.g., an onsite diesel generator).
- A loss of all offsite power and loss of all emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being back-fed from Unit 3.
- A loss of emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being fed or back-fed from an offsite power source.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of power.

Escalation of the emergency classification level would be via IC MS1.

This hot condition EAL is equivalent to the cold condition EAL CU2.1.

Under normal conditions, four 345 kV lines will be in service to connect the Millstone Station to the main electric system. The emergency buses powering equipment for safe shutdown are 4.16 kV emergency buses 24C (A3), 24D (A4), and 24E (A5). 4.16 kV emergency bus 24E (A5) may be fed from either 24C or 24D. The associated 480 V emergency buses are 22E (fed from 24C) and 22F (fed from 24D). MPS2 has the following methods available to provide power to the 4.16 kV emergency buses (ref. 1, 2, 3):

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 24A (A1) and 24B (A2). Bus ties connect to 4.16 kV emergency buses 24C (A3) and 24D (A4). Additionally, the NSST may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer by removing links for the main generator and providing power through the 15G-8T-2 or 15G-9T-2 breakers.

- The Unit 2 Reserve Station Service Transformer (RSST). This is the connection to the utility system for the preferred power supply. During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the Unit 2 RSST directly to 4.16 kV emergency buses 24C (A3) and 24D (A4).
- Diesel Generator 15G-12U(H7A) supplying 4.16 kV emergency bus 24C(A3).
- Diesel Generator 15G-13U(H7B) supplying 4.16 kV emergency bus 24D(A4).
- Unit 3 supplying power through bus 34A/34B to emergency bus 24E. This is controlled by procedure. However, there are power limitations when using this method to supply MPS2 buses when Millstone 3 bus 34A/34B is being used.

Reference(s):

1. Technical Specifications Section 3.8.1.1, "Electrical Power Systems Operating"
2. UFSAR Section 8.2, "4160-Volt and 6900-Volt Systems"
3. UFSAR Section 8.3, "Emergency Generators"
4. AOP 2580, "Degraded Voltage"
5. Dwg. No. 25203-30004, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Buses 24A & 24B (A1 & A2)"
6. Dwg. No. 25203-30005, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Emergency Buses 24C, 24D (A3, A4)"
7. NEI 99-01 SA1

Category: M – System Malfunction

Subcategory: 1 – Loss of Emergency AC Power

Initiating Condition: Loss of **all** offsite power and **all** onsite AC power to emergency buses for 15 minutes or longer

EAL:

MS1.1 Site Area Emergency

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 24C and 24D for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators, provided it can be aligned within the 15 minute classification criteria.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC addresses a total loss of AC power that compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. In addition, fission product barrier monitoring capabilities may be degraded under these conditions. This IC represents a condition that involves actual or likely major failures of plant functions needed for the protection of the public.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs RG1, FG1 or MG1.

This hot condition EAL is equivalent to the cold condition EAL CA2.1.

Under normal conditions, four 345 kV lines will be in service to connect the Millstone Station to the main electric system. The emergency buses powering equipment for safe shutdown are 4.16 kV emergency buses 24C (A3), 24D (A4), and 24E (A5). 4.16 kV emergency bus 24E (A5) may be fed from either 24C or 24D. The associated 480 V emergency buses are 22E (fed from 24C) and 22F (fed from 24D). MPS2 has the following methods available to provide power to the 4.16 kV emergency buses (ref. 1, 2, 3):

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 24A (A1) and 24B (A2). Bus ties connect to 4.16 kV emergency buses 24C (A3) and 24D (A4). Additionally, the NSST may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer by removing links for the main generator and providing power through the 15G-8T-2 or 15G-9T-2 breakers.
- The Unit 2 Reserve Station Service Transformer (RSST). This is the connection to the utility system for the preferred power supply. During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the Unit 2 RSST directly to 4.16 kV emergency buses 24C (A3) and 24D (A4).
- Diesel Generator 15G-12U(H7A) supplying 4.16 kV emergency bus 24C(A3).
- Diesel Generator 15G-13U(H7B) supplying 4.16 kV emergency bus 24D(A4).
- Unit 3 supplying power through bus 34A/34B to emergency bus 24E. This is controlled by procedure. However, there are power limitations when using this method to supply MPS2 buses when Millstone 3 bus 34A/34B is being used.

Reference(s):

1. Technical Specifications Section 3.8.1.1, "Electrical Power Systems Operating"
2. UFSAR Section 8.2, "4160-Volt and 6900-Volt Systems"
3. UFSAR Section 8.3, "Emergency Generators"
4. AOP 2580, "Degraded Voltage"
5. Dwg. No. 25203-30004, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Buses 24A & 24B (A1 & A2)"
6. Dwg. No. 25203-30005, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Emergency Buses 24C, 24D (A3, A4)"
7. NEI 99-01 SS1

Category: M –System Malfunction
Subcategory: 1 – Loss of Vital AC Power
Initiating Condition: Prolonged loss of **all** offsite and **all** onsite AC power to emergency buses

EAL:

MG1.1 General Emergency

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 24C and 24D

AND

CETs > 1200°F

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators (ref. 1, 2, 3, 4).

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

The SAMGs use a CET temperature of 1200 °F as an entry condition and the temperature indicative of a badly damaged core (ref. 5)

This IC addresses a prolonged loss of all power sources to AC emergency buses that results in degraded core cooling. A loss of all AC power compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. A prolonged loss of these buses will eventually lead to a loss of one or more fission product barriers. In addition, fission product barrier monitoring capabilities may be degraded under these conditions.

For extended loss of emergency bus AC power events that do not result in a breach of the RCS barrier, this EAL should require declaration of a General Emergency prior to meeting the thresholds for IC FG1. This will allow additional time for implementation of offsite protective actions.

The EAL will require a General Emergency declaration if the loss of AC power results in parameters that indicate an inability to adequately remove decay heat from the core.

Under normal conditions, four 345 kV lines will be in service to connect the Millstone Station to the main electric system. The emergency buses powering equipment for safe shutdown are 4.16 kV emergency buses 24C (A3), 24D (A4), and 24E (A5). 4.16 kV emergency bus 24E (A5) may be fed from either 24C or 24D. The associated 480 V emergency buses are 22E (fed from 24C) and 22F (fed from 24D). MPS2 has the following methods available to provide power to the 4.16 kV emergency buses (ref. 1, 2, 3):

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 24A (A1) and 24B (A2). Bus ties connect to 4.16 kV emergency buses 24C (A3) and 24D (A4). Additionally, the NSST may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer by removing links for the main generator and providing power through the 15G-8T-2 or 15G-9T-2 breakers.
- The Unit 2 Reserve Station Service Transformer (RSST). This is the connection to the utility system for the preferred power supply. During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the Unit 2 RSST directly to 4.16 kV emergency buses 24C (A3) and 24D (A4).
- Diesel Generator 15G-12U(H7A) supplying 4.16 kV emergency bus 24C(A3).
- Diesel Generator 15G-13U(H7B) supplying 4.16 kV emergency bus 24D(A4).
- Unit 3 supplying power through bus 34A/34B to emergency bus 24E. This is controlled by procedure. However, there are power limitations when using this method to supply MPS2 buses when Millstone 3 bus 34A/34B is being used.

Reference(s):

1. Technical Specifications Section 3.8.1.1, "Electrical Power Systems Operating"
2. UFSAR Section 8.2, "4160-Volt and 6900-Volt Systems"
3. UFSAR Section 8.3, "Emergency Generators"
4. EOP 2530, "Station Blackout"
5. SAMG 4211, "Phase 1 Initial Diagnosis"
6. Dwg. No. 25203-30004, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Buses 24A & 24B (A1 & A2)"
7. Dwg. No. 25203-30005, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Emergency Buses 24C, 24D (A3, A4)"
8. NEI 99-01 SG1

Category: M – System Malfunction
Subcategory: 2 – Loss of Vital DC Power
Initiating Condition: Loss of all vital DC power for 15 minutes or longer
EAL:

MS2.1 Site Area Emergency

Indicated voltage is < 105 VDC on **both** vital 125 VDC buses 201A **AND** 201B for ≥15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

The Vital (Class 1E) 125 V DC power system consists of two physically and electrically separated redundant buses - Bus 201A and Bus 201B. Each bus has a 60 cell battery bank with an eight-hour rating of 2300 amp-hours, with an operating range of 105 Volts to 140 Volts. In the event that either battery system is out of service, it is possible through interlocked circuit breakers to feed both battery buses from one battery. Each of the vital batteries supplies control logic, field flashing and breaker control for one diesel generator. They also supply power to a backup DC motor driven starting air compressor for each diesel generator, emergency lighting, turbine driven auxiliary feedwater pump (Terry Turbine) controls (bus 201B), and 125 VDC/120 VAC inverters for vital instrumentation.

The four vital 120 VAC instrumentation panels power reactor protection, engineered safety features and vital instrumentation which is normally supplied by four physically isolated and electrically independent inverters, two of which are supplied by each of the two redundant batteries 201A and 201B (ref. 1). Each of the four vital instrumentation panels has an alternate power supply via a "zero break" static transfer switch. Vital 120 VAC instrumentation channels 1 and 2 can be fed from separate DC/AC inverters whose power source is the turbine battery (i.e. 201D). Vital 120 VAC instrumentation channels 3 and 4 can be fed from one of the two regulated AC instrument power panels.

This IC addresses a loss of vital DC power which compromises the ability to monitor and control SAFETY SYSTEMS. In modes above Cold Shutdown, this condition involves a major failure of plant functions needed for the protection of the public.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs RG1, FG1 or MG1.

This hot condition EAL equivalent of the cold condition EAL CU4.1.

Reference(s):

1. UFSAR Section 8.5, "Battery System"
2. NEI 99-01 SS8

Category: M – System Malfunction
Subcategory: 2 – Loss of Vital DC Power
Initiating Condition: Loss of all emergency AC and vital DC power sources for 15 minutes or longer

EAL:

MG2.1 General Emergency

Loss of all offsite and all onsite AC power to 4.16 kV emergency buses 24C and 24D for ≥ 15 min. (Note 1)

AND

Indicated voltage is < 105 VDC on both vital 125 VDC buses 201A **AND** 201B for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This IC addresses a concurrent and prolonged loss of both emergency AC and vital DC power. A loss of all emergency AC power compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. A loss of vital DC power compromises the ability to monitor and control SAFETY SYSTEMS. A sustained loss of both emergency AC and vital DC power will lead to multiple challenges to fission product barriers.

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators, provided it can be aligned within the 15 minute classification criteria (ref. 1, 2, 3, 4).

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

The Vital (Class 1E) 125 V DC power system consists of two physically and electrically separated redundant buses - Bus 201A and Bus 201B. Each bus has a 60 cell battery bank with an eight-hour rating of 2300 amp-hours, with an operating range of 105 Volts to 140 Volts. In the event that either battery system is out of service, it is possible through interlocked circuit breakers to feed both battery buses from one battery. Each of the vital batteries supplies control logic, field flashing and breaker control for one diesel generator. They also supply power to a backup DC motor driven starting air compressor for each diesel generator, emergency lighting, turbine driven auxiliary feedwater pump (Terry Turbine) controls (bus 201B), and 125 VDC/120 VAC inverters for vital instrumentation.

The four vital 120 VAC instrumentation panels power reactor protection, engineered safety features and vital instrumentation which is normally supplied by four physically isolated and electrically independent inverters, two of which are supplied by each of the two redundant batteries 201A and 201B (ref. 5). Each of the four vital instrumentation panels has an alternate power supply via a "zero break" static transfer switch. Vital 120 VAC instrumentation channels 1 and 2 can be fed from separate DC/AC inverters whose power source is the turbine battery (i.e. 201D). Vital 120 VAC instrumentation channels 3 and 4 can be fed from one of the two regulated AC instrument power panels.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses. The 15-minute emergency declaration clock begins at the point when both EAL thresholds are met.

Under normal conditions, four 345 kV lines will be in service to connect the Millstone Station to the main electric system. The emergency buses powering equipment for safe shutdown are 4.16 kV emergency buses 24C (A3), 24D (A4), and 24E (A5). 4.16 kV emergency bus 24E (A5) may be fed from either 24C or 24D. The associated 480 V emergency buses are 22E (fed from 24C) and 22F (fed from 24D). MPS2 has the following methods available to provide power to the 4.16 kV emergency buses (ref. 1, 2, 3):

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 24A (A1) and 24B (A2). Bus ties connect to 4.16 kV emergency buses 24C (A3) and 24D (A4). Additionally, the NSST may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer by removing links for the main generator and providing power through the 15G-8T-2 or 15G-9T-2 breakers.
- The Unit 2 Reserve Station Service Transformer (RSST). This is the connection to the utility system for the preferred power supply. During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the Unit 2 RSST directly to 4.16 kV emergency buses 24C (A3) and 24D (A4).
- Diesel Generator 15G-12U(H7A) supplying 4.16 kV emergency bus 24C(A3).
- Diesel Generator 15G-13U(H7B) supplying 4.16 kV emergency bus 24D(A4).
- Unit 3 supplying power through bus 34A/34B to emergency bus 24E. This is controlled by procedure. However, there are power limitations when using this method to supply MPS2 buses when Millstone 3 bus 34A/34B is being used.

Reference(s):

1. Technical Specifications Section 3.8.1.1, "Electrical Power Systems Operating"
2. UFSAR Section 8.2, "4160-Volt and 6900-Volt Systems"
3. UFSAR Section 8.,3 "Emergency Generators"
4. EOP 2530, "Station Blackout"
5. UFSAR Section 8.5, "Battery System"
6. Dwg. No. 25203-30004, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Buses 24A & 24B (A1 &A2)"
7. Dwg. No. 25203-30005, "Millstone Nuclear Power Station - Unit No. 2 Single Line Meter & Relay Diagram 4.16 kV Emergency Buses 24C, 24D (A3, A4)"
8. NEI 99-01 SG8

Category: M – System Malfunction
Subcategory: 3 – Loss of Control Room Indications
Initiating Condition: UNPLANNED loss of Control Room indications for 15 minutes or longer

EAL:

MU3.1 Unusual Event

An UNPLANNED event results in the inability to monitor one or more Table M-2 parameters from within the Control Room for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-2 Safety System Parameters

- Reactor power
- RCS level
- RCS pressure
- CET temperature
- Level in at least one SG
- Auxiliary feedwater flow to at least one SG

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

Applicable safety system parameters are listed in Table M-2.

The Plant Process Computer System and Safety Parameter Display System (SPDS) serve as redundant indicators which may be utilized as compensatory measures in lieu of the Control Room indicators associated with safety functions (ref. 1, 2).

This IC addresses the difficulty associated with monitoring normal plant conditions without the ability to obtain SAFETY SYSTEM parameters from within the Control Room. This condition is a precursor to a more significant event and represents a potential degradation in the level of safety of the plant.

As used in this EAL, an "inability to monitor" means that values for one or more of the listed parameters cannot be determined from within the Control Room. This situation would require a loss of all of the Control Room sources for the given parameter(s). For example, the reactor power level cannot be determined from any analog, digital and recorder source within the Control Room.

An event involving a loss of plant indications, annunciators and/or display systems is evaluated in accordance with 10 CFR 50.72 (and associated guidance in NUREG-1022) to determine if an NRC event report is required. The event would be reported if it significantly impaired the capability to perform emergency assessments. In particular, emergency assessments necessary to implement abnormal operating procedures, emergency operating procedures, and emergency plan implementing procedures addressing emergency classification, accident assessment, or protective action decision-making.

This EAL is focused on a selected subset of plant parameters associated with the key safety functions of reactivity control, core cooling and RCS heat removal. The loss of the ability to determine one or more of these parameters from within the Control Room is considered to be more significant than simply a reportable condition. In addition, if all indication sources for one or more of the listed parameters are lost, then the ability to determine the values of other SAFETY SYSTEM parameters may be impacted as well. For example, if the value for RCS water level cannot be determined from the indications and recorders on a main control board, the SPDS or the plant computer, the availability of other parameter values may be compromised as well.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation of the emergency classification level would be via IC MA3.

Reference(s):

1. UFSAR Section 7.5, "Instrumentation System"
2. AOP 2518, "Loss of Plant Process Computer"
3. NEI 99-01 SU2

Category: M – System Malfunction
Subcategory: 3 – Loss of Control Room Indications
Initiating Condition: UNPLANNED loss of Control Room indications for 15 minutes or longer with a significant transient in progress

EAL:

MA3.1 Alert

An UNPLANNED event results in the inability to monitor one or more Table M-2 parameters from within the Control Room for ≥ 15 min. (Note 1)

AND

Any significant transient is in progress, Table M-3

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-2 Safety System Parameters

- Reactor power
- RCS level
- RCS pressure
- CET temperature
- Level in at least one SG
- Auxiliary feedwater flow to at least one SG

Table M-3 Significant Transients

- Electrical load rejection > 25% full electrical load
- Reactor Trip
- SIAS actuation

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

Applicable safety system parameters are listed in Table M-2.

Significant transients are listed in Table M-3.

The Plant Process Computer System and Safety Parameter Display System (SPDS) serve as redundant indicators which may be utilized as compensatory measures in lieu of the Control Room indicators associated with safety functions (ref. 1, 2).

This IC addresses the difficulty associated with monitoring rapidly changing plant conditions during a transient without the ability to obtain SAFETY SYSTEM parameters from within the Control Room. During this condition, the margin to a potential fission product barrier challenge is reduced. It thus represents a potential substantial degradation in the level of safety of the plant.

As used in this EAL, an "inability to monitor" means that values for one or more of the listed parameters cannot be determined from within the Control Room. This situation would require a loss of all of the Control Room sources for the given parameter(s). For example, the reactor power level cannot be determined from any analog, digital and recorder source within the Control Room.

An event involving a loss of plant indications, annunciators and/or display systems is evaluated in accordance with 10 CFR 50.72 (and associated guidance in NUREG-1022) to determine if an NRC event report is required. The event would be reported if it significantly impaired the capability to perform emergency assessments. In particular, emergency assessments necessary to implement abnormal operating procedures, emergency operating procedures, and emergency plan implementing procedures addressing emergency classification, accident assessment, or protective action decision-making.

This EAL is focused on a selected subset of plant parameters associated with the key safety functions of reactivity control, core cooling and RCS heat removal. The loss of the ability to determine one or more of these parameters from within the Control Room is considered to be more significant than simply a reportable condition. In addition, if all indication sources for one or more of the listed parameters are lost, then the ability to determine the values of other SAFETY SYSTEM parameters may be impacted as well. For example, if the value for RCS water level cannot be determined from the indications and recorders on a main control board, the SPDS or the plant computer, the availability of other parameter values may be compromised as well.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation of the emergency classification level would be via ICs FS1 or RS1

Reference(s):

1. UFSAR Section 7.5, "Instrumentation System"
2. AOP 2518, "Loss of Plant Process Computer"
3. NEI 99-01 SA2

Category: M – System Malfunction

Subcategory: 4 – RCS Activity

Initiating Condition: RCS activity greater than Technical Specification allowable limits

EAL:

MU4.1 Unusual Event

Dose rate at 1 ft. from an unpressurized RCS sample \geq Table M-4

Table M-4 Tech. Spec. Coolant Activity Dose Rates	
Time > Shutdown (hrs)	mR/hr/ml
≤ 2	0.7
$> 2 - \leq 8$	0.5
> 8	0.3

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

This IC addresses a reactor coolant activity value that exceeds an allowable limit specified in Technical Specifications. This condition is a precursor to a more significant event and represents a potential degradation of the level of safety of the plant.

Per Engineering Calculation RA-0059, dose rate is assumed to result from radioactive iodines (I-131 thru I-135) in RCS in concentrations corresponding to 60 $\mu\text{Ci/gm}$ DEI-131. This value corresponds to the Technical Specification coolant activity limit for iodine spike at full power operations. The values contained in Table M-4 (Tech. Spec. Coolant Activity Dore Rates) represent expected one foot dose rates per ml of sample based on time since reactor shutdown to the time when the sample is taken. Time after shutdown values are provided to account for radioactive decay. The expected dose rate is a near linear relationship with the volume of the sample, so any volume collected can be determined by dividing the measured dose rate by the sample volume and comparing to the threshold value from Table M-4 for the applicable time frame. These dose rates assume no emergency core cooling system (ECCS) injection so there is no dilution credited which would vary coolant volume. Values in the table have been rounded for ease of use. The > 8 hour threshold is conservative up to 24 hours following reactor shutdown. After 24 hours, the expected response from radioactive iodine levels off. Therefore, the value shown for > 8 hours applies for all samples taken 8 hours or more since reactor shutdown.

The values specified in Table M-4 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Values were chosen to minimize error from the highest to lowest dose rate within each range.

It should be noted that this EALs is primarily directed toward mechanical damage to the clad not involving inadequate core cooling (ICC) sequences. Clad damage due to ICC sequences is addressed by the fuel clad and CTMT fission product barrier thresholds (Category F).

Escalation of the emergency classification level would be via IC FA1 or the Category R ICs.

Reference(s):

1. RA-0059, "Detector Response to an RCS Sample for EAL Classification of Fuel Clad Degradation and Barrier Loss"
2. Technical Specification Section 3.4.8, "RCS Specific Activity"
3. AOP 2511, "High Activity in RCS"
4. NEI 99-01 SU3

Category: M – System Malfunction

Subcategory: 4 – RCS Activity

Initiating Condition: Reactor coolant activity greater than Technical Specification allowable limits

EAL:

MU4.2 Unusual Event

Sample analysis indicates that a reactor coolant activity value is > an allowable limit specified in Technical Specification 3.4.8

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

This IC addresses a reactor coolant activity value that exceeds an allowable limit specified in Technical Specifications. This condition is a precursor to a more significant event and represents a potential degradation of the level of safety of the plant.

Escalation of the emergency classification level would be via ICs FA1 or the Category R ICs.

Reference(s):

1. Technical Specification Section 3.4.8, "RCS Specific Activity"
2. AOP 2511, "High Activity in RCS"
3. NEI 99-01 SU3

Category: M – System Malfunction
Subcategory: 5 – RCS Leakage
Initiating Condition: RCS leakage for 15 minutes or longer
EAL:

MU5.1 Unusual Event

RCS unidentified or pressure boundary leakage > 10 gpm for ≥ 15 min.

OR

RCS identified leakage > 25 gpm for ≥ 15 min.

OR

Leakage from the RCS to a location outside containment > 25 gpm for ≥ 15 min.

(Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

Basis:

The 15-minute threshold duration allows sufficient time for prompt operator actions to isolate the leakage, if possible.

Once the RCS leak rate has been quantified to be greater than the specified value, failure to isolate the leak within 15 minutes, or if known that the leak cannot be isolated within 15 minutes, from the time of leak rate quantification, requires immediate classification.

This IC addresses RCS leakage which may be a precursor to a more significant event. In this case, RCS leakage has been detected and operators, following applicable procedures, have been unable to promptly isolate the leak. This condition is considered to be a potential degradation of the level of safety of the plant.

The first and second EAL conditions are focused on a loss of mass from the RCS due to "unidentified leakage", "pressure boundary leakage" or "identified leakage" (as these leakage types are defined in the plant Technical Specifications). The third condition addresses an RCS mass loss caused by an UNISOLABLE leak through an interfacing system. These conditions thus apply to leakage into the containment, a secondary-side system (e.g., steam generator tube leakage) or a location outside of containment.

The leak rate values for each condition were selected because they are usually observable with normal Control Room indications. Lesser values typically require time-consuming calculations to determine (e.g., a mass balance calculation). The first condition uses a lower value that reflects the greater significance of unidentified or pressure boundary leakage.

The release of mass from the RCS due to the as-designed/expected operation of a relief valve does not warrant an emergency classification. An emergency classification would be required

if a mass loss is caused by a relief valve that is not functioning as designed/expected (e.g., a relief valve sticks open and the line flow cannot be isolated).

Escalation of the emergency classification level would be via ICs of Category R or F.

Reference(s):

1. Technical Specification Section 1.1, "Definitions"
2. Technical Specification Section 3.4.6, "RCS Operational Leakage"
3. AOP 2568, "Reactor Coolant Leakage"
4. NEI 99-01 SU4

Category: M – System Malfunction

Subcategory: 6 – RPS Failure

Initiating Condition: Automatic or manual trip fails to shut down the reactor

EAL:

MU6.1 Unusual Event

An automatic trip did **not** shut down the reactor as indicated by reactivity control Safety Function Status Check acceptance criteria **not** met after **any** RPS setpoint is exceeded

AND

A subsequent automatic trip or **EITHER** manual trip (RX TRIP TCBS BUTTONS **OR** MG Set Output Breakers (B0505 and B0608)) are successful in shutting down the reactor as indicated by reactivity control Safety Function Status Check acceptance criteria met (Note 8)

Note 8: A manual trip action is **any** operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does **not** include manually driving in control rods or implementation of boron injection strategies.

Mode Applicability:

1 - Power Operation, 2 - Startup

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

This EAL addresses a failure of the RPS to initiate or complete an automatic trip that results in a reactor shutdown, and either a subsequent operator manual action taken at the reactor control consoles or an automatic trip (i.e., any subsequent RPS setpoint trip) is successful in shutting down the reactor. This event is a precursor to a more significant condition and thus represents a potential degradation of the level of safety of the plant.

This EAL is applicable in Mode 2 as well as Mode 1 since the power level specified in the reactivity control Safety Function Status Check acceptance criteria is below the Mode 2 to Mode 1 transition power of 5% (ref. 1).

Following the failure on an automatic reactor trip, operators will promptly initiate manual actions at the reactor control consoles to shutdown the reactor (e.g., initiate a manual reactor trip by either pushing the RX TRIP TCBS buttons or tripping the MG Set Output Breakers (B0505 AND B0608)). If either of these manual actions are successful in shutting down the reactor, core heat generation will quickly fall to a level within the capabilities of the plant's decay heat removal systems.

A manual action at the reactor control console (Panel C04) is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor trip by either pushing the RX TRIP TCBS buttons or tripping the MG Set Output Breakers (B0505 AND B0608)). This action does not include opening the reactor trip feeder breakers locally, manually driving in control rods or implementation of boron injection

strategies. Actions taken at other locations within the Control Room, or any location outside the Control Room, are not considered to be "at the reactor control consoles".

The plant response to the failure of an automatic trip will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other concurrent plant conditions, etc. If subsequent operator manual actions taken at the reactor control consoles are also unsuccessful in shutting down the reactor, then the emergency classification level will escalate to an Alert via IC MA6. Depending upon the plant response, escalation is also possible via IC FA1. Absent the plant conditions needed to meet either IC MA6 or FA1, an Unusual Event declaration is appropriate for this event.

A reactor shutdown is determined in accordance with applicable Emergency Operating Procedure criteria (ref. 3, 4).

Should a reactor trip signal be generated as a result of plant work (e.g., RPS setpoint testing), the following classification guidance should be applied.

- If the signal causes a plant transient that should have included an automatic reactor trip and the RPS fails to automatically shutdown the reactor, then this IC and the EALs are applicable, and should be evaluated.
- If the signal does not cause a plant transient and the trip failure is determined through other means (e.g., assessment of test results), then this IC and the EALs are not applicable and no classification is warranted.

In the event that the operator identifies a reactor trip is IMMINENT and initiates a successful manual reactor trip before the automatic RPS trip setpoint is reached, no declaration is required. The successful manual trip of the reactor before it reaches its automatic trip setpoint or reactor trip signals caused by instrumentation channel failures do not lead to a potential fission product barrier loss. However, if subsequent manual reactor trip actions fail to shutdown the reactor, the event escalates to the Alert under EAL MA6.1.

Reference(s):

1. Technical Specification Table 1.1, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Protective Instrumentation"
3. EOP 2525, "Standard Post Trip Actions"
4. EOP 2526, "Reactor Trip Recovery"
5. NEI 99-01 SU5

Category: M – System Malfunction
Subcategory: 6 – RPS Failure
Initiating Condition: Automatic or manual trip fails to shut down the reactor
EAL:

MU6.2 Unusual Event

A manual trip (RX TRIP TCBS BUTTONS OR MG Set Output Breakers (B0505 and B0608)) did **not** shut down the reactor as indicated by reactivity control Safety Function Status Check acceptance criteria **not** met

AND

A subsequent manual trip OR automatic trip is successful in shutting down the reactor as indicated by reactivity control Safety Function Status Check acceptance criteria met (Note 8)

Note 8: A manual trip action is **any** operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does **not** include manually driving in control rods or implementation of boron injection strategies.

Mode Applicability:

1 - Power Operation, 2 - Startup

Definition(s):

None

Basis:

This EAL addresses a failure of a manual reactor trip that results in a reactor shutdown, and either a subsequent operator manual action taken at the reactor control consoles or an automatic trip is successful in shutting down the reactor. This event is a precursor to a more significant condition and thus represents a potential degradation of the level of safety of the plant.

This EAL is applicable in Mode 2 as well as Mode 1 since the power level specified in the reactivity control Safety Function Status Check acceptance criteria is below the Mode 2 to Mode 1 transition power of 5% (ref. 1).

If an initial manual reactor trip is unsuccessful, operators will promptly take manual actions at other location(s) away from the reactor control consoles to shutdown the reactor (e.g., initiate a manual reactor trip by opening the reactor trip feeder breakers locally) (ref. 3). However, those actions are not credited as a successful manual reactor trip for this EAL.

Depending upon several factors, the initial effort to manually trip the reactor, or a concurrent plant condition, may lead to the generation of an automatic reactor trip signal. If a subsequent automatic trip is successful in shutting down the reactor, core heat generation will quickly fall to a level within the capabilities of the plant's decay heat removal systems.

A manual action at the reactor control console (Panel C04) is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor trip by pushing the RX TRIP TCBS buttons or tripping the MG Set Output

Breakers (B0505 AND B0608)). This action does not include opening the reactor trip feeder breakers locally, manually driving in control rods or implementation of boron injection strategies. Actions taken at other locations within the Control Room, or any location outside the Control Room, are not considered to be "at the reactor control consoles".

The plant response to the failure of a manual reactor trip will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other concurrent plant conditions, etc. Depending upon the plant response, escalation is also possible via IC FA1. Absent the plant conditions needed to meet either IC MA6 or FA1, an Unusual Event declaration is appropriate for this event.

A reactor shutdown is determined in accordance with applicable Emergency Operating Procedure criteria (ref. 3, 4).

Should a reactor trip signal be generated as a result of plant work (e.g., RPS setpoint testing), the following classification guidance should be applied.

- If the signal causes a plant transient that should have included an automatic reactor trip and the RPS fails to automatically shutdown the reactor, then this IC and the EALs are applicable, and should be evaluated.
- If the signal does not cause a plant transient and the trip failure is determined through other means (e.g., assessment of test results), then this IC and the EALs are not applicable and no classification is warranted.

Reference(s):

1. Technical Specification Table 1.1, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Protective Instrumentation"
3. EOP 2525, "Standard Post Trip Actions"
4. EOP 2526, "Reactor Trip Recovery"
5. NEI 99-01 SU5

Category: M – System Malfunction
Subcategory: 2 – RPS Failure
Initiating Condition: Automatic or manual trip fails to shut down the reactor and subsequent manual actions taken at the reactor control consoles are **not** successful in shutting down the reactor

EAL:

MA6.1 Alert

An automatic or manual trip (RX TRIP TCBS BUTTONS **OR** MG Set Output Breakers (B0505 and B0608)) did **not** shut down the reactor as indicated by reactivity control Safety Function Status Check acceptance criteria **not** met

AND

Subsequent automatic or manual trip actions (RX TRIP TCBS buttons **AND** MG Set Output Breakers (B0505 and B0608)) are **not** successful in shutting down the reactor as indicated by reactivity control Safety Function Status Check acceptance criteria **not** met (Note 8)

Note 8: A manual trip action is **any** operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does **not** include manually driving in control rods or implementation of boron injection strategies.

Mode Applicability:

1 - Power Operation, 2 - Startup

Definition(s):

None

Basis:

This EAL addresses a failure of the RPS to initiate or complete an automatic reactor trip or failure of a manual reactor trip that results in a reactor shutdown, and subsequent operator manual actions taken at the reactor control consoles to shutdown the reactor are also unsuccessful. This condition represents an actual or potential substantial degradation of the level of safety of the plant. An emergency declaration is required even if the reactor is subsequently shutdown by an action taken away from the reactor control consoles since this event entails a significant failure of the RPS.

This EAL is applicable in Mode 2 as well as Mode 1 since the power level specified in the reactivity control Safety Function Status Check acceptance criteria is below the Mode 2 to Mode 1 transition power of 5% (ref. 1).

A manual action at the reactor control console (Panel C04) is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor trip by pushing the RX TRIP TCBS buttons or tripping the MG Set Output Breakers (B0505 AND B0608)). This action does not include opening the reactor trip feeder breakers locally, manually driving in control rods or implementation of boron injection strategies. If this action(s) is unsuccessful, operators would immediately pursue additional manual actions at locations away from the reactor control console (opening the reactor trip

feeder breakers locally). Actions taken at other locations within the Control Room, or any location outside the Control Room, are not considered to be "at the reactor control console".

The plant response to the failure of an automatic or manual reactor trip will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other concurrent plant conditions, etc. If the failure to shut down the reactor is prolonged enough to cause a challenge to the core cooling or RCS heat removal safety functions, the emergency classification level will escalate to a Site Area Emergency via IC MS6. Depending upon plant responses and symptoms, escalation is also possible via IC FS1. Absent the plant conditions needed to meet either IC MS6 or FS1, an Alert declaration is appropriate for this event.

It is recognized that plant responses or symptoms may also require an Alert declaration in accordance with the Category F ICs; however, this IC and EAL are included to ensure a timely emergency declaration.

A reactor shutdown is determined in accordance with applicable Emergency Operating Procedure criteria (ref. 3, 4).

Reference(s):

1. Technical Specification Table 1.1, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Protective Instrumentation"
3. EOP 2525, "Standard Post Trip Actions"
4. EOP 2526, "Reactor Trip Recovery"
5. NEI 99-01 SA5

Category: M – System Malfunction

Subcategory: 2 – RPS Failure

Initiating Condition: Inability to shut down the reactor causing a challenge to core cooling or RCS heat removal

EAL:

MS6.1 Site Area Emergency

An automatic or manual trip did **not** shut down the reactor as indicated by reactivity control Safety Function Status Check acceptance criteria **not** met

AND

All actions taken to shut down the reactor are **not** successful as indicated by reactivity control Safety Function Status Check acceptance criteria not met

AND EITHER:

- CETs >1200°F
- Applicable RCS and Core Heat Removal (HR) Safety Function Status Check acceptance criteria **not** met

Mode Applicability:

1 - Power Operation, 2 - Startup

Definition(s):

None

Basis:

This EAL addresses a failure of the RPS to initiate or complete an automatic reactor trip or failure of a manual reactor trip that results in a reactor shutdown, all subsequent operator actions to manually shutdown the reactor are unsuccessful, and continued power generation is challenging the capability to adequately remove heat from the core and/or the RCS. This condition will lead to fuel damage if additional mitigation actions are unsuccessful and thus warrants the declaration of a Site Area Emergency.

This EAL is applicable in Mode 2 as well as Mode 1 since the power level specified in the reactivity control Safety Function Status Check acceptance criteria is below the Mode 2 to Mode 1 transition power of 5% (ref. 1).

Reactor shutdown achieved by use of other actions such as opening supply breakers, emergency boration, or manually driving control rods are also credited for achieving a successful shutdown provided reactivity control Safety Function Status Check acceptance criteria are met before indications of an extreme challenge to either core cooling or heat removal exist.

In some instances, the emergency classification resulting from this IC/EAL may be higher than that resulting from an assessment of the plant responses and symptoms against the Category F ICs/EALs. This is appropriate in that the Category F ICs/EALs do not address the additional

threat posed by a failure to shut down the reactor. The inclusion of this IC and EAL ensures the timely declaration of a Site Area Emergency in response to prolonged failure to shut down the reactor.

A reactor shutdown is determined in accordance with applicable Emergency Operating Procedure criteria (ref. 3, 4).

The SAMGs use a CET temperature of 1200 °F as an entry condition and the temperature indicative of a badly damaged core (ref. 7, 8). This reading indicates temperatures within the core are sufficient to cause significant superheating of reactor coolant.

Failure to meet the applicable HR acceptance criteria indicates an extreme challenge to the ability to remove RCS heat using the steam generators (i.e., loss of an effective secondary-side heat sink).

There are three defined and potentially applicable RCS and Core Heat Removal (HR) Safety Function Status Check acceptance criteria conditions (ref. 5, 6):

- HR-1 – SG Heat Sink With No SI Operating
- HR-2 – SG Heat Sink With SI Operating
- HR-3 – Once-Through-Cooling

Escalation of the emergency classification level would be via IC RG1 or FG1.

