

## Appendix 3B. Figures

Figure 3-1. Wind Pressure Distribution On The Reactor Building

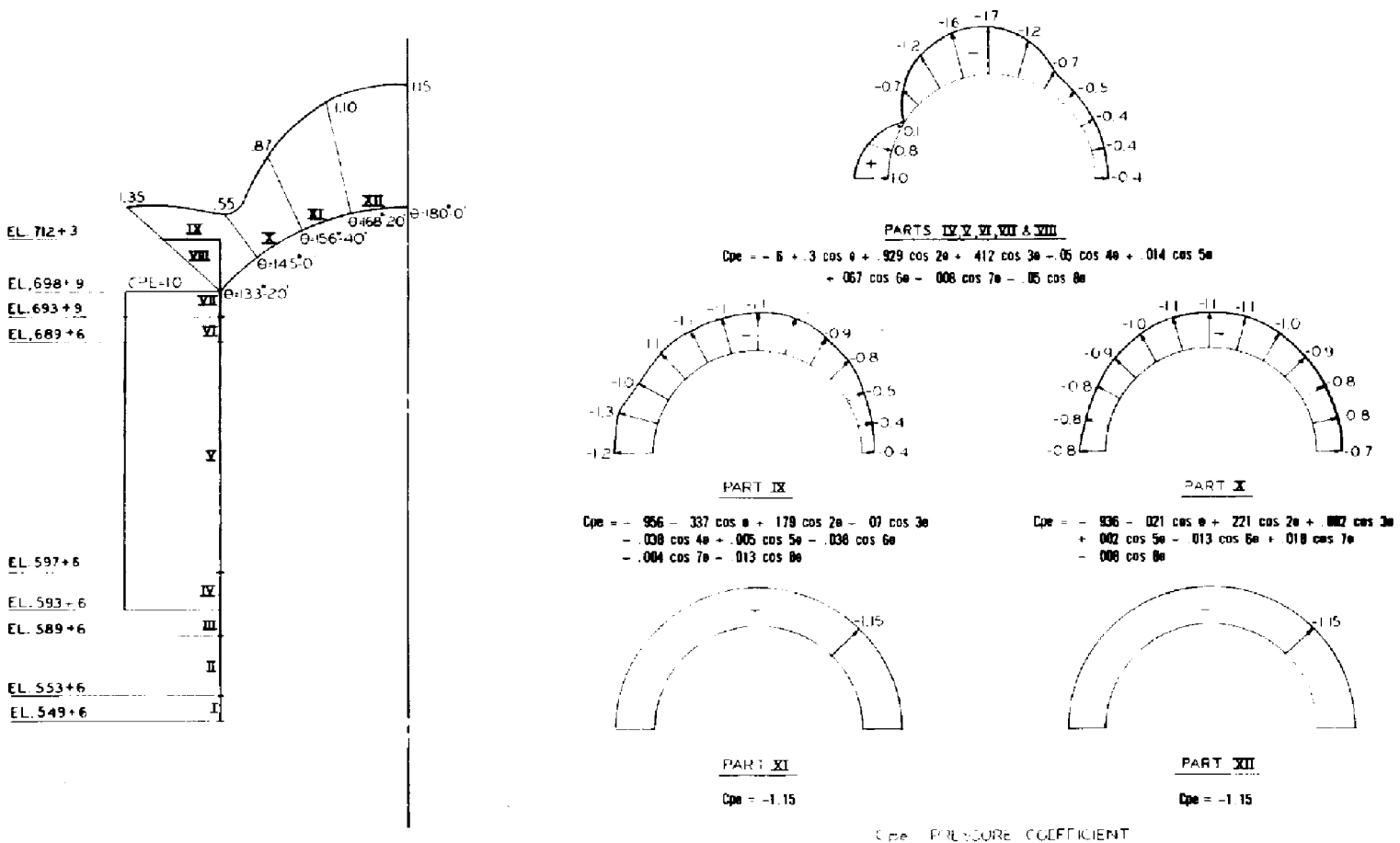


