



**Consumers
Power
Company**

General Offices: 212 West Michigan Avenue, Jackson, Michigan 49201 • Area Code 517 788-0550

March 13, 1978

Mr James G Keppler
Office of Inspection and Enforcement
Region III
US Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - UPDATED
LER-78-004 AND LER-78-006

Attached are two updated licensee event reports for the Palisades Plant.

David P Hoffman
Assistant Nuclear Licensing Administrator

CC: Director, Office of Nuclear Reactor Regulation
Director, Office of Inspection and Enforcement

MAR 15 1978

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

CONTROL BLOCK:

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 ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

M I P A L L		O O O O O O O O O O		L I L I L I		C A T	
LICENSEE CODE		LICENSE NUMBER		LICENSE TYPE		CAT 58	
CON'T							
REPORT SOURCE		DOCKET NUMBER		EVENT DATE		REPORT DATE	
L 0 5 0 0 0 2 5 5 7 0 1 3 1 7 8		60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80					
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES							
During calibration of steam generator pressure instrumentation which controls initiation of main steam line isolation, the setpoint of PIC 0752B for initiating main steam line isolation was 493 psia. The setpoint of PIC 0751 for removal of the main steam line isolation bypass was 552.5 psia. These setpoints are outside the limits of TS 3.16. Redundancy was provided by three other instruments in each channel, and each channel was operable.							
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE							
I B		E		E		I N S T R U C Z	
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	
7 8		7 8		0 0 4		0 3	
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB PRIME COMP. SUPPLIER COMPONENT MANUFACTURER							
E Z		Z		Z		0 0 0 0 N Y N S 1 1 8 5	
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS							
The occurrence is attributed to setpoint drift. The magnitude of the drift has been evaluated and found to be reasonable. The instruments were recalibrated, and no further action is contemplated.							
FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION LOCATION OF RELEASE							
H		0 0 0 0 N/A		R Calibration results			
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY							
Z		Z					
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION							
0 0 0 Z							
PERSONNEL INJURIES NUMBER DESCRIPTION							
0 0 0							
LOSS OR DAMAGE TO FACILITY TYPE DESCRIPTION							
Z							
PUBLCITY ISSUED DESCRIPTION							
N							

Palisades

U. S. NUCLEAR REGULATORY COMMISSION

CONTROL BLOCK: | | | | | | | ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)																																																																																							
During movement of the incore shipping cask into and out of the contain-																																																																																							
ment building on 12 and 14 February, respectively, the edge of the cask																																																																																							
was permitted to pass to the east of the cast boundary of the tilt pits.																																																																																							
This is contrary to the requirements of TS 3.21.lb. The cask did not																																																																																							
travel over the spent fuel pool. At the time of occurrence, the cask																																																																																							
was held by a dual lifting sling and was being lifted by the fuel handling																																																																																							
building crane 100-ton hook. Event had no adverse effect on public /																																																																																							
safety.																																																																																							
<table border="1" style="width:100%; border-collapse: collapse; font-size: small;"> <tr> <td colspan="2">SYSTEM CODE Z Z (11)</td> <td colspan="2">CAUSE CODE A (12)</td> <td colspan="2">CAUSE SUBCODE C (13)</td> <td colspan="2">COMPONENT CODE Z Z Z Z Z Z Z (14)</td> <td colspan="2">COMP. SUBCODE Z (15)</td> <td colspan="2">VALVE SUBCODE Z (16)</td> </tr> <tr> <td colspan="2">LER/RO REPORT NUMBER 7 8 (17)</td> <td colspan="2">EVENT YEAR 7 8 (21) 22</td> <td colspan="2">SEQUENTIAL REPORT NO. 0 0 6 (24) 26</td> <td colspan="2">OCCURRENCE CODE 0 1 (28) 29</td> <td colspan="2">REPORT TYPE X (30) 31</td> <td colspan="2">REVISION NO. 1 (32) 32</td> </tr> <tr> <td>ACTION TAKEN Z (33) 34</td> <td>FUTURE ACTION G (35) 36</td> <td>EFFECT ON PLANT Z (37) 38</td> <td>SHUTDOWN METHOD Z (39) 40</td> <td>HOURS 0 0 0 0 (42) 44</td> <td>ATTACHMENT SUBMITTED Y (46) 47</td> <td>NPRD-4 FORM SUB. N (48) 49</td> <td>PRIME COMP. SUPPLIER Z (52) 53</td> <td colspan="4">COMPONENT MANUFACTURER Z 9 9 9 (56) 58</td> </tr> </table>																												SYSTEM CODE Z Z (11)		CAUSE CODE A (12)		CAUSE SUBCODE C (13)		COMPONENT CODE Z Z Z Z Z Z Z (14)		COMP. SUBCODE Z (15)		VALVE SUBCODE Z (16)		LER/RO REPORT NUMBER 7 8 (17)		EVENT YEAR 7 8 (21) 22		SEQUENTIAL REPORT NO. 0 0 6 (24) 26		OCCURRENCE CODE 0 1 (28) 29		REPORT TYPE X (30) 31		REVISION NO. 1 (32) 32		ACTION TAKEN Z (33) 34	FUTURE ACTION G (35) 36	EFFECT ON PLANT Z (37) 38	SHUTDOWN METHOD Z (39) 40	HOURS 0 0 0 0 (42) 44	ATTACHMENT SUBMITTED Y (46) 47	NPRD-4 FORM SUB. N (48) 49	PRIME COMP. SUPPLIER Z (52) 53	COMPONENT MANUFACTURER Z 9 9 9 (56) 58																											
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CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)																																																																																							
Personnel in charge of the movement were not adequately instructed in																																																																																							
order to make them aware of the requirements of TS 3.21. A procedure																																																																																							
which encompasses the requirements of TS 3.21 was issued prior to																																																																																							
subsequent movements of the incore shipping cask into the containment																																																																																							
building.																																																																																							
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Consumers Power Company, Palisades Nuclear Plant
Attachment to LER 78-006/OLX-1, Docket 50-255

On February 12 and 14, 1978 the incore shipping cask was moved through the fuel handling building en route to and from the containment building. These movements are permitted by Technical Specification 3.21 which was issued as part of Amendment 35 to the Palisades Provisional Operating License. During both of these moves, the edge of the cask closest to the spent fuel pool was to the east of east boundary of the tilt pits, contrary to the requirements of Technical Specification 3.21.1.b.

The occurrence was caused by not ensuring that the personnel directly supervising the move were fully aware of the requirements of Specification 3.21. A procedure was written and issued prior to the second shipment of incore detectors. This procedure listed the Technical Specification requirements and provided a drawing which showed the required path for the incore shipping cask movement.