Reference(s):

1. Technical Specification Table 1.1, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Protective Instrumentation"
3. EOP 2525, "Standard Post Trip Actions"
4. EOP 2526, "Reactor Trip Recovery"
5. EOP 2540, "Functional Recovery"
6. EOP 2540D, "Functional Recovery of Heat Removal"
7. SAMG 4211, "Phase 1 Initial Diagnosis"
8. MP-26-EPI-FAP11-001, "Core Damage Assessment: Core Exit Temperatures"
9. NEI 99-01 SS5

Category: M – System Malfunction

Subcategory: 7 – Loss of Communications

Initiating Condition: Loss of **all** onsite or offsite communications capabilities

EAL:

MU7.1 Unusual Event

Loss of **all** Table M-5 onsite communication methods

OR

Loss of **all** Table M-5 State and local agency communication methods

OR

Loss of **all** Table M-5 NRC communication methods

Table M-5 Communication Methods			
System	Onsite	State/ Local	NRC
ENRS / ARCOS		X	
Station Radio System	X	X	
Plant Phone System	X	X	
Public Address System	X		
Gaitronics / Maintenance Jacks	X		
Federal Telephone System (ENS)			X
Commercial Telephone System		X	X
Satellite Phones		X	X
Dedicated Hotlines		X	

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

This IC addresses a significant loss of on-site or offsite communications capabilities. While not a direct challenge to plant or personnel safety, this event warrants prompt notifications to State and local agencies and the NRC.

This IC should be assessed only when extraordinary means are being utilized to make communications possible (e.g., use of non-plant, privately owned equipment, relaying of on-site information via individuals or multiple radio transmission points, individuals being sent to offsite locations, etc.).

The first EAL condition addresses a total loss of the communications methods used in support of routine plant operations.

The second EAL condition addresses a total loss of the communications methods used to notify all State and local agencies of an emergency declaration. The State and local agencies referred to here are the State of Connecticut and local communities.

The third EAL addresses a total loss of the communications methods used to notify the NRC of an emergency declaration.

This hot condition EAL is equivalent to the cold condition EAL CU5.1.

Reference(s):

1. MPS Emergency Plan Section 7.9, "Communication Systems"
2. MP-26-EPI-FAP07, "Notifications and Communications"
3. NEI 99-01 SU6

Category: M – System Malfunction

Subcategory: 8 – Containment Failure

Initiating Condition: Failure to isolate containment or loss of containment pressure control

EAL:

MU8.1 Unusual Event

Any penetration is **not** closed within 15 min. of a VALID CIAS actuation signal

OR

CTMT pressure > 10 psig with < one full train of CTMT heat removal systems
(Note 11) operating per design for ≥ 15 min.

(Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 11: One full train of containment heat removal systems consist of one Containment Spray pump and two containment air recirculation units in the Containment Air Recirculation and Cooling System.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This EAL addresses a failure of one or more containment penetrations to automatically isolate (close) when required by an actuation signal. It also addresses an event that results in high containment pressure with a concurrent failure of containment pressure control systems. Absent challenges to another fission product barrier, either condition represents potential degradation of the level of safety of the plant.

For the first condition, the containment isolation signal (CIAS) must be generated as the result on an off-normal/accident condition (e.g., a safety injection or high containment pressure); a failure resulting from testing or maintenance does not warrant classification. The determination of containment and penetration status – isolated or not isolated – should be made in accordance with the appropriate criteria contained in the plant AOPs and EOPs. The 15-minute criterion is included to allow operators time to manually isolate the required penetrations, if possible.

The second condition addresses a condition where containment pressure is greater than the setpoint (9.48 psig rounded to 10 psig for readability) at which containment energy (heat) removal systems are designed to automatically actuate, and less than one full train of equipment is capable of operating per design (ref. 1, 2). One full train of containment heat

removal systems consist of one Containment Spray pump and two containment air recirculation units. The combination of required equipment can be obtained from using equipment on either emergency bus in order to meet the "one full train" requirement (ref. 3).

The 15-minute criterion is included to allow operators time to manually start equipment that may not have automatically started, if possible. The inability to start the required equipment indicates that containment heat removal/depressurization systems (e.g., containment sprays) are either lost or performing in a degraded manner.

This event would escalate to a Site Area Emergency in accordance with IC FS1 if there were a concurrent loss or potential loss of either the Fuel Clad or RCS fission product barriers.

Reference(s):

1. UFSAR Section 6.4, "Containment Spray System"
2. UFSAR Section 7.3, "Engineered Safety Features Actuation System"
3. UFSAR Section 6.5, "Containment Air Recirculation and Cooling System"
4. NEI 99-01 SU7

Category: M – System Malfunction
Subcategory: 9 – Hazardous Event Affecting Safety Systems
Initiating Condition: Hazardous event affecting SAFETY SYSTEMS needed for the current operating mode

EAL:

MA9.1 Alert

The occurrence of **any** Table M-6 hazardous event

AND

Event damage has caused indications of degraded performance on one train of a SAFETY SYSTEM needed for the current operating mode

AND EITHER:

- Event damage has caused indications of degraded performance to the second train of the SAFETY SYSTEM needed for the current operating mode
- Event damage has resulted in **VISIBLE DAMAGE** to the second train of the SAFETY SYSTEM needed for the current operating mode

(Notes 9, 10)

Note 9: If the affected SAFETY SYSTEM train was already inoperable or out of service before the hazardous event occurred, then emergency classification is **not** warranted.

Note 10: If the hazardous event **only** resulted in **VISIBLE DAMAGE**, with **no** indications of degraded performance to at least one train of a SAFETY SYSTEM, then this emergency classification is **not** warranted.

Table M-6 Hazardous Events
<ul style="list-style-type: none">• Seismic event (earthquake)• Internal or external FLOODING event• High winds or tornado strike• FIRE• EXPLOSION• Other events with similar hazard characteristics as determined by the DSEO/ADTS

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

EXPLOSION - A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding,

arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

VISIBLE DAMAGE - Damage to a SAFETY SYSTEM train that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected SAFETY SYSTEM train.

Basis:

This IC addresses a hazardous event that causes damage to SAFETY SYSTEMS needed for the current operating mode. In order to provide the appropriate context for consideration of an ALERT classification, the hazardous event must have caused indications of degraded SAFETY SYSTEM performance in one train, and there must be either indications of performance issues with the second SAFETY SYSTEM train or VISIBLE DAMAGE to the second train such that the potential exists for this second SAFETY SYSTEM train to have performance issues. In other words, in order for this EAL to be classified, the hazardous event must occur, at least one SAFETY SYSTEM train must have indications of degraded performance, and the second SAFETY SYSTEM train must have indications of degraded performance or VISIBLE DAMAGE such that the potential exists for performance issues. Note that this second SAFETY SYSTEM train is from the same SAFETY SYSTEM that has indications of degraded performance; commercial nuclear power plants are designed to be able to support single system issues without compromising public health and safety from radiological events.

Indications of degraded performance addresses damage to a SAFETY SYSTEM train that is in service/operation since indications for it will be readily available. The indications of degraded performance should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

VISIBLE DAMAGE addresses damage to a SAFETY SYSTEM train that is not in service/operation and that potentially could cause performance issues. Operators will make this determination based on the totality of available event and damage report information. This is intended to be a brief assessment not requiring lengthy analysis or quantification of the

damage. This VISIBLE DAMAGE should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

An event affecting equipment common to two or more trains of a safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the common equipment) should be classified as an Alert under this EAL, as appropriate to the plant mode. By affecting the functionality of multiple trains of a safety system, the loss of the common equipment effectively meets the two-train impact criteria that underlie the EALs and bases.

An event affecting a single-train safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the one train) would not be classified under this EAL because the two-train impact criteria that underlie the EALs and bases would not be met. If an event affects a single-train safety system, then the emergency classification should be made based on plant parameters/symptoms meeting the EALs for another IC. Depending upon the circumstances, classification may also occur based on Shift Manager/DSEO/ADTS judgement.

An event that affects two trains of a safety system (e.g., one train has indications of degraded performance and the other VISIBLE DAMAGE) that also has one or more additional trains should be classified as an Alert under this EAL, as appropriate to the plant mode. This approach maintains consistency with the two-train impact criteria that underlie the EALs and bases, and is warranted because the event was severe enough to affect the functionality of two trains of a safety system despite plant design criteria associated with system and system train separation and protection. Such an event may have caused other plant impacts that are not immediately apparent.

Escalation of the emergency classification level would be via IC FS1 or RS1.

This hot condition EAL is equivalent of the cold condition EAL CA6.1.

Reference(s):

1. AOP 2560, "Storms, High Winds and High Tides"
2. AOP 2562, "Earthquake"
3. EP FAQ 2016-002
4. NEI 99-01 SA9

Background

NEI 99-01 Revision 6 ICs AA3 and HA5 prescribe declaration of an Alert based on IMPEDED access to rooms or areas (due to either area radiation levels or hazardous gas concentrations) where equipment necessary for normal plant operations, cooldown or shutdown is located. These areas are intended to be plant operating mode dependent. Specifically the Developers Notes for AA3 and HA5 states:

The "site-specific list of plant rooms or areas with entry-related mode applicability identified" should specify those rooms or areas that contain equipment which require a manual/local action as specified in operating procedures used for normal plant operation, cooldown and shutdown. Do not include rooms or areas in which actions of a contingent or emergency nature would be performed (e.g., an action to address an off-normal or emergency condition such as emergency repairs, corrective measures or emergency operations). In addition, the list should specify the plant mode(s) during which entry would be required for each room or area.

The list should not include rooms or areas for which entry is required solely to perform actions of an administrative or record keeping nature (e.g., normal rounds or routine inspections).

Further, as specified in IC HA5:

The list need not include the Control Room if adequate engineered safety/design features are in place to preclude a Control Room evacuation due to the release of a hazardous gas. Such features may include, but are not limited to, capability to draw air from multiple air intakes at different and separate locations, inner and outer atmospheric boundaries, or the capability to acquire and maintain positive pressure within the Control Room envelope.

MPS2 Table R-2 and H-2 Bases

A review of station operating procedures identified the following mode dependent in-plant actions and associated areas that are required for normal plant operation, cooldown or shutdown:

In-Plant Actions (MPS2)	Safe Shutdown Area	Modes
Perform "Boration During Cooldown"	AB EI 14'6" BA Batching Tank AB EI -5'6" West Area AB EI -45'6" General Area	3
Commence SDC Preps: Align RB to SDC Reduce Non-Essential RB Loads Place In-Service RB Hx in "Summer Control Perform Line-up 2310-001 Place SI-306 in RC Open SI-400 Obtain SDC Boron Commence Warmup & Pressurization Throttle LPSI Min Flow for Press. Control Throttle RB for SDC Verify LPSI Suction Pressure Realign to RWST (if necessary) Close SI-709	AB EI -45'6" General Area EB EI -45'6" "A" & "B" ESF Rooms AB EI -5'6" East Near SFP Cooling AB EI -25'6" RB Hx Area EB EI -5'6" East & West Piping Rooms AB EI 14'6" Near B61	3
Install Closing Coils for All SIT Isolation Valves	AB EI 14'6" B51 & B61 Enclosures	3
Pressure < 1400 psia, Place 2 nd letdown FCV/PCV	EB EI -5'6" West Pipe Penetration Room	3
Close 2-SI-651 Disconnect	AB EI 14'6" SE Across From MCC B51	4
Initiate SDC: Close LRR-39 Vent Throttle SI-444 or 432 Close SI-444 or 432 Throttle LPSI Min. Flow Close SI-400 Adjust Temp.	EB EI -45'6" "A" ESF Room EB EI -45'6" "A" & "B" ESF Rooms EB EI -45'6" "A" & "B" ESF Rooms EB EI -45'6" "B" ESF Room AB EI 14'6" Near B61 AB EI -45'6" General Area	4
Establish Excess Letdown	EB EI -5'6" West Pipe Penetration Room EB EI -45'6" "A" ESF Room SW AB EI -5'6" VCT Block Wall Area	5
Remove Gagging Device from 2-SI 468	EB EI -5'6" West Area	5

Control Room ventilation systems have adequate engineered safety/design features in place to preclude a Control Room evacuation due to the external release of a hazardous gas (UFSAR Section 9.9.11). Therefore, the Control Room is not included in this assessment or in Table H-2.

Ref: OP 2201, "Plant Heatup"
OP 2204, "Load Changes"
OP 2206, "Redactor Shutdown"
OP 2207, "Plant Cooldown"

Table R-2 & H-2 Results

Table R-2/H-2 Safe Operation & Shutdown Rooms/Areas		
Room/Area		Mode
Aux. Building EI -5'6"	West Area	3
Aux. Building EI -5'6"	East Near SFP Cooling	
Aux. Building EI 14'6"	B51 & B61 Enclosures	
Aux. Building EI 14'6"	Boric Acid Batching Tank	
Aux. Building EI -25'6"	RB Hx Area	
Enc. Building EI -5'6"	East Pipe Penetration	
Aux. Building EI 14'6"	By B61	3, 4
Enc. Building EI -45'6"	"A" & "B" ESF Rooms	
Aux. Building EI -45'6"	General Area	
Enc. Building EI -5'6"	West Pipe Penetration	3, 5
Aux. Building EI 14'6"	SE Across From MCC B51	4
Aux. Building EI -5'6"	VCT Block Wall Area	5
Enc. Building EI -45'6"	"A" ESF Room	

ATTACHMENT 3C

MPS3 EAL TECHNICAL BASES DOCUMENT (Final)

**Dominion Energy Nuclear Connecticut, Inc. (DENC)
Millstone Power Station Unit 3 and ISFSI**

**Emergency Action Level Technical Bases Document
Millstone Power Station – Unit 3**

(Final)

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1.0 INTRODUCTION

This document provides an explanation and rationale for each Emergency Action Level (EAL) included in the NEI 99-01, Revision 6, EAL Upgrade Project for Millstone Power Station Unit 3 (MPS3). It should be used to facilitate review of the MPS3 EALs and provide historical documentation for future reference. Decision-makers responsible for implementation of MP-26-EPI-FAP06, Classification and PARs, may use this document as a technical reference in support of EAL interpretation. This information may assist the Director of Station Emergency Operations (DSEO)/Assistant Director Technical Support (ADTS) in making classifications, particularly those involving judgment or multiple events. The basis information may also be useful in training and for explaining event classifications to off-site officials.

The expectation is that emergency classifications are to be made as soon as conditions are present and recognizable for the classification, but within 15 minutes or less in all cases of conditions present. Use of this document for assistance is not intended to delay the emergency classification.

Since the information in a basis document can affect emergency classification decision-making (e.g., the DSEO/ADTS refers to it during an event), the NRC staff expects that changes to the basis document will be evaluated in accordance with the provisions of 10 CFR 50.54(q). For Dominion Energy sites, a 10 CFR 50.54(q)(3) screening/evaluation will be performed to evaluate changes to this document.

Dominion Energy fleet procedure CM-AA-400, "10 CFR 50.59 and 10 CFR 72.48 – Changes, Tests and Experiments" provides a method to determine the impacts to licensing basis documents when changes are proposed to procedures, including changes to Abnormal Operating Procedures (AOPs) and Emergency Operating Procedures (EOPs). The 50.59/72.48 applicability review specifically requires that the effect of a proposed procedure change on the Emergency Plan (and associated EALs) be reviewed/assessed. When impacts to the Emergency Plan are identified, a separate review in accordance to 10 CFR 50.54(q) will be performed to determine the acceptability of the proposed procedure change.

2.0 DISCUSSION

2.1 Background

EALs are the plant-specific indications, conditions or instrument readings that are utilized to classify emergency conditions defined in the Millstone Power Station Emergency Plan.

In 1992, the NRC endorsed NUMARC/NESP-007, "Methodology for Development of Emergency Action Levels" as an alternative guidance to the original Standard Review Plan and NUREG-0654 EAL schemes.

NEI 99-01 (NUMARC/NESP-007), Revisions 4 and 5, were subsequently issued for industry implementation. Enhancements over earlier revisions included:

- Consolidating the system malfunction initiating conditions and example emergency action levels which address conditions that may be postulated to occur during plant shutdown conditions.

- Initiating conditions and example emergency action levels that fully address conditions that may be postulated to occur at permanently Defueled Stations and Independent Spent Fuel Storage Installations (ISFSIs).
- Simplifying the fission product barrier EAL threshold for a Site Area Emergency.

Subsequently, Revision 6 of NEI 99-01 has been issued which incorporates resolutions to numerous implementation issues including the NRC EAL Frequently Asked Questions (FAQs). Using NEI 99-01 Revision 6, "Methodology for the Development of Emergency Action Levels for Non-Passive Reactors," November 2012 (ref. 4.1.1), MPS3 conducted an EAL implementation upgrade project that produced the EALs discussed herein.

2.2 Fission Product Barriers

Fission product barrier thresholds represent threats to the defense in depth design concept that precludes the release of radioactive fission products to the environment. This concept relies on multiple physical barriers, any one of which, if maintained intact, precludes the release of significant amounts of radioactive fission products to the environment.

Many of the EALs derived from the NEI methodology are fission product barrier threshold based. That is, the conditions that define the EALs are based upon thresholds that represent the loss or potential loss of one or more of the three fission product barriers. "Loss" and "Potential Loss" signify the relative damage and threat of damage to the barrier. A "Loss" threshold means the barrier no longer assures containment of radioactive materials. A "Potential Loss" threshold implies a greater probability of barrier loss and reduced certainty of maintaining the barrier.

The primary fission product barriers are:

- Fuel Clad Barrier (FC): The Fuel Clad Barrier consists of the cladding material that contains the fuel pellets.
- Reactor Coolant System Barrier (RCS): The RCS Barrier includes the RCS primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary isolation valves.
- Containment Barrier (CTMT): The Containment Barrier includes the containment building and connections up to and including the outermost containment isolation valves. This barrier also includes the main steam, feedwater, and blowdown line extensions outside the containment building up to and including the outermost secondary side isolation valve. Containment Barrier thresholds are used as criteria for escalation of the Emergency Classification Level (ECL) from an Alert to a Site Area Emergency or a General Emergency.

2.3 Fission Product Barrier Classification Criteria

The following criteria are the bases for event classification related to fission product barrier loss or potential loss:

Alert:

Any loss or any potential loss of either Fuel Clad or RCS Barrier

Site Area Emergency:

Loss or potential loss of any two barriers

General Emergency:

Loss of any two barriers and loss or potential loss of the third barrier

2.4 EAL Organization

The MPS3 EAL scheme includes the following features:

- Division of the EAL set into three broad groups:
 - EALs applicable under any plant operational modes – This group would be reviewed by the EAL-user any time emergency classification is considered.
 - EALs applicable only under hot operational modes – This group would only be reviewed by the EAL-user when the plant is in Hot Shutdown, Hot Standby, Startup, or Power Operation mode.
 - EALs applicable only under cold operating modes – This group would only be reviewed by the EAL-user when the plant is in Cold Shutdown, Refueling or Defueled mode.

The purpose of the groups is to avoid review of hot condition EALs when the plant is in a cold condition and avoid review of cold condition EALs when the plant is in a hot condition. This approach significantly minimizes the total number of EALs that must be reviewed by the EAL-user for a given plant condition, reduces EAL-user reading burden and, thereby, speeds identification of the EAL that applies to the emergency.

- Within each group, assignment of EALs to categories and subcategories:

Category and subcategory titles are selected to represent conditions that are operationally significant to the EAL-user. The MPS3 EAL categories are aligned to and represent the NEI 99-01 "Recognition Categories." Subcategories are used in the MPS3 scheme as necessary to further divide the EALs of a category into logical sets of possible emergency classification thresholds. The MPS3 EAL categories and subcategories are listed below.

The EALs are pre-determined, site-specific, observable thresholds for determining whether an Initiating Condition (IC) has occurred and that an EAL threshold was met or exceeded. Thus failure to evaluate the IC and EAL together could result in an incorrect declaration.

The primary tool for determining the emergency classification level is the EAL Classification Matrix. The user of the EAL Classification Matrix may (but is not required to) consult the EAL technical bases in order to obtain additional information concerning the EALs under classification consideration. The user should consult Section 3.0 and Attachment 1 of this document for such information.

EAL Groups, Categories and Subcategories

EAL Group/Category	EAL Subcategory
<u>Any Operating Mode:</u>	
R – Abnormal Rad Levels / Rad Effluent	1 – Radiological Effluent 2 – Irradiated Fuel Event 3 – Area Radiation Levels
H – Hazards and Other Conditions Affecting Plant Safety	1 – Security 2 – Seismic Event 3 – Natural or Technological Hazard 4 – Fire 5 – Hazardous Gas 6 – Control Room Evacuation 7 – DSEO/ADTS Judgment
E – Independent Spent Fuel Storage Installation (ISFSI)	1 – Confinement Boundary
<u>Hot Conditions:</u>	
M – System Malfunction	1 – Loss of Emergency AC Power 2 – Loss of Vital DC Power 3 – Loss of Control Room Indications 4 – RCS Activity 5 – RCS Leakage 6 – RPS Failure 7 – Loss of Communications 8 – Containment Failure 9 – Hazardous Event Affecting Safety Systems
F – Fission Product Barrier Degradation	None
<u>Cold Conditions:</u>	
C – Cold Shutdown / Refueling System Malfunction	1 – RCS Level 2 – Loss of Emergency AC Power 3 – RCS Temperature 4 – Loss of Vital DC Power 5 – Loss of Communications 6 – Hazardous Event Affecting Safety Systems

2.5 Technical Bases Information

EAL technical bases are provided in Attachment 1 for each EAL according to EAL group (Any, Hot, Cold), EAL category (R, C, E, F, H and M) and EAL subcategory. A summary explanation of each category and subcategory is given at the beginning of the technical bases discussions of the EALs included in the category. For each EAL, the following information is provided:

Category Letter & Title

Subcategory Number & Title

Initiating Condition (IC)

Site-specific description of the generic IC given in NEI 99-01 Rev. 6.

EAL Identifier (enclosed in rectangle)

Each EAL is assigned a unique identifier to support accurate communication of the emergency classification to onsite and offsite personnel. Four characters define each EAL identifier:

1. First character (letter): Corresponds to the EAL category as described above (R, C, E, F, H or M)
2. Second character (letter): The emergency classification (G, S, A or U)
 - G = General Emergency
 - S = Site Area Emergency
 - A = Alert
 - U = Unusual Event
3. Third character (number): Subcategory number within the given category. Subcategories are sequentially numbered beginning with the number one (1). If a category does not have a subcategory, this character is assigned the number one (1).
4. Fourth character (number): The numerical sequence of the EAL within the EAL subcategory. If the subcategory has only one EAL, it is given the number one (1).

Classification (enclosed in rectangle):

General Emergency (G), Site Area Emergency (S), Alert (A) or Unusual Event (U).

EAL (enclosed in rectangle)

Exact wording of the EAL as it appears in the EAL Classification Matrix.

Notes (as applicable)

Mode Applicability

One or more of the following plant operating conditions comprise the mode to which each EAL is applicable: 1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown, 5 - Cold Shutdown, 6 - Refueling, DEF - Defueled, or All. (See Section 2.6 for operating mode definitions)

Definition(s):

If the EAL wording contains a defined term, the definition of the term is included in this section. These definitions can also be found in Section 5.1.

Basis:

An EAL basis section that provides MPS3-relevant information concerning the EAL as well as a description of the rationale for the EAL as provided in NEI 99-01 Rev. 6.

Reference(s):

Source documentation from which the EAL is derived

2.6 Operational Mode Applicability

MODE	K_{eff}	THERMAL POWER *	T_{AVG}
1. Power Operation	≥ 0.99	$> 5\%$	$\geq 350^{\circ}\text{F}$
2. Startup	≥ 0.99	$\leq 5\%$	$\geq 350^{\circ}\text{F}$
3. Hot Standby	< 0.99	0	$\geq 350^{\circ}\text{F}$
4. Hot Shutdown	< 0.99	0	$> 200^{\circ}\text{F} \text{ \& } < 350^{\circ}\text{F}$
5. Cold Shutdown	< 0.98	0	$\leq 200^{\circ}\text{F}$
6. Refueling **	≤ 0.95	0	$\leq 140^{\circ}\text{F}$
Defueled	NA	NA	NA – no fuel in reactor vessel

* Excluding Decay Heat

** Fuel in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed

The plant operating mode that exists at the time that the event occurs (prior to any protective system or operator action being initiated in response to the condition) should be compared to the mode applicability of the EALs. If a lower or higher plant operating mode is reached before the emergency classification is made, the declaration shall be based on the mode that existed at the time the event occurred.

3.0 GUIDANCE ON MAKING EMERGENCY CLASSIFICATIONS

3.1 General Considerations

When making an emergency classification, the DSEO/ADTS must consider all information having a bearing on the proper assessment of an Initiating Condition (IC). This includes the EAL plus any associated Operational Mode Applicability, Notes, and the informing basis information. In the Category F matrices, EALs are based on loss or potential loss of Fission Product Barrier thresholds.

3.1.1 Classification Timeliness

NRC regulations require the licensee to establish and maintain the capability to assess, classify, and declare an emergency condition within 15 minutes after the availability of indications to plant operators that an emergency action level has been exceeded and to promptly declare the emergency condition as soon as possible following identification of the appropriate emergency classification level. The NRC staff has provided guidance on implementing this requirement in NSIR/DPR-ISG-01, "Interim Staff Guidance, Emergency Planning for Nuclear Power Plants" (ref. 4.1.8).

3.1.2 Valid Indications

All emergency classification assessments shall be based upon valid indications, reports or conditions. A valid indication, report, or condition, is one that has been verified through appropriate means such that there is no doubt regarding the indicator's operability, the condition's existence, or the report's accuracy.

An indication, report, or condition is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

3.1.3 Imminent Conditions

For ICs and EALs that have a stipulated time duration (e.g., 15 minutes, 30 minutes, etc.), the DSEO/ADTS should not wait until the applicable time has elapsed, but should declare the event as soon as it is determined that the condition has exceeded, or will likely exceed, the applicable time. If an ongoing radiological release is detected and the release start time is unknown, it should be assumed that the release duration specified in the IC/EAL has been exceeded, absent data to the contrary.

3.1.4 Planned vs. Unplanned Events

A planned work activity that results in an expected event or condition which meets or exceeds an EAL does not warrant an emergency declaration provided that: 1) the activity proceeds as planned, and 2) the plant remains within the limits imposed by the operating license. Such activities include planned work to test, manipulate, repair, maintain or modify a system or component. In these cases, the controls associated with the planning, preparation and execution of the work will ensure that compliance is maintained with all aspects of the operating license provided that the activity proceeds and concludes as expected. Events or conditions of this type may be subject to the reporting requirements of 10CFR 50.72 (ref. 4.1.4).

3.1.5 Classification Based on Analysis

The assessment of some EALs is based on the results of analyses that are necessary to ascertain whether a specific EAL threshold has been exceeded (e.g., dose assessments, chemistry sampling, RCS leak rate calculation, etc.). For these EALs, the wording of the EAL or associated basis discussion will identify the necessary analysis. In these cases, the 15-minute declaration period starts with the availability of the analysis results that show the threshold to be exceeded (i.e., this is the time that the EAL information is first available). The NRC expects licensees to establish the capability to initiate and complete EAL-related analyses within a reasonable period of time (e.g., maintain the necessary expertise on-shift).

3.1.6 DSEO/ADTS Judgment

While the EALs have been developed to address a full spectrum of possible events and conditions which may warrant emergency classification, a provision for classification based on operator/management experience and judgment is still necessary. The NEI 99-01 EAL scheme provides the DSEO/ADTS with the ability to classify events and conditions based upon judgment using EALs that are consistent with the Emergency Classification Level (ECL) definitions (refer to Category H). The DSEO/ADTS will need to determine if the effects or consequences of the event or condition reasonably meet or exceed a particular ECL definition. A similar provision is incorporated in the Fission Product Barrier Tables; judgment may be used to determine the status of a fission product barrier.

3.2 Classification Methodology

To make an emergency classification, the user will compare an event or condition (i.e., the relevant plant indications and reports) to an EAL(s) and determine if the EAL has been met or exceeded. The evaluation of an EAL must be consistent with the related Operating Mode Applicability and Notes. If an EAL has been met or exceeded, the associated IC is likewise met, the emergency classification process "clock" starts, and the ECL must be declared in accordance with plant procedures no later than 15 minutes after the process "clock" started.

When assessing an EAL that specifies a time duration for the potentially classifiable condition, the "clock" for the EAL time duration runs concurrently with the emergency classification process "clock." For a full discussion of this timing requirement, refer to NSIR/DPR-ISG-01 (ref. 4.1.8).

3.2.1 Classification of Multiple Events and Conditions

When multiple emergency events or conditions are present, the user will identify all met or exceeded EALs. The highest applicable ECL identified during this review is declared. For example:

- If an Alert EAL and a Site Area Emergency EAL are met, whether at one unit or at two units, a Site Area Emergency should be declared.

There is no "additive" effect from multiple EALs meeting the same ECL. For example:

- If two Alert EALs are met, whether at one unit or at two units, an Alert should be declared.

Related guidance concerning classification of rapidly escalating events or conditions is provided in Regulatory Issue Summary (RIS) 2007-02, *Clarification of NRC Guidance for Emergency Notifications During Quickly Changing Events* (ref. 4.1.2).

3.2.2 Consideration of Mode Changes During Classification

The mode in effect at the time that an event or condition occurred, and prior to any plant or operator response, is the mode that determines whether or not an IC is applicable. If an event or condition occurs, and results in a mode change before the emergency is declared, the emergency classification level is still based on the mode that existed at the time that the event or condition was initiated (and not when it was declared). Once a different mode is reached, any new event or condition, not related to the original event or condition, requiring emergency classification should be evaluated against the ICs and EALs applicable to the operating mode at the time of the new event or condition.

For events that occur in Cold Shutdown or Refueling, escalation is via EALs that are applicable in the Cold Shutdown or Refueling modes, even if Hot Shutdown (or a higher mode) is entered during the subsequent plant response. In particular, the fission product barrier EALs are applicable only to events that initiate in the Hot Shutdown mode or higher.

3.2.3 Classification of Imminent Conditions

Although EALs provide specific thresholds, the DSEO/ADTS must remain alert to events or conditions that could lead to meeting or exceeding an EAL within a relatively short period of time (i.e., a change in the ECL is IMMINENT). If, in the judgment of the DSEO/ADTS, meeting an EAL is IMMINENT, the emergency classification should be made as if the EAL has been met. While applicable to all emergency classification levels, this approach is particularly important at the higher emergency classification levels since it provides additional time for implementation of protective measures.

3.2.4 Emergency Classification Level Upgrading and Downgrading

An ECL may be downgraded when the event or condition that meets the highest IC and EAL no longer exists, and other site-specific downgrading requirements are met. If downgrading the ECL is deemed appropriate, the new ECL would then be based on a lower applicable IC(s) and EAL(s). The ECL may also simply be terminated.

As noted above, guidance concerning classification of rapidly escalating events or conditions is provided in RIS 2007-02 (ref. 4.1.2).

3.2.5 Classification of Short-Lived Events

Event-based ICs and EALs define a variety of specific occurrences that have potential or actual safety significance. By their nature, some of these events may be short-lived and, thus, over before the emergency classification assessment can be completed. If an event occurs that meets or exceeds an EAL, the associated ECL must be declared regardless of its continued presence at the time of declaration. Examples of such events include an earthquake or a failure of the reactor protection system to automatically scram the reactor followed by a successful manual scram.

3.2.6 Classification of Transient Conditions

Many of the ICs and/or EALs employ time-based criteria. These criteria will require that the IC/EAL conditions be present for a defined period of time before an emergency declaration is

warranted. In cases where no time-based criterion is specified, it is recognized that some transient conditions may cause an EAL to be met for a brief period of time (e.g., a few seconds to a few minutes). The following guidance should be applied to the classification of these conditions.

EAL momentarily met during expected plant response - In instances in which an EAL is briefly met during an expected (normal) plant response, an emergency declaration is not warranted provided that associated systems and components are operating as expected, and operator actions are performed in accordance with procedures.

EAL momentarily met but the condition is corrected prior to an emergency declaration - If an operator takes prompt manual action to address a condition, and the action is successful in correcting the condition prior to the emergency declaration, then the applicable EAL is not considered met and the associated emergency declaration is not required. For illustrative purposes, consider the following example:

An ATWS occurs and the high pressure ECCS systems fail to automatically start. The plant enters an inadequate core cooling condition (a potential loss of both the Fuel Clad and RCS Barriers). If an operator manually starts a high pressure ECCS system in accordance with an EOP step and clears the inadequate core cooling condition prior to an emergency declaration, then the classification should be based on the ATWS only.

It is important to stress that the 15-minute emergency classification assessment period (process clock) is not a "grace period" during which a classification may be delayed to allow the performance of a corrective action that would obviate the need to classify the event. Emergency classification assessments must be deliberate and timely, with no undue delays. The provision discussed above addresses only those rapidly evolving situations when an operator is able to take a successful corrective action prior to the DSEO/ADTS completing the review and steps necessary to make the emergency declaration. This provision is included to ensure that any public protective actions resulting from the emergency classification are truly warranted by the plant conditions.

3.2.7 After-the-Fact Discovery of an Emergency Event or Condition

In some cases, an EAL may be met but the emergency classification was not made at the time of the event or condition. This situation can occur when personnel discover that an event or condition existed which met an EAL, but no emergency was declared, and the event or condition no longer exists at the time of discovery. This may be due to the event or condition not being recognized at the time or an error that was made in the emergency classification process.

In these cases, no emergency declaration is warranted; however, the guidance contained in NUREG-1022 (ref. 4.1.3) is applicable. Specifically, the event should be reported to the NRC in accordance with 10CFR 50.72 (ref. 4.1.4) within one hour of the discovery of the undeclared event or condition. The licensee should also notify appropriate State and local agencies in accordance with the agreed upon arrangements.

3.2.8 Retraction of an Emergency Declaration

Guidance on the retraction of an emergency declaration reported to the NRC is discussed in NUREG-1022 (ref. 4.1.3).

4.0 REFERENCES

4.1 Developmental

- 4.1.1 NEI 99-01, Revision 6, "Methodology for the Development of Emergency Action Levels for Non-Passive Reactors," (ADAMS Accession No. ML12326A805)
- 4.1.2 RIS 2007-02, "Clarification of NRC Guidance for Emergency Notifications During Quickly Changing Events", February 2, 2007.
- 4.1.3 NUREG-1022, "Event Reporting Guidelines: 10CFR50.72 and 50.73"
- 4.1.4 10 CFR 50.72, "Immediate Notification Requirements for Operating Nuclear Power Reactors"
- 4.1.5 10 CFR 50.73, "License Event Report System"
- 4.1.6 MPS3 Technical Specifications Table 1.2, "Operational Modes"
- 4.1.7 MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
- 4.1.8 NSIR/DPR-ISG-01, "Interim Staff Guidance, Emergency Planning for Nuclear Power Plants"
- 4.1.9 MPS Emergency Plan
- 4.1.10 NUH-003, "UFSAR for the Horizontal Modular Storage System for Irradiated Fuel"
- 4.1.11 MPS2 ETE-NAF-2010-0004, "MPS Unit 2 ISFSI 10 CFR72.212 Evaluation Report"
- 4.1.12 MPS3 FSAR Chapter 9.1, "Fuel Storage and Handling"
- 4.1.13 SP 3613F.3-001, "Containment Boundary During Movement of Fuel Within The Containment Building"

4.2 Implementing

- 4.2.1 MP-26-EPI-FAP06, "Classification and PARs"
- 4.2.2 NEI 99-01 Rev. 6 to MPS3 EAL Comparison Matrix
- 4.2.3 MPS3 EAL Matrix

5.0 DEFINITIONS, ACRONYMS & ABBREVIATIONS

5.1 Definitions (ref. 4.1.1 except as noted)

Selected terms used in Initiating Condition, EAL statements and EAL bases are set in all capital letters (e.g., ALL CAPS). These are defined terms that have specific meanings as used in this document. The definitions of these terms are provided below.

ALERT

Events are in progress, or have occurred, which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. **Any** releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.

CONFINEMENT BOUNDARY

The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielding Canister (DSC) (ref. 4.1.10).

CONTAINMENT CLOSURE

The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist (ref. 4.1.13):

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

EMERGENCY ACTION LEVEL (EAL)

A pre-determined, site-specific, observable threshold for an INITIATING CONDITION that, when met or exceeded, places the plant in a given emergency classification level.

EMERGENCY CLASSIFICATION LEVEL (ECL)

One of a set of names or titles established by the US Nuclear Regulatory Commission (NRC) for grouping off-normal events or conditions according to (1) potential or actual effects or consequences, and (2) resulting onsite and offsite response actions. The emergency classification levels, in ascending order of severity, are:

- Unusual Event (UE)
- Alert
- Site Area Emergency (SAE)
- General Emergency (GE)

EXPLOSION

A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding, arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FAULTED

The term applied to a steam generator that has a steam leak on the secondary side of sufficient size to cause an uncontrolled drop in steam generator pressure or the steam generator to become completely depressurized.

FIRE

Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FISSION PRODUCT BARRIER THRESHOLD

A pre-determined, site-specific, observable threshold indicating the loss or potential loss of a fission product barrier.

FLOODING

A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

GENERAL EMERGENCY

Events are in progress or have occurred which involve actual or **IMMINENT** substantial core degradation or melting with potential for loss of containment integrity or **HOSTILE ACTION** that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA PAG exposure levels offsite for more than the immediate site area.

HOSTAGE

A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION

An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take **HOSTAGES**, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, **PROJECTILES**, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the **OWNER CONTROLLED AREA**).