Figure 3-2. Wind Pressure Distribution For Rectangular Structures

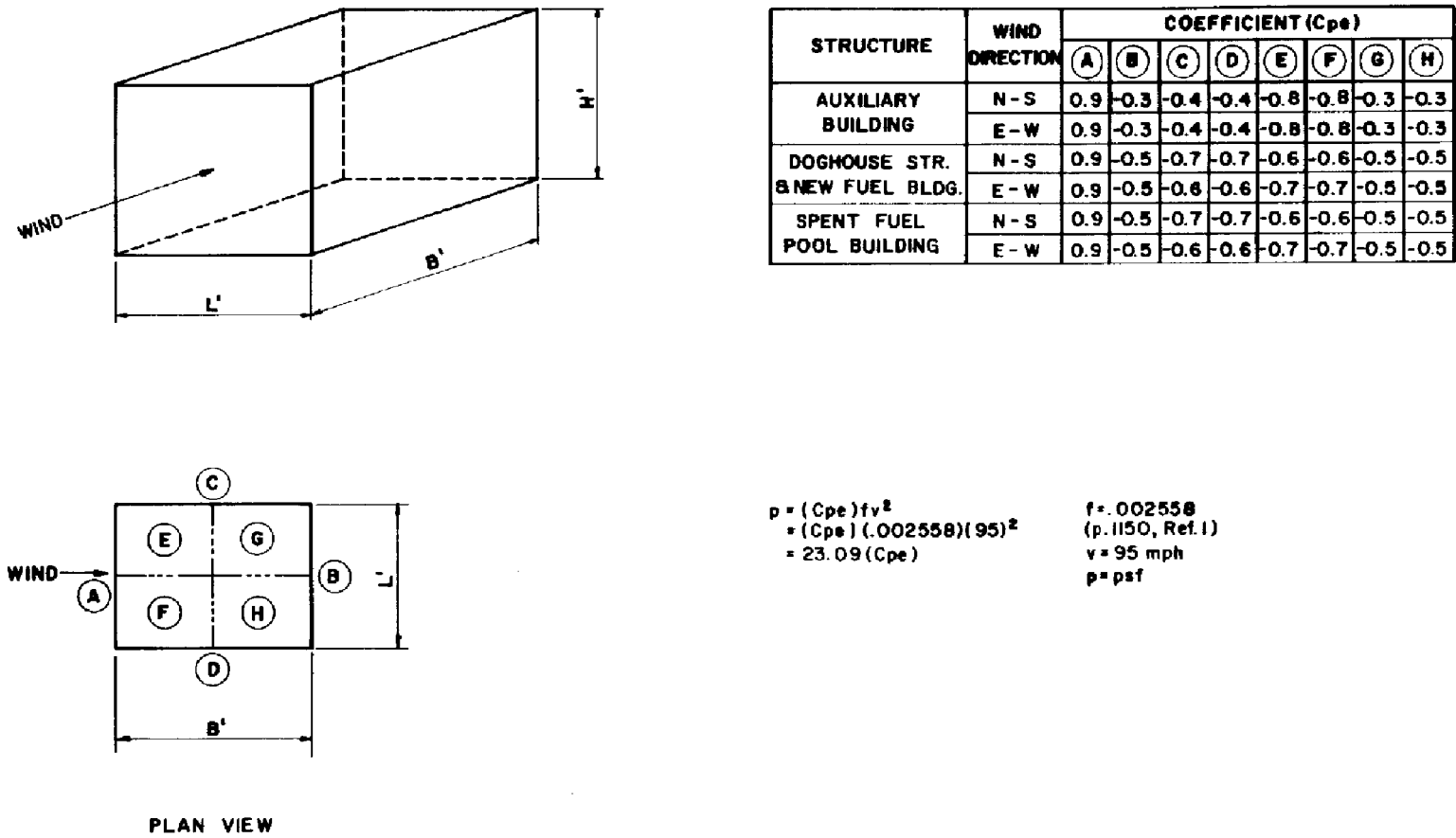


Figure 3-3. Turbine Building Tornado Velocity Distribution