HOSTILE FORCE

One or more individuals who are engaged in a determined assault, overtly or by stealth and deception, equipped with suitable weapons capable of killing, maiming, or causing destruction.

IMMINENT

The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

IMPEDE(D)

Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

INDEPENDENT SPENT FUEL STORAGE INSTALLATION (ISFSI)

A complex that is designed and constructed for the interim storage of spent nuclear fuel and other radioactive materials associated with spent fuel storage.

INITIATING CONDITION (IC)

An event or condition that aligns with the definition of one of the four emergency classification levels by virtue of the potential or actual effects or consequences.

OWNER CONTROLLED AREA (OCA)

The area within the SITE BOUNDARY including the PROTECTED AREA (ref. 4.1.9).

PROJECTILE

An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA

The area within the Millstone Power Station security fence (ref. 4.1.9).

REFUELING PATHWAY

Refueling cavity, fuel transfer canal, and spent fuel pool (SFP), but **not** including the reactor vessel, comprise the refueling pathway (ref. 4.1.12).

RUPTURED

The condition of a steam generator in which primary-to-secondary leakage is of sufficient magnitude to require a safety injection.

SAFETY SYSTEM

A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

SECURITY CONDITION

Any security event as listed in the approved security contingency plan that constitutes a threat/compromise to site security, threat/risk to site personnel, or a potential degradation to the level of safety of the plant. A Security Condition does **not** involve a HOSTILE ACTION.

SITE AREA EMERGENCY

Events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage or malicious acts; (1) toward site personnel or equipment that could lead to the likely failure of or; (2) that prevent effective access to equipment needed for the protection of the public. **Any** releases are **not** expected to result in exposure levels which exceed EPA PAG exposure levels beyond the SITE BOUNDARY.

SITE BOUNDARY

That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA (ref. 4.1.7)

UNISOLABLE

An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED

A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

UNUSUAL EVENT

Events are in progress or have occurred which indicate a potential degradation in the level of safety of the plant or indicate a security threat to facility protection has been initiated. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs.

VALID

An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

VISIBLE DAMAGE

Damage to a SAFETY SYSTEM train that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected SAFETY SYSTEM train.

5.2 Abbreviations/Acronyms

°F	Degrees Fahrenheit
°	Degrees
μCi.....	Micro Curie
AC	Alternating Current
ADTS	Assistant Director Technical Support
AFW	Auxiliary Feedwater
AOP	Abnormal Operating Procedure
ARM	Area Radiation Monitor
ATWS.....	Anticipated Transient Without Scram
CDA	Containment Depressurization Actuation
CDE	Committed Dose Equivalent
CET	Core Exit Thermocouple
CFR.....	Code of Federal Regulations
CIAS.....	Containment Isolation Actuation Signal
CPM	Counts Per Minute
CSFST	Critical Safety Function Status Check
CTMT	Containment
DBA.....	Design Basis Accident
DEF	Defueled
DC	Direct Current
D/G	Diesel Generator
DSEO	Director of Station Emergency Operations
EAL	Emergency Action Level
ECCS	Emergency Core Cooling System
ECL	Emergency Classification Level
EDG	Emergency Diesel Generator
EOF	Emergency Operations Facility
EOP	Emergency Operating Procedure
EPA.....	Environmental Protection Agency
EPI	Emergency Plan Implementing
FAA	Federal Aviation Administration
FBI	Federal Bureau of Investigation
FC	Fuel Clad Barrier
FEMA	Federal Emergency Management Agency
FSAR	Final Safety Analysis Report
GE	General Emergency
GPM	Gallons Per Minute

HR.....	Heat Removal
Hr.	Hour
HSM	Horizontal Storage Module
IC	Initiating Condition
ICC.....	Inadequate Core Cooling
ISFSI	Independent Spent Fuel Storage Installation
K _{eff}	Effective Neutron Multiplication Factor
LCO.....	Limiting Condition of Operation
LER	Licensee Event Report
LOCA	Loss of Coolant Accident
LPSI	Low Pressure Safety Injection
LRW	Liquid Radwaste
LWR.....	Light Water Reactor
MCB	Main Control Board
Min.	Minute
MPH	Miles Per Hour
MPS	Millstone Power Station
mR, mRem, mrem, mREM	milli-Roentgen Equivalent Man
MW.....	Megawatt
NEI	Nuclear Energy Institute
NEIC	National Earthquake Information Center
NPP.....	Nuclear Power Plant
NRC	Nuclear Regulatory Commission
NSSS	Nuclear Steam Supply System
NORAD	North American Aerospace Defense Command
OBE	Operating Basis Earthquake
OCA	Owner Controlled Area
PAG	Protective Action Guideline
PORV	Power Operated Relief Valve
PSIG	Pounds per Square Inch Gauge
PTS.....	Pressurized Thermal Shock
QSS	Quench Spray System
R	Roentgen
RCS	Reactor Coolant System
Rem, rem, REM	Roentgen Equivalent Man
REMODCM	Radiological Effluent Monitoring Manual/Off-site Dose Calculation Manual
RPS.....	Reactor Protection System
RSS.....	Recirculation Spray System
RVLIS.....	Reactor Vessel Level Instrumentation System

RVLMS..... Reactor Vessel Level Monitoring System
SBO Station Blackout
SCIP..... Security Contingency Plan Implementing Procedures
SCBA Self-Contained Breathing Apparatus
SFP Spent Fuel Pool (Pit)
SG..... Steam Generator
SI..... Safety Injection
SM..... Shift Manager
SPDS Safety Parameter Display System
SRO Senior Reactor Operator
SRV..... Safety Relief Valve
TC (T/C) Thermocouple
TEDE Total Effective Dose Equivalent
TAF Top of Active Fuel
TIC Thermally Induced Current
TS Technical Specifications
TSC..... Technical Support Center
UE Unusual Event
UFSAR..... Updated Final Safety Analysis Report
USGS..... United States Geological Survey

6.0 MPS3-TO-NEI 99-01, Rev. 6 EAL CROSS-REFERENCE

This cross-reference is provided to facilitate association and location of a MPS3 EAL within the NEI 99-01 IC/EAL identification scheme. Further information regarding the development of the MPS3 EALs based on the NEI guidance can be found in the EAL Comparison Matrix.

MPS3	NEI 99-01, Rev. 6	
	IC	Example EAL
RU1.1	AU1	1
RU1.2	AU1	2
RU1.3	AU1	3
RU1.4	AU1	1
RU1.5	AU1	1
RU2.1	AU2	1
RA1.1	AA1	1
RA1.2	AA1	2
RA1.3	AA1	3
RA1.4	AA1	4
RA2.1	AA2	1
RA2.2	AA2	2
RA2.3	AA2	3
RA3.1	AA3	1
RA3.2	AA3	2
RS1.1	AS1	1
RS1.2	AS1	2
RS1.3	AS1	3
RS2.1	AS2	1
RG1.1	AG1	1
RG1.2	AG1	2

MPS3	NEI 99-01, Rev. 6	
EAL	IC	Example EAL
RG1.3	AG1	3
RG2.1	AG2	1
CU1.1	CU1	1
CU1.2	CU1	2
CU2.1	CU2	1
CU3.1	CU3	1
CU3.2	CU3	2
CU4.1	CU4	1
CU5.1	CU5	1, 2, 3
CA1.1	CA1	1
CA1.2	CA1	2
CA2.1	CA2	1
CA3.1	CA3	1, 2
CA6.1	CA6	1
CS1.1	CS1	1
CS1.2	CS1	2
CS1.3	CS1	3
CG1.1	CG1	1
CG1.2	CG1	2
EU1.1	EU1	1
FA1.1	FA1	1
FS1.1	FS1	1
FG1.1	FG1	1
HU1.1	HU1	1, 2, 3

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MPS3	NEI 99-01, Rev. 6	
EAL	IC	Example EAL
HU2.1	HU2	1
HU3.1	HU3	1
HU3.2	HU3	2
HU3.3	HU3	3
HU3.4	HU3	4
HU4.1	HU4	1
HU4.2	HU4	2
HU4.3	HU4	3
HU4.4	HU4	4
HU7.1	HU7	1
HA1.1	HA1	1, 2
HA5.1	HA5	1
HA6.1	HA6	1
HA7.1	HA7	1
HS1.1	HS1	1
HS6.1	HS6	1
HS7.1	HS7	1
HG7.1	HG7	1
MU1.1	SU1	1
MU3.1	SU2	1
MU4.1	SU3	1
MU4.2	SU3	2
MU5.1	SU4	1, 2, 3
MU6.1	SU5	1

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MPS3	NEI 99-01, Rev. 6	
EAL	IC	Example EAL
MU6.2	SU5	2
MU7.1	SU6	1, 2, 3
MU8.1	SU7	1, 2
MA1.1	SA1	1
MA3.1	SA2	1
MA6.1	SA5	1
MA9.1	SA9	1
MS1.1	SS1	1
MS2.1	SS8	1
MS6.1	SS5	1
MG1.1	SG1	1
MG2.1	SG8	1

7.0 ATTACHMENTS

- 7.1 Attachment 1, Emergency Action Level Technical Bases
- 7.2 Attachment 2, Safe Operation & Shutdown Areas Tables R-2 & H-2 Bases

Category R – Abnormal Rad Release / Rad Effluent

EAL Group: ANY (EALs in this category are applicable to any plant condition, hot or cold.)

Many EALs are based on actual or potential degradation of fission product barriers because of the elevated potential for offsite radioactivity release. Degradation of fission product barriers though is not always apparent via non-radiological symptoms. Therefore, direct indication of elevated radiological effluents or area radiation levels are appropriate symptoms for emergency classification.

At lower levels, abnormal radioactivity releases may be indicative of a failure of containment systems or precursors to more significant releases. At higher release rates, offsite radiological conditions may result which require offsite protective actions. Elevated area radiation levels in plant may also be indicative of the failure of containment systems or preclude access to plant vital equipment necessary to ensure plant safety.

Events of this category pertain to the following subcategories:

1. Radiological Effluent

Direct indication of effluent radiation monitoring systems provides a rapid assessment mechanism to determine releases in excess of classifiable limits. Projected offsite doses, actual offsite field measurements or measured release rates via sampling indicate doses or dose rates above classifiable limits.

2. Irradiated Fuel Events

Conditions indicative of a loss of adequate shielding or damage to irradiated fuel may preclude access to vital plant areas or result in radiological releases that warrant emergency classification.

3. Area Radiation Levels

Sustained general area radiation levels which may preclude access to areas required to safely operate and shutdown the plant also warrant emergency classification.

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1a – Radiological Effluent (Liquid)
Initiating Condition: Release of liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.1 Unusual Event

Reading on **EITHER** of the following radiation monitors > 2x the “alarm” setpoint for ≥60 min. (Notes 1, 2, 3)

- 3SSR-RE08 SG Blowdown
- 3DAS-RE50 Turbine Building Floor Drain

(Notes 1, 2, 3)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no longer VALID** for classification purposes.

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any liquid radiological release, monitored or unmonitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

This EAL addresses normally occurring continuous radioactivity releases from monitored liquid effluent pathways (ref. 1).

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1a – Radiological Effluent (Liquid)
Initiating Condition: Release of liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.2 Unusual Event

Reading on **EITHER** of the following effluent radiation monitors > 2 x the “alarm” setpoint established by a current radioactivity discharge permit for ≥ 60 min.

- 3LWS-RE70 Liquid Waste Effluent
- 3CND-RE07 Waste Neutralization Sump

(Notes 1, 2, 3)

- Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.
- Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.
- Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator’s operability, the condition’s existence, or the report’s accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

3CND-RE07 Waste Neutralizer Discharge radiation monitor can only be read on the RMS Console (3-RMS-CNSL2).

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any liquid radiological release, monitored or unmonitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

This EAL addresses radioactivity releases that cause effluent radiation monitor readings to exceed 2 times the limit established by a radioactivity discharge permit. Such releases are typically associated with planned batch releases from non-continuous release pathways (e.g., radwaste) (ref. 1, 2).

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. RP 17-24, "Millstone Abnormal Rad Release Liquid EAL Thresholds based on NEI 99-01, Rev. 6"
3. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1a – Radiological Effluent (Liquid)
Initiating Condition: Release of liquid radioactivity greater than 2 times the allocated REMODCM limits for 60 minutes or longer

EAL:

RU1.3 Unusual Event

Sample analysis for a liquid release indicates a concentration or release rate
> 2 x the allocated REMODCM limits for ≥ 60 min. (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any liquid radiological release, monitored or unmonitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

This EAL addresses uncontrolled liquid releases that are detected by sample analyses or environmental surveys, particularly on unmonitored pathways (e.g., spills of radioactive liquids into storm drains, heat exchanger leakage in ocean water systems, etc.).

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1b – Radiological Effluent (Gaseous)

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1 mrem TEDE

EAL:

RU1.4 Unusual Event

Reading on **any** Table R-1 effluent radiation monitor > column "UE" for ≥ 60 min.
(Notes 1, 2, 3)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Table R-1 Unit 3 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Ventilation Vent Normal/High Range (HVR10A/B)	5.9E+00 $\mu\text{Ci/cc}$	5.9E-01 $\mu\text{Ci/cc}$	5.9E-02 $\mu\text{Ci/cc}$	5.9E-03 $\mu\text{Ci/cc}$
Stack (SLCRS) Normal/High Range (HVR19A/B)	3.6E+02 $\mu\text{Ci/cc}$	3.6E+01 $\mu\text{Ci/cc}$	3.6E+00 $\mu\text{Ci/cc}$	3.6E-01 $\mu\text{Ci/cc}$
Main Steam Release (MSS75-78)	5.3E+01 $\mu\text{Ci/cc}$	5.3E+00 $\mu\text{Ci/cc}$	5.3E-01 $\mu\text{Ci/cc}$	N/A
Aux FDWTR Pump Mntr (MSS79)	1.6E+01 $\mu\text{Ci/cc}$	1.6E+00 $\mu\text{Ci/cc}$	1.6E-01 $\mu\text{Ci/cc}$	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous radiological release, monitored or unmonitored

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

This EAL addresses normally occurring continuous radioactivity releases from monitored gaseous effluent pathways (ref. 1, 2).

The basis for the gaseous Unusual Event IC and associated thresholds has been revised to correspond to any unplanned release of gaseous effluent radioactivity to the environment that will result in greater than 1 mrem TEDE. This Unusual Event gaseous release criterion is being used consistently across all operating nuclear units at Dominion Energy nuclear stations at Millstone, North Anna and Surry. The reason this alternative criterion is required is due to the fact that for some effluent gaseous release pathways, the resulting calculated Unusual Event threshold following the NEI 99-01 guidance of two times the site-specific effluent release limit would result in a Unusual Event threshold value greater than the corresponding calculated ALERT threshold based on exceeding 10 mrem TEDE. For the other gaseous release pathways that did not show an incongruent relationship when compared to the ALERT threshold, many showed Unusual Event values essentially equivalent to 1 mrem TEDE when applying the guidance in NEI 99-01 of a value set at two times the site-specific effluent release limit. The fact that, (1) many of the gaseous release pathway Unusual Event values following NEI 99-01 guidance were essentially equivalent to 1 mrem TEDE, (2) application of an alternative definition set at a value of 1 mrem TEDE results in a more limiting value for those release paths that showed incongruent comparison to the corresponding ALERT threshold, and (3) Unusual Event criterion set at a value ten (10) times lower than the ALERT threshold provides a logical and consistent escalation between each classification level, provides justification for the Unusual Event criterion of 1 mrem TEDE. This single Initiating Condition (IC) definition for gaseous releases at the Unusual Event level is being applied to maintain consistency across the Dominion Energy nuclear fleet and to reduce confusion and human error potential if two different (IC) definitions were applied.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Due to the fact that there are no REMODCM limits on steam safeties or auxiliary feedwater exhausts and the limited ability for these respective radiation monitors to detect low level activity in these steam line configurations, the UE classification thresholds for the steam safeties and auxiliary feedwater exhaust are being labeled N/A (not applicable).

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. RP 18-03, "MP3 Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01, Rev. 6"
3. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1b – Radiological Effluent (Gaseous)
Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1 mrem TEDE

EAL:

RU1.5 Unusual Event

Sample analysis for a gaseous release indicates a concentration or release rate > 2 x the allocated REMODCM limits for ≥ 60 min. (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous radiological release, monitored or unmonitored.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

Calculation RP 18-03 (ref. 2) demonstrates how a release rate limit based on 2 x the allocated REMODCM limit will produce essentially 1 mrem TEDE assuming most prevalent meteorological dispersion.

Most prevalent meteorology represents conditions that would most likely to exist (based on most prevalent stability class and average wind speed within that stability class). Dispersion based on most prevalent meteorology differs from that assumed in the REMODCM which uses annual average meteorology. Dispersion based on actual meteorological conditions at the time

of the emergency (most prevalent) can be 10 – 20 times higher than the annual average dispersion prescribed for use in an ODCM.

This EAL addresses uncontrolled gaseous releases that are detected by sample analyses or environmental surveys, particularly on unmonitored pathways.

Escalation of the emergency classification level would be via IC RA1.

Reference(s):

1. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
2. RP 18-03, "MP3 Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
3. NEI 99-01 AU1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.1 Alert

Reading on **any** Table A-1 effluent radiation monitor > column "ALERT" for ≥ 15 min.
(Notes 1, 2, 3, 4)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no longer VALID** for classification purposes.

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Table R-1 Unit 3 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Ventilation Vent Normal/High Range (HVR10A/B)	5.9E+00 $\mu\text{Ci/cc}$	5.9E-01 $\mu\text{Ci/cc}$	5.9E-02 $\mu\text{Ci/cc}$	5.9E-03 $\mu\text{Ci/cc}$
Stack (SLCRS) Normal/High Range (HVR19A/B)	3.6E+02 $\mu\text{Ci/cc}$	3.6E+01 $\mu\text{Ci/cc}$	3.6E+00 $\mu\text{Ci/cc}$	3.6E-01 $\mu\text{Ci/cc}$
Main Steam Release (MSS75-78)	5.3E+01 $\mu\text{Ci/cc}$	5.3E+00 $\mu\text{Ci/cc}$	5.3E-01 $\mu\text{Ci/cc}$	N/A
Aux FDWTR Pump Mntr (MSS79)	1.6E+01 $\mu\text{Ci/cc}$	1.6E+00 $\mu\text{Ci/cc}$	1.6E-01 $\mu\text{Ci/cc}$	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Escalation of the emergency classification level would be via IC RS1.

Reference(s):

1. RP 18-03 MP3, "Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.2 Alert

Dose assessment using actual meteorology indicates doses > 10 mrem TEDE at or beyond the SITE BOUNDARY (Note 4)

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Actual meteorology is specifically identified since it gives the most accurate dose assessment. Actual meteorology (including forecasts) should be used whenever possible.

Escalation of the emergency classification level would be via IC RS1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.3 Alert

Analysis of a liquid effluent sample indicates a concentration or release rate that would result in doses > 10 mrem TEDE at or beyond the SITE BOUNDARY for 60 min. of exposure (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

This EAL is assessed per the REMODCM (ref. 2). REMODCM software can be used to produce a dose to the maximum individual.

Escalation of the emergency classification level would be via IC RS1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"

2. MP-22-REC-BAP01, "Radiological Effluent Monitoring and Offsite Dose Calculation Manual"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE

EAL:

RA1.4 Alert

Field survey results indicate closed window dose rates > 10 mR/hr expected to continue for ≥60 min. at or beyond the SITE BOUNDARY (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

Escalation of the emergency classification level would be via IC RS1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-26-EPI-FAP04, "Emergency Operations Facility Activation and Operation"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AA1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 100 mrem TEDE

EAL:

RS1.1 Site Area Emergency

Reading on **any** Table R-1 effluent radiation monitor > column "SAE" for ≥ 15 min.
(Notes 1, 2, 3, 4)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available

Table R-1 Unit 3 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Ventilation Vent Normal/High Range (HVR10A/B)	5.9E+00 $\mu\text{Ci/cc}$	5.9E-01 $\mu\text{Ci/cc}$	5.9E-02 $\mu\text{Ci/cc}$	5.9E-03 $\mu\text{Ci/cc}$
Stack (SLCRS) Normal/High Range (HVR19A/B)	3.6E+02 $\mu\text{Ci/cc}$	3.6E+01 $\mu\text{Ci/cc}$	3.6E+00 $\mu\text{Ci/cc}$	3.6E-01 $\mu\text{Ci/cc}$
Main Steam Release (MSS75-78)	5.3E+01 $\mu\text{Ci/cc}$	5.3E+00 $\mu\text{Ci/cc}$	5.3E-01 $\mu\text{Ci/cc}$	N/A
Aux FDWTR Pump Mntr (MSS79)	1.6E+01 $\mu\text{Ci/cc}$	1.6E+00 $\mu\text{Ci/cc}$	1.6E-01 $\mu\text{Ci/cc}$	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to 10% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude are associated with the failure of plant systems needed for the protection of the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 10% of the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Escalation of the emergency classification level would be via IC RG1.

Reference(s):

1. RP 18-03, "MP3 Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AS1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 100 mrem TEDE

EAL:

RS1.2 Site Area Emergency

Dose assessment using actual meteorology indicates doses > 100 mrem TEDE at or beyond the SITE BOUNDARY (Note 4)

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to 10% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude are associated with the failure of plant systems needed for the protection of the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 10% of the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Actual meteorology is specifically identified since it gives the most accurate dose assessment. Actual meteorology (including forecasts) should be used whenever possible.

Escalation of the emergency classification level would be via IC RG1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AS1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 1 – Radiological Effluent
Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 100 mrem TEDE

EAL:

RS1.3 Site Area Emergency

Field survey results indicate closed window dose rates > 100 mR/hr expected to continue for ≥60 min. at or beyond the SITE BOUNDARY (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to 10% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude are associated with the failure of plant systems needed for the protection of the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 10% of the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

Escalation of the emergency classification level would be via IC RG1.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-26-EPI-FAP04, "Emergency Operations Facility Activation and Operation"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AS1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE

EAL:

RG1.1 General Emergency

Reading on **any** Table R-1 effluent radiation monitor > column "GE" for ≥ 15 min.
(Notes 1, 2, 3, 4)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Note 3: If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is **no** longer VALID for classification purposes.

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Table R-1 Unit 3 Gaseous Effluent Monitor Classification Thresholds				
Release Point & Monitor	GE	SAE	Alert	UE
Ventilation Vent Normal/High Range (HVR10A/B)	5.9E+00 $\mu\text{Ci/cc}$	5.9E-01 $\mu\text{Ci/cc}$	5.9E-02 $\mu\text{Ci/cc}$	5.9E-03 $\mu\text{Ci/cc}$
Stack (SLCRS) Normal/High Range (HVR19A/B)	3.6E+02 $\mu\text{Ci/cc}$	3.6E+01 $\mu\text{Ci/cc}$	3.6E+00 $\mu\text{Ci/cc}$	3.6E-01 $\mu\text{Ci/cc}$
Main Steam Release (MSS75-78)	5.3E+01 $\mu\text{Ci/cc}$	5.3E+00 $\mu\text{Ci/cc}$	5.3E-01 $\mu\text{Ci/cc}$	N/A
Aux FDWTR Pump Mntr (MSS79)	1.6E+01 $\mu\text{Ci/cc}$	1.6E+00 $\mu\text{Ci/cc}$	1.6E-01 $\mu\text{Ci/cc}$	N/A

Mode Applicability:

All

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude will require implementation of protective actions for the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer VALID for classification purposes.

Classification thresholds within Table R-1 were generated using the MIDAS dose assessment code. Inputs to MIDAS use most prevalent meteorological data and expected release point parameters. An assumed one-hour decay since shutdown and a one-hour release duration are applied. Mitigating reduction mechanisms (e.g., decay, sprays, filters) input into MIDAS for each accident type determined the radiological release source term consistent with the guidance provided in NUREG-1228.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Reference(s):

1. RP 18-03 MP3, "Abnormal Rad Release Gaseous EAL Thresholds based on NEI 99-01", Rev. 6
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AG1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE

EAL:

RG1.2 General Emergency

Dose assessment using actual meteorology indicates doses > 1,000 mrem TEDE at or beyond the SITE BOUNDARY (Note 4)

Note 4: The pre-calculated effluent monitor values presented in EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude will require implementation of protective actions for the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 2) and consistent with direction provided by the States of Connecticut and New York.

Since dose assessment is based on actual meteorology whereas the monitor reading EALs are not, the results from these assessments may indicate that the classification is not warranted, or may indicate that a higher classification is warranted. For this reason, emergency implementing procedures call for the timely performance of dose assessments using actual meteorology and release information. If the results of these dose assessments are available when the classification is made (e.g., initiated at a lower classification level), the dose assessment results override the monitor readings listed in Table R-1.

Actual meteorology is specifically identified since it gives the most accurate dose assessment. Actual meteorology (including forecasts) should be used whenever possible.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
3. NEI 99-01 AG1

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 1 – Radiological Effluent

Initiating Condition: Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE

EAL:

RG1.3 General Emergency

Field survey results indicate closed window dose rates > 1,000 mR/hr expected to continue for ≥60 min. at or beyond the SITE BOUNDARY (Notes 1, 2)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 2: If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded the specified time limit.

Mode Applicability:

All

Definition(s):

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude will require implementation of protective actions for the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at the EPA PAG of 1,000 mrem TEDE.

The thyroid CDE component has been eliminated from this EAL as allowed by EPA-400 (ref. 3) and consistent with direction provided by the States of Connecticut and New York.

Reference(s):

1. MP-26-EPI-FAP10, "Dose Assessment"
2. MP-26-EPI-FAP04, "Emergency Operations Facility Activation and Operation"
3. EPA-400, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incident"
4. NEI 99-01 AG1

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 2 – Irradiated Fuel Events
Initiating Condition: UNPLANNED loss of water level above irradiated fuel
EAL:

RU2.1 Unusual Event

UNPLANNED water level drop in the REFUELING PATHWAY as indicated by **any** of the following:

- FUEL POOL LEVEL LO alarm MB1A 3-4
- FUEL POOL WATER LEVEL LO alarm FP 1-3
- Report of dropping level in refueling cavity or SFP

AND

UNPLANNED rise in corresponding area radiation levels as indicated by **any** of the following radiation monitors:

- RMS01 Manipulator Crane
- RMS41/42 Fuel Drop
- RMS08 SFP Bridge
- RMS36 Fuel Pool

Mode Applicability:

All

Definition(s):

UNPLANNED- A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

REFUELING PATHWAY- Refueling cavity, fuel transfer canal, and spent fuel pool (SFP), but **not** including the reactor vessel, comprise the refueling pathway.

Basis:

This IC addresses a decrease in water level above irradiated fuel sufficient to cause elevated radiation levels. This condition could be a precursor to a more serious event and is also indicative of a minor loss in the ability to control radiation levels within the plant. It is therefore a potential degradation in the level of safety of the plant.

A water level decrease will be primarily determined by indications from available level instrumentation. Other sources of level indications may include reports from plant personnel (e.g., from a refueling crew) or video camera observations (if available). A significant drop in the water level may also cause a loss of SFP cooling suction flow and an increase in the radiation levels of adjacent areas that can be detected by monitors in those locations.

The specified radiation monitors are those expected to see increase area radiation levels as a result of a loss of REFUELING PATHWAY inventory (ref. 1, 2, 3, 4). Increasing radiation indications on these monitors in the absence of indications of decreasing REFUELING PATHWAY level are not classifiable under this EAL.

The effects of planned evolutions should be considered. For example, a refueling bridge area radiation monitor reading may increase due to planned evolutions such as lifting of the reactor vessel head or movement of a fuel assembly. Note that this EAL is applicable only in cases where the elevated reading is due to an unplanned loss of water level.

A drop in water level above irradiated fuel within the reactor vessel may be classified in accordance Category C during the Cold Shutdown and Refueling modes.

Escalation of the emergency classification level would be via IC RA2.

Reference(s):

1. AOP 3572, "Failure of Refueling Cavity Seal"
2. EOP 3505A, "Loss of Spent Fuel Pool Cooling"
3. EOP 3502, "Fuel Handling Accident"
4. OP 3362, "Radiation Monitor System Display and Control System"
5. NEI 99-01 AU2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Significant lowering of water level above, or damage to, irradiated fuel

EAL:

RA2.1 Alert

IMMINENT uncovering of irradiated fuel in the REFUELING PATHWAY

Mode Applicability:

All

Definition(s):

CONFINEMENT BOUNDARY- The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielding Canister (DSC).

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

REFUELING PATHWAY- Refueling cavity, fuel transfer canal, and spent fuel pool (SFP), but **not** including the reactor vessel, comprise the refueling pathway.

Basis:

This IC addresses events that have caused IMMINENT or actual damage to an irradiated fuel assembly, or a significant lowering of water level within the REFUELING PATHWAY. These events present radiological safety challenges to plant personnel and are precursors to a release of radioactivity to the environment. As such, they represent an actual or potential substantial degradation of the level of safety of the plant.

For irradiated fuel that is licensed for dry storage, this EAL applies up to the point that the loaded storage cask is sealed. Once sealed, damage to a loaded cask causing loss of the CONFINEMENT BOUNDARY is classified in accordance with IC EU1.

Escalation of the emergency would be based on either Category R or C ICs.

This EAL escalates from RU2.1 in that the loss of level, in the affected portion of the REFUELING PATHWAY, is of sufficient magnitude to have resulted in uncovering of irradiated fuel. Indications of irradiated fuel uncovering may include direct or indirect visual observation (e.g., reports from personnel or camera images), as well as significant changes in water and radiation levels, or other plant parameters. Computational aids may also be used (e.g., a boil-off curve). Classification of an event using this EAL should be based on the totality of available indications, reports and observations.

While an area radiation monitor could detect an increase in a dose rate due to a lowering of water level in some portion of the REFUELING PATHWAY, the reading may not be a reliable indication of whether or not the fuel is actually uncovered. To the degree possible, readings should be considered in combination with other available indications of inventory loss.

A drop in water level above irradiated fuel within the reactor vessel may be classified in

accordance with Category C during the Cold Shutdown and Refueling modes.

Reference(s):

1. NEI 99-01 AA2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Significant lowering of water level above, or damage to, irradiated fuel

EAL:

RA2.2 Alert

Damage to irradiated fuel resulting in a release of radioactivity

AND

VALID alarm on **any** of the following radiation monitors:

- CMS22 CTMT G/A
- RM01 Manipulator Crane
- RMS41/42 Fuel Drop
- HVR17 Fuel Bldg
- RMS08 SFP Bridge
- RMS36 Fuel Pool

Mode Applicability:

All

Definition(s):

CONFINEMENT BOUNDARY- The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielded Canister (DSC).

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

The specified radiation monitors are those expected to see increased area radiation levels as a result of damage to irradiated fuel (ref. 1, 2, 3, 4).

This EAL addresses events that have caused actual damage to an irradiated fuel assembly. These events present radiological safety challenges to plant personnel and are precursors to a release of radioactivity to the environment. As such, they represent an actual or potential substantial degradation of the level of safety of the plant.

For irradiated fuel that is licensed for dry storage, this EAL applies up to the point that the loaded storage cask is sealed. Once sealed, damage to a loaded cask causing loss of the CONFINEMENT BOUNDARY is classified in accordance with IC EU1.

This EAL addresses a release of radioactive material caused by mechanical damage to irradiated fuel. Damaging events may include the dropping, bumping or binding of an assembly, or dropping a heavy load onto an assembly. A rise in readings on radiation monitors should be considered in conjunction with in-plant reports or observations of a potential fuel damaging event (e.g., a fuel handling accident).

Escalation of the emergency would be based on either Category R or C ICs.

Reference(s):

1. AOP 3572, "Failure of Refueling Cavity Seal"
2. EOP 3505A, "Loss of Spent Fuel Pool Cooling"
3. EOP 3502, "Fuel Handling Accident"
4. OP 3362, "Radiation Monitor System Display and Control System"
5. NEI 99-01 AA2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Significant lowering of water level above, or damage to, irradiated fuel

EAL:

RA2.3 Alert

Lowering of spent fuel pool level to 10 ft. (Level 2) on 3SFC-LI55A or 3SFC-LI55B

Mode Applicability:

All

Definition(s):

None

Basis:

This EAL addresses events that have caused a significant lowering of water level within the spent fuel pool. These events present radiological safety challenges to plant personnel and are precursors to a release of radioactivity to the environment. As such, they represent an actual or potential substantial degradation of the level of safety of the plant.

Spent fuel pool water level at this value is within the lower end of the level range necessary to prevent significant dose consequences from direct gamma radiation to personnel performing operations in the vicinity of the spent fuel pool. This condition reflects a significant loss of spent fuel pool water inventory and thus it is also a precursor to a loss of the ability to adequately cool the irradiated fuel assemblies stored in the pool.

Escalation of the emergency classification level would be via ICs RS1 or RS2.

Post-Fukushima order EA-12-051 required the installation of reliable SFP level indication (3SFC-LI55A or 3SFC-LI55B) capable of identifying normal level (Level 1 – EL 47 ft. 1 in.), SFP level 10 ft. above the top of the fuel racks (Level 2 – EL 35 ft. 11 in.) and SFP level at 1 ft. above the top of the fuel racks (Level 3 – EL 26 ft. 11 in.) (ref. 1, 2).

Reference(s):

1. ETE-CPR-2012-0008, "Millstone Unit 3 – Beyond Design Basis FLEX Strategy Basis Document and Final Integrated Plan"
2. DC MP3-13-01012, "Beyond Design Basis Spent Fuel Pool Level Instrumentation, Millstone Unit 3"
3. NEI 99-01 AA2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Spent fuel pool level at the top of the fuel racks

EAL:

RS2.1 Site Area Emergency

Lowering of spent fuel pool level to 1 ft. (Level 3) on 3SFC-LI55A or 3SFC-LI55B

Mode Applicability:

All

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

This EAL addresses a significant loss of spent fuel pool inventory control and makeup capability leading to IMMINENT fuel damage. This condition entails major failures of plant functions needed for protection of the public and thus warrant a Site Area Emergency declaration.

It is recognized that this IC would likely not be met until well after another Site Area Emergency IC was met; however, it is included to provide classification diversity.

Escalation of the emergency classification level would be via IC RG1 or RG2.

Post-Fukushima order EA-12-051 required the installation of reliable SFP level indication (3SFC-LI55A or 3SFC-LI55B) capable of identifying normal level (Level 1 – EL 47 ft. 1 in.), SFP level 10 ft. above the top of the fuel racks (Level 2 – EL 35 ft. 11 in.) and SFP level at 1 ft. above the top of the fuel racks (Level 3 – EL 26 ft. 11 in.) (ref. 1, 2).

Reference(s):

1. ETE-CPR-2012-0008, "Millstone Unit 3 – Beyond Design Basis FLEX Strategy Basis Document and Final Integrated Plan"
2. DC MP3-13-01012, "Beyond Design Basis Spent Fuel Pool Level Instrumentation, Millstone Unit 3"
3. NEI 99-01 AS2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 2 – Irradiated Fuel Events

Initiating Condition: Spent fuel pool level **cannot** be restored to at least the top of the fuel racks for 60 minutes or longer

EAL:

RG2.1 General Emergency

Spent fuel pool level **cannot** be restored to at least 1 ft. (Level 3) on 3SFC-LI55A or 3SFC-LI55B for ≥ 60 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

All

Definition(s):

None

Basis:

This EAL addresses a significant loss of spent fuel pool inventory control and makeup capability leading to a prolonged uncover of spent fuel. This condition will lead to fuel damage and a radiological release to the environment.

It is recognized that this EAL would likely not be met until well after another General Emergency EAL was met; however, it is included to provide classification diversity.

Post-Fukushima order EA-12-051 required the installation of reliable SFP level indication (3SFC-LI55A or 3SFC-LI55B) capable of identifying normal level (Level 1 – EL 47 ft. 1 in.), SFP level 10 ft. above the top of the fuel racks (Level 2 – EL 35 ft. 11 in.) and SFP level at 1 ft. above the top of the fuel racks (Level 3 – EL 26 ft. 11 in.) (ref. 1, 2).

Reference(s):

1. ETE-CPR-2012-0008, "Millstone Unit 3 – Beyond Design Basis FLEX Strategy Basis Document and Final Integrated Plan"
2. DC MP3-13-01012, "Beyond Design Basis Spent Fuel Pool Level Instrumentation, Millstone Unit 3"
3. NEI 99-01 AG2

Category: R – Abnormal Rad Levels / Rad Effluent

Subcategory: 3 – Area Radiation Levels

Initiating Condition: Radiation levels that IMPEDE access to equipment necessary for normal plant operations, cooldown or shutdown

EAL:

RA3.1 Alert

Dose rate > 15 mR/hr in **EITHER** of the following areas:

- Control Room
- Central Alarm Station

Mode Applicability:

All

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

Basis:

This IC addresses elevated radiation levels in certain plant rooms/areas sufficient to preclude or IMPEDE personnel from performing actions necessary to maintain normal plant operation, or to perform a normal plant cooldown and shutdown. As such, it represents an actual or potential substantial degradation of the level of safety of the plant. The DSEO/ADTS should consider the cause of the increased radiation levels and determine if another IC may be applicable.

Areas that meet this threshold include the Control Room (CR) and the Central Alarm Station (CAS). The Control Room is monitored for excessive radiation by one detector, RMS-22 (ref. 1). The CAS is included in this EAL because of its importance to permitting access to areas required to assure safe plant operations.

Reference(s):

1. AOP 3573, "Radiation Monitor Alarm Response"
2. NEI 99-01 AA3

Category: R – Abnormal Rad Levels / Rad Effluent
Subcategory: 3 – Area Radiation Levels
Initiating Condition: Radiation levels that IMPEDE access to equipment necessary for normal plant operations, cooldown or shutdown

EAL:

RA3.2 Alert

An UNPLANNED event results in radiation levels that prohibit or IMPEDE access to **any** Table R-2 room or area (Note 5)

Note 5: If the equipment in the listed room or area was already inoperable or out-of-service before the event occurred, then no emergency classification is warranted.

Table R-2 Safe Operation & Shutdown Rooms/Areas	
Room/Area	Mode
Aux. Building EI 43'	1, 2, 3
Aux. Building EI 24' East MCC/RCA EI 24' West MCC/RCA EI 24'	3, 4
Aux. Building EI 3' 8"	3
Containment ESF Building A RHR Cubicle ESF Building B RHR Cubicle ESF Building EI 36' ESF Building EI 24' East Switchgear West Switchgear	4

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

UNPLANNED- A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses elevated radiation levels in certain plant rooms/areas sufficient to preclude or IMPEDE personnel from performing actions necessary to maintain normal plant operation, or to perform a normal plant cooldown and shutdown. As such, it represents an actual or

potential substantial degradation of the level of safety of the plant. The DSEO/ADTS should consider the cause of the increased radiation levels and determine if another IC may be applicable.

For RA3.2, an Alert declaration is warranted if entry into the affected room/area is, or may be, procedurally required during the plant operating mode in effect at the time of the elevated radiation levels. The emergency classification is not contingent upon whether entry is actually necessary at the time of the increased radiation levels. Access should be considered as IMPEDED if extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., installing temporary shielding, requiring use of non-routine protective equipment, requesting an extension in dose limits beyond normal administrative limits).

If the equipment in the listed room or area was already inoperable, or out-of-service, before the event occurred, then no emergency should be declared since the event will have no adverse impact beyond that already allowed by Technical Specifications at the time of the event.

The list of plant rooms or areas with entry-related mode applicability identified specify those rooms or areas that contain equipment which require a manual/local action as specified in operating procedures used for normal plant operation, cooldown and shutdown. Rooms or areas in which actions of a contingent or emergency nature would be performed (e.g., an action to address an off-normal or emergency condition such as emergency repairs, corrective measures or emergency operations) are not included. In addition, the list specifies the plant mode(s) during which entry would be required for each room or area (ref. 1).

An emergency declaration is **not** warranted if any of the following conditions apply:

- The plant is in an operating mode different than the mode specified for the affected room/area (i.e., entry is not required during the operating mode in effect at the time of the elevated radiation levels). For example, the plant is in Mode 1 when the radiation increase occurs, and the procedures used for normal operation, cooldown and shutdown do not require entry into the affected room until Mode 4.
- The increased radiation levels are a result of a planned activity that includes compensatory measures which address the temporary inaccessibility of a room or area (e.g., radiography, spent filter or resin transfer, etc.).
- The action for which room/area entry is required is of an administrative or record keeping nature (e.g., normal rounds or routine inspections).
- The access control measures are of a conservative or precautionary nature, and would not actually prevent or IMPEDE a required action.

Reference(s):

1. Attachment 2, "Safe Operation & Shutdown Areas Tables R-2 & H-2 Bases"
2. NEI 99-01 AA3

Category C – Cold Shutdown / Refueling System Malfunction

EAL Group: Cold Conditions (RCS temperature $\leq 200^{\circ}\text{F}$); EALs in this category are applicable only in one or more cold operating modes.

Category C EALs are directly associated with cold shutdown or refueling system safety functions. Given the variability of plant configurations (e.g., systems out-of-service for maintenance, containment open, reduced AC power redundancy, time since shutdown) during these periods, the consequences of any given initiating event can vary greatly. For example, a loss of decay heat removal capability that occurs at the end of an extended outage has less significance than a similar loss occurring during the first week after shutdown. Compounding these events is the likelihood that instrumentation necessary for assessment may also be inoperable. The cold shutdown and refueling system malfunction EALs are based on performance capability to the extent possible with consideration given to RCS integrity, CONTAINMENT CLOSURE, and fuel clad integrity for the applicable operating modes (5 - Cold Shutdown, 6 - Refueling, DEF – Defueled).

The events of this category pertain to the following subcategories:

1. RCS Level

RCS water level is directly related to the status of adequate core cooling and, therefore, fuel clad integrity.

2. Loss of Emergency AC Power

Loss of vital plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of onsite and offsite power sources for 4.16 KV emergency buses.

3. RCS Temperature

Uncontrolled or inadvertent temperature or pressure rises are indicative of a potential loss of safety functions.

4. Loss of Vital DC Power

Loss of emergency plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of power to or degraded voltage on the 125V DC vital buses.

5. Loss of Communications

Certain events that degrade plant operator ability to effectively communicate with essential personnel within or external to the plant warrant emergency classification.

6. Hazardous Event Affecting Safety Systems

Certain hazardous natural and technological events may result in VISIBLE DAMAGE to or degraded performance of safety systems warranting classification.

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: UNPLANNED loss of RCS inventory

EAL:

CU1.1 Unusual Event

UNPLANNED loss of reactor coolant results in RCS water level < a required lower limit for ≥ 15 min. (Note 1) .

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

UNPLANNED-. A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

RCS water level less than a required lower limit is meant to be less than the lower end of the level control band being procedurally maintained for the current condition or evolution.

With the plant in Cold Shutdown, RCS water level is normally maintained within a pressurizer level control band (ref. 1). However, if RCS level is being controlled below the normal pressurizer level control band, or if level is being maintained in a designated band in the reactor vessel it is the inability to maintain level above the low end of the designated control band due to a loss of inventory resulting from a leak in the RCS that is the concern.

With the plant in Refueling mode, RCS water level is normally maintained at or above the reactor vessel flange (ref. 2, 3).

This IC addresses the inability to restore and maintain water level to a required minimum level (or the lower limit of a level band), or a loss of the ability to monitor RCS level concurrent with indications of coolant leakage. Either of these conditions is considered to be a potential degradation of the level of safety of the plant.

Refueling evolutions that decrease RCS water inventory are carefully planned and controlled. An UNPLANNED event that results in water level decreasing below a procedurally required limit warrants the declaration of an Unusual Event due to the reduced water inventory that is available to keep the core covered.

This EAL recognizes that the minimum required RCS level can change several times during the course of a refueling outage as different plant configurations and system lineups are implemented. This EAL is met if the minimum level, specified for the current plant conditions, cannot be maintained for 15 minutes or longer. The minimum level is typically specified in the applicable operating procedure but may be specified in another controlling document.

The 15-minute threshold duration allows sufficient time for prompt operator actions to restore and maintain the expected water level. This criterion excludes transient conditions causing a brief lowering of water level.

Continued loss of RCS inventory may result in escalation to the Alert emergency classification level via either IC CA1 or CA3.

Reference(s):

1. OP 3208, "Plant Cooldown"
2. OP 3216, "RCS Drain (ICCE)"
3. OP 3210B, "Refueling Operations"
4. NEI 99-01 CU1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: UNPLANNED loss of RCS inventory

EAL:

CU1.2 Unusual Event

RCS water level **cannot** be monitored

AND EITHER:

- UNPLANNED rise in **any** Table C-1 sump or tank level due to a loss of RCS inventory
- Visual observation of UNISOLABLE RCS leakage

Table C-1 Sumps/Tanks
<ul style="list-style-type: none"> • PRT • CDTT • PDTT • Containment Sump • Unidentified Leakage Sump • Auxiliary Bldg. Sump • ESF Bldg. Sump

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED-. A parameter changes or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses the inability to restore and maintain water level to a required minimum level (or the lower limit of a level band), or a loss of the ability to monitor RCS level concurrent with indications of coolant leakage. Either of these conditions is considered to be a potential degradation of the level of safety of the plant.

Refueling evolutions that decrease RCS water inventory are carefully planned and controlled. An UNPLANNED event that results in water level decreasing below a procedurally required limit warrants the declaration of an Unusual Event due to the reduced water inventory that is available to keep the core covered.

This EAL addresses a condition where all means to determine RCS level have been lost. In this condition, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels (Table C-1) (ref. 1, 2, 3). Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the RCS.

Continued loss of RCS inventory may result in escalation to the Alert emergency classification level via either IC CA1 or CA3.

Reference(s):

1. OP 3216, "RCS Drain (ICCE)"
2. EOP 3505, "Loss of Shutdown Cooling and/or RCS Inventory"
3. MPS3 FSAR Section 5.2, "Integrity of Reactor Coolant Pressure Boundary"
4. NEI 99-01 CU1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Significant Loss of RCS inventory

EAL:

CA1.1 Alert

Loss of RCS inventory as indicated by **EITHER**:

- < 82% RVLMS plenum level
- ≤ 15 in above centerline of hotleg on tygon tube indicator

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

None

Basis:

This IC addresses conditions that are precursors to a loss of the ability to adequately cool irradiated fuel (i.e., a precursor to a challenge to the fuel clad barrier). This condition represents a potential substantial reduction in the level of plant safety.

For this EAL, a lowering of RCS water level below the specified value(s) indicates that operator actions have not been successful in restoring and maintaining RCS water level. The heat-up rate of the coolant will increase as the available water inventory is reduced. A continuing decrease in water level will lead to core uncover. The specified levels are the lowest RCS levels that supports continued decay heat removal pump (RHR) operations (ref. 1, 2).

Although related, this EAL is concerned with the loss of RCS inventory and not the potential concurrent effects on systems needed for decay heat removal (e.g., loss of a Residual Heat Removal suction point). An increase in RCS temperature caused by a loss of decay heat removal capability is evaluated under IC CA3.

A decreasing RCS level can be indicated by (ref. 1, 2):

- Tygon tube level indicator
- ICC Reactor Vessel Level Monitoring System (RVLMS)

If RCS water level continues to lower, then escalation to Site Area Emergency would be via IC CS1.

Reference(s):

1. EOP 3505, "Loss of Shutdown Cooling and/or RCS Inventory"
2. OP 3216, "RCS Drain (ICCE)"
3. NEI 99-01 CA1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Significant Loss of RCS inventory

EAL:

CA1.2 Alert

RCS water level **cannot** be monitored for ≥ 15 min. (Note 1)

AND EITHER

- UNPLANNED rise in **any** Table C-1 sump or tank level due to a loss of RCS inventory
- Visual observation of UNISOLABLE RCS leakage

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table C-1 Sumps/Tanks
<ul style="list-style-type: none"> • PRT • CDTT • PDTT • Containment Sump • Unidentified Leakage Sump • Auxiliary Bldg. Sump • ESF Bldg. Sump

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses conditions that are precursors to a loss of the ability to adequately cool irradiated fuel (i.e., a precursor to a challenge to the fuel clad barrier). This condition represents a potential substantial reduction in the level of plant safety.

For this EAL, the inability to monitor RCS level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation. If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels (ref. 1, 2). Sump and/or tank level (Table C-1)

changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the RCS (ref 1, 2, 3).

The 15-minute duration for the loss of level indication was chosen because it is half of the EAL duration specified in IC CS1.

If the RCS inventory level continues to lower, then escalation to Site Area Emergency would be via IC CS1.

Reference(s):

1. OP 3216, "RCS Drain (ICCE)"
2. EOP 3505, "Loss of Shutdown Cooling and/or RCS Inventory"
3. MPS3 FSAR Section 5.2, "Integrity of Reactor Coolant Pressure Boundary"
4. NEI 99-01 CA1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Loss of RCS inventory affecting core decay heat removal capability

EAL:

CS1.1 Site Area Emergency

RVLMS plenum level reading 19% (Note 12)

Note 12: This EAL is only applicable if a RVLMS 19% sensor is operable.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

This IC addresses a significant and prolonged loss of RCS inventory control and makeup capability leading to IMMINENT fuel damage. The lost inventory may be due to a RCS component failure, a loss of configuration control or prolonged boiling of reactor coolant. These conditions entail major failures of plant functions needed for protection of the public and thus warrant a Site Area Emergency declaration.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in RCS level. If RCS level cannot be restored, fuel damage is probable.

Millstone 3 includes in its Inadequate Core Cooling instrumentation a reactor vessel level monitoring system (RVLMS) that is displayed to the operators and can measure discrete reactor vessel water levels from the top of the reactor vessel head to the top of the core plate (19% sensor - approximately 10 inches above the top of the active fuel). The bottom of this instrument's span in the reactor vessel plenum is the lowest available reactor vessel level indicator and is used in this EAL to represent approximately the top of active fuel (ref. 1).

RVLMS is only required to be operable in modes 1, 2 and 3 (ref. 2). For plant conditions in which RVLMS is disconnected or otherwise inoperable, such as in the Refueling mode, classification should be made based on CS1.2 when RCS water level cannot be monitored.

This EAL addresses concerns raised by Generic Letter 88-17, Loss of Decay Heat Removal; SECY 91-283, Evaluation of Shutdown and Low Power Risk Issues; NUREG-1449, Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States; and NUMARC 91-06, Guidelines for Industry Actions to Assess Shutdown Management.

Escalation of the emergency classification level would be via ICs CG1 or RG1.

Reference(s):

1. MPS3 FSAR Section 7.5, "Safety Related Display Instrumentation"
2. MPS3 Technical Specifications Section 3.3.3.6, "Accident Monitoring Instrumentation"

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Attachment 1 Emergency Action Level Technical Bases

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3. EOP 35 FR-C.2, "Response to Degraded Core Cooling Bases"
4. Calculation W3-517-981 RE, "Millstone 3 EOP Setpoint Documentation"
5. NEI 99-01 CS1

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 1 – RCS Level

Initiating Condition: Loss of RCS inventory affecting core decay heat removal capability

EAL:

CS1.2 Site Area Emergency

RCS level **cannot** be monitored for ≥ 30 min. (Note 1)

AND

Core uncover is indicated by **any** of the following:

- UNPLANNED rise in **any** Table C-1 sump or tank level of sufficient magnitude to indicate core uncover
- Visual observation of UNISOLABLE RCS leakage of sufficient magnitude to indicate core uncover
- **Any** CTMT area radiation monitor reading > 3 R/hr (Refueling Mode)
- Erratic source range monitor indications

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table C-1 Sumps/Tanks
<ul style="list-style-type: none"> • PRT • CDTT • PDTT • Containment Sump • Unidentified Leakage Sump • Auxiliary Bldg. Sump • ESF Bldg. Sump

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses a significant and prolonged loss of RCS inventory control and makeup capability leading to IMMINENT fuel damage. The lost inventory may be due to a RCS component failure, a loss of configuration control or prolonged boiling of reactor coolant. These conditions entail major failures of plant functions needed for protection of the public and thus warrant a Site Area Emergency declaration.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in reactor vessel level. If RCS level cannot be restored, fuel damage is probable.

In this EAL, the 30-minute criterion is tied to a readily recognizable event start time (i.e., the total loss of ability to monitor level), and allows sufficient time to monitor, assess and correlate reactor and plant conditions to determine if core uncover has actually occurred (i.e., to account for various accident progression and instrumentation uncertainties). It also allows sufficient time for performance of actions to terminate leakage, recover inventory control/makeup equipment and/or restore level monitoring.

The inability to monitor RCS level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation (ref. 1, 2). If water level cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels (Table C-1). Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the RCS (ref. 1, 2, 3).

If the make-up rate to the RCS unexplainably rises above the pre-established rate, a loss of RCS inventory may be occurring even if the source of the leakage cannot be immediately identified. Visual observation of leakage from systems connected to the RCS that cannot be isolated could also be indicative of a loss of RCS inventory.

In the Refueling mode, as water level in the reactor vessel lowers, the dose rate above the core will rise. The dose rate due to this core shine should result in on-scale indications of > 3 R/hr on CTMT area radiation monitors (ref. 4).

Post-TMI accident studies indicated that the installed PWR nuclear instrumentation will operate erratically when the core is uncovered and that this should be used as a tool for making such determinations.

This EAL addresses concerns raised by Generic Letter 88-17, Loss of Decay Heat Removal; SECY 91-283, Evaluation of Shutdown and Low Power Risk Issues; NUREG-1449, Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States; and NUMARC 91-06, Guidelines for Industry Actions to Assess Shutdown Management.

Escalation of the emergency classification level would be via IC CG1 or AG1

Reference(s):

1. OP 3216, "RCS Drain (ICCE)"
2. EOP 3505, "Loss of Shutdown Cooling and/or RCS Inventory"
3. MPS3 FSAR Section 5.2, "Integrity of Reactor Coolant Pressure Boundary"

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4. Calculation RA-0078, "Verification of Rad Monitor Response to Core Uncovery"
5. Millstone Unit 3 Radiation Monitor Manual
6. NEI 99-01 CS1

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 1 – RCS Level
Initiating Condition: Loss of RCS inventory affecting fuel clad integrity with containment challenged

EAL:

CG1.1 General Emergency

RVLMS plenum level reading 19% for ≥ 30 min. (Notes 1, 12)

AND

Any Containment Challenge indication, Table C-2

- Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.
- Note 6: If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, declaration of a General Emergency is **not** required.
- Note 12: This EAL is only applicable if a RVLMS 19% sensor is operable.

Table C-2 Containment Challenge Indications

- CONTAINMENT CLOSURE **not** established (Note 6)
- CTMT hydrogen concentration > 4%
- UNPLANNED increase in CTMT pressure

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

Millstone 3 includes in its Inadequate Core Cooling instrumentation a reactor vessel level monitoring system (RVLMS) that is displayed to the operators and can measure discrete reactor vessel water levels from the top of the reactor vessel head to the top of the core plate (19% sensor - approximately 10 inches above the top of the active fuel). The bottom of this instrument's span in the reactor vessel plenum is the lowest available reactor vessel level indicator and is used in this EAL to represent approximately the top of active fuel (ref. 1, 3).

RVLMS is only required to be operable in modes 1, 2 and 3 (ref. 2). For plant conditions in which RVLMS is disconnected or otherwise inoperable, such as in the Refueling mode, classification should be made based on CG1.2 when RCS water level cannot be monitored.

This IC addresses the inability to restore and maintain RCS level above the top of active fuel with containment challenged. This condition represents actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity. Releases can be reasonably expected to exceed EPA PAG exposure levels offsite for more than the immediate site area.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in reactor vessel level. If RCS level cannot be restored, fuel damage is probable.

Three conditions are associated with a challenge to containment's capability to serve as an effective barrier to fission product release:

1. With CONTAINMENT CLOSURE not established, there is a high potential for a direct and unmonitored release of radioactivity to the environment. If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, then declaration of a General Emergency is not required.
2. The existence of an explosive mixture means, at a minimum, that the containment atmospheric hydrogen concentration is sufficient to support a hydrogen burn (i.e., at the lower deflagration limit of 4%). A hydrogen burn will raise containment pressure and could result in collateral equipment damage leading to a loss of containment integrity. It therefore represents a challenge to containment integrity.

In the early stages of a core uncover event, it is unlikely that hydrogen buildup due to a core uncover could result in an explosive gas mixture in containment. However, containment monitoring and/or sampling should be performed to verify this assumption and a General Emergency declared if it is determined that hydrogen concentration has exceeded the minimum necessary to support a hydrogen burn (4%) (ref. 4). If all installed hydrogen gas monitors are out-of-service during an event leading to fuel cladding damage, it may not be possible to obtain a containment hydrogen gas concentration reading as ambient conditions within the containment will preclude personnel access. During periods when installed containment hydrogen gas monitors are out-of-service, operators may use the other listed indications to assess whether or not containment is challenged.

3. Any UNPLANNED rise in containment pressure in the Cold Shutdown or Refueling mode indicates a potential challenge of CONTAINMENT CLOSURE capability due to likely reduced pressure capability of the containment (temporary barriers installed for CONTAINMENT CLOSURE). UNPLANNED containment pressure rise indicates CONTAINMENT CLOSURE cannot be assured and the containment cannot be relied upon as a barrier to fission product release.

This EAL addresses concerns raised by Generic Letter 88-17, *Loss of Decay Heat Removal*; SECY 91-283, *Evaluation of Shutdown and Low Power Risk Issues*; NUREG-1449, *Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States*; and NUMARC 91-06, *Guidelines for Industry Actions to Assess Shutdown Management*.

Reference(s):

1. MPS3 FSAR Section 7.5, "Safety Related Display Instrumentation"
2. MPS3 Technical Specifications Section 3.3.3.6, "Accident Monitoring Instrumentation"
3. EOP 35 FR-C.2, "Response to Degraded Core Cooling Bases"
4. MP-CALC-ENG-PRA98NQA-01555S3, "SAMG – Computational Aids and Setpoints"
5. NEI 99-01 CG1

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 1 – RCS Level
Initiating Condition: Loss of RCS inventory affecting fuel clad integrity with containment challenged

EAL:

CG1.2 General Emergency

RCS level **cannot** be monitored for ≥ 30 min. (Note 1)

AND

Core uncover is indicated by **any** of the following:

- UNPLANNED rise in **any** Table C-1 sump or tank level of sufficient magnitude to indicate core uncover
- Visual observation of UNISOLABLE RCS leakage of sufficient magnitude to indicate core uncover
- **Any** CTMT area radiation monitor reading > 3 R/hr (Refueling Mode)
- Erratic source range monitor indications

AND

Any Containment Challenge indication, Table C-2

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 6: If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, declaration of a General Emergency is **not** required.

Table C-1 Sumps/Tanks

- PRT
- CDTT
- PDTT
- Containment Sump
- Unidentified Leakage Sump
- Auxiliary Bldg. Sump
- ESF Bldg. Sump

Table C-2 Containment Challenge Indications

- CONTAINMENT CLOSURE **not** established (Note 6)
- CTMT hydrogen concentration > 4%
- UNPLANNED increase in CTMT pressure

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

This IC addresses the inability to restore and maintain RCS level above the top of active fuel with containment challenged. This condition represents actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity. Releases can be reasonably expected to exceed EPA PAG exposure levels offsite for more than the immediate site area.

Following an extended loss of core decay heat removal and inventory makeup, decay heat will cause reactor coolant boiling and a further reduction in reactor vessel level. If RCS level cannot be restored, fuel damage is probable.

The inability to monitor RCS level may be caused by instrumentation and/or power failures, or water level dropping below the range of available instrumentation (ref. 1, 2). If water level

cannot be monitored, operators may determine that an inventory loss is occurring by observing changes in sump and/or tank levels. Sump and/or tank level changes must be evaluated against other potential sources of water flow to ensure they are indicative of leakage from the RCS (ref. 2, 3, 4).

If the make-up rate to the RCS unexplainably rises above the pre-established rate, a loss of RCS inventory may be occurring even if the source of the leakage cannot be immediately identified. Visual observation of leakage from systems connected to the RCS that cannot be isolated could also be indicative of a loss of RCS inventory.

In the Refueling mode, as water level in the reactor vessel lowers, the dose rate above the core will rise. The dose rate due to this core shine should result in on-scale indications on area radiation monitors. A reading > 3 R/hr on any CTMT area radiation monitor such as RMS-4A and RMS-5A is indicative of likely core uncover while in the Refueling mode. Although RMS-4A and RMS-5A are not in direct line of sight of the reactor core when the vessel head is removed, scatter from the air and containment dome would result in readings of approximately 3 R/hr when water level is at the top of active fuel (ref. 4, 5).

Post-TMI accident studies indicated that the installed PWR nuclear instrumentation will operate erratically when the core is uncovered and that this should be used as a tool for making such determinations.

The 30-minute criterion is tied to a readily recognizable event start time (i.e., the total loss of ability to monitor level), and allows sufficient time to monitor, assess and correlate reactor and plant conditions to determine if core uncover has actually occurred (i.e., to account for various accident progression and instrumentation uncertainties). It also allows sufficient time for performance of actions to terminate leakage, recover inventory control/makeup equipment and/or restore level monitoring.

Three conditions are associated with a challenge to containment's capability to serve as an effective barrier to fission product release:

1. With CONTAINMENT CLOSURE not established, there is a high potential for a direct and unmonitored release of radioactivity to the environment. If CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute time limit, then declaration of a General Emergency is not required.
2. The existence of an explosive mixture means, at a minimum, that the containment atmospheric hydrogen concentration is sufficient to support a hydrogen burn (i.e., at the lower deflagration limit of 4%). A hydrogen burn will raise containment pressure and could result in collateral equipment damage leading to a loss of containment integrity. It therefore represents a challenge to containment integrity.

In the early stages of a core uncover event, it is unlikely that hydrogen buildup due to a core uncover could result in an explosive gas mixture in containment. However, containment monitoring and/or sampling should be performed to verify this assumption and a General Emergency declared if it is determined that hydrogen concentration has exceeded the minimum necessary to support a hydrogen burn (4%) (ref. 6). If all installed hydrogen gas monitors are out-of-service during an event leading to fuel cladding damage, it may not be possible to obtain a containment hydrogen gas concentration reading as ambient conditions within the containment will preclude

personnel access. During periods when installed containment hydrogen gas monitors are out-of-service, operators may use the other listed indications to assess whether or not containment is challenged.

3. Any UNPLANNED rise in containment pressure in the Cold Shutdown or Refueling mode indicates a potential challenge of CONTAINMENT CLOSURE capability due to likely reduced pressure capability of the containment (temporary barriers installed for CONTAINMENT CLOSURE). UNPLANNED containment pressure rise indicates CONTAINMENT CLOSURE cannot be assured and the containment cannot be relied upon as a barrier to fission product release.

This EAL addresses concerns raised by Generic Letter 88-17, *Loss of Decay Heat Removal*; SECY 91-283, *Evaluation of Shutdown and Low Power Risk Issues*; NUREG-1449, *Shutdown and Low-Power Operation at Commercial Nuclear Power Plants in the United States*; and NUMARC 91-06, *Guidelines for Industry Actions to Assess Shutdown Management*.

Reference(s):

1. OP 3216, "RCS Drain (ICCE)"
2. EOP 3505, "Loss of Shutdown Cooling and/or RCS Inventory"
3. MPS3 FSAR Section 5.2, "Integrity of Reactor Coolant Pressure Boundary"
4. Calculation RA-0078, "Verification of Rad Monitor Response to Core Uncovery"
5. Millstone Unit 3 "Radiation Monitor Manual"
6. MP-CALC-ENG-PRA98NQA-01555S3, "SAMG – Computational Aids and Setpoints"
7. NEI 99-01 CG1

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 2 – Loss of Emergency AC Power
Initiating Condition: Loss of **all** but one AC power source to emergency buses for 15 minutes or longer

EAL:

CU2.1 Unusual Event

AC power capability, Table C-3, to 4.16 kV emergency buses 34C and 34D reduced to a single power source for ≥ 15 min. (Note 1)

AND

Any additional single power source failure will result in loss of **all** AC power to SAFETY SYSTEMS

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table C-3 AC Power Sources	
Offsite	
<ul style="list-style-type: none"> • Unit 3 Normal Station Service Transformer A (NSST) • Unit 3 Reserve Station Service Transformer (RSST) 	
Onsite	
<ul style="list-style-type: none"> • EDG A • EDG B • SBO Diesel Generator (if already aligned) 	

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling, DEF - Defueled

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

Table C-3 provides a list of offsite and onsite AC electrical power sources credited for this EAL. The AC power source (SBO Diesel Generator) annotated "(if already aligned)" requires more than 15 minutes to establish and therefore is only credited if the source was already aligned at the time of AC power loss.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC describes a significant degradation of offsite and onsite AC power sources such that any additional single failure would result in a loss of all AC power to SAFETY SYSTEMS. In this condition, the sole AC power source may be powering one, or more than one, train of safety-related equipment.

When in the cold shutdown, refueling, or defueled mode, this condition is not classified as an Alert because of the increased time available to restore another power source to service. Additional time is available due to the reduced core decay heat load, and the lower temperatures and pressures in various plant systems. Thus, when in these modes, this condition is considered to be a potential degradation of the level of safety of the plant.

An "AC power source" is a source recognized in AOPs and EOPs, and capable of supplying required power to an emergency bus. Some examples of this condition are presented below.

- A loss of all offsite power with a concurrent failure of all but one emergency power source (e.g., an onsite diesel generator).
- A loss of all offsite power and loss of all emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being back-fed from the unit main transformer.
- A loss of emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being fed from an offsite power source.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of power.

The subsequent loss of the remaining single power source would escalate the event to an Alert in accordance with IC CA2.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.

- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

This cold condition EAL is equivalent to the hot condition EAL MA1.1.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS FSAR Section 8.2, "Off Site Power System"
3. MPS FSAR Section 8.3, "On Site Power System"
4. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
5. NEI 99-01 CU2

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 2 – Loss of Emergency AC Power

Initiating Condition: Loss of **all** offsite and **all** onsite AC power to emergency buses for 15 minutes or longer

EAL:

CA2.1 Alert

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 34C and 34D for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling, DEF - Defueled

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators, provided it can be aligned within the 15 minute classification criteria.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC addresses a total loss of AC power that compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink.

When in the cold shutdown, refueling, or defueled mode, this condition is not classified as a Site Area Emergency because of the increased time available to restore an emergency bus to service. Additional time is available due to the reduced core decay heat load, and the lower temperatures and pressures in various plant systems. Thus, when in these modes, this condition represents an actual or potential substantial degradation of the level of safety of the plant.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs CS1 or RS1.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

This cold condition EAL is equivalent to the hot condition EAL MS1.1.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS FSAR Section 8.2, "Off Site Power System"
3. MPS FSAR Section 8.3, "On Site Power System"
4. EOP 3501, "Loss of All AC Power (Mode 5, 6 and Zero)"
5. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
6. NEI 99-01 CU2

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 3 – RCS Temperature

Initiating Condition: UNPLANNED increase in RCS temperature

EAL:

CU3.1 Unusual Event

UNPLANNED rise in RCS temperature to > 200°F

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

UNPLANNED- A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

In the absence of reliable RCS temperature indication caused by the loss of decay heat removal capability, classification should be based on time to boil data when in Mode 6 or the RCS is not intact in Mode 5 (ref. 1). If the RCS is intact, classification should be based on the RCS pressure rise criteria of CA3.1. Guidance for calculating RCS time to 200°F is provided on the Shutdown Safety Assessment Checklist (ref. 3).

This EAL addresses an UNPLANNED increase in RCS temperature above the Technical Specification cold shutdown temperature limit and represents a potential degradation of the level of safety of the plant (ref. 1). If the RCS is not intact and CONTAINMENT CLOSURE is not established during this event, the DSEO/ADTS should also refer to EAL CA3.1.

A momentary UNPLANNED excursion above the Technical Specification cold shutdown temperature limit when the heat removal function is available does not warrant a classification.

This EAL involves a loss of decay heat removal capability, or an addition of heat to the RCS in excess of that which can currently be removed, such that reactor coolant temperature cannot be maintained below the cold shutdown temperature limit specified in Technical Specifications.

During this condition, there is no immediate threat of fuel damage because the core decay heat load has been reduced since the cessation of power operation.

During an outage, the level in the reactor vessel will normally be maintained at or above the reactor vessel flange. Refueling evolutions that lower water level below the reactor vessel flange are carefully planned and controlled. A loss of forced decay heat removal at reduced inventory may result in a rapid increase in reactor coolant temperature depending on the time after shutdown (ref. 3).

Escalation to Alert would be via IC CA1 based on an inventory loss or IC CA3 based on exceeding plant configuration-specific time criteria.

Reference(s):

1. Technical Specifications Table 1.2
2. EOP 3505, "Loss of Shutdown Cooling And/Or RCS Inventory"
3. OU-M3-201, "Shutdown Safety Assessment Checklist"
4. NEI 99-01 CU3

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 3 – RCS Temperature

Initiating Condition: UNPLANNED rise in RCS temperature

EAL:

CU3.2 Unusual Event

Loss of all RCS temperature and RCS water level indication for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6- Refueling

Definition(s):

CONTAINMENT CLOSURE - The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

Basis:

This EAL addresses the inability to determine RCS temperature and level, and represents a potential degradation of the level of safety of the plant. If the RCS is not intact and CONTAINMENT CLOSURE is not established during this event, the DSEO/ADTS should also refer to EAL CA3.1.

This EAL reflects a condition where there has been a significant loss of instrumentation capability necessary to monitor RCS conditions and operators would be unable to monitor key parameters necessary to assure core decay heat removal. During this condition, there is no immediate threat of fuel damage because the core decay heat load has been reduced since the cessation of power operation.

RCS level indications include (ref. 2):

- 3RCS-LT51A RHR A Transmitter
- 3RCS-LT51B RHR B Transmitter
- 3RCS-LT51C Letdown Transmitter
- ICC Reactor Vessel Level Monitoring System (RVLMS)

- Tygon Tube

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation to Alert would be via IC CA1 based on an inventory loss or IC CA3 based on exceeding plant configuration-specific time criteria.

Reference(s):

1. EOP 3505, "Loss of Shutdown Cooling And/Or RCS Inventory"
2. OP 3216, "Reactor Coolant System Drain (ICCE)"
3. NEI 99-01 CU3

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 3 – RCS Temperature

Initiating Condition: Inability to maintain plant in cold shutdown

EAL:

CA3.1 Alert

UNPLANNED rise in RCS temperature to > 200°F for > Table C-4 duration
(Notes 1, 13)

OR

UNPLANNED RCS pressure rise > 10 psi
(This pressure threshold does not apply during water-solid plant conditions)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded.

Note 13: If an RCS heat removal system is in operation within the applicable Table C-4 heat-up duration and RCS temperature is being reduced, the EAL is **not** applicable.

Table C-4 RCS Heat-up Duration Thresholds		
RCS Status	CONTAINMENT CLOSURE Status	Heat-up Duration
Intact <u>AND</u> not reduced inventory		60 min.
Not intact <u>OR</u> reduced inventory	Established	20 min.
	Not established	0 min.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

CONTAINMENT CLOSURE - The procedurally defined conditions or actions taken to secure containment and associated structures, systems, and components as a functional barrier to fission product release under shutdown conditions.

Containment Closure is established when all of the following conditions exist:

- The equipment access hatch is closed and held in place by a minimum of four bolts **AND** any closed hatch penetrations do **not** provide direct access from the containment atmosphere to the outside atmosphere.
- A minimum of one door in the personnel access hatch is closed.
- Each penetration providing direct access from the containment atmosphere to the outside atmosphere (except the equipment access hatch and personnel access hatch) shall be closed by an isolation valve, blind flange, or manual valve.

UNPLANNED- A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

In the absence of reliable RCS temperature indication caused by the loss of decay heat removal capability, classification should be based on the RCS pressure rise criteria when the RCS is intact in Mode 4 or based on time to boil data when in Mode 5 or the RCS is not intact in Mode 4 (ref. 2, 3). Guidance for calculating RCS time to 200°F is provided on the Shutdown Safety Assessment Checklist (ref.4).

The RCS is considered to be at reduced inventory when RCS level is below 19 ft. 6 in. elevation (ref. 3).

This EAL addresses conditions involving a loss of decay heat removal capability or an addition of heat to the RCS in excess of that which can currently be removed. Either condition represents an actual or potential substantial degradation of the level of safety of the plant.

A momentary UNPLANNED excursion above the Technical Specification cold shutdown temperature limit when the heat removal function is available does not warrant a classification.

The RCS Heat-up Duration Thresholds table addresses an increase in RCS temperature when CONTAINMENT CLOSURE is established but the RCS is not intact, or RCS inventory is reduced (e.g., mid-loop operation in PWRs). The 20-minute criterion was included to allow time for operator action to address the temperature increase.

The RCS Heat-up Duration Thresholds table also addresses an increase in RCS temperature with the RCS intact. The status of CONTAINMENT CLOSURE is not crucial in this condition since the intact RCS is providing a high pressure barrier to a fission product release. The 60-minute time frame should allow sufficient time to address the temperature increase without a substantial degradation in plant safety.

Finally, in the case where there is an increase in RCS temperature, the RCS is not intact or is at reduced inventory, and CONTAINMENT CLOSURE is not established, no heat-up duration is allowed (i.e., 0 minutes). This is because 1) the evaporated reactor coolant may be released directly into the containment atmosphere and subsequently to the environment, and 2) there is reduced reactor coolant inventory above the top of irradiated fuel.

The RCS pressure rise threshold provides a pressure-based indication of RCS heat-up in the absence of RCS temperature monitoring capability.

Escalation of the emergency classification level would be via IC CS1 or RS1.

Reference(s):

1. Technical Specifications Table 1.2
2. EOP 3505, "Loss of Shutdown Cooling And/Or RCS Inventory"
3. OP 3216, "Reactor Coolant System Drain (ICCE)"
4. OU-M3-201, "Shutdown Safety Assessment Checklist"
5. NEI 99-01 CA3

Category: C – Cold Shutdown / Refueling System Malfunction

Subcategory: 4 – Loss of Vital DC Power

Initiating Condition: Loss of vital DC power for 15 minutes or longer

EAL:

CU4.1 Unusual Event

Indicated voltage is < 105 VDC on **required** vital 125 VDC battery bus 1 **OR** 2 for ≥15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis

The vital (Class 1E) 125 V DC power system consists of four physically and electrically separated channels (buses). Each bus has a 60 cell battery bank with a two-hour rating before reaching the minimum bus voltage of 105 Volts (1.75 volts/cell).