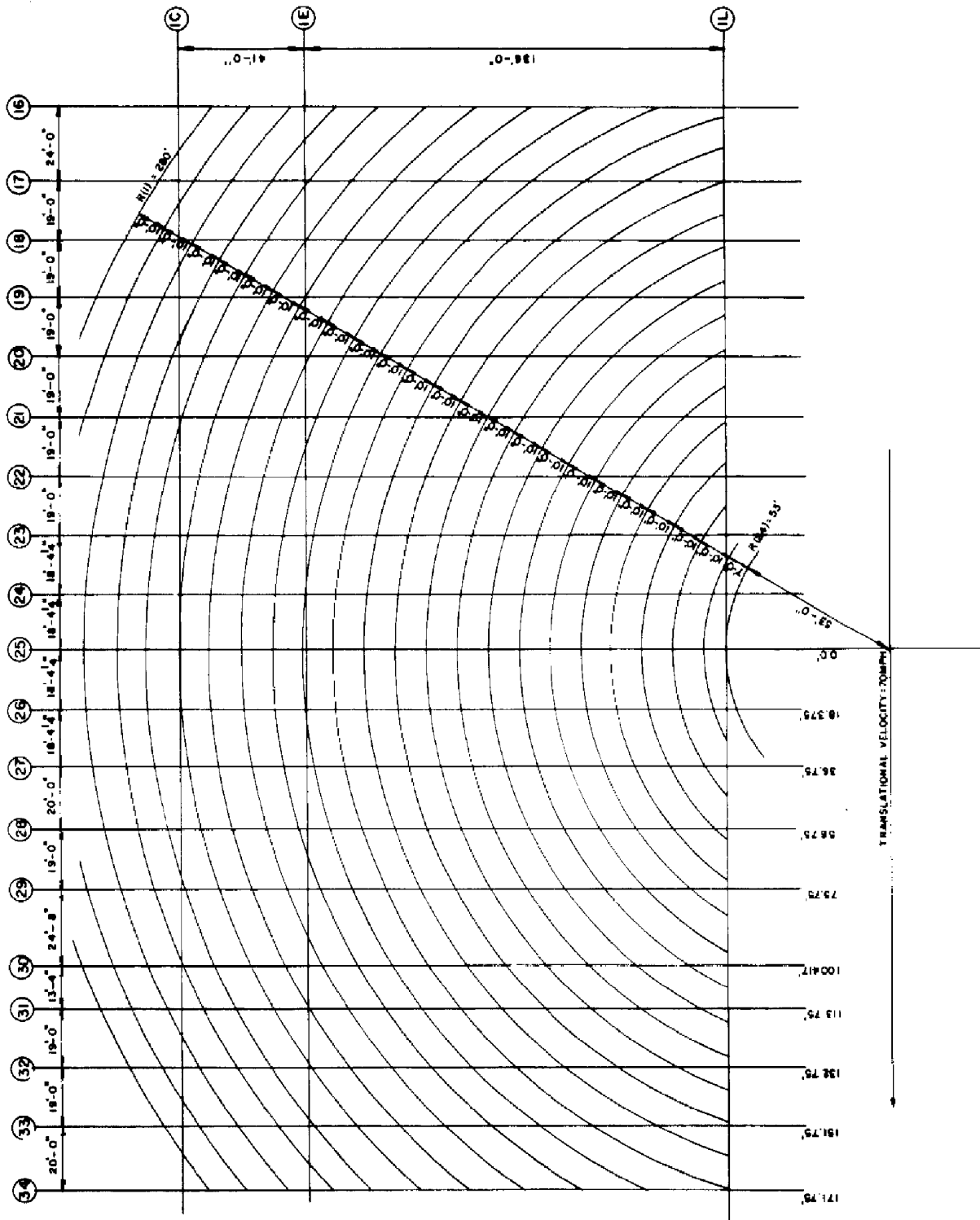


Figure 3-4. Tornado Wind Loads - Design Velocity Vs. Radius From Center Of Tornado