Battery buses 1 and 2 power reactor protection, engineered safety features, vital loads and instrumentation (ref. 1, 2, 3). Battery buses 3 and 4 are not credited for this EAL.

This IC addresses a loss of vital DC power which compromises the ability to monitor and control operable SAFETY SYSTEMS when the plant is in the cold shutdown or refueling mode. In these modes, the core decay heat load has been significantly reduced, and coolant system temperatures and pressures are lower; these conditions increase the time available to restore a vital DC bus to service. Thus, this condition is considered to be a potential degradation of the level of safety of the plant.

As used in this EAL, "required" means the vital DC buses necessary to support operation of the in-service, or operable, train or trains of SAFETY SYSTEM equipment. For example, if Train A is out-of-service (inoperable) for scheduled outage maintenance work and Train B is in-service (operable), then a loss of vital DC power affecting Train B would require the

declaration of an Unusual Event. A loss of vital DC power to Train A would not warrant an emergency classification.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Depending upon the event, escalation of the emergency classification level would be via IC CA1 or CA3, or an IC in Category M.

This cold condition EAL is equivalent to the hot condition EAL MS2.1.

Reference(s):

1. MPS FSAR Section 8.3.2, "DC Power Systems"
2. AOP 3563, "Loss of DC Bus Power"
3. Technical Specifications Section 3.8.2.2, "D.C. Sources – Shutdown"
4. NEI 99-01 CU4

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 5 – Loss of Communications
Initiating Condition: Loss of **all** onsite or offsite communications capabilities
EAL:

CU5.1 Unusual Event

Loss of **all** Table C-5 onsite communication methods

OR

Loss of **all** Table C-5 State and local agency communication methods

OR

Loss of **all** Table C-5 NRC communication methods

Table C-5 Communication Methods			
System	Onsite	State/ Local	NRC
ENRS / ARCOS		X	
Station Radio System	X	X	
Plant Phone System	X	X	
Public Address System	X		
Gaitronics / Maintenance Jacks	X		
Federal Telephone System (ENS)			X
Commercial Telephone System		X	X
Satellite Phones		X	X
Dedicated Hotlines		X	

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling, DEF – Defueled

Definition(s):

None

Basis:

This IC addresses a significant loss of on-site or offsite communications capabilities. While not a direct challenge to plant or personnel safety, this event warrants prompt notifications to State and local agencies and the NRC.

This IC should be assessed only when extraordinary means are being utilized to make communications possible (e.g., use of non-plant, privately owned equipment, relaying of on-site information via individuals or multiple radio transmission points, individuals being sent to offsite locations, etc.).

The first EAL condition addresses a total loss of the communications methods used in support of routine plant operations.

The second EAL condition addresses a total loss of the communications methods used to notify all State and local agencies of an emergency declaration. The State and local agencies referred to here are the State of Connecticut and affected local communities.

The third EAL condition addresses a total loss of the communications methods used to notify the NRC of an emergency declaration.

This cold condition EAL is equivalent to the hot condition EAL MU7.1.

Reference(s):

1. MPS Emergency Plan Section 7.9, "Communication Systems"
2. MP-26-EPI-FAP07, "Notifications and Communications"
3. NEI 99-01 CU5

Category: C – Cold Shutdown / Refueling System Malfunction
Subcategory: 6 – Hazardous Event Affecting Safety Systems
Initiating Condition: Hazardous event affecting SAFETY SYSTEMS needed for the current operating mode

EAL:

CA6.1 Alert

The occurrence of **any** Table C-6 hazardous event

AND

Event damage has caused indications of degraded performance on one train of a SAFETY SYSTEM needed for the current operating mode

AND EITHER:

- Event damage has caused indications of degraded performance to the second train of the SAFETY SYSTEM needed for the current operating mode
- Event damage has resulted in **VISIBLE DAMAGE** to the second train of the SAFETY SYSTEM needed for the current operating mode

(Notes 9, 10)

Note 9: If the affected SAFETY SYSTEM train was already inoperable or out of service before the hazardous event occurred, then emergency classification is **not** warranted.

Note 10: If the hazardous event **only** resulted in **VISIBLE DAMAGE**, with **no** indications of degraded performance to at least one train of a SAFETY SYSTEM, then this emergency classification is **not** warranted.

Table C-6 Hazardous Events

- Seismic event (earthquake)
- Internal or external FLOODING event
- High winds or tornado strike
- FIRE
- EXPLOSION
- Other events with similar hazard characteristics as determined by the DSEO/ADTS

Mode Applicability:

5 - Cold Shutdown, 6 - Refueling

Definition(s):

EXPLOSION - A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding, arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

VISIBLE DAMAGE - Damage to a SAFETY SYSTEM train that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected SAFETY SYSTEM train.

Basis:

This IC addresses a hazardous event that causes damage to SAFETY SYSTEMS needed for the current operating mode. In order to provide the appropriate context for consideration of an ALERT classification, the hazardous event must have caused indications of degraded SAFETY SYSTEM performance in one train, and there must be either indications of performance issues with the second SAFETY SYSTEM train or VISIBLE DAMAGE to the second train such that the potential exists for this second SAFETY SYSTEM train to have performance issues. In other words, in order for this EAL to be classified, the hazardous event must occur, at least one SAFETY SYSTEM train must have indications of degraded performance, and the second SAFETY SYSTEM train must have indications of degraded performance or VISIBLE DAMAGE such that the potential exists for performance issues. Note that this second SAFETY SYSTEM train is from the same SAFETY SYSTEM that has indications of degraded performance; commercial nuclear power plants are designed to be able to support single system issues without compromising public health and safety from radiological events.

Indications of degraded performance addresses damage to a SAFETY SYSTEM train that is in service/operation since indications for it will be readily available. The indications of degraded performance should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

VISIBLE DAMAGE addresses damage to a SAFETY SYSTEM train that is not in service/operation and that potentially could cause performance issues. Operators will make this determination based on the totality of available event and damage report information. This is intended to be a brief assessment not requiring lengthy analysis or quantification of the damage. This VISIBLE DAMAGE should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

An event affecting equipment common to two or more trains of a safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the common equipment) should be classified as an Alert under this EAL, as appropriate to the plant mode. By affecting the functionality of multiple trains of a safety system, the loss of the common equipment effectively meets the two-train impact criteria that underlie the EALs and bases.

An event affecting a single-train safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the one train) would not be classified under this EAL because the two-train impact criteria that underlie the EALs and bases would not be met. If an event affects a single-train safety system, then the emergency classification should be made based on plant parameters/symptoms meeting the EALs for another IC. Depending upon the circumstances, classification may also occur based on Shift Manager/DSEO/ADTS judgement.

An event that affects two trains of a safety system (e.g., one train has indications of degraded performance and the other VISIBLE DAMAGE) that also has one or more additional trains should be classified as an Alert under this EAL, as appropriate to the plant mode. This approach maintains consistency with the two-train impact criteria that underlie the EALs and bases, and is warranted because the event was severe enough to affect the functionality of two trains of a safety system despite plant design criteria associated with system and system train separation and protection. Such an event may have caused other plant impacts that are not immediately apparent.

Escalation of the emergency classification level would be via IC CS1 or RS1.

This cold condition EAL is equivalent to the hot condition EAL MA8.1.

Reference(s):

1. AOP 3569, "Severe Weather Conditions"
2. AOP 3570, "Earthquake"
3. EP FAQ 2016-002
4. NEI 99-01 CA6

Category E – Independent Spent Fuel Storage Installation (ISFSI)

EAL Group: ANY (EALs in this category are applicable to any plant condition, hot or cold.)

An independent spent fuel storage installation (ISFSI) is a complex that is designed and constructed for the interim storage of spent nuclear fuel and other radioactive materials associated with spent fuel storage. A significant amount of the radioactive material contained within a canister must escape its packaging and enter the biosphere for there to be a significant environmental effect resulting from an accident involving the dry storage of spent nuclear fuel.

A Notification of Unusual Event is declared on the basis of the occurrence of an event of sufficient magnitude that a loaded cask CONFINEMENT BOUNDARY is damaged or violated.

The MPS ISFSI is located wholly within the MPS plant PROTECTED AREA. Therefore any security event related to the ISFSI is classified under Category H1 security event related EALs.

Category: ISFSI

Subcategory: Confinement Boundary

Initiating Condition: Damage to a loaded cask CONFINEMENT BOUNDARY

EAL:

EU1.1 Unusual Event

Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by an on-contact radiation reading on the surface of a loaded spent fuel cask (HSM) > **any** of the following:

- 1,700 mrem/hr on the HSM front bird screen
- 400 mrem/hr on the outside HSM door
- 12 mrem/hr on the HSM end shield wall

Mode Applicability:

All

Definition(s):

CONFINEMENT BOUNDARY- The barrier(s) between spent fuel and the environment once the spent fuel is processed for dry storage. Confinement Boundary is defined as the NUHOMS Dry Shielded Canister (DSC).

INDEPENDENT SPENT FUEL STORAGE INSTALLATION (ISFSI): A complex that is designed and constructed for the interim storage of spent nuclear fuel and other radioactive materials associated with spent fuel storage.

Basis:

This IC addresses an event that results in damage to the CONFINEMENT BOUNDARY of a storage cask containing spent fuel. It applies to irradiated fuel that is licensed for dry storage beginning at the point that the loaded storage cask is sealed. The issues of concern are the creation of a potential or actual release path to the environment, degradation of one or more fuel assemblies due to environmental factors, and configuration changes which could cause challenges in removing the cask or fuel from storage.

The existence of "damage" is determined by radiological survey. The specified EAL threshold values correspond to 2 times the NUHOMS Horizontal Storage Module (HSM) technical specification external surface dose rate limits (ref. 2). The technical specification multiple of "2 times", which is also used in Category R IC RU1, is used here to distinguish between non-emergency and emergency conditions. The emphasis for this classification is the degradation in the level of safety of the spent fuel cask and not the magnitude of the associated dose or dose rate. It is recognized that in the case of extreme damage to a loaded cask, the fact that the "on-contact" dose rate limit is exceeded may be determined based on measurement of a dose rate at some distance from the cask (ref. 1, 2.3).

Security-related events for ISFSIs are covered under Category H ICs.

Reference(s):

1. NUH-003, "UFSAR for the Horizontal Modular Storage System for Irradiated Fuel"
2. MPS2 ETE-NAF-2010-0004, "MPS Unit 2 ISFSI 10 CFR72.212 Evaluation Report Attachment 1 Section 5.4 HSM or HSM-H Dose Rate Evaluation Program"
3. RPM 2.5.9, "Dry Shielded Canisters (DSC) Surveys (ISFSI)"
4. NEI 99-01 E-HU1

Category F – Fission Product Barrier Degradation

EAL Group: Hot Conditions (RCS temperature > 200°F); EALs in this category are applicable only in one or more hot operating modes.

EALs in this category represent threats to the defense in depth design concept that precludes the release of highly radioactive fission products to the environment. This concept relies on multiple physical barriers any one of which, if maintained intact, precludes the release of significant amounts of radioactive fission products to the environment. The primary fission product barriers are:

- A. Fuel Clad Barrier (FC): The Fuel Clad Barrier consists of the cladding material that contains the fuel pellets.
- B. Reactor Coolant System Barrier (RCS): The RCS Barrier includes the RCS primary side and its connections up to and including the pressurizer safety and relief valves, and other connections up to and including the primary isolation valves.
- C. Containment Barrier (CTMT): The Containment Barrier includes the containment building and connections up to and including the outermost containment isolation valves. This barrier also includes the main steam, feedwater, and blowdown line extensions outside the containment building up to and including the outermost secondary side isolation valve. Containment Barrier thresholds are used as criteria for escalation of the Emergency Classification Level (ECL) from an Alert to a Site Area Emergency or a General Emergency.

The EALs in this category require evaluation of the loss and potential loss thresholds listed in the fission product barrier matrix of Table F-1. "Loss" and "Potential Loss" signify the relative damage and threat of damage to the barrier. "Loss" means the barrier no longer assures containment of radioactive materials. "Potential Loss" means integrity of the barrier is threatened and could be lost if conditions continue to degrade. The number of barriers that are lost or potentially lost and the following criteria determine the appropriate emergency classification level:

Alert:

Any loss or any potential loss of either Fuel Clad or RCS Barrier

Site Area Emergency:

Loss or potential loss of any two barriers

General Emergency:

Loss of any two barriers and loss or potential loss of third barrier

The logic used for emergency classification based on fission product barrier monitoring should reflect the following considerations:

- The Fuel Clad Barrier and the RCS Barrier are weighted more heavily than the Containment Barrier.
- Unusual Event ICs associated with RCS and Fuel Clad Barriers are addressed under System Malfunction ICs.

- For accident conditions involving a radiological release, evaluation of the fission product barrier thresholds will need to be performed in conjunction with dose assessments to ensure correct and timely escalation of the emergency classification. For example, an evaluation of the fission product barrier thresholds may result in a Site Area Emergency classification while a dose assessment may indicate that an EAL for General Emergency IC RG1 has been exceeded.
- The fission product barrier thresholds specified within a scheme reflect plant-specific MPS3 design and operating characteristics.
- As used in this category, the term RCS leakage encompasses not just those types defined in Technical Specifications but also includes the loss of RCS mass to any location— inside the containment, an interfacing system, or outside of the containment. The release of liquid or steam mass from the RCS due to the as-designed/expected operation of a relief valve is not considered to be RCS leakage.
- At the Site Area Emergency level, EAL users should maintain cognizance of how far present conditions are from meeting a threshold that would require a General Emergency declaration. For example, if the Fuel Clad and RCS fission product barriers were both lost, then there should be frequent assessments of containment radioactive inventory and integrity. Alternatively, if both the Fuel Clad and RCS fission product barriers were potentially lost, the DSEO/ADTS would have more assurance that there was no immediate need to escalate to a General Emergency.

Category: Fission Product Barrier Degradation

Subcategory: N/A

Initiating Condition: Any loss or any potential loss of either Fuel Clad or RCS

EAL:

FA1.1 Alert

Any loss or any potential loss of **EITHER** Fuel Clad or RCS barrier (Table F-1)

Mode Applicability:

1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown

Definition(s):

None

Basis:

Fuel Clad, RCS and Containment comprise the fission product barriers. Table F-1 lists the fission product barrier thresholds, bases and references.

At the Alert classification level, Fuel Clad and RCS barriers are weighted more heavily than the Containment barrier. Unlike the Containment barrier, loss or potential loss of either the Fuel Clad or RCS barrier may result in the relocation of radioactive materials or degradation of core cooling capability. Note that the loss or potential loss of Containment barrier in combination with loss or potential loss of either Fuel Clad or RCS barrier results in declaration of a Site Area Emergency under EAL FS1.1

Reference(s):

1. NEI 99-01 FA1

Category: Fission Product Barrier Degradation
Subcategory: N/A
Initiating Condition: Loss or potential loss of **any** two barriers
EAL:

FS1.1 Site Area Emergency

Loss or potential loss of **any** two barriers (Table F-1)

Mode Applicability:

1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

Fuel Clad, RCS and Containment comprise the fission product barriers. Table F-1 lists the fission product barrier thresholds, bases and references.

At the Site Area Emergency classification level, each barrier is weighted equally. A Site Area Emergency is therefore appropriate for any combination of the following conditions:

- One barrier loss and a second barrier loss (i.e., loss - loss)
- One barrier loss and a second barrier potential loss (i.e., loss - potential loss)
- One barrier potential loss and a second barrier potential loss (i.e., potential loss - potential loss)

At the Site Area Emergency classification level, the ability to dynamically assess the proximity of present conditions with respect to the threshold for a General Emergency is important. For example, the existence of Fuel Clad and RCS Barrier loss thresholds in addition to offsite dose assessments would require continual assessments of radioactive inventory and Containment integrity in anticipation of reaching a General Emergency classification. Alternatively, if both Fuel Clad and RCS potential loss thresholds existed, they would have greater assurance that escalation to a General Emergency is less *IMMINENT*.

Reference(s):

1. NEI 99-01 FS1

Category: Fission Product Barrier Degradation

Subcategory: N/A

Initiating Condition: Loss of **any** two barriers and loss or potential loss of the third barrier

EAL:

FG1.1 General Emergency

Loss of **any** two barriers

AND

Loss or potential loss of the third barrier (Table F-1)

Mode Applicability:

1 - Power Operation, 2 - Startup, 3 - Hot Standby, 4 - Hot Shutdown

Definition(s):

None

Basis:

Fuel Clad, RCS and Containment comprise the fission product barriers. Table F-1 lists the fission product barrier thresholds, bases and references.

At the General Emergency classification level each barrier is weighted equally. A General Emergency is therefore appropriate for any combination of the following conditions:

- Loss of Fuel Clad, RCS and Containment Barriers
- Loss of Fuel Clad and RCS Barriers with potential loss of Containment Barrier
- Loss of RCS and Containment Barriers with potential loss of Fuel Clad Barrier
- Loss of Fuel Clad and Containment Barriers with potential loss of RCS Barrier

Reference(s):

1. NEI 99-01 FG1

Table F-1 Fission Product Barrier Threshold Matrix & Bases

Table F-1 lists the threshold conditions that define the Loss and Potential Loss of the three fission product barriers (Fuel Clad, Reactor Coolant System, and Containment). The table is structured so that each of the three barriers occupies adjacent columns. Each fission product barrier column is further divided into two columns; one for Loss thresholds and one for Potential Loss thresholds.

The first column of the table (to the left of the Fuel Clad Loss column) lists the categories (types) of fission product barrier thresholds. The fission product barrier categories are:

- A. RCS or SG Tube Leakage
- B. Inadequate Heat removal
- C. CTMT Radiation / RCS Activity
- D. CTMT Integrity or Bypass
- E. DSEO/ADTS Judgment

Each category occupies a row in Table F-1 thus forming a matrix defined by the categories. The intersection of each row with each Loss/Potential Loss column forms a cell in which one or more fission product barrier thresholds appear. If NEI 99-01 does not define a threshold for a barrier Loss/Potential Loss, the word "None" is entered in the cell.

Thresholds are assigned sequential numbers within each barrier column beginning with number one.

If a cell in Table F-1 contains more than one numbered threshold, each of the numbered thresholds, if exceeded, signifies a Loss or Potential Loss of the barrier. It is not necessary to exceed all of the thresholds in a category before declaring a barrier Loss/Potential Loss.

Subdivision of Table F-1 by category facilitates association of plant conditions to the applicable fission product barrier Loss and Potential Loss thresholds. This structure promotes a systematic approach to assessing the classification status of the fission product barriers.

When equipped with knowledge of plant conditions related to the fission product barriers, the EAL-user first scans down the category column of Table F-1, locates the likely category and then reads across the fission product barrier Loss and Potential Loss thresholds in that category to determine if a threshold has been exceeded. If a threshold has not been exceeded, the EAL-user proceeds to the next likely category and continues review of the thresholds in the new category.

If the EAL-user determines that any threshold has been exceeded, by definition, the barrier is lost or potentially lost – even if multiple thresholds in the same barrier column are exceeded, only that one barrier is lost or potentially lost. The EAL-user must examine each of the three fission product barriers to determine if other barrier thresholds in the category are lost or potentially lost. For example, if containment radiation is sufficiently high, a Loss of the Fuel Clad and RCS Barriers and a Potential Loss of the Containment Barrier can occur. Barrier Losses and Potential Losses are then applied to the algorithms given in EALs FG1.1, FS1.1, and FA1.1 to determine the appropriate emergency classification.

Millstone Power Station Unit 3
Emergency Action Level Technical Bases Document
Attachment 1 Emergency Action Level Technical Bases

Serial No. 18-364
Docket Nos. 50-423; 72-47
Enclosure 3; Attachment 3C

Table F-1 Fission Product Barrier Threshold Matrix

	Fuel Clad Barrier (FC)		Reactor Coolant System Barrier (RCS)		Containment Barrier (CTMT)	
Category	Loss	Potential Loss	Loss	Potential Loss	Loss	Potential Loss
A RCS or SG Tube Leakage	None	None	1. An automatic or manual Safety Injection (SI) actuation required by <u>EITHER</u> : <ul style="list-style-type: none">• UNISOLABLE RCS leakage• SG tube RUPTURE	1. Operation of a second charging pump is required by <u>EITHER</u> : <ul style="list-style-type: none">• UNISOLABLE RCS leakage• SG tube leakage 2. Integrity-RED path conditions met	1. A leaking or RUPTURED SG is FAULTED outside of CTMT	None
B Inadequate Heat Removal	1. Core Cooling-RED PATH conditions met	1. Core Cooling-ORANGE Path conditions met 2. Heat Sink-RED Path conditions met AND Required feedwater flow cannot be established within 15 min. (Note 1)	None	3. Heat Sink-RED Path conditions met AND Required feedwater flow cannot be established within 15 min. (Note 1)	None	1. Core Cooling-RED PATH conditions met AND Restoration procedures not effective within 15 min. (Note 1)
C CTMT Radiation / RCS Activity	2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column Fuel Clad Loss (Note 14) 3. Coolant activity > 300 µCi/gm dose equivalent l-131 4. Dose rate at 1 ft. from an unpressurized RCS sample ≥Table F-3 5. Sample line dose rate threshold ≥Table F-4	None	2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column RCS Loss (Note 14)	None	None	2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column CTMT Potential Loss (Note 14)
D CTMT Integrity or Bypass	None	None	None	None	2. CTMT isolation (CIA) Phase A is required AND EITHER : <ul style="list-style-type: none">• CTMT integrity has been lost based on DSEO/ADTS judgment• UNISOLABLE pathway from CTMT atmosphere to the environment exists 3. Indications of RCS leakage outside of CTMT	3. CTMT pressure > 60 psia 4. CTMT hydrogen concentration > 4% 5. CTMT pressure > 25 psia with < one full train of CTMT heat removal systems (Note 11) operating per design for ≥15 min. (Note 1)
E DSEO/ADTS Judgment	6. Any condition in the opinion of the DSEO/ADTS that indicates loss of the fuel clad barrier	3. Any condition in the opinion of the DSEO/ADTS that indicates potential loss of the fuel clad barrier	3. Any condition in the opinion of the DSEO/ADTS that indicates loss of the RCS barrier	4. Any condition in the opinion of the DSEO/ADTS that indicates potential loss of the RCS barrier	4. Any condition in the opinion of the DSEO/ADTS that indicates loss of the CTMT barrier	6. Any condition in the opinion of the DSEO/ADTS that indicates potential loss of the CTMT barrier

Barrier: Fuel Clad

Category: A. RCS or SG Tube Leakage

Degradation Threat: Loss

Threshold:

None

Barrier: Fuel Clad

Category: A. RCS or SG Tube Leakage

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Fuel Clad

Category: B. Inadequate Heat Removal

Degradation Threat: Loss

Threshold:

- | |
|---|
| 1. Core Cooling-RED Path conditions met |
|---|

Definition(s):

None

Basis:

This condition indicates temperatures within the core are sufficient to cause significant superheating of reactor coolant.

The loss threshold is based on meeting the Core Cooling Red Path criteria for Core Exit Thermocouple readings ($> 1,200^{\circ}\text{F}$) (ref. 1, 2).

Reference(s):

1. EOP 35 F-0.2, "CSFST Core Cooling"
2. EOP 35 FR-C.1, "Response to Inadequate Core Cooling"
3. NEI 99-01 Inadequate Heat Removal Fuel Clad Loss 2.A

Barrier: Fuel Clad

Category: B. Inadequate Heat Removal

Degradation Threat: Potential Loss

Threshold:

- | |
|--|
| 1. Core Cooling-ORANGE Path conditions met |
|--|

Definition(s):

None

Basis:

This condition indicates a reduction in reactor vessel water level sufficient to allow the onset of heat-induced cladding damage.

The potential loss threshold is based on meeting the Core Cooling Orange Path criteria for Core Exit Thermocouple readings ($> 718^{\circ}\text{F}$) or RVLMS plenum level indication ($< 19\%$) with a loss of subcooling margin (ref. 1, 2).

Reference(s):

1. EOP 35 F-0.2, "CSFST Core Cooling"
2. EOP 35 FR-C.1, "Response to Inadequate Core Cooling"
3. NEI 99-01 Inadequate Heat Removal Fuel Clad Potential Loss 2.A

Barrier: Fuel Clad
Category: B. Inadequate Heat Removal
Degradation Threat: Potential Loss
Threshold:

2. Heat Sink-RED Path conditions met

AND

Required feedwater flow **cannot** be established within 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Definition(s):

None

Basis:

The potential loss threshold is based on meeting the Heat Sink Red Path criteria of both of the following conditions existing (ref. 1):

- Narrow Range level in all SGs < 8% (42% adverse CTMT)
- Total feedwater flow to SGs < 530 gpm

This condition indicates an extreme challenge to the ability to remove RCS heat using the steam generators (i.e., loss of an effective secondary-side heat sink). This condition represents a potential loss of the Fuel Clad Barrier. In accordance with EOPs, there may be unusual accident conditions during which operators intentionally reduce the heat removal capability of the steam generators; during these conditions, classification using this threshold is not warranted.

If inventory could not be restored, steam generator dry out could occur within about 30 minutes. If feedwater flow could not be reestablished, FR-H.1 would require establishment of RCS bleed and feed to maintain core cooling and to assure sufficient RCS inventory. If neither of these actions can be accomplished, then fuel clad heatup will begin.

The phrase "*Required feedwater flow **cannot** be established within 15 min.*" precludes the need for classification for conditions that indicate the feed flow requirements have been satisfied in accordance with FR-H.1. For example, FR-H.1 is entered from CSFST Heat Sink-Red. Step 1 tells the operator to determine if heat sink is required by checking that RCS pressure is greater than any non-faulted SG pressure and RCS T_{hot} is greater than 350°F. If these conditions exist, Heat Sink is required. Otherwise, the operator is to either go to the procedure and step in effect or place RHR in service for heat removal. For large LOCA events inside the Containment, the SGs are irrelevant because heat removal through the containment heat removal systems takes place. Therefore, Heat Sink Red is not applicable and should not be assessed for EAL classification because a LOCA event alone should not require higher than an Alert classification (ref. 1, 2).

However, required feed flow may be less than 530 gpm (Heat Sink Red Path entry criteria) and still be acceptable based on the exit criteria from FR-H.1. Per FR-H.1, feed flow can be less than 530 gpm and be considered acceptable if reduced feed flow is due to operator action or if WR level in at least one SG is increasing and Core Exit TCs are stable or decreasing.

The 15 minute allowance provides time for equipment actuation, operator action to restore feedwater flow, and verification that adequate feedwater flow has been established.

Meeting this threshold results in a Site Area Emergency because this threshold is identical to RCS Barrier Potential Loss threshold B.3; both will be met. This condition warrants a Site Area Emergency declaration because inadequate RCS heat removal may result in fuel heat-up sufficient to damage the cladding and increase RCS pressure to the point where mass will be lost from the system.

Reference(s):

1. EOP 35 F-0.3, "CSFST Heat Sink"
2. EOP 35 FR-H.1, "Response to Loss of Secondary Heat Sink"
3. NEI 99-01 Inadequate Heat Removal Fuel Clad Potential Loss 2.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column Fuel Clad Loss (Note 14)

Note 14: Readings are considered sustained when effects of TIC have dissipated.

Table F-2 CTMT High Range Radiation Monitor Barrier Thresholds RE-04A/05A			
Time > Shutdown (hrs)	Fuel Clad Loss (R/hr)	RCS Loss (R/hr)	CTMT Potential Loss (R/hr)
≤ 2	140	5	560
$> 2 - \leq 4$	90	5	360
$> 4 - \leq 8$	50	5	200
$> 8 - \leq 14$	25	5	100
> 14	12	5	48

Definition(s):

None

Basis:

Containment radiation monitor readings greater than the Table F-2 Fuel Clad Loss column threshold indicate the release of reactor coolant, with elevated activity indicative of fuel damage, into the containment. The reading is derived assuming the instantaneous release and dispersal of the reactor coolant noble gas and iodine inventory associated with a concentration of 5% clad failure into the containment atmosphere. Reactor coolant concentrations of this magnitude are several times larger than the maximum concentrations (including iodine spiking) allowed within Technical Specifications and are therefore indicative of fuel damage (approximately 5 % clad failure depending on core inventory and RCS volume) (ref. 1, 2).

The values specified in Table F-2 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Time periods were chosen to fit monitor response (fast changes in response early following reactor shutdown are broken up into smaller time periods to better approximate expected change). Values were chosen within each time period to minimize error (<50%) to the highest and lowest response within the range.

Time after shutdown values are provided to account for radioactive decay.

The effects of thermally induced currents (TIC) are significant but of relatively short duration. Generally, the effects of TIC will dissipate within 2-5 minutes. Until the TIC affects have dissipated, the RE04A/RE05A readings should not be considered valid.

The radiation monitor reading in this threshold is higher than that specified for RCS Barrier Loss threshold C.2 since it indicates a loss of both the Fuel Clad barrier and the RCS barrier. Note that a combination of the two monitor readings appropriately escalates the ECL to a Site Area Emergency.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Damage Computer Program
2. Calculation RA-0075, "Millstone Unit 3 Expected Containment High Range Radiation Monitor Response to a LOCA Based on Fuel Rod Gap Fractions Defined in NUREG 1228"
3. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.A

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

3. Coolant activity > 300 $\mu\text{Ci/gm}$ dose equivalent I-131

Definition(s):

None

Basis:

This threshold indicates that RCS radioactivity concentration is greater than 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

It is recognized that sample collection and analysis of reactor coolant with highly elevated activity levels could require several hours to complete. Nonetheless, a sample-related threshold is included as a backup to other indications.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

4. Dose rate at 1 ft. from an unpressurized RCS sample \geq Table F-3

Table F-3 FC Loss Coolant Activity Dose Rates	
Time > Shutdown (hrs)	mR/hr/ml
≤ 2	15
$> 2 - \leq 8$	8
> 8	3

Definition(s):

None

Basis:

This threshold indicates that RCS radioactivity concentration is greater than 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

It is recognized that sample collection and analysis of reactor coolant with highly elevated activity levels could require several hours to complete. Nonetheless, a sample-related threshold is included as a backup to other indications. This EAL provides the ability to take a dose rate off of an RCS sample to determine fuel clad barrier loss, without the need to analyze the sample before making this determination. This EAL saves significant time by allowing evaluation of contained radioactivity within the RCS by a direct dose rate measurement.

Per Engineering Calculation RA-0059, dose rate is assumed to result from radioactive iodines (I-131 thru I-135) in RCS in concentrations corresponding to the loss of 5% of gap radioactivity of the core. For 5% loss of gap radioactivity ($\sim 300 \mu\text{Ci/gm}$ dose equivalent I-131), 2% of the core inventory of radioactive iodines are assumed to be contained in the gap. The values contained in Table F-3 (FC Loss Coolant Activity Dose Rates) represent expected one foot dose rates per ml of sample based on time since reactor shutdown to the time when the sample is taken. Time after shutdown values are provided to account for radioactive decay. The expected dose rate is a near linear relationship with the volume of the sample, so any volume collected can be determined by dividing the measured dose rate by the sample volume and comparing to the threshold value from Table F-3 for the applicable time frame. These dose rates assume no ECCS injection so there is no dilution credited which would vary coolant

volume. Values in the table have been rounded for ease of use. The > 8 hour threshold is conservative up to 24 hours following reactor shutdown. After 24 hours, the expected response from radioactive iodine levels off. Therefore, the value shown for > 8 hours applies for all samples taken 8 hours or more since reactor shutdown (ref. 1, 2).

The values specified in Table F-3 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Values were chosen to minimize error from the highest to lowest dose rate within each range.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Emergency Plan Procedure MP-26-EPI-FAP11, "Core Damage Assessment"
2. Calculation RA-0059, "Detector Response to an RCS Sample for EAL Classification of Fuel Clad Degradation and Barrier Loss"
3. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Loss

Threshold:

5. Sample line dose rate \geq Table F-4

Table F-4 FC Loss RCS Sample Line Dose Rates	
Time > Shutdown (hrs)	R/hr
≤ 2	4
$> 2 - \leq 8$	2
> 8	1

Definition(s):

None

Basis:

This threshold indicates that RCS radioactivity concentration is greater than 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

Per Engineering Calculation RA-0079, dose rate is assumed to result from radioactive iodines in the RCS in concentrations corresponding to the loss of 5% of gap radioactivity of the core. The values contained in Table F-4 (FC Loss RCS Sample Line Dose Rates) represent fuel clad failure thresholds when measured approximately 2" from the outside of the RCS hot leg sample line. RCS sample line locations have been predetermined for use with this EAL. Other RCS lines could be used if analyzed on a case-by-case basis. Values in the table have been rounded for ease of use. The sample line dose rates have been calculated for various time ranges after shutdown to account for radioactive decay (ref. 1).

The values specified in Table F-4 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Values were chosen to minimize error from the highest to lowest dose rate within each range.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Engineering Calculation RA-0079
2. NEI 99-01 CTMT Radiation / RCS Activity FC Loss 3.B

Barrier: Fuel Clad

Category: C. CTMT Radiation / RCS Activity

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Fuel Clad

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

None

Barrier: Fuel Clad

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Fuel Clad

Category: E. DSEO/ADTS Judgment

Degradation Threat: Loss

Threshold:

6. **Any** condition in the opinion of the DSEO/ADTS that indicates loss of the Fuel Clad barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that are to be used by the DSEO/ADTS in determining whether the Fuel Clad barrier is lost.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Fuel Clad Loss 6.A

Barrier: Fuel Clad

Category: F. DSEO/ADTS Judgment

Degradation Threat: Potential Loss

Threshold:

3. **Any** condition in the opinion of the DSEO/ADTS that indicates potential loss of the Fuel Clad barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that are to be used by the DSEO/ADTS in determining whether the Fuel Clad barrier is potentially lost. The DSEO/ADTS should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Potential Fuel Clad Loss 6.A

Barrier: Reactor Coolant System

Category: A. RCS or S/G Tube Leakage

Degradation Threat: Loss

Threshold:

1. An automatic or manual Safety Injection (SI) actuation required by **EITHER**:
 - UNISOLABLE RCS leakage
 - SG tube RUPTURE

Definition(s):

UNISOLABLE - An open or breached system line that cannot be isolated, remotely or locally.

RUPTURE - The condition of a steam generator in which primary-to-secondary leakage is of sufficient magnitude to require a safety injection.

Basis:

This threshold is based on an UNISOLABLE RCS leak of sufficient size to require an automatic or manual actuation of the Emergency Core Cooling System (ECCS). This condition clearly represents a loss of the RCS Barrier.

This threshold is applicable to unidentified and pressure boundary leakage, as well as identified leakage. It is also applicable to UNISOLABLE RCS leakage through an interfacing system. The mass loss may be into any location – inside containment, to the secondary-side (i.e., steam generator tube leakage) or outside of containment.

A steam generator with primary-to-secondary leakage of sufficient magnitude to require a safety injection is considered to be RUPTURED. If a RUPTURED steam generator is also FAULTED outside of containment, the declaration escalates to a Site Area Emergency since the Containment Barrier Loss threshold A.1 will also be met.

If EOPs direct operators to open the Pressurizer pressure relief valves to implement a core cooling strategy (i.e., a “feed and bleed” cooldown), then there will exist a reactor coolant flow path from the RCS, past the “pressurizer safety and relief valves” and into the containment that operators cannot isolate without compromising the effectiveness of the strategy (i.e., for the strategy to be effective, the valves must be kept in the open position); therefore, the flow through the pressure relief line is UNISOLABLE. In this case, the ability of the RCS pressure boundary to serve as an effective barrier to a release of fission products has been eliminated and thus this condition constitutes a loss of the RCS barrier.

A SI actuation resulting from a non-RCS leak event (ex.: FAULTED SG) does not meet the intent of this threshold.

Reference(s):

1. EOP 35 E-0, "Reactor Trip or Safety Injection"
2. EOP 35 E-3, "Steam Generator Tube Rupture"
3. NEI 99-01 RCS or SG Tube Leakage Reactor Coolant System Loss 1.A

Barrier: Reactor Coolant System

Category: A. RCS or S/G Tube Leakage

Degradation Threat: Potential Loss

Threshold:

1. Operation of a second charging pump is required by **EITHER**:
- UNISOLABLE RCS leakage
 - SG tube leakage

Definition(s):

UNISOLABLE - An open or breached system line that cannot be isolated, remotely or locally.

Basis:

This threshold is based on an UNISOLABLE RCS leak that results in the inability to maintain pressurizer level within specified limits by operation of a normally used charging pump, but an ECCS (SI) actuation has not occurred. The threshold is met when an operating procedure, or operating crew supervision, directs that a second charging pump be placed in service to restore and maintain pressurizer level (ref. 1, 2).

This threshold is applicable to unidentified and pressure boundary leakage, as well as identified leakage. It is also applicable to UNISOLABLE RCS leakage through an interfacing system. The mass loss may be into any location – inside containment, to the secondary-side (i.e., steam generator tube leakage) or outside of containment.

If the leaking steam generator requiring the start of a second charging pump is also FAULTED outside of containment, the declaration escalates to a Site Area Emergency since the Containment Barrier Loss threshold A.1 will also be met.

When classifying RCS leaks, the classification should be based on actual leakage vice the impact of normal plant response to Operator directed actions. Example: A rapid down power may have the impact of increasing charging flow. The increase in charging should not be considered increased leakage.

Reference(s):

1. AOP 3555, "Reactor Coolant System Leak"
2. AOP 3576, "Steam Generator Tube Leak"
3. NEI 99-01 RCS or SG Tube Leakage Reactor Coolant System Potential Loss 1.A

Barrier: Reactor Coolant System

Category: A. RCS or S/G Tube Leakage

Degradation Threat: Potential Loss

Threshold:

2. Integrity-RED path conditions met

Definition(s):

None

Basis:

This condition indicates an extreme challenge to the integrity of the RCS pressure boundary due to pressurized thermal shock – a transient that causes rapid RCS cooldown while the RCS is in Mode 3 or higher (i.e., hot and pressurized).

The potential loss threshold is defined by the CSFST Integrity - RED path. CSFST Integrity - Red Path plant conditions (> 100°F/hr cold leg cooldown) and associated PTS Limit A Curve indicates an extreme challenge to the safety function when plant parameters are to the right of the limit curve following excessive RCS cooldown under pressure (ref. 1).

Reference(s):

1. EOP 35 F-0.4, "CSFST Integrity"
2. NEI 99-01 RCS or SG Tube Leakage Reactor Coolant System Potential Loss 1.B

Barrier: Reactor Coolant System

Category: B. Inadequate Heat Removal

Degradation Threat: Loss

Threshold:

None

Barrier: Reactor Coolant System

Category: B. Inadequate Heat Removal

Degradation Threat: Potential Loss

Threshold:

3. Heat Sink-RED Path conditions met

AND

Required feedwater flow **cannot** be established within 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Definition(s):

None

Basis:

The potential loss threshold is based on meeting the Heat Sink Red Path criteria of both of the following conditions existing (ref. 1):

- Narrow Range level in all SGs < 8% (42% adverse CTMT)
- Total feedwater flow to SGs < 530 gpm

This condition indicates an extreme challenge to the ability to remove RCS heat using the steam generators (i.e., loss of an effective secondary-side heat sink). This condition represents a potential loss of the RCS Barrier. In accordance with EOPs, there may be unusual accident conditions during which operators intentionally reduce the heat removal capability of the steam generators; during these conditions, classification using this threshold is not warranted.

If inventory could not be restored, steam generator dry out could occur within about 30 minutes. If feedwater flow could not be reestablished, FR-H.1 would require establishment of RCS bleed and feed to maintain core cooling and to assure sufficient RCS inventory. If neither of these actions can be accomplished, then fuel clad heatup will begin.

The phrase "*Required feedwater flow **cannot** be established within 15 min.*" precludes the need for classification for conditions that indicate the feed flow requirements have been satisfied in accordance with FR-H.1. For example, FR-H.1 is entered from CSFST Heat Sink-Red. Step 1 tells the operator to determine if heat sink is required by checking that RCS pressure is greater than any non-faulted SG pressure and RCS T_{hot} is greater than 350°F. If these conditions exist, Heat Sink is required. Otherwise, the operator is to either go to the procedure and step in effect or place RHR in service for heat removal. For large LOCA events inside the Containment, the SGs are irrelevant because heat removal through the containment heat removal systems takes place. Therefore, Heat Sink Red is not applicable and should not be assessed for EAL classification because a LOCA event alone should not require higher than an Alert classification. (ref. 1, 2).

However, required feed flow may be less than 530 gpm (Heat Sink Red Path entry criteria) and still be acceptable based on the exit criteria from FR-H.1. Per FR-H.1, feed flow can be less than 530 gpm and be considered acceptable if reduced feed flow is due to operator action or if WR level in at least one SG is increasing and Core Exit TCs are stable or decreasing.

The 15 minute allowance provides time for equipment actuation, operator action to restore feedwater flow, and verification that adequate feedwater flow has been established.

Meeting this threshold results in a Site Area Emergency because this threshold is identical to Fuel Clad Barrier Potential Loss threshold B.2; both will be met. This condition warrants a Site Area Emergency declaration because inadequate RCS heat removal may result in fuel heat-up sufficient to damage the cladding and increase RCS pressure to the point where mass will be lost from the system.

Reference(s):

1. EOP 35 F-0.3, "CSFST Heat Sink"
2. EOP 35 FR-H.1, "Response to Loss of Secondary Heat Sink"
3. NEI 99-01 Inadequate Heat Removal RCS Loss 2.B

Barrier: Reactor Coolant System

Category: C. CTMT Radiation/ RCS Activity

Degradation Threat: Loss

Threshold:

2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column RCS Loss (Note 14)

Note 14: Readings are considered sustained when effects of TIC have dissipated.

Table F-2 CTMT High Range Radiation Monitor Barrier Thresholds RE-04A/05A			
Time > Shutdown (hrs)	Fuel Clad Loss (R/hr)	RCS Loss (R/hr)	CTMT Potential Loss (R/hr)
≤ 2	140	5	560
$> 2 - \leq 4$	90	5	360
$> 4 - \leq 8$	50	5	200
$> 8 - \leq 14$	25	5	100
> 14	12	5	48

Definition(s):

None

Basis:

A reading > 5 R/hr (minimum practical reading) on RE-04A or 05A is indicative of a breach in the RCS barrier (ref. 1, 2).

The radiation monitor reading corresponds to an instantaneous release of all reactor coolant mass into the containment, assuming that reactor coolant activity equals Technical Specification allowable limits. This value is lower than that specified for Fuel Clad barrier loss threshold C.2 since it indicates a loss of the RCS Barrier only.

Because of the very high fuel clad integrity, only small amounts of noble gases would be dissolved in the primary coolant. Conservative estimates indicated that the readings from release of the normal RCS inventory would be below normal readings on the monitor while the station was operating. Therefore, a value 5 times the normal containment radiation monitor (RE-04A/05A) reading of ~ 1 R/hr is used. The reading is less than that specified for fuel cladding barrier loss because no damage to the fuel cladding is assumed. Only leakage from the RCS is assumed for this barrier loss threshold. The value is high enough to preclude erroneous classification of barrier loss due to normal plant operations and is set at the mid-point of the lowest decade of monitor indication (ref. 2).

The effects of thermally induced currents (TIC) are significant but of relatively short duration. Generally, the effects of TIC will dissipate within 2-5 minutes. Until the TIC affects have dissipated, the RE04A/RE05A readings should not be considered valid.

There is no Potential Loss threshold associated with RCS Activity / Containment Radiation.

Reference(s):

1. Damage Computer Program
2. Calculation RA-0075, "Millstone Unit 3 Expected Containment High Range Radiation Monitor Response to a LOCA Based on Fuel Rod Gap Fractions Defined in NUREG 1228"
3. NEI 99-01 CMT Radiation / RCS Activity RCS Loss 3.A

Barrier: Reactor Coolant System

Category: C. CTMT Radiation/ RCS Activity

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Reactor Coolant System

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

None

Barrier: Reactor Coolant System

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Reactor Coolant System

Category: E. DSEO/ADTS Judgment

Degradation Threat: Loss

Threshold:

3. Any condition in the opinion of the DSEO/ADTS that indicates loss of the RCS barrier
--

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the DSEO/ADTS in determining whether the RCS barrier is lost.

Reference(s):

1. NEI 99-01 Emergency Director Judgment RCS Loss 6.A

Barrier: Reactor Coolant System

Category: E. DSEO/ADTS Judgment

Degradation Threat: Potential Loss

Threshold:

4. Any condition in the opinion of the DSEO/ADTS that indicates potential loss of the RCS barrier
--

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the DSEO/ADTS in determining whether the RCS barrier is potentially lost. The DSEO/ADTS should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Reference(s):

1. NEI 99-01 Emergency Director Judgment RCS Potential Loss 6.A

Barrier: Containment

Category: A. RCS or SG Tube Leakage

Degradation Threat: Loss

Threshold:

1. A leaking or RUPTURED SG is FAULTED outside of CTMT

Definition(s):

FAULTED - The term applied to a steam generator that has a steam leak on the secondary side of sufficient size to cause an uncontrolled drop in steam generator pressure or the steam generator to become completely depressurized.

RUPTURED - The condition of a steam generator in which primary-to-secondary leakage is of sufficient magnitude to require a safety injection.

Basis:

This threshold addresses a leaking or RUPTURED Steam Generator (SG) that is also FAULTED outside of containment. The condition of the SG, whether leaking or RUPTURED, is determined in accordance with the thresholds for RCS Barrier Potential Loss A.1 and Loss A.1, respectively. This condition represents a bypass of the containment barrier.

FAULTED is a defined term within the NEI 99-01 methodology; this determination is not necessarily dependent upon entry into, or diagnostic steps within, an EOP. For example, if the pressure in a steam generator is decreasing uncontrollably (part of the FAULTED definition) and the FAULTED steam generator isolation procedure is not entered because EOP user rules are dictating implementation of another procedure to address a higher priority condition, the steam generator is still considered FAULTED for emergency classification purposes.

The FAULTED criterion establishes an appropriate lower bound on the size of a steam release that may require an emergency classification. Steam releases of this size are readily observable with normal Control Room indications. The lower bound for this aspect of the containment barrier is analogous to the lower bound criteria specified in IC MU4 for the fuel clad barrier (i.e., RCS activity values) and IC MU5 for the RCS barrier (i.e., RCS leak rate values).

This threshold also applies to prolonged steam releases necessitated by operational considerations such as the forced steaming of a leaking or RUPTURED steam generator directly to atmosphere to cooldown the plant, or to drive an auxiliary (emergency) feed water pump. These types of conditions will result in a significant and sustained release of radioactive steam to the environment (and are thus similar to a FAULTED condition). The inability to isolate the steam flow without an adverse effect on plant cooldown meets the intent of a loss of containment.

Steam releases associated with the expected operation of a SG power operated relief valve or safety relief valve do not meet the intent of this threshold. Such releases may occur

intermittently for a short period of time following a reactor trip as operators process through emergency operating procedures to bring the plant to a stable condition and prepare to initiate a plant cooldown. Steam releases associated with the unexpected operation of a valve (e.g., a stuck-open safety valve) do meet this threshold.

Following an SG tube leak or rupture, there may be minor radiological releases through a secondary-side system component (e.g., air ejectors, gland seal exhausters, valve packing, etc.). These types of releases do not constitute a loss or potential loss of containment but should be evaluated using the Category R ICs.

The emergency classification levels resulting from primary-to-secondary leakage, with or without a steam release from the FAULTED SG, are summarized below.

P-to-S Leak Rate	Affected SG is FAULTED Outside of Containment?	
	Yes	No
Less than or equal to 25 gpm	No classification	No classification
Greater than 25 gpm	Unusual Event per MU5.1	Unusual Event per MU5.1
Requires operation of a standby charging pump (<i>RCS Barrier Potential Loss</i>)	Site Area Emergency per FS1.1	Alert per FA1.1
Requires an automatic or manual ECCS (SI) actuation (<i>RCS Barrier Loss</i>)	Site Area Emergency per FS1.1	Alert per FA1.1

There is no Potential Loss threshold associated with RCS or SG Tube Leakage.

Reference(s):

1. NEI 99-01 RCS or SG Tube Leakage Containment Loss 1.A

Barrier: Containment

Category: A. RCS or SG Tube Leakage

Degradation Threat: Potential Loss

Threshold:

None

Barrier: Containment

Category: B. Inadequate Heat Removal

Degradation Threat: Loss

None

Barrier: Containment

Category: B. Inadequate Heat Removal

Degradation Threat: Potential Loss

Threshold:

1. Core Cooling-RED Path conditions met
- AND**
- Restoration procedures **not** effective within 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Definition(s):

IMMINENT: The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

The potential loss threshold is based on meeting the Core Cooling Red Path criteria for Core Exit Thermocouple readings ($> 1,200$ °F) and restoration procedures not effective within 15 minutes (ref. 1, 2).

This condition represents an IMMINENT core melt sequence which, if not corrected, could lead to vessel failure and an increased potential for containment failure. For this condition to occur there must already have been a loss of the RCS Barrier and the Fuel Clad Barrier. If implementation of a procedure(s) to restore adequate core cooling is not effective (successful) within 15 minutes, it is assumed that the event trajectory will likely lead to core melting and a subsequent challenge of the Containment Barrier.

The restoration procedure is considered "effective" if core exit thermocouple readings are decreasing and/or if reactor vessel level is increasing. Whether or not the procedure(s) will be effective should be apparent within 15 minutes. The DSEO/ADTS should escalate the emergency classification level to a General Emergency as soon as it is determined that the procedure(s) will not be effective.

Severe accident analyses (e.g., NUREG-1150) have concluded that functional restoration procedures can arrest core degradation in a significant fraction of core damage scenarios, and that the likelihood of containment failure is very small in these events. Given this, it is appropriate to provide 15 minutes beyond the required entry point to determine if procedural actions can reverse the core melt sequence.

Reference(s):

1. EOP 35 F-0.2, "CSFST Core Cooling"
2. EOP 35 FR-C.1, "Response to Inadequate Core Cooling"
3. NEI 99-01 Inadequate Heat Removal Containment Potential Loss 2.A

Barrier: Containment

Category: C. CTMT Radiation/RCS Activity

Degradation Threat: Loss

Threshold:

None

Barrier: Containment

Category: C. CTMT Radiation/RCS Activity

Degradation Threat: Potential Loss

Threshold:

2. Sustained CTMT high range radiation monitor RE-04A/05A reading > Table F-2 column CTMT Potential Loss (Note 14)

Note 14: Readings are considered sustained when effects of TIC have dissipated.

Table F-2 CTMT High Range Radiation Monitor Barrier Thresholds RE-04A/05A			
Time > Shutdown (hrs)	Fuel Clad Loss (R/hr)	RCS Loss (R/hr)	CTMT Potential Loss (R/hr)
≤ 2	140	5	560
$> 2 - \leq 4$	90	5	360
$> 4 - \leq 8$	50	5	200
$> 8 - \leq 14$	25	5	100
> 14	12	5	48

Definition(s):

None

Basis:

The radiation monitor reading corresponds to an instantaneous release of all reactor coolant mass into the containment, assuming that 20% of the fuel cladding has failed. This level of fuel clad failure is well above that used to determine the analogous Fuel Clad Barrier Loss and RCS Barrier Loss thresholds (ref. 1, 2).

Time after shutdown values are provided to account for radioactive decay.

The values specified in Table F-2 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Time periods were chosen to fit monitor response (fast changes in response early following reactor shutdown are broken up into smaller time periods to better approximate expected change). Values were chosen within each time period to minimize error (<50%) to the highest and lowest response within the range.

NUREG-1228, Source Estimations During Incident Response to Severe Nuclear Power Plant Accidents, indicates the fuel clad failure must be greater than approximately 20% in order for there to be a major release of radioactivity requiring offsite protective actions. For this condition to exist, there must already have been a loss of the RCS barrier and the Fuel Clad

barrier. It is therefore prudent to treat this condition as a potential loss of containment which would then escalate the emergency classification level to a General Emergency.

The effects of thermally induced currents (TIC) are significant but of relatively short duration. Generally, the effects of TIC will dissipate within 2-5 minutes. Until the TIC affects have dissipated, the RE04A/RE05A readings should not be considered valid.

Reference(s):

1. Damage Computer Program
2. Calculation RA-0075, "Millstone Unit 3 Expected Containment High Range Radiation Monitor Response to a LOCA Based on Fuel Rod Gap Fractions Defined in NUREG 1228"
3. NEI 99-01 CMT Radiation / RCS Activity Containment Potential Loss 3.A

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

2. CTMT isolation (CIA) Phase A is required

AND EITHER:

- CTMT integrity has been lost based on DSEO/ADTS judgment
- UNISOLABLE pathway from CTMT atmosphere to the environment exists

Definition(s):

UNISOLABLE - An open or breached system line that cannot be isolated, remotely or locally.

Basis:

The status of the containment barrier during an event involving steam generator tube leakage is assessed using Loss Threshold A.1. Therefore this threshold is not applicable to steam generator tube leakage.

These thresholds address a situation where containment isolation is required and one of two conditions exists as discussed below. Users are reminded that there may be accident and release conditions that simultaneously meet both bulleted thresholds.

First Threshold – Containment integrity has been lost, i.e., the actual containment atmospheric leak rate likely exceeds that associated with allowable leakage (or sometimes referred to as design leakage). Following the release of RCS mass into containment, containment pressure will fluctuate based on a variety of factors; a loss of containment integrity condition may (or may not) be accompanied by a noticeable drop in containment pressure. Recognizing the inherent difficulties in determining a containment leak rate during accident conditions, it is expected that the DSEO/ADTS will assess this threshold using judgment, and with due consideration given to current plant conditions, and available operational and radiological data (e.g., containment pressure, readings on radiation monitors outside containment, operating status of containment pressure control equipment, etc.).

Refer to the middle piping run of Figure 1. Two simplified examples are provided. One is leakage from a penetration and the other is leakage from an in-service system valve. Depending upon radiation monitor locations and sensitivities, the leakage could be detected by any of the four monitors depicted in the figure.

Another example would be a loss or potential loss of the RCS barrier, and the simultaneous occurrence of two FAULTED locations on a steam generator where one fault is located inside containment (e.g., on a steam or feedwater line) and the other outside of containment. In this case, the associated steam line provides a pathway for the containment atmosphere to escape to an area outside the containment.

Following the leakage of RCS mass into containment and a rise in containment pressure, there may be minor radiological releases associated with allowable (design) containment leakage through various penetrations or system components. These releases do not constitute a loss or potential loss of containment but should be evaluated using the Category A ICs.

Second Threshold – Conditions are such that there is an UNISOLABLE pathway for the migration of radioactive material from the containment atmosphere to the environment. As used here, the term “environment” includes the atmosphere of a room or area, outside the containment, that may, in turn, communicate with the outside-the-plant atmosphere (e.g., through discharge of a ventilation system or atmospheric leakage). Depending upon a variety of factors, this condition may or may not be accompanied by a noticeable drop in containment pressure.

Refer to the top piping run of Figure 1. In this simplified example, the inboard and outboard isolation valves remained open after a containment isolation was required (i.e., containment isolation was not successful). There is now an UNISOLABLE pathway from the containment to the environment.

The existence of a filter is not considered in the threshold assessment. Filters do not remove fission product noble gases. In addition, a filter could become ineffective due to iodine and/or particulate loading beyond design limits (i.e., retention ability has been exceeded) or water saturation from steam/high humidity in the release stream.

Leakage between two interfacing liquid systems, by itself, does not meet this threshold.

Refer to the bottom piping run of Figure 1. In this simplified example, leakage in an RCP seal cooler is allowing radioactive material to enter the Auxiliary Building. The radioactivity would be detected by the Process Monitor. If there is no leakage from the closed water cooling system to the Auxiliary Building, then no threshold has been met. If the pump developed a leak that allowed steam/water to enter the Auxiliary Building, then the second threshold would be met. Depending upon radiation monitor locations and sensitivities, this leakage could be detected by any of the four monitors depicted in the figure and cause the first threshold to be met as well.

Following the leakage of RCS mass into containment and a rise in containment pressure, there may be minor radiological releases associated with allowable containment leakage through various penetrations or system components. Minor releases may also occur if a containment isolation valve(s) fails to close but the containment atmosphere escapes to an enclosed system. These releases do not constitute a loss or potential loss of containment but should be evaluated using the Category R ICs.

Reference(s):

1. NEI 99-01 CMT Integrity or Bypass Containment Loss 4.A

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Loss

Threshold:

3. Indications of RCS leakage outside of CTMT

Definition(s):

None

Basis:

To ensure proper escalation of the emergency classification, the RCS leakage outside of containment must be related to the mass loss that is causing the RCS Loss and/or Potential Loss threshold A.1 to be met.

The status of the containment barrier during an event involving steam generator tube leakage is assessed using Containment Loss Threshold A.1. Therefore this threshold is not applicable to steam generator tube leakage.

This threshold **does not** apply to an UNISOLABLE RSHX tube leak outside containment. Such leaks are properly addressed under the category R radiological release based EALs.

Containment sump, temperature, pressure and/or radiation levels will increase if reactor coolant mass is leaking into the containment. If these parameters have not increased, then the reactor coolant mass may be leaking outside of containment (i.e., a containment bypass sequence). Increases in sump, temperature, pressure, flow and/or radiation level readings outside of the containment may indicate that the RCS mass is being lost outside of containment.

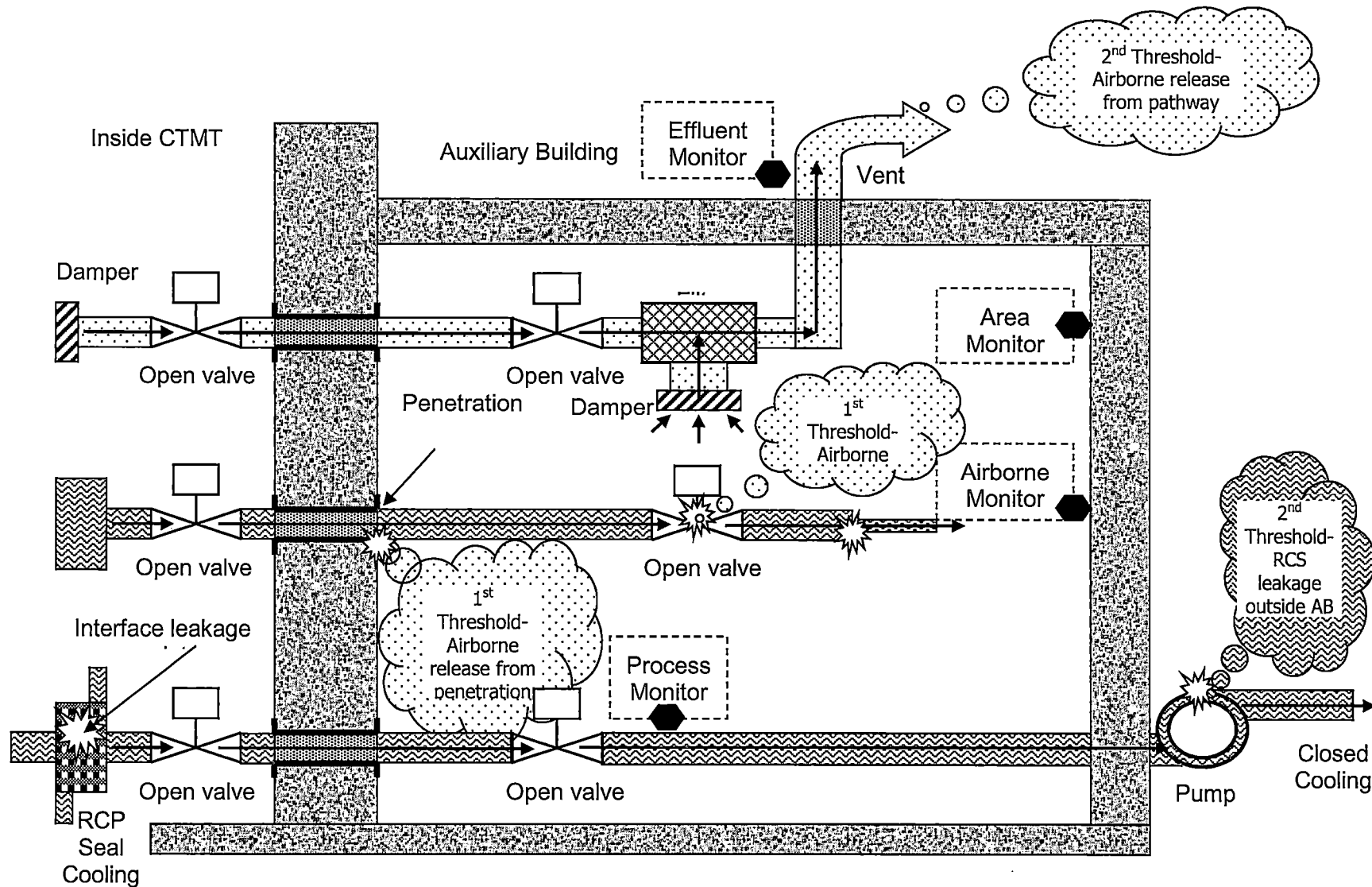
Unexpected elevated readings and alarms on radiation monitors with detectors outside containment should be corroborated with other available indications to confirm that the source is a loss of RCS mass outside of containment. If the fuel clad barrier has not been lost, radiation monitor readings outside of containment may not increase significantly; however, other unexpected changes in sump levels, area temperatures or pressures, flow rates, etc. should be sufficient to determine if RCS mass is being lost outside of the containment.

Refer to the middle piping run of Figure 1. In this simplified example, a leak has occurred at a reducer on a pipe carrying reactor coolant in the Auxiliary Building. Depending upon radiation monitor locations and sensitivities, the leakage could be detected by any of the four monitors depicted in the figure and cause loss threshold D.2 to be met as well.

Reference(s):

1. NEI 99-01 CMT Integrity or Bypass Containment Loss 4.B

Figure 1: Containment Integrity or Bypass Examples



Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

3. CTMT pressure > 60 psia

Definition(s):

None

Basis:

If containment pressure exceeds the design pressure of 60 psia (ref. 1), there exists a potential to lose the containment barrier. To reach this level, there must be an inadequate core cooling condition for an extended period of time; therefore, the RCS and Fuel Clad barriers would already be lost. Thus, this threshold is a discriminator between a Site Area Emergency and General Emergency since there is now a potential to lose the third barrier.

Reference(s):

1. MPS3 FSAR Section 6.2.1.1.1, "Containment Structure Design Bases"
2. NEI 99-01 CMT Integrity or Bypass Containment Potential Loss 4.A

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

4. CTMT hydrogen concentration > 4%

Definition(s):

None

Basis:

The existence of an explosive mixture means, at a minimum, that the containment atmospheric hydrogen concentration is sufficient to support a hydrogen burn (i.e., at the lower deflagration limit). A hydrogen burn will raise containment pressure and could result in collateral equipment damage leading to a loss of containment integrity. It therefore represents a potential loss of the containment barrier.

A containment hydrogen concentration of 4% conservatively represents the lowest threshold for flammability in the presence of oxygen (ref. 1).

Reference(s):

1. Calculation PRA98NQA-01555S3, "SAMG – Computational Aids and Setpoints"
2. NEI 99-01 CMT Integrity or Bypass Containment Potential Loss 4.B

Barrier: Containment

Category: D. CTMT Integrity or Bypass

Degradation Threat: Potential Loss

Threshold:

5. CTMT pressure > 25 psia with < one full train of CTMT heat removal systems (Note 11) operating per design for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 11: One full train of containment heat removal systems consist of one train of Recirculation Spray System (RSS) and one train of Quench Spray System (QSS).

Definition(s):

None

Basis:

This threshold describes a condition where containment pressure is greater than the setpoint (25 psia) at which containment energy (heat) removal systems are designed to automatically actuate, and less than one full train of equipment is capable of operating per design (ref. 1, 2, 3). The 15-minute criterion is included to allow operators time to manually start equipment that may not have automatically started, if possible. This threshold represents a potential loss of containment in that containment heat removal/depressurization systems (e.g., containment sprays but not including containment venting strategies) are either lost or performing in a degraded manner.

The Quench Spray System (QSS) is activated immediately upon receipt of a CDA signal. QSS becomes effective within 70 seconds of initiation.

The containment Recirculation Spray System (RSS) is activated on receipt of the RWST Low-Low level signal coincident with a CDA signal. There is a maximum delay of 5 minutes before recirculation spray becomes effective.

One full train of containment heat removal systems consist of one train of Recirculation Spray System (RSS) and one train of Quench Spray System (QSS) (ref. 1, 2). The combination of required equipment can be obtained from using equipment on either emergency bus in order to meet the "one full train" requirement.

Reference(s):

1. MPS3 FSAR Section 6.2.2, "Containment Heat Removal System"
2. MPS3 Technical Specifications 3.6.2.1, "Containment Quench Spray and Recirculation Spray System"
3. EOP 35 FR-Z.1, "Response to High Containment Pressure"
4. NEI 99-01 CMT Integrity or Bypass Containment Potential Loss 4.C

Barrier: Containment

Category: E. DSEO/ADTS Judgment

Degradation Threat: Loss

Threshold:

4. **Any** condition in the opinion of the DSEO/ADTS that indicates loss of the CTMT barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the DSEO/ADTS in determining whether the containment barrier is lost.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Containment Loss 6.A

Barrier: Containment

Category: E. DSEO/ADTS Judgment

Degradation Threat: Potential Loss

Threshold:

6. **Any** condition in the opinion of the DSEO/ADTS that indicates potential loss of the CTMT barrier

Definition(s):

None

Basis:

This threshold addresses any other factors that may be used by the DSEO/ADTS in determining whether the containment barrier is potentially lost. The DSEO/ADTS should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Reference(s):

1. NEI 99-01 Emergency Director Judgment Containment Potential Loss 6.A

Category H – Hazards and Other Conditions Affecting Plant Safety

EAL Group: ANY (EALs in this category are applicable to any plant condition, hot or cold.)

Hazards are non-plant, system-related events that can directly or indirectly affect plant operation, reactor plant safety or personnel safety.

1. Security

Unauthorized entry attempts into the PROTECTED AREA, bomb threats, sabotage attempts, and actual security compromises threatening loss of physical control of the plant.

2. Seismic Event

Natural events such as earthquakes have potential to cause plant structure or equipment damage of sufficient magnitude to threaten personnel or plant safety.

3. Natural or Technological Hazard

Other natural and non-naturally occurring events that can cause damage to plant facilities include tornados, FLOODING, hazardous material releases and events restricting site access warranting classification.

4. Fire

FIRES can pose significant hazards to personnel and reactor safety. Appropriate for classification are FIRES within the plant PROTECTED AREA or which may affect operability of equipment needed for safe shutdown

5. Hazardous Gas

Toxic, corrosive, asphyxiant or flammable gas leaks can affect normal plant operations or preclude access to plant areas required to safely shutdown the plant.

6. Control Room Evacuation

Events that are indicative of loss of Control Room habitability. If the Control Room must be evacuated, additional support for monitoring and controlling plant functions is necessary through the emergency response facilities.

7. DSEO/ADTS Judgment

The EALs defined in other categories specify the predetermined symptoms or events that are indicative of emergency or potential emergency conditions and thus warrant classification. While these EALs have been developed to address the full spectrum of possible emergency conditions which may warrant classification and subsequent implementation of the Emergency Plan, a provision for classification of emergencies based on operator/management experience and judgment is still necessary. The EALs of this category provide the DSEO/ADTS the latitude to classify emergency conditions consistent with the established classification criteria based upon DSEO/ADTS judgment.

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 1 – Security

Initiating Condition: Confirmed SECURITY CONDITION or threat

EAL:

HU1.1 Unusual Event

A SECURITY CONDITION that does **not** involve a HOSTILE ACTION as reported by MPS Security Shift Supervision

OR

Notification of a credible security threat directed at the site

OR

A validated notification from the NRC providing information of an aircraft threat

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA (OCA) - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;

- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

SECURITY CONDITION - **Any** security event as listed in the approved security contingency plan that constitutes a threat/compromise to site security, threat/risk to site personnel, or a potential degradation to the level of safety of the plant. A security condition does **not** involve a HOSTILE ACTION.

Basis:

This IC addresses events that pose a threat to plant personnel or SAFETY SYSTEM equipment, and thus represent a potential degradation in the level of plant safety. Security events which do not meet one of these EALs are adequately addressed by the requirements of 10CFR73.71 or 10CFR50.72. Security events assessed as HOSTILE ACTIONS are classifiable under ICs HA1 and HS1. Guidance on assessing Security Conditions is included in the Security Contingency Plan Implementing Procedures (SCIP). The SCIPs are implementing procedures for the Station Safeguards Contingency Plan.

Timely and accurate communications between Security Shift Supervision and the Control Room is essential for proper classification of a security-related event. Classification of these events will initiate appropriate threat-related notifications to plant personnel and Offsite Response Organizations.

Security plans and terminology are based on the guidance provided by NEI 03-12, *Template for the Security Plan, Training and Qualification Plan, Safeguards Contingency Plan [and Independent Spent Fuel Storage Installation Security Program]*.

The first threshold references the Security Shift Supervision because these are the individuals trained to confirm that a security event is occurring or has occurred. Training on security event confirmation and classification is controlled due to the nature of Safeguards and 10CFR2.39 information.

The second threshold addresses the receipt of a credible security threat. The credibility of the threat is assessed in accordance with the Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program (ref. 1).

The third threshold addresses the threat from the impact of an aircraft on the plant. The NRC Headquarters Operations Officer (HOO) will communicate to the licensee if the threat involves an aircraft. The status and size of the plane may also be provided by NORAD through the NRC. Validation of the threat is performed in accordance with OP-AA-900 Authentication (ref. 2).

Emergency plans and implementing procedures are public documents; therefore, EALs should not incorporate Security-sensitive information. This includes information that may be advantageous to a potential adversary, such as the particulars concerning a specific threat or threat location. Security-sensitive information should be contained in non-public documents such as the Security Plan for MPS (ref. 1).

Escalation of the emergency classification level would be via IC HA1.

Reference(s):

1. Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program
2. OP-AA-900, "Authentication"
3. NEI 99-01 HU1

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 1 – Security
Initiating Condition: HOSTILE ACTION within the OWNER CONTROLLED AREA or airborne attack threat within 30 minutes

EAL:

HA1.1 Alert

A HOSTILE ACTION is occurring or has occurred within the OWNER CONTROLLED AREA as reported by MPS Security Shift Supervision

OR

A validated notification from NRC of an aircraft attack threat within 30 min. of the site

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

HOSTILE FORCE - One or more individuals who are engaged in a determined assault, overtly or by stealth and deception, equipped with suitable weapons capable of killing, maiming, or causing destruction.

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses the occurrence of a HOSTILE ACTION within the OWNER CONTROLLED AREA or notification of an aircraft attack threat. This event will require rapid response and assistance due to the possibility of the attack progressing to the PROTECTED AREA, or the need to prepare the plant and staff for a potential aircraft impact.

Timely and accurate communications between Security Shift Supervision and the Control Room is essential for proper classification of a security-related event.

Security plans and terminology are based on the guidance provided by NEI 03-12; *Template for the Security Plan, Training and Qualification Plan, Safeguards Contingency Plan [and Independent Spent Fuel Storage Installation Security Program]*.

As time and conditions allow, these events require a heightened state of readiness by the plant staff and implementation of onsite protective measures (e.g., evacuation, dispersal or sheltering). The Alert declaration will also heighten the awareness of State and local agencies, allowing them to be better prepared should it be necessary to consider further actions.

This EAL does not apply to incidents that are accidental events, acts of civil disobedience, or otherwise are not a HOSTILE ACTION perpetrated by a HOSTILE FORCE. Examples include the crash of a small aircraft, shots from hunters, physical disputes between employees, etc. Reporting of these types of events is adequately addressed by other EALs, or the requirements of 10CFR73.71 or 10CFR50.72.

The first threshold is applicable for any HOSTILE ACTION occurring, or that has occurred, in the OWNER CONTROLLED AREA.

The second threshold addresses the threat from the impact of an aircraft on the plant, and the anticipated arrival time is within 30 minutes. The intent of this EAL is to ensure that threat-related notifications are made in a timely manner so that plant personnel and State and local agencies are in a heightened state of readiness. This EAL is met when the threat-related information has been validated in accordance with OP-AA-900 Authentication (ref. 2).

The NRC Headquarters Operations Officer (HOO) will communicate to the licensee if the threat involves an aircraft. The status and size of the plane may be provided by NORAD through the NRC.

In some cases, it may not be readily apparent if an aircraft impact within the OWNER CONTROLLED AREA was intentional (i.e., a HOSTILE ACTION). It is expected, although not certain, that notification by an appropriate Federal agency to the site would clarify this point. In this case, the appropriate federal agency is intended to be NORAD, FBI, FAA or NRC. The emergency declaration, including one based on other ICs/EALs, should not be unduly delayed while awaiting notification by a Federal agency.

Emergency plans and implementing procedures are public documents; therefore, EALs should not incorporate Security-sensitive information. This includes information that may be advantageous to a potential adversary, such as the particulars concerning a specific threat or threat location. Security-sensitive information should be contained in non-public documents such as the Security Plan for MPS (ref. 1).

Escalation of the emergency classification level would be via IC HS1.

Reference(s):

1. Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program
2. OP-AA-900, "Authentication"
3. NEI 99-01 HA1

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 1 – Security

Initiating Condition: HOSTILE ACTION within the PROTECTED AREA

EAL:

HS1.1 Site Area Emergency

A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by MPS Security Shift Supervision

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

HOSTILE FORCE - One or more individuals who are engaged in a determined assault, overtly or by stealth and deception, equipped with suitable weapons capable of killing, maiming, or causing destruction.

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses the occurrence of a HOSTILE ACTION within the PROTECTED AREA. This event will require rapid response and assistance due to the possibility for damage to plant equipment.

Timely and accurate communications between Security Shift Supervision and the Control Room is essential for proper classification of a security-related event (ref. 1).

Security plans and terminology are based on the guidance provided by NEI 03-12, *Template for the Security Plan, Training and Qualification Plan, Safeguards Contingency Plan [and Independent Spent Fuel Storage Installation Security Program]*.

As time and conditions allow, these events require a heightened state of readiness by the plant staff and implementation of onsite protective measures (e.g., evacuation, dispersal or sheltering). The Site Area Emergency declaration will mobilize State and local agency resources and have them available to develop and implement public protective actions in the unlikely event that the attack is successful in impairing multiple safety functions.

This EAL does not apply to incidents that are accidental events, acts of civil disobedience, or otherwise are not a HOSTILE ACTION perpetrated by a HOSTILE FORCE. Examples include the crash of a small aircraft, shots from hunters, physical disputes between employees, etc. Reporting of these types of events is adequately addressed by other EALs, or the requirements of 10CFR73.71 or 10CFR50.72.

Emergency plans and implementing procedures are public documents; therefore, EALs should not incorporate Security-sensitive information. This includes information that may be advantageous to a potential adversary, such as the particulars concerning a specific threat or threat location. Security-sensitive information should be contained in non-public documents such as the Security Plan for MPS (ref. 1).

Reference(s):

1. Millstone, North Anna and Surry Power Stations' Security Plan, Training and Qualification Plan, Safeguards Contingency Plan and Independent Spent Fuel Storage Installation Security Program
2. NEI 99-01 HS1

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 2 – Seismic Event

Initiating Condition: Seismic event greater than OBE levels

EAL:

HU2.1 Unusual Event

Seismic event > OBE as indicated by OBE alarm light on Panel 3ERS-PNLSM1C

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses a seismic event that results in accelerations at the plant site greater than those specified for an Operating Basis Earthquake (OBE). An earthquake greater than an OBE but less than a Safe Shutdown Earthquake (SSE) should have no significant impact on safety-related systems, structures and components; however, some time may be required for the plant staff to ascertain the actual post-event condition of the plant (e.g., performs walk-downs and post-event inspections). Given the time necessary to perform walk-downs and inspections, and fully understand any impacts, this event represents a potential degradation of the level of safety of the plant.

Event verification with external sources should not be necessary during or following an OBE. Earthquakes of this magnitude should be readily felt by on-site personnel and recognized as a seismic event (e.g., lateral accelerations in excess of 0.09g). The Shift Manager or DSEO/ADTS may seek external verification if deemed appropriate (e.g., a call to the U.S. Geological Survey (USGS), check internet news sources, etc.); however, the verification action must not preclude a timely emergency declaration.

For both MPS2 and MPS3, the OBE ground acceleration thresholds are > 0.09g horizontal or > 0.06g vertical. The MPS3 Control Room has real time OBE exceedance alarm indications (ref. 1, 2, 3). Therefore classification shall be based upon the receipt of the MPS3 OBE alarm light on MPS3 Panel 3ERS-PNLSM1C (ref. 2).

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or MA9.

Reference(s):

1. AOP 3570, "Earthquake"
2. MPS3 FSAR Section 2.1.1, "Site Location and Description"
3. NEI 99-01 HU2

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.1 Unusual Event

A tornado strike within the PROTECTED AREA

Mode Applicability:

All

Definition(s):

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

This EAL addresses a tornado striking (touching down) within the PROTECTED AREA.

Escalation of the emergency classification level would be based on ICs in Categories R, F, M or C.

If damage is confirmed visually or by other in-plant indications, the event may be escalated to an Alert under IC CA6 or MA9.

A tornado striking (touching down) within the PROTECTED AREA warrants declaration of an Unusual Event regardless of the measured wind speed at the meteorological tower. A tornado is defined as a violently rotating column of air in contact with the ground and extending from the base of a thunderstorm (ref. 1).

Reference(s):

1. AOP 3569, "Severe Weather Conditions"
2. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.2 Unusual Event

Internal room or area FLOODING of a magnitude sufficient to require manual or automatic electrical isolation of a SAFETY SYSTEM component required by Technical Specifications for the current operating mode

Mode Applicability:

All

Definition(s):

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

This EAL addresses FLOODING of a building room or area that results in operators isolating power to a SAFETY SYSTEM component due to water level or other wetting concerns.

Classification is also required if the water level or related wetting causes an automatic isolation of a SAFETY SYSTEM component from its power source (e.g., a breaker or relay trip). To warrant classification, operability of the affected component must be required by Technical Specifications for the current operating mode (ref. 1, 2).

Escalation of the emergency classification level would be based on ICs in Categories R, F, M or C.

Refer to EAL CA6.1 or MA9.1 for internal flooding affecting more than one SAFETY SYSTEM train.

Reference(s):

1. AOP 3569, "Severe Weather Conditions"
2. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.3 Unusual Event

Movement of personnel within the PROTECTED AREA is IMPEDED due to an event external to the PROTECTED AREA involving hazardous materials (e.g., an offsite chemical spill or toxic gas release)

Mode Applicability:

All

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

This EAL addresses a hazardous materials event originating at a location outside the PROTECTED AREA and of sufficient magnitude to IMPEDE the movement of personnel within the PROTECTED AREA.

Escalation of the emergency classification level would be based on ICs in Categories R, F, M or C.

Reference(s):

1. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 3 – Natural or Technological Hazard

Initiating Condition: Hazardous event

EAL:

HU3.4 Unusual Event

A hazardous event that results in on-site conditions sufficient to prohibit the plant staff from accessing the site via personal vehicles (Note 7)

Note 7: This EAL does **not** apply to routine traffic impediments such as fog, snow, ice, or vehicle breakdowns or accidents.

Mode Applicability:

All

Definition(s):

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

Basis:

This IC addresses hazardous events that are considered to represent a potential degradation of the level of safety of the plant.

This EAL addresses a hazardous event that causes an on-site impediment to vehicle movement and significant enough to prohibit the plant staff from accessing the site using personal vehicles. Examples of such an event include site FLOODING caused by a hurricane, heavy rains, up-river water releases, dam failure, etc., or an on-site train derailment blocking the access road.

This EAL is not intended to apply to routine impediments such as fog, snow, ice, or vehicle breakdowns or accidents, but rather to more significant conditions such as the Hurricane Andrew strike on Turkey Point in 1992, the FLOODING around the Cooper Station during the Midwest floods of 1993, or the FLOODING around Ft. Calhoun Station in 2011.

Escalation of the emergency classification level would be based on ICs in Categories R, F, M or C.

Reference(s):

1. NEI 99-01 HU3

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.1 Unusual Event

A FIRE is **not** extinguished within 15 min. of **any** of the following fire detection indications (Note 1):

- Report from the field (i.e., visual observation)
- Receipt of multiple (more than 1) fire alarms or indications
- Field verification of a single fire alarm

AND

The FIRE is located within **any** Table H-1 area

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table H-1 MPS3 Fire Areas

- Containment Building
- Auxiliary Building
- Control Building
- Emergency Generator Enclosure
- ESF Building
- Main Steam Valve Building
- A & B Train Service Water Cubicles
- North & South Cable Tunnels
- Yard Areas
 - RWST
 - DWST

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

The 15 minute requirement begins with a credible notification that a FIRE is occurring, or receipt of multiple VALID fire detection system alarms or field validation of a single fire alarm. The alarm is to be validated using available Control Room indications or alarms to prove that it is not spurious, or by reports from the field.

Table H-1 Fire Areas are those areas that contain equipment necessary for safe operation and shutdown of the plant (ref. 1, 2).

The intent of the 15-minute duration is to size the FIRE and to discriminate against small FIRES that are readily extinguished (e.g., smoldering waste paper basket). In addition to alarms, other indications of a FIRE could be a drop in fire main pressure, automatic activation of a suppression system, etc.

Upon receipt, operators will take prompt actions to confirm the validity of an initial fire alarm, indication, or report. For EAL assessment purposes, the emergency declaration clock starts at the time that the initial alarm, indication, or report was received, and not the time that a subsequent verification action was performed. Similarly, the fire duration clock also starts at the time of receipt of the initial alarm, indication or report.

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or MA9.

Reference(s):

1. EOP 3509, "Fire Emergency"
2. CM-AA-FPA-102, "MP3 Branch Technical Position 9.5-1 Compliance Report"
3. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.2 Unusual Event

Receipt of a single fire alarm (i.e., **no** other indications of a FIRE)

AND

The fire alarm is indicating a FIRE within **any** Table H-1 area (excluding Containment Building)

AND

The existence of a FIRE is **not** verified within 30 min. of alarm receipt (Notes 1, 15)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 15: A Containment Building fire alarm is considered VALID upon receipt of multiple (more than one) fire zone alarms.

Table H-1 MPS3 Fire Areas
<ul style="list-style-type: none"> • Containment Building • Auxiliary Building • Control Building • Emergency Generator Enclosure • ESF Building • Main Steam Valve Building • A & B Train Service Water Cubicles • North & South Cable Tunnels • Yard Areas <ul style="list-style-type: none"> ○ RWST ○ DWST

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

The 30 minute requirement begins upon receipt of a single VALID fire detection system alarm. The alarm is to be validated using available Control Room indications or alarms to prove that it is not spurious, or by reports from the field. Actual field reports must be made within the 30 minute time limit or a classification must be made. If a fire is verified to be occurring by field report, classification shall be made based on EAL HU4.1, with the 15 minute requirement beginning with the verification of the fire by field report.

Table H-1 Fire Areas are those areas that contain equipment necessary for safe operation and shutdown of the plant (ref. 1, 2).

This EAL addresses receipt of a single fire alarm, and the existence of a FIRE is not verified (i.e., proved or disproved) within 30-minutes of the alarm. Upon receipt, operators will take prompt actions to confirm the validity of a single fire alarm. For EAL assessment purposes, the 30-minute clock starts at the time that the initial alarm was received, and not the time that a subsequent verification action was performed.

With regard to Containment Building fire alarms, there is constant air movement in the enclosed containment due to the operation of the containment ventilation system. The operating cooling units are drawing air to the units past the smoke detectors. It can be reasonably expected that a fire that burns for 15 minutes would produce sufficient products of combustion to cause fire detectors in multiple zones to alarm. Therefore, a single Containment Building fire alarm is not considered VALID.

A single fire alarm, absent other indication(s) of a FIRE, may be indicative of equipment failure or a spurious activation, and not an actual FIRE. For this reason, additional time is allowed to verify the validity of the alarm. The 30-minute period is a reasonable amount of time to determine if an actual FIRE exists; however, after that time, and absent information to the contrary, it is assumed that an actual FIRE is in progress.

If an actual FIRE is verified by a report from the field, then HU4.1 is immediately applicable, and the emergency must be declared if the FIRE is not extinguished within 15-minutes of the report. If the alarm is verified to be due to an equipment failure or a spurious activation, and this verification occurs within 30-minutes of the receipt of the alarm, then this EAL is not applicable and no emergency declaration is warranted.

Basis-Related Requirements from Appendix R (Although MPS3 is a BTP 9.5-1 plant (ref. 2), the following is provided as justification for the use of the 30 minute criteria)

10 CFR 50 Appendix R, states in part:

Criterion 3 of Appendix A to this part specifies that "Structures, systems, and components important to safety shall be designed and located to minimize, consistent with other safety requirements, the probability and effect of fires and explosions."

When considering the effects of fire, those systems associated with achieving and maintaining safe shutdown conditions assume major importance to safety because damage to them can lead to core damage resulting from loss of coolant through boil-off.

Because fire may affect safe shutdown systems and because the loss of function of systems used to mitigate the consequences of design basis accidents under post-fire conditions does not per se impact public safety, the need to limit fire damage to systems required to achieve and maintain safe shutdown conditions is greater than the need to limit fire damage to those systems required to mitigate the consequences of design basis accidents.

In addition, Appendix R to 10 CFR 50, requires, among other considerations, the use of 1-hour fire barriers for the enclosure of cable and equipment and associated non-safety circuits of one redundant train (G.2.c). As used in HU4.2, the 30-minutes to verify a single alarm is well within this worst-case 1-hour time period.

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or MA9.

Reference(s):

1. EOP 3509, "Fire Emergency"
2. CM-AA-FPA-102, "MP3 Branch Technical Position 9.5-1 Compliance Report"
4. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.3 Unusual Event

A FIRE within the PROTECTED AREA **not** extinguished within 60 min. of the initial report, alarm or indication (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

PROTECTED AREA - An area encompassed by physical barriers (i.e., the security fence) and to which access is controlled.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

In addition to a FIRE addressed by EAL HU4.1 or HU4.2, a FIRE within the plant PROTECTED AREA not extinguished within 60-minutes may also potentially degrade the level of plant safety.

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or MA9.

Reference(s):

1. EOP 3509, "Fire Emergency"
2. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety

Subcategory: 4 – Fire

Initiating Condition: FIRE potentially degrading the level of safety of the plant

EAL:

HU4.4 Unusual Event

A FIRE within the PROTECTED AREA that requires an offsite fire department to assist with extinguishment

Mode Applicability:

All

Definition(s):

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

PROTECTED AREA - An area encompassed by physical barriers (i.e., the security fence) and to which access is controlled.

Basis:

This IC addresses the magnitude and extent of FIRES that may be indicative of a potential degradation of the level of safety of the plant.

If a FIRE within the plant PROTECTED AREA is of sufficient size to require a response by an offsite firefighting agency (e.g., a local town Fire Department), then the level of plant safety is potentially degraded. The dispatch of an offsite firefighting agency to the site requires an emergency declaration only if it is needed to actively support firefighting efforts because the fire is beyond the capability of the Fire Brigade to extinguish. Declaration is not necessary if the agency resources are placed on stand-by, or supporting post-extinguishment recovery or investigation actions.

The Shift Fire Brigade Advisor or Shift Fire Brigade Leader will assess whether the fire conditions warrant outside assistance (ref. 1).

Depending upon the plant mode at the time of the event, escalation of the emergency classification level would be via IC CA6 or MA9.

Reference(s):

1. EOP 3509, "Fire Emergency"
2. NEI 99-01 HU4

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 5 – Hazardous Gases
Initiating Condition: Gaseous release IMPEDING access to equipment necessary for normal plant operations, cooldown or shutdown

EAL:

HA5.1 Alert

Release of a toxic, corrosive, asphyxiant or flammable gas into **any** Table H-2 room or area

AND

Entry into the room or area is prohibited or IMPEDED (Note 5)

Note 5: If the equipment in the listed room or area was already inoperable or out-of-service before the event occurred, then no emergency classification is warranted.

Table R-2 Safe Operation & Shutdown Rooms/Areas	
Room/Area	Mode
Aux. Building EI 43'	1, 2, 3
Aux. Building EI 24' East MCC/RCA EI 24' West MCC/RCA EI 24'	3, 4
Aux. Building EI 3' 8"	3
Containment ESF Building A RHR Cubicle ESF Building B RHR Cubicle ESF Building EI 36' ESF Building EI 24' East Switchgear West Switchgear	4

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

IMPEDE(D) - Personnel access to a room or area is hindered to an extent that extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is **not** routinely employed).

Basis:

This IC addresses an event involving a release of a hazardous gas that precludes or IMPEDES access to equipment necessary to maintain normal plant operation, or required for a normal plant cooldown and shutdown. This condition represents an actual or potential substantial degradation of the level of safety of the plant.

An Alert declaration is warranted if entry into the affected room/area is, or may be, procedurally required during the plant operating mode in effect at the time of the gaseous release. The emergency classification is not contingent upon whether entry is actually necessary at the time of the release.

Evaluation of the IC and EAL do not require atmospheric sampling; it only requires the DSEO/ADTS's judgment that the gas concentration in the affected room/area is sufficient to preclude or significantly IMPEDE procedurally required access. This judgment may be based on a variety of factors including an existing job hazard analysis, report of ill effects on personnel, advice from a subject matter expert or operating experience with the same or similar hazards. Access should be considered as IMPEDED if extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., requiring use of protective equipment, such as SCBAs, that is not routinely employed).

An emergency declaration is **not** warranted if any of the following conditions apply:

- The plant is in an operating mode different than the mode specified for the affected room/area (i.e., entry is not required during the operating mode in effect at the time of the gaseous release). For example, the plant is in Mode 1 when the gaseous release occurs, and the procedures used for normal operation, cooldown and shutdown do not require entry into the affected room until Mode 4.
- The gas release is a planned activity that includes compensatory measures which address the temporary inaccessibility of a room or area (e.g., fire suppression system testing).
- The action for which room/area entry is required is of an administrative or record keeping nature (e.g., normal rounds or routine inspections).
- The access control measures are of a conservative or precautionary nature, and would not actually prevent or IMPEDE a required action.
- If the equipment in the listed room or area was already inoperable, or out-of-service, before the event occurred, then no emergency should be declared since the event will have no adverse impact beyond that already allowed by Technical Specifications at the time of the event.

An asphyxiant is a gas capable of reducing the level of oxygen in the body to dangerous levels. Most commonly, asphyxiants work by merely displacing air in an enclosed environment. This reduces the concentration of oxygen below the normal level of around 19%, which can lead to breathing difficulties, unconsciousness or even death.

This EAL does not apply to firefighting activities that generate smoke and that automatically or manually activate a fire suppression system in an area.

Escalation of the emergency classification level would be via Category R, C or F ICs.

Reference(s):

1. Attachment 2, "Safe Operation & Shutdown Areas Tables A-3 & H-2 Bases"
2. NEI 99-01 HA5

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 6 – Control Room Evacuation
Initiating Condition: Control Room evacuation resulting in transfer of plant control to alternate locations

EAL:

HA6.1 Alert

An event has resulted in plant control being transferred from the Control Room to the Auxiliary Shutdown Panel

Mode Applicability:

All

Definition(s):

None

Basis:

This IC addresses an evacuation of the Control Room that results in transfer of plant control to alternate locations outside the Control Room. The loss of the ability to control the plant from the Control Room is considered to be a potential substantial degradation in the level of plant safety.

Following a Control Room evacuation, control of the plant will be transferred to alternate shutdown locations. The necessity to control a plant shutdown from outside the Control Room, in addition to responding to the event that required the evacuation of the Control Room, will present challenges to plant operators and other on-shift personnel. Activation of the ERO and emergency response facilities will assist in responding to these challenges.

Transfer of plant control begins when the last licensed operator leaves the Control Room.

Control will be established at the Auxiliary Shutdown Panel if the Control Room was evacuated for any reason (ref. 1, 2, 3).

Escalation of the emergency classification level would be via IC HS6.

Reference(s):

1. EOP 3503, "Shutdown Outside the Control Room"
2. EOP 3504, "Cooldown Outside the Control Room"
3. EOP 3509,.1 "Control Room, Cable Spreading Room Area or Instrument Rack Room Fire"
4. NEI 99-01 HA6

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 6 – Control Room Evacuation
Initiating Condition: Inability to control a key safety function from outside the Control Room
EAL:

HS6.1 Site Area Emergency

An event has resulted in plant control being transferred from the Control Room to the Auxiliary Shutdown Panel

AND

Control of **any** of the following key safety functions is **not** re-established within 15 min. of the last licensed operator leaving the Control Room (Note 1):

- Reactivity (modes 1, 2 and 3 **only**)
- Core cooling
- RCS heat removal

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 - Hot Standby, 4 – Hot Shutdown, 5 – Cold Shutdown, 6 – Refueling

Definition(s):

None

Basis:

This IC addresses an evacuation of the Control Room that results in transfer of plant control to alternate locations, and the control of a key safety function cannot be reestablished in a timely manner. The failure to gain control of a key safety function following a transfer of plant control to alternate locations is a precursor to a challenge to one or more fission product barriers within a relatively short period of time.

The determination of whether or not “control” is established at the remote safe shutdown location(s) is based on DSEO/ADTS judgment. The DSEO/ADTS is expected to make a reasonable, informed judgment within 15 minutes whether or not the operating staff has control of key safety functions from the remote safe shutdown location(s).

Transfer of plant control and the time period to establish control begins when the last licensed operator leaves the Control Room.

Control will be established at the Auxiliary Shutdown Panel if the Control Room was evacuated for any reason (ref. 1, 2, 3).

Establishment of the reactivity safety function is only applicable in Modes 1, 2 and 3. Sufficient shutdown margin has already been established once in modes 4, 5 and 6 (ref.4).

Escalation of the emergency classification level would be via IC FG1 or CG1

Reference(s):

1. EOP 3503, "Shutdown Outside the Control Room"
2. EOP 3504, "Cooldown Outside the Control Room"
3. EOP 3509.1, "Control Room, Cable Spreading Room Area or Instrument Rack Room Fire"
4. NRC EP FAQ 2015-014
5. NEI 99-01 HS6

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions existing that in the judgment of the DSEO warrant declaration of a Unusual Event

EAL:

HU7.1 Unusual Event

Other conditions exist which in the judgment of the DSEO indicate that events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate a security threat to facility protection has been initiated. **No** releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of SAFETY SYSTEMS occurs.

Mode Applicability:

All

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the DSEO to fall under the emergency classification level description for an Unusual Event.

Reference(s):

1. NEI 99-01 HU7

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions exist that in the judgment of the DSEO/ADTS warrant declaration of an Alert

EAL:

HA7.1 Alert

Other conditions exist which, in the judgment of the DSEO/ADTS, indicate that events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. **Any** releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the DSEO/ADTS to fall under the emergency classification level description for an Alert.

Reference(s):

1. NEI 99-01 HA7

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions existing that in the judgment of the DSEO/ADTS warrant declaration of a Site Area Emergency

EAL:

HS7.1 Site Area Emergency

Other conditions exist which in the judgment of the DSEO/ADTS indicate that events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage or malicious acts, (1) toward site personnel or equipment that could lead to the likely failure of or, (2) that prevent effective access to equipment needed for the protection of the public. **Any** releases are **not** expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels beyond the SITE BOUNDARY

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should **not** be construed to include acts of civil disobedience or felonious acts that are **not** part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

SITE BOUNDARY - That line beyond which the land is not owned, leased or otherwise controlled by MPS. Also see OWNER CONTROLLED AREA.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the DSEO/ADTS to fall under the emergency classification level description for a SITE AREA EMERGENCY.

Reference(s):

1. NEI 99-01 HS7

Category: H – Hazards and Other Conditions Affecting Plant Safety
Subcategory: 7 – DSEO/ADTS Judgment
Initiating Condition: Other conditions exist that in the judgment of the DSEO/ADTS warrant declaration of a General Emergency

EAL:

HG7.1 General Emergency

Other conditions exist which in the judgment of the DSEO/ADTS indicate that events are in progress or have occurred which involve actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity or HOSTILE ACTION that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels offsite for more than the immediate site area.

Mode Applicability:

All

Definition(s):

HOSTAGE - A person(s) held as leverage against the station to ensure that demands will be met by the station.

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

HOSTILE ACTION - An act toward MPS or its personnel that includes the use of violent force to destroy equipment, take HOSTAGES, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, PROJECTILES, vehicles, or other devices used to deliver destructive force. Other acts that satisfy the overall intent may be included. Hostile action should not be construed to include acts of civil disobedience or felonious acts that are not part of a concerted attack on MPS. Non-terrorism-based EALs should be used to address such activities (i.e., this may include violent acts between individuals in the OWNER CONTROLLED AREA).

OWNER CONTROLLED AREA - The area within the SITE BOUNDARY including the PROTECTED AREA.

PROJECTILE - An object directed toward a Nuclear Power Plant that could cause concern for its continued operability, reliability, or personnel safety.

PROTECTED AREA - The area within the Millstone Power Station security fence.

Basis:

This IC addresses unanticipated conditions not addressed explicitly elsewhere but that warrant declaration of an emergency because conditions exist which are believed by the DSEO/ADTS to fall under the emergency classification level description for a GENERAL EMERGENCY.

Reference(s):

1. NEI 99-01 HG7

Category M – System Malfunction

EAL Group: Hot Conditions (RCS temperature > 200°F); EALs in this category are applicable only in one or more hot operating modes.

Numerous system-related equipment failure events that warrant emergency classification have been identified in this category. They may pose actual or potential threats to plant safety.

The events of this category pertain to the following subcategories:

1. Loss of Emergency AC Power

Loss of emergency plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of onsite and offsite power sources for 4.16 KV emergency buses.

2. Loss of Vital DC Power

Loss of emergency plant electrical power can compromise plant safety system operability including decay heat removal and emergency core cooling systems which may be necessary to ensure fission product barrier integrity. This category includes loss of power to or degraded voltage on the 125V DC vital buses.

3. Loss of Control Room Indications

Certain events that degrade plant operator ability to effectively assess plant conditions within the plant warrant emergency classification. Losses of indicators are in this subcategory.

4. RCS Activity

During normal operation, reactor coolant fission product activity is very low. Small concentrations of fission products in the coolant are primarily from the fission of tramp uranium in the fuel clad or minor perforations in the clad itself. Any significant rise from these base-line levels (2% - 5% clad failures) is indicative of fuel failures and is covered under the Fission Product Barrier Degradation category. However, lesser amounts of clad damage may result in coolant activity exceeding Technical Specification limits. These fission products will be circulated with the reactor coolant and can be detected by coolant sampling.

5. RCS Leakage

The reactor vessel provides a volume for the coolant that covers the reactor core. The reactor pressure vessel and associated pressure piping (reactor coolant system) together provide a barrier to limit the release of radioactive material should the reactor fuel clad integrity fail. Excessive RCS leakage greater than Technical Specification limits indicates potential pipe cracks that may propagate to an extent threatening fuel clad, RCS and containment integrity.

6. RPS Failure

This subcategory includes events related to failure of the Reactor Protection System (RPS) to initiate and complete reactor trips. In the plant licensing basis, postulated failures of the RPS to complete a reactor trip comprise a specific set of analyzed events referred to as Anticipated Transient Without Scram (ATWS) events. For EAL classification, however, ATWS is intended to mean any scram failure event that does not achieve reactor shutdown. If RPS actuation fails to properly result in reactor shutdown, positive control of reactivity is at risk and could cause a threat to fuel clad, RCS and containment integrity.

7. Loss of Communications

Certain events that degrade plant operator ability to effectively communicate with essential personnel within or external to the plant warrant emergency classification.

8. Containment Failure

Failure of containment isolation capability (under conditions in which the containment is not currently challenged) warrants emergency classification. Failure of containment pressure control capability also warrants emergency classification.

9. Hazardous Event Affecting Safety Systems

Various natural and technological events that result in degraded plant safety system train performance or significant VISIBLE DAMAGE warrant emergency classification under this subcategory.

Category: M – System Malfunction
Subcategory: 1 – Loss of Emergency AC Power
Initiating Condition: Loss of **all** offsite AC power capability to emergency buses for 15 minutes or longer

EAL:

MU1.1 Unusual Event

Loss of **all** offsite AC power capability, Table M-1, to 4.16 kV emergency buses 34C and 34D for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-1 AC Power Sources
Offsite <ul style="list-style-type: none">• Unit 3 Normal Station Service Transformer (NSST)• Unit 3 Reserve Station Service Transformer (RSST)
Onsite <ul style="list-style-type: none">• EDG A• EDG B• SBO Diesel Generator (if already aligned)

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

Table M-1 provides a list of offsite AC electrical power sources credited for this EAL.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC addresses a prolonged loss of offsite power. The loss of offsite power sources renders the plant more vulnerable to a complete loss of power to AC emergency buses. This condition represents a potential reduction in the level of safety of the plant.

For emergency classification purposes, "capability" means that an offsite AC power source(s) is available to the emergency buses, whether or not the buses are powered from it.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of offsite power.

Escalation of the emergency classification level would be via IC MA1.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS3 FSAR Section 8.2, "Off Site Power System"
3. MPS3 FSAR Section 8.3, "On Site Power System"
4. AOP 3577, "Loss of Normal and Offsite Power to a 4.16 kV Emergency Bus"
5. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
6. NEI 99-01 SU1

Category: M – System Malfunction
Subcategory: 1 – Loss of Emergency AC Power
Initiating Condition: Loss of **all but one** AC power source to emergency buses for 15 minutes or longer

EAL:

MA1.1 Alert

AC power capability, Table M-1, to 4.16 kV emergency buses 34C and 34D reduced to a single power source for ≥ 15 min. (Note 1)

AND

Any additional single power source failure will result in loss of **all** AC power to SAFETY SYSTEMS

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-1 AC Power Sources	
Offsite	
<ul style="list-style-type: none"> Unit 3 Normal Station Service Transformer (NSST) Unit 3 Reserve Station Service Transformer (RSST) 	
Onsite	
<ul style="list-style-type: none"> EDG A EDG B SBO Diesel Generator (if already aligned) 	

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

Table M-1 provides a list of offsite and onsite AC electrical power sources credited for this EAL. The AC power source (SBO Diesel Generator) annotated "(if already aligned)" requires more than 15 minutes to establish and therefore is only credited if the source was already aligned at the time of AC power loss.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC describes a significant degradation of offsite and onsite AC power sources such that any additional single failure would result in a loss of all AC power to SAFETY SYSTEMS. In this condition, the sole AC power source may be powering one, or more than one, train of safety-related equipment. This IC provides an escalation path from IC MU1.

An "AC power source" is a source recognized in AOPs and EOPs, and capable of supplying required power to an emergency bus. Some examples of this condition are presented below.

- A loss of all offsite power with a concurrent failure of all but one emergency power source (e.g., an onsite diesel generator).
- A loss of all offsite power and loss of all emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being fed from the SBO Diesel Generator.
- A loss of emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being fed from an offsite power source.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of power.

Escalation of the emergency classification level would be via IC MS1.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

This hot condition EAL is equivalent to the cold condition EAL CU2.1.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS3 FSAR Section 8.2, "Off Site Power System"
3. MPS3 FSAR Section 8.3, "On Site Power System"
4. AOP 3577, "Loss of Normal and Offsite Power to a 4.16 kV Emergency Bus"
5. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
6. NEI 99-01 SA1

Category: M – System Malfunction

Subcategory: 1 – Loss of Emergency AC Power

Initiating Condition: Loss of **all** offsite power and **all** onsite AC power to emergency buses for 15 minutes or longer

EAL:

MS1.1 Site Area Emergency

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 34C and 34D for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators, provided it can be aligned within the 15 minute classification criteria.

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC addresses a total loss of AC power that compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. In addition, fission product barrier monitoring capabilities may be degraded under these conditions. This IC represents a condition that involves actual or likely major failures of plant functions needed for the protection of the public.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs RG1, FG1 or MG1.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

This hot condition EAL is equivalent to the cold condition EAL CA2.1.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS3 FSAR Section 8.2, "Off Site Power System"
3. MPS3 FSAR Section 8.3, "On Site Power System"
4. AOP 3577, "Loss of Normal and Offsite Power to a 4.16 kV Emergency Bus"
5. EOP 35 ECA-0.0, "Loss of All AC Power"
6. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
7. NEI 99-01 SS1

Category: M –System Malfunction
Subcategory: 1 – Loss of Vital AC Power
Initiating Condition: Prolonged loss of **all** offsite and **all** onsite AC power to emergency buses

EAL:

MG1.1 General Emergency

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 34C and 34D

AND

Core Cooling-RED Path conditions met

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators (ref. 1, 2, 3, 4, 5).

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

This IC addresses a prolonged loss of all power sources to AC emergency buses that results in degraded core cooling. A loss of all AC power compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. A prolonged loss of these buses will eventually lead to a loss of one or more fission product barriers. In addition, fission product barrier monitoring capabilities may be degraded under these conditions.

The degraded core cooling threshold is based on meeting the Core Cooling Red Path criteria for Core Exit Thermocouple readings (> 1,200 °F) (ref. 6).

For extended loss of emergency bus AC power events that do not result in a breach of the RCS barrier, this EAL should require declaration of a General Emergency prior to meeting the thresholds for IC FG1. This will allow additional time for implementation of offsite protective actions.

The EAL will require a General Emergency declaration if the loss of AC power results in parameters that indicate an inability to adequately remove decay heat from the core.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS3 FSAR Section 8.2, "Off Site Power System"
3. MPS3 FSAR Section 8.3, "On Site Power System"
4. AOP 3577, "Loss of Normal and Offsite Power to a 4.16 kV Emergency Bus"
5. EOP 35 ECA-0.0, "Loss of All AC Power"
6. EOP 35 F-0.2, "CSFST Core Cooling"
7. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
8. NEI 99-01 SG1

Category: M – System Malfunction
Subcategory: 2 – Loss of Vital DC Power
Initiating Condition: Loss of **all** vital DC power for 15 minutes or longer
EAL:

MS2.1 Site Area Emergency

Indicated voltage is < 105 VDC on **both** vital 125 VDC battery buses 1 **AND** 2 for ≥15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

The vital (Class 1E) 125 V DC power system consists of four physically and electrically separated channels (buses). Each bus has a 60 cell battery bank with a two-hour rating before reaching the minimum bus voltage of 105 Volts (1.75 volts/cell).

Battery buses 1 and 2 power reactor protection, engineered safety features, vital loads and instrumentation (ref. 1, 2, 3). Battery buses 3 and 4 are not credited for this EAL.

This IC addresses a loss of vital DC power which compromises the ability to monitor and control SAFETY SYSTEMS. In modes above Cold Shutdown, this condition involves a major failure of plant functions needed for the protection of the public.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs RG1, FG1 or MG1.

This hot condition EAL equivalent of the cold condition EAL CU4.1.

Reference(s):

1. MPS3 FSAR Section 8.3.2, "DC Power Systems"
2. AOP 3563, "Loss of DC Bus Power"
3. Technical Specifications Section 3.8.2.1, "D.C. Sources"
4. NEI 99-01 SS8

Category: M –System Malfunction
Subcategory: 2 – Loss of Vital DC Power
Initiating Condition: Loss of **all** emergency AC and vital DC power sources for 15 minutes or longer

EAL:

MG2.1 General Emergency

Loss of **all** offsite and **all** onsite AC power to 4.16 kV emergency buses 34C and 34D for ≥ 15 min. (Note 1)

AND

Indicated voltage is < 105 VDC on **both** vital 125 VDC battery buses 1 **AND** 2 for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

Basis:

This IC addresses a concurrent and prolonged loss of both emergency AC and vital DC power. A loss of all emergency AC power compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. A loss of vital DC power compromises the ability to monitor and control SAFETY SYSTEMS. A sustained loss of both emergency AC and vital DC power will lead to multiple challenges to fission product barriers.

For this EAL credit can be taken for any AC power source that has sufficient capability to operate equipment necessary to maintain a safe shutdown condition, such as the Unit 3 SBO diesel or FLEX generators, provided it can be aligned within the 15 minute classification criteria (ref. 1, 2, 3, 4, 5).

The SBO Diesel Generator meets the definition for an Alternate AC power source per 10CFR50.2.

Safe shutdown equipment is powered from 4.16 kV emergency buses 34C and 34D. During normal operation, buses 34C and 34D are powered from normal station service transformer A.

When the unit is shutdown, these buses are normally powered from offsite power through either the Normal Station Service Transformer or Reserve Station Service Transformer A.

MPS3 has the following methods available to provide power to the 4.16 kV emergency buses:

- The Normal Station Service Transformer (NSST). During normal power operation, power is supplied to buses 34A and 34B. Bus ties connect to 4.16 kV emergency buses 34C and 34D. Additionally, the NSST, back-fed via the Main Transformer, may be used during shutdown and when maintenance is being performed on the Reserve Station Service Transformer.
- The MPS3 Reserve Station Service Transformer (RSST). During other periods, such as startup and shutdown when the NSST is not used, power is supplied from the MPS3 RSST directly to 4.16 kV emergency buses 34C and 34D.
- Diesel Generator 15G-14U supplying 4.16 kV emergency bus 34C.
- Diesel Generator 15G-15U supplying 4.16 kV emergency bus 34D.
- SBO Diesel Generator

The vital (Class 1E) 125 V DC power system consists of four physically and electrically separated channels (buses). Each bus has a 60 cell battery bank with a two-hour rating before reaching the minimum bus voltage of 105 Volts (1.75 volts/cell).

Battery buses 1 and 2 power reactor protection, engineered safety features, vital loads and instrumentation (ref. 6, 7, 8). Battery buses 3 and 4 are not credited for this EAL.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses. The 15-minute emergency declaration clock begins at the point when both EAL thresholds are met.

Reference(s):

1. Technical Specifications Section 3.8.1.2, "AC Sources Shutdown"
2. MPS3 FSAR Section 8.2, "Off Site Power System"
3. MPS3 FSAR Section 8.3, "On Site Power System"
4. AOP 3577, "Loss of Normal and Offsite Power to a 4.16 kV Emergency Bus"
5. EOP 35 ECA-0.0, "Loss of All AC Power"
6. MPS3 FSAR Section 8.3.2, "DC Power Systems"
7. AOP 3563, "Loss of DC Bus Power"
8. Technical Specifications Section 3.8.2.1, "D.C. Sources"
9. Dwg. No. 12179-EE-1A-11, "Millstone Unit 3 Main One Line/Phasing Diagram Pwr Distr Sys Composite"
10. NEI 99-01 SG8

Category: M – System Malfunction
Subcategory: 3 – Loss of Control Room Indications
Initiating Condition: UNPLANNED loss of Control Room indications for 15 minutes or longer

EAL:

MU3.1 Unusual Event

An UNPLANNED event results in the inability to monitor one or more Table M-2 parameters from within the Control Room for ≥ 15 min. (Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-2 Safety System Parameters

- Reactor power
- RCS level
- RCS pressure
- CET temperature
- Level in at least one SG
- Auxiliary feedwater flow to at least one SG

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

Applicable safety system parameters are listed in Table M-2.

The Plant Process Computer System and Safety Parameter Display System (SPDS) serve as redundant indicators which may be utilized as compensatory measures in lieu of the Control Room indicators associated with safety functions (ref. 1, 2, 3).

This IC addresses the difficulty associated with monitoring normal plant conditions without the ability to obtain SAFETY SYSTEM parameters from within the Control Room. This condition is a precursor to a more significant event and represents a potential degradation in the level of safety of the plant.

As used in this EAL, an "inability to monitor" means that values for one or more of the listed parameters cannot be determined from within the Control Room. This situation would require a loss of all of the Control Room sources for the given parameter(s). For example, the reactor power level cannot be determined from any analog, digital and recorder source within the Control Room.

An event involving a loss of plant indications, annunciators and/or display systems is evaluated in accordance with 10 CFR 50.72 (and associated guidance in NUREG-1022) to determine if an NRC event report is required. The event would be reported if it significantly impaired the capability to perform emergency assessments. In particular, emergency assessments necessary to implement abnormal operating procedures, emergency operating procedures, and emergency plan implementing procedures addressing emergency classification, accident assessment, or protective action decision-making.

This EAL is focused on a selected subset of plant parameters associated with the key safety functions of reactivity control, core cooling and RCS heat removal. The loss of the ability to determine one or more of these parameters from within the Control Room is considered to be more significant than simply a reportable condition. In addition, if all indication sources for one or more of the listed parameters are lost, then the ability to determine the values of other SAFETY SYSTEM parameters may be impacted as well. For example, if the value for RCS water level cannot be determined from the indications and recorders on a main control board, the SPDS or the plant computer, the availability of other parameter values may be compromised as well.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation of the emergency classification level would be via IC MA3.

Reference(s):

1. MPS3 FSAR Section 7.5, "Safety Related Display Instrumentation"
2. AOP 3571, "Instrument Failure Response"
3. AOP 3574, "Loss of Main Board Annunciation"
4. NEI 99-01 SU2

Category: M – System Malfunction
Subcategory: 3 – Loss of Control Room Indications
Initiating Condition: UNPLANNED loss of Control Room indications for 15 minutes or longer with a significant transient in progress

EAL:

MA3.1 Alert

An UNPLANNED event results in the inability to monitor one or more Table M-2 parameters from within the Control Room for ≥ 15 min. (Note 1)

AND

Any significant transient is in progress, Table M-3

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Table M-2 Safety System Parameters

- Reactor power
- RCS level
- RCS pressure
- CET temperature
- Level in at least one SG
- Auxiliary feedwater flow to at least one SG

Table M-3 Significant Transients

- Automatic turbine runback > 25% reactor power
- Electrical load rejection > 25% full electrical load
- Reactor Trip
- SI Actuation

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

UNPLANNED - A parameter change or an event that is **not** 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Basis:

Applicable safety system parameters are listed in Table M-2.

Significant transients are listed in Table M-3.

The Plant Process Computer System and Safety Parameter Display System (SPDS) serve as redundant indicators which may be utilized as compensatory measures in lieu of the Control Room indicators associated with safety functions (ref. 1, 2, 3). This IC addresses the difficulty associated with monitoring rapidly changing plant conditions during a transient without the ability to obtain SAFETY SYSTEM parameters from within the Control Room. During this condition, the margin to a potential fission product barrier challenge is reduced. It thus represents a potential substantial degradation in the level of safety of the plant.

As used in this EAL, an "inability to monitor" means that values for one or more of the listed parameters cannot be determined from within the Control Room. This situation would require a loss of all of the Control Room sources for the given parameter(s). For example, the reactor power level cannot be determined from any analog, digital and recorder source within the Control Room.

An event involving a loss of plant indications, annunciators and/or display systems is evaluated in accordance with 10 CFR 50.72 (and associated guidance in NUREG-1022) to determine if an NRC event report is required. The event would be reported if it significantly impaired the capability to perform emergency assessments. In particular, emergency assessments necessary to implement abnormal operating procedures, emergency operating procedures, and emergency plan implementing procedures addressing emergency classification, accident assessment, or protective action decision-making.

This EAL is focused on a selected subset of plant parameters associated with the key safety functions of reactivity control, core cooling and RCS heat removal. The loss of the ability to determine one or more of these parameters from within the Control Room is considered to be more significant than simply a reportable condition. In addition, if all indication sources for one or more of the listed parameters are lost, then the ability to determine the values of other

SAFETY SYSTEM parameters may be impacted as well. For example, if the value for RCS water level cannot be determined from the indications and recorders on a main control board, the SPDS or the plant computer, the availability of other parameter values may be compromised as well.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation of the emergency classification level would be via ICs FS1 or RS1

Reference(s):

1. MPS3 FSAR Section 7.5, "Safety Related Display Instrumentation"
2. AOP 3571, "Instrument Failure Response"
3. AOP 3574, "Loss of Main Board Annunciation"
4. NEI 99-01 SA2

Category: M – System Malfunction

Subcategory: 4 – RCS Activity

Initiating Condition: RCS activity greater than Technical Specification allowable limits

EAL:

MU4.1 Unusual Event

Dose rate at 1 ft. from an unpressurized RCS sample \geq Table M-4

Table M-4 Tech. Spec. Coolant Activity Dose Rates	
Time > Shutdown (hrs)	mR/hr/ml
≤ 2	0.7
$> 2 - \leq 8$	0.5
> 8	0.3

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

This IC addresses a reactor coolant activity value that exceeds an allowable limit specified in Technical Specifications. This condition is a precursor to a more significant event and represents a potential degradation of the level of safety of the plant.

Per Engineering Calculation RA-0059 (ref. 1), dose rate is assumed to result from radioactive iodines (I-131 thru I-135) in RCS in concentrations corresponding to 60 $\mu\text{Ci/gm}$ DEI-131. This value corresponds to the Technical Specification coolant activity limit for iodine spike at full power operations (ref. 2). The values contained in Table M-4 (Tech. Spec. Coolant Activity Dose Rates) represent expected one foot dose rates per ml of sample based on time since reactor shutdown to the time when the sample is taken (ref. 3). Time after shutdown values are provided to account for radioactive decay. The expected dose rate is a near linear relationship with the volume of the sample, so any volume collected can be determined by dividing the measured dose rate by the sample volume and comparing to the threshold value from Table M-4 for the applicable time frame. These dose rates assume no emergency core cooling system (ECCS) injection so there is no dilution credited which would vary coolant volume. Values in the table have been rounded for ease of use. The > 8 hour threshold is conservative up to 24 hours following reactor shutdown. After 24 hours, the expected response from radioactive iodine levels off. Therefore, the value shown for > 8 hours applies for all samples taken 8 hours or more since reactor shutdown.

The values specified in Table M-4 were developed using a method to minimize error (+/-) for the threshold value within each defined time period. Values were chosen to minimize error from the highest to lowest dose rate within each range.

It should be noted that this EALs is primarily directed toward mechanical damage to the clad not involving inadequate core cooling (ICC) sequences. Clad damage due to ICC sequences is addressed by the fuel clad and CTMT fission product barrier thresholds (Category F).

Escalation of the emergency classification level would be via IC FA1 or the Category R ICs.

Reference(s):

1. RA-0059, "Detector Response to an RCS Sample for EAL Classification of Fuel Clad Degradation and Barrier Loss"
2. MPS3 Technical Specification 3.4.8, "RCS Specific Activity"
3. AOP 3553, "High RCS Activity"
4. NEI 99-01 SU3

Category: M – System Malfunction

Subcategory: 4 – RCS Activity

Initiating Condition: Reactor coolant activity greater than Technical Specification allowable limits

EAL:

MU4.2 Unusual Event

Sample analysis indicates that a reactor coolant activity value is > an allowable limit specified in Technical Specification 3.4.8

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

This IC addresses a reactor coolant activity value that exceeds an allowable limit specified in Technical Specifications. This condition is a precursor to a more significant event and represents a potential degradation of the level of safety of the plant.

Escalation of the emergency classification level would be via ICs FA1 or the Category R ICs.

Reference(s):

1. Technical Specification Section 3.4.8, "RCS Specific Activity"
2. AOP 3553, "High RCS Activity"
3. NEI 99-01 SU3

Category: M – System Malfunction
Subcategory: 5 – RCS Leakage
Initiating Condition: RCS leakage for 15 minutes or longer
EAL:

MU5.1 Unusual Event

RCS unidentified or pressure boundary leakage > 10 gpm for ≥ 15 min.

OR

RCS identified leakage > 25 gpm for ≥ 15 min.

OR

Leakage from the RCS to a location outside containment > 25 gpm for ≥ 15 min.

(Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

UNISOLABLE - An open or breached system line that **cannot** be isolated, remotely or locally.

Basis:

The 15-minute threshold duration allows sufficient time for prompt operator actions to isolate the leakage, if possible.

Once the RCS leak rate has been quantified to be greater than the specified value, failure to isolate the leak within 15 minutes, or if known that the leak cannot be isolated within 15 minutes, from the time of leak rate quantification, requires immediate classification.

This IC addresses RCS leakage which may be a precursor to a more significant event. In this case, RCS leakage has been detected and operators, following applicable procedures, have been unable to promptly isolate the leak. This condition is considered to be a potential degradation of the level of safety of the plant.

The first and second EAL conditions are focused on a loss of mass from the RCS due to "unidentified leakage", "pressure boundary leakage" or "identified leakage" (as these leakage types are defined in the plant Technical Specifications) (ref. 1, 2). The third condition addresses an RCS mass loss caused by an UNISOLABLE leak through an interfacing system. These conditions thus apply to leakage into the containment, a secondary-side system (e.g., steam generator tube leakage) or a location outside of containment.

The leak rate values for each condition were selected because they are usually observable with normal Control Room indications. Lesser values typically require time-consuming calculations to determine (e.g., a mass balance calculation). The first condition uses a lower value that reflects the greater significance of unidentified or pressure boundary leakage.

The release of mass from the RCS due to the as-designed/expected operation of a relief valve does not warrant an emergency classification. An emergency classification would be required if a mass loss is caused by a relief valve that is not functioning as designed/expected (e.g., a relief valve sticks open and the line flow cannot be isolated).

Escalation of the emergency classification level would be via ICs of Category R or F.

Reference(s):

1. Technical Specification Section 1.0, "Definitions"
2. Technical Specification Section 3.4.6, "RCS Leakage"
3. AOP 3555, "RCS Leak"
4. NEI 99-01 SU4

Category: M – System Malfunction

Subcategory: 6 – RPS Failure

Initiating Condition: Automatic or manual trip fails to shut down the reactor

EAL:

MU6.1 Unusual Event

An automatic trip did **not** shut down the reactor as indicated by reactor power > 5% after any RPS setpoint is exceeded

AND

A subsequent automatic trip or **EITHER** manual trip (Rx Trip Switches **OR** opening breakers 32B and 32N) are successful in shutting down the reactor as indicated by reactor power $\leq 5\%$ (Note 8)

Note 8: A manual trip action is **any** operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does **not** include manually driving in control rods or implementation of boron injection strategies.

Mode Applicability:

1 - Power Operation

Definition(s):

IMMINENT - The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

Basis:

This EAL addresses a failure of the RPS to initiate or complete an automatic reactor trip that results in a reactor shutdown, and either a subsequent operator manual action taken at the reactor control consoles or an automatic trip (i.e., any subsequent RPS setpoint trip) is successful in shutting down the reactor. This event is a precursor to a more significant condition and thus represents a potential degradation of the level of safety of the plant.

This EAL is applicable to failure to trip events initiated in Mode 1 only, since the power level specified for a successful shutdown (5%) aligns with the Mode 1 transition power (5%) (ref. 1, 5).

Following the failure on an automatic reactor trip, operators will promptly initiate manual actions at the reactor control consoles to shutdown the reactor (e.g., initiate a manual reactor trip using Rx Trip Switches or opening breakers 32B and 32N). If these manual actions are successful in shutting down the reactor, core heat generation will quickly fall to a level within the capabilities of the plant's decay heat removal systems ($\leq 5\%$).

A manual action at the reactor control consoles is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor trip using Rx Trip Switches or opening breakers 32B and 32N). This action does not include manually driving in control rods or implementation of boron injection strategies. Actions taken at other locations within the Control Room, or any location outside the Control Room, are not considered to be "at the reactor control consoles".

The plant response to the failure of an automatic trip will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other concurrent plant conditions, etc. If subsequent operator manual actions taken at the reactor control consoles are also unsuccessful in shutting down the reactor, then the emergency classification level will escalate to an Alert via IC MA6. Depending upon the plant response, escalation is also possible via IC FA1. Absent the plant conditions needed to meet either IC MA6 or FA1, an Unusual Event declaration is appropriate for this event.

A reactor shutdown ($\leq 5\%$) is determined in accordance with EOP 35 F-0.1 Subcriticality Red Path criteria (ref. 3, 4, 5).

Should a reactor trip signal be generated as a result of plant work (e.g., RPS setpoint testing), the following classification guidance should be applied.

- If the signal causes a plant transient that should have included an automatic reactor trip and the RPS fails to automatically shutdown the reactor, then this IC and the EALs are applicable, and should be evaluated.
- If the signal does not cause a plant transient and the trip failure is determined through other means (e.g., assessment of test results), then this IC and the EALs are not applicable and no classification is warranted.

In the event that the operator identifies a reactor trip is IMMINENT and initiates a successful manual reactor trip before the automatic RPS trip setpoint is reached, no declaration is required. The successful manual trip of the reactor before it reaches its automatic trip setpoint or reactor trip signals caused by instrumentation channel failures do not lead to a potential fission product barrier loss. However, if subsequent manual reactor trip actions fail to shutdown the reactor, the event escalates to the Alert under EAL MA6.1.

Reference(s):

1. Technical Specification Table 1.2, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Trip System Instrumentation"
3. EOP 35, E-0 "Reactor Trip or Safety Injection"
4. EOP 35, F-0.1 "Subcriticality"
5. EOP FR-S.1, "Response to Nuclear Power Generation/ATWS"
6. NEI 99-01 SU5

Category: M – System Malfunction
Subcategory: 6 – RPS Failure
Initiating Condition: Automatic or manual trip fails to shut down the reactor
EAL:

MU6.2 Unusual Event

A manual trip (Rx Trip Switches **OR** opening breakers 32B and 32N) did **not** shut down the reactor as indicated by reactor power > 5%

AND

A subsequent manual trip **OR** automatic trip is successful in shutting down the reactor as indicated by reactor power $\leq 5\%$ (Note 8)

Note 8: A manual trip action is **any** operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does **not** include manually driving in control rods or implementation of boron injection strategies.

Mode Applicability:

1 - Power Operation

Definition(s):

None

Basis:

This EAL addresses a failure of a manual reactor trip that results in a reactor shutdown, and either a subsequent operator manual action taken at the reactor control consoles or an automatic trip is successful in shutting down the reactor. This event is a precursor to a more significant condition and thus represents a potential degradation of the level of safety of the plant.

This EAL is applicable to failure to trip events initiated in Mode 1 only, since the power level specified for a successful shutdown (5%) aligns with the Mode 1 transition power (5%) (ref. 1, 5).

If an initial manual reactor trip is unsuccessful, operators will promptly take manual actions at other location(s) on the reactor control consoles to shutdown the reactor (e.g., initiate a manual reactor trip using a different switch or breaker).

Depending upon several factors, the initial or subsequent effort to manually trip the reactor, or a concurrent plant condition, may lead to the generation of an automatic reactor trip signal. If a subsequent manual or automatic trip is successful in shutting down the reactor, core heat generation will quickly fall to a level within the capabilities of the plant's decay heat removal systems ($\leq 5\%$) (ref. 4, 5).

A manual action at the reactor control consoles is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor trip using Rx Trip Switches or opening breakers 32B and 32N). This action does not include locally tripping reactor trip and bypass breakers or tripping the turbine, manually driving in control rods or implementation of boron injection strategies. Actions taken at other locations

within the Control Room, or any location outside the Control Room, are not considered to be "at the reactor control consoles".

The plant response to the failure of a manual reactor trip will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other concurrent plant conditions, etc. If subsequent operator manual actions taken at the reactor control consoles are also unsuccessful in shutting down the reactor, then the emergency classification level will escalate to an Alert via IC MA6. Depending upon the plant response, escalation is also possible via IC FA1. Absent the plant conditions needed to meet either IC MA6 or FA1, an Unusual Event declaration is appropriate for this event.

A reactor shutdown ($\leq 5\%$) is determined in accordance with EOP 35 F-0.1 Subcriticality Red Path criteria (ref. 3, 4, 5).

Should a reactor trip signal be generated as a result of plant work (e.g., RPS setpoint testing), the following classification guidance should be applied.

- If the signal causes a plant transient that should have included an automatic reactor trip and the RPS fails to automatically shutdown the reactor, then this IC and the EALs are applicable, and should be evaluated.
- If the signal does not cause a plant transient and the trip failure is determined through other means (e.g., assessment of test results), then this IC and the EALs are not applicable and no classification is warranted.

Reference(s):

1. Technical Specification Table 1.2, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Trip System Instrumentation"
3. EOP 35 E-0, "Reactor Trip or Safety Injection"
4. EOP 35 F-0.1, "Subcriticality"
5. EOP FR-S.1, "Response to Nuclear Power Generation/ATWS"
6. NEI 99-01 SU5

Category: M – System Malfunction

Subcategory: 2 – RPS Failure

Initiating Condition: Automatic or manual trip fails to shut down the reactor and subsequent manual actions taken at the reactor control consoles are **not** successful in shutting down the reactor

EAL:

MA6.1 Alert

An automatic or manual trip (Rx Trip Switches **OR** opening breakers 32B and 32N) did **not** shut down the reactor as indicated by reactor power > 5%

AND

Subsequent automatic or manual trip actions (Rx Trip Switches **AND** opening breakers 32B and 32N) are **not** successful in shutting down the reactor as indicated by reactor power > 5% (Note 8)

Note 8: A manual trip action is **any** operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does **not** include manually driving in control rods or implementation of boron injection strategies.

Mode Applicability:

1 - Power Operation

Definition(s):

None

Basis:

This IC addresses a failure of the RPS to initiate or complete an automatic reactor trip or failure of a manual reactor trip that results in a reactor shutdown, and subsequent operator manual actions taken at the reactor control consoles to shutdown the reactor are also unsuccessful. This condition represents an actual or potential substantial degradation of the level of safety of the plant. An emergency declaration is required even if the reactor is subsequently shutdown by an action taken away from the reactor control consoles since this event entails a significant failure of the RPS.

This EAL is applicable to failure to trip events initiated in Mode 1 only, since the power level specified for a successful shutdown (5%) aligns with the Mode 1 transition power (5%) (ref. 1, 5).

A manual action at the reactor control consoles is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor trip using Rx Trip Switches or opening breakers 32B and 32N). This action does not include locally tripping reactor trip and bypass breakers or tripping the turbine, manually driving in control rods or implementation of boron injection strategies. If this action(s) is unsuccessful, operators would immediately pursue additional manual actions at locations away from the reactor control consoles (e.g., locally opening breakers). Actions taken at locations within the Control Room, or any location outside the Control Room, are not considered to be "at the reactor control consoles".

The plant response to the failure of an automatic or manual reactor trip will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other concurrent plant conditions, etc. If the failure to shut down the reactor is prolonged enough to cause a challenge to the core cooling or RCS heat removal safety functions, the emergency classification level will escalate to a Site Area Emergency via IC MS6. Depending upon plant responses and symptoms, escalation is also possible via IC FS1. Absent the plant conditions needed to meet either IC MS6 or FS1, an Alert declaration is appropriate for this event.

It is recognized that plant responses or symptoms may also require an Alert declaration in accordance with the Category F ICs; however, this IC and EAL are included to ensure a timely emergency declaration.

A reactor shutdown ($\leq 5\%$) is determined in accordance with applicable EOP 35 F-0.1 SUBCRITICALITY Red Path criteria (ref. 3, 4, 5).

Reference(s):

1. Technical Specification Table 1.2, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Trip System Instrumentation"
3. EOP 35 E-0, "Reactor Trip or Safety Injection"
4. EOP 35 F-0.1, "Subcriticality"
5. EOP FR-S.1, "Response to Nuclear Power Generation/ATWS"
6. NEI 99-01 SA5

Category: M – System Malfunction

Subcategory: 2 – RPS Failure

Initiating Condition: Inability to shut down the reactor causing a challenge to core cooling or RCS heat removal

EAL:

MS6.1 Site Area Emergency

An automatic or manual trip did **not** shut down the reactor as indicated by reactor power > 5%

AND

All actions taken to shut down the reactor are **not** successful as indicated by reactor power > 5%

AND EITHER:

- Core Cooling-RED Path conditions met
- Heat Sink-RED Path conditions met

Mode Applicability:

1 - Power Operation

Definition(s):

None

Basis:

This EAL addresses a failure of the RPS to initiate or complete an automatic or manual reactor trip that results in a reactor shutdown, all subsequent operator actions to manually shutdown the reactor are unsuccessful, and continued power generation is challenging the capability to adequately remove heat from the core and/or the RCS. This condition will lead to fuel damage if additional mitigation actions are unsuccessful and thus warrants the declaration of a Site Area Emergency.

This EAL is applicable to failure to trip events initiated in Mode 1 only, since the power level specified for a successful shutdown (5%) aligns with the Mode 1 transition power (5%) (ref. 1, 5).

Reactor shutdown achieved by use of other trip actions such as opening locally supply breakers, emergency boration, or manually driving control rods are also credited as a successful manual trip if reactor power is $\leq 5\%$ before indications of an extreme challenge to either core cooling or heat removal exist (ref. 1, 2, 3, 4).

In some instances, the emergency classification resulting from this IC/EAL may be higher than that resulting from an assessment of the plant responses and symptoms against the Category F ICs/EALs. This is appropriate in that the Category F ICs/EALs do not address the additional threat posed by a failure to shut down the reactor. The inclusion of this IC and EAL ensures

the timely declaration of a Site Area Emergency in response to prolonged failure to shut down the reactor.

A reactor shutdown ($\leq 5\%$) is determined in accordance with applicable EOP 35 F-0.1 SUBCRITICALITY Red Path criteria (ref. 4).

A severe challenge to adequate core cooling is based on meeting the Core Cooling Red Path criteria for Core Exit Thermocouple readings ($> 1,200$ °F) (ref. 5, 6).

The severe challenge to RCS heat removal is based on meeting the Heat Sink Red Path criteria of both of the following conditions existing (ref. 7, 8):

- Narrow Range level in all SGs $< 8\%$ (42% adverse CTMT)
- Total feedwater flow to SGs < 530 gpm

Escalation of the emergency classification level would be via IC RG1 or FG1.

Reference(s):

1. Technical Specification Table 1.2, "Operational Modes"
2. Technical Specification Table 3.3-1, "Reactor Trip System Instrumentation"
3. EOP 35 E-0, "Reactor Trip or Safety Injection"
4. EOP 35 F-0.1, "Subcriticality"
5. EOP 35 F-0.2, "CSFST Core Cooling"
6. EOP 35 FR-C.1, "Response to Inadequate Core Cooling"
7. EOP 35 F-0.3, "CSFST Heat Sink"
8. EOP 35 FR-H.1, "Response to Loss of Secondary Heat Sink"
9. NEI 99-01 SS5

Category: M – System Malfunction

Subcategory: 7 – Loss of Communications

Initiating Condition: Loss of **all** onsite or offsite communications capabilities

EAL:

MU7.1 Unusual Event

Loss of **all** Table M-5 onsite communication methods

OR

Loss of **all** Table M-5 State and local agency communication methods

OR

Loss of **all** Table M-5 NRC communication methods

Table M-5 Communication Methods			
System	Onsite	State/ Local	NRC
ENRS / ARCOS		X	
Station Radio System	X	X	
Plant Phone System	X	X	
Public Address System	X		
Gaitronics / Maintenance Jacks	X		
Federal Telephone System (ENS)			X
Commercial Telephone System		X	X
Satellite Phones		X	X
Dedicated Hotlines		X	

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

None

Basis:

This IC addresses a significant loss of on-site or offsite communications capabilities. While not a direct challenge to plant or personnel safety, this event warrants prompt notifications to State and local agencies and the NRC.

This IC should be assessed only when extraordinary means are being utilized to make communications possible (e.g., use of non-plant, privately owned equipment, relaying of on-site information via individuals or multiple radio transmission points, individuals being sent to offsite locations, etc.).

The first EAL condition addresses a total loss of the communications methods used in support of routine plant operations.

The second EAL condition addresses a total loss of the communications methods used to notify all State and local agencies of an emergency declaration. The State and local agencies referred to here are the State of Connecticut and local communities.

The third EAL addresses a total loss of the communications methods used to notify the NRC of an emergency declaration.

This hot condition EAL is equivalent to the cold condition EAL CU5.1.

Reference(s):

1. MPS Emergency Plan Section 7.9, "Communication Systems"
2. MP-26-EPI-FAP07, "Notifications and Communications"
3. NEI 99-01 SU6

Category: M – System Malfunction

Subcategory: 8 – Containment Failure

Initiating Condition: Failure to isolate containment or loss of containment pressure control

EAL:

MU8.1 Unusual Event

Any penetration is **not** closed within 15 min. of a VALID Phase A or B isolation signal

OR

CTMT pressure > 25 psia with < one full train of CTMT heat removal systems
(Note 11) operating per design for ≥ 15 min.

(Note 1)

Note 1: The DSEO/ADTS should declare the event promptly upon determining that the time limit has been exceeded, or will likely be exceeded.

Note 11: One full train of containment heat removal systems consist of one train of Recirculation Spray System (RSS) and one train of Quench Spray System (QSS).

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

VALID - An indication, report, or condition, is considered to be valid when it is verified by (1) an instrument channel check, or (2) indications on related or redundant indicators, or (3) by direct observation by plant personnel, such that doubt related to the indicator's operability, the condition's existence, or the report's accuracy is removed. Implicit in this definition is the need for timely assessment.

Basis:

This EAL addresses a failure of one or more containment penetrations to automatically isolate (close) when required by an actuation signal. It also addresses an event that results in high containment pressure with a concurrent failure of containment pressure control systems. Absent challenges to another fission product barrier, either condition represents potential degradation of the level of safety of the plant.

For the first condition, the containment isolation signal (Phase A or B) must be generated as the result on an off-normal/accident condition (e.g., a safety injection or high containment pressure); a failure resulting from testing or maintenance does not warrant classification. The determination of containment and penetration status – isolated or not isolated – should be made in accordance with the appropriate criteria contained in the plant AOPs and EOPs. The 15-minute criterion is included to allow operators time to manually isolate the required penetrations, if possible (ref. 4).

The second condition addresses a condition where containment pressure is greater than the setpoint (25 psia) at which containment energy (heat) removal systems are designed to automatically actuate, and less than one full train of equipment is capable of operating per design. One full train of containment heat removal systems consist of one train of Recirculation

Spray System (RSS) and one train of Quench Spray System (QSS) (ref. 1, 2). The combination of required equipment can be obtained from using equipment on either emergency busses in order to meet the "one full train" requirement.

The Quench Spray System (QSS) is activated immediately upon receipt of a CDA signal. QSS becomes effective within 70 seconds of initiation.

The containment Recirculation Spray System (RSS) is activated on receipt of the RWST Low-Low level signal coincident with a CDA signal

The 15-minute criterion is included to allow operators time to manually start equipment that may not have automatically started, if possible. The inability to start the required equipment indicates that containment heat removal/depressurization systems (e.g., containment sprays) are either lost or performing in a degraded manner.

This event would escalate to a Site Area Emergency in accordance with IC FS1 if there were a concurrent loss or potential loss of either the Fuel Clad or RCS fission product barriers.

Reference(s):

1. MPS3 FSAR Section 6.2.2, "Containment Heat Removal System"
2. MPS3 Technical Specifications 3.6.2.1, "Containment Quench Spray and Recirculation Spray System"
3. EOP 35 FR-Z.1, "Response to High Containment Pressure"
4. MPS3 FSAR Section 6.2.4, "Containment Isolation System"
5. NEI 99-01 SU7

Category: M – System Malfunction
Subcategory: 9 – Hazardous Event Affecting Safety Systems
Initiating Condition: Hazardous event affecting SAFETY SYSTEMS needed for the current operating mode

EAL:

MA9.1 Alert

The occurrence of **any** Table M-6 hazardous event

AND

Event damage has caused indications of degraded performance on one train of a SAFETY SYSTEM needed for the current operating mode

AND EITHER:

- Event damage has caused indications of degraded performance to the second train of the SAFETY SYSTEM needed for the current operating mode
- Event damage has resulted in **VISIBLE DAMAGE** to the second train of the SAFETY SYSTEM needed for the current operating mode

(Notes 9, 10)

Note 9: If the affected SAFETY SYSTEM train was already inoperable or out of service before the hazardous event occurred, then emergency classification is **not** warranted.

Note 10: If the hazardous event **only** resulted in **VISIBLE DAMAGE**, with **no** indications of degraded performance to at least one train of a SAFETY SYSTEM, then this emergency classification is **not** warranted.

Table M-6 Hazardous Events
<ul style="list-style-type: none">• Seismic event (earthquake)• Internal or external FLOODING event• High winds or tornado strike• FIRE• EXPLOSION• Other events with similar hazard characteristics as determined by the DSEO/ADTS

Mode Applicability:

1 – Power Operation, 2 – Startup, 3 – Hot Standby, 4 – Hot Shutdown

Definition(s):

EXPLOSION - A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high energy lines or components) or an electrical component failure (caused by short circuits, grounding,

arcing, etc.) should **not** automatically be considered an explosion. Such events require a post-event inspection to determine if the attributes of an explosion are present.

FIRE - Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do **not** constitute fires. Observation of flame is preferred but is **not** required if large quantities of smoke and heat are observed.

FLOODING - A condition where water is entering a room or area faster than installed equipment is capable of removal, resulting in a rise of water level within the room or area.

SAFETY SYSTEM - A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related (as defined in 10CFR50.2):

Those structures, systems and components that are relied upon to remain functional during and following design basis events to assure:

- (1) The integrity of the reactor coolant pressure boundary;
- (2) The capability to shut down the reactor and maintain it in a safe shutdown condition;
- (3) The capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures.

VISIBLE DAMAGE - Damage to a SAFETY SYSTEM train that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected SAFETY SYSTEM train.

Basis:

This IC addresses a hazardous event that causes damage to SAFETY SYSTEMS needed for the current operating mode. In order to provide the appropriate context for consideration of an ALERT classification, the hazardous event must have caused indications of degraded SAFETY SYSTEM performance in one train, and there must be either indications of performance issues with the second SAFETY SYSTEM train or VISIBLE DAMAGE to the second train such that the potential exists for this second SAFETY SYSTEM train to have performance issues. In other words, in order for this EAL to be classified, the hazardous event must occur, at least one SAFETY SYSTEM train must have indications of degraded performance, and the second SAFETY SYSTEM train must have indications of degraded performance or VISIBLE DAMAGE such that the potential exists for performance issues. Note that this second SAFETY SYSTEM train is from the same SAFETY SYSTEM that has indications of degraded performance; commercial nuclear power plants are designed to be able to support single system issues without compromising public health and safety from radiological events.

Indications of degraded performance addresses damage to a SAFETY SYSTEM train that is in service/operation since indications for it will be readily available. The indications of degraded performance should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

VISIBLE DAMAGE addresses damage to a SAFETY SYSTEM train that is not in service/operation and that potentially could cause performance issues. Operators will make this determination based on the totality of available event and damage report information. This is intended to be a brief assessment not requiring lengthy analysis or quantification of the

damage. This VISIBLE DAMAGE should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

An event affecting equipment common to two or more trains of a safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the common equipment) should be classified as an Alert under this EAL, as appropriate to the plant mode. By affecting the functionality of multiple trains of a safety system, the loss of the common equipment effectively meets the two-train impact criteria that underlie the EALs and bases.

An event affecting a single-train safety system (i.e., there are indications of degraded performance and/or VISIBLE DAMAGE affecting the one train) would not be classified under this EAL because the two-train impact criteria that underlie the EALs and bases would not be met. If an event affects a single-train safety system, then the emergency classification should be made based on plant parameters/symptoms meeting the EALs for another IC. Depending upon the circumstances, classification may also occur based on Shift Manager/DSEO/ADTS judgement.

An event that affects two trains of a safety system (e.g., one train has indications of degraded performance and the other VISIBLE DAMAGE) that also has one or more additional trains should be classified as an Alert under this EAL, as appropriate to the plant mode. This approach maintains consistency with the two-train impact criteria that underlie the EALs and bases, and is warranted because the event was severe enough to affect the functionality of two trains of a safety system despite plant design criteria associated with system and system train separation and protection. Such an event may have caused other plant impacts that are not immediately apparent.

Escalation of the emergency classification level would be via IC FS1 or RS1.

This hot condition EAL is equivalent of the cold condition EAL CA6.1.

Reference(s):

1. AOP 3569, "Severe Weather Conditions"
2. AOP 3570, "Earthquake"
3. EP FAQ 2016-002
4. NEI 99-01 SA9

Attachment 2 Safe Operation & Shutdown Rooms/Areas Tables R-2 & H-2 Bases

Background

NEI 99-01 Revision 6 ICs AA3 and HA5 prescribe declaration of an Alert based on IMPEDED access to rooms or areas (due to either area radiation levels or hazardous gas concentrations) where equipment necessary for normal plant operations, cooldown or shutdown is located. These areas are intended to be plant operating mode dependent. Specifically the Developers Notes for AA3 and HA5 states:

The "site-specific list of plant rooms or areas with entry-related mode applicability identified" should specify those rooms or areas that contain equipment which require a manual/local action as specified in operating procedures used for normal plant operation, cooldown and shutdown. Do not include rooms or areas in which actions of a contingent or emergency nature would be performed (e.g., an action to address an off-normal or emergency condition such as emergency repairs, corrective measures or emergency operations). In addition, the list should specify the plant mode(s) during which entry would be required for each room or area.

The list should not include rooms or areas for which entry is required solely to perform actions of an administrative or record keeping nature (e.g., normal rounds or routine inspections).

Further, as specified in IC HA5:

The list need not include the Control Room if adequate engineered safety/design features are in place to preclude a Control Room evacuation due to the release of a hazardous gas. Such features may include, but are not limited to, capability to draw air from multiple air intakes at different and separate locations, inner and outer atmospheric boundaries, or the capability to acquire and maintain positive pressure within the Control Room envelope.

**Millstone Power Station Unit 3
Emergency Action Level Technical Bases Document**

Serial No. 18-364
Docket Nos. 50-423; 72-47
Enclosure 3; Attachment 3C

Attachment 2 Safe Operation & Shutdown Rooms/Areas Tables R-2 & H-2 Bases

MPS3 Table R-2 and H-2 Bases

A review of station operating procedures identified the following mode dependent in-plant actions and associated areas that are required for normal plant operation, cooldown or shutdown:

In-Plant Actions (MPS3)	Safe Shutdown Area	Modes
Chemistry sample RCS for boron concentration	AB EI 43'	1
Chemistry sample RCS for boron concentration	AB EI 43'	2
Chemistry sample RCS for boron concentration	AB EI 43'	3
Throttle seal injection flows	AB 3'8"	3
Isolate SI Accumulators	AB 24' East MCC/RCA 24' West MCC/RCA 24'	3
Mode 4 gas accumulation testing for all trains for TS 3.4.1.3	Containment ESF Bldg A RHR Cubicle ESF Bldg B RHR Cubicle	4
Establish mass addition protection	East Switchgear West Switchgear	4
Align RHR A for Cooldown	ESF 24' ESF 36' AB 24' MCC/RCA 24'	4
Warmup RHR A Train	ESF 36'	4
Align RHR B for Cooldown	ESF 24' ESF 36' AB 24' MCC/RCA 24'	4
Warmup RHR B Train	ESF 36'	4

Control Room ventilation systems have adequate engineered safety/design features in place to preclude a Control Room evacuation due to the external release of a hazardous gas (FSAR Section 9.4.1). Therefore, the Control Room is not included in this assessment or in Table H-2.

Ref: OP 3204, "At Power Operation"
OP 3206, "Plant Shutdown"
OP 3208, "Reactor Shutdown"

Attachment 2 Safe Operation & Shutdown Rooms/Areas Tables R-2 & H-2 Bases

Table R-2 & H-2 Results

Table R-2/H-2 Safe Operation & Shutdown Rooms/Areas	
Room/Area	Mode
Aux. Building EI 43'	1, 2, 3
Aux. Building EI 24' East MCC/RCA EI 24' West MCC/RCA EI 24'	3, 4
Aux. Building EI 3' 8"	3
Containment ESF Building A RHR Cubicle ESF Building B RHR Cubicle ESF Building EI 36' ESF Building EI 24' East Switchgear West Switchgear	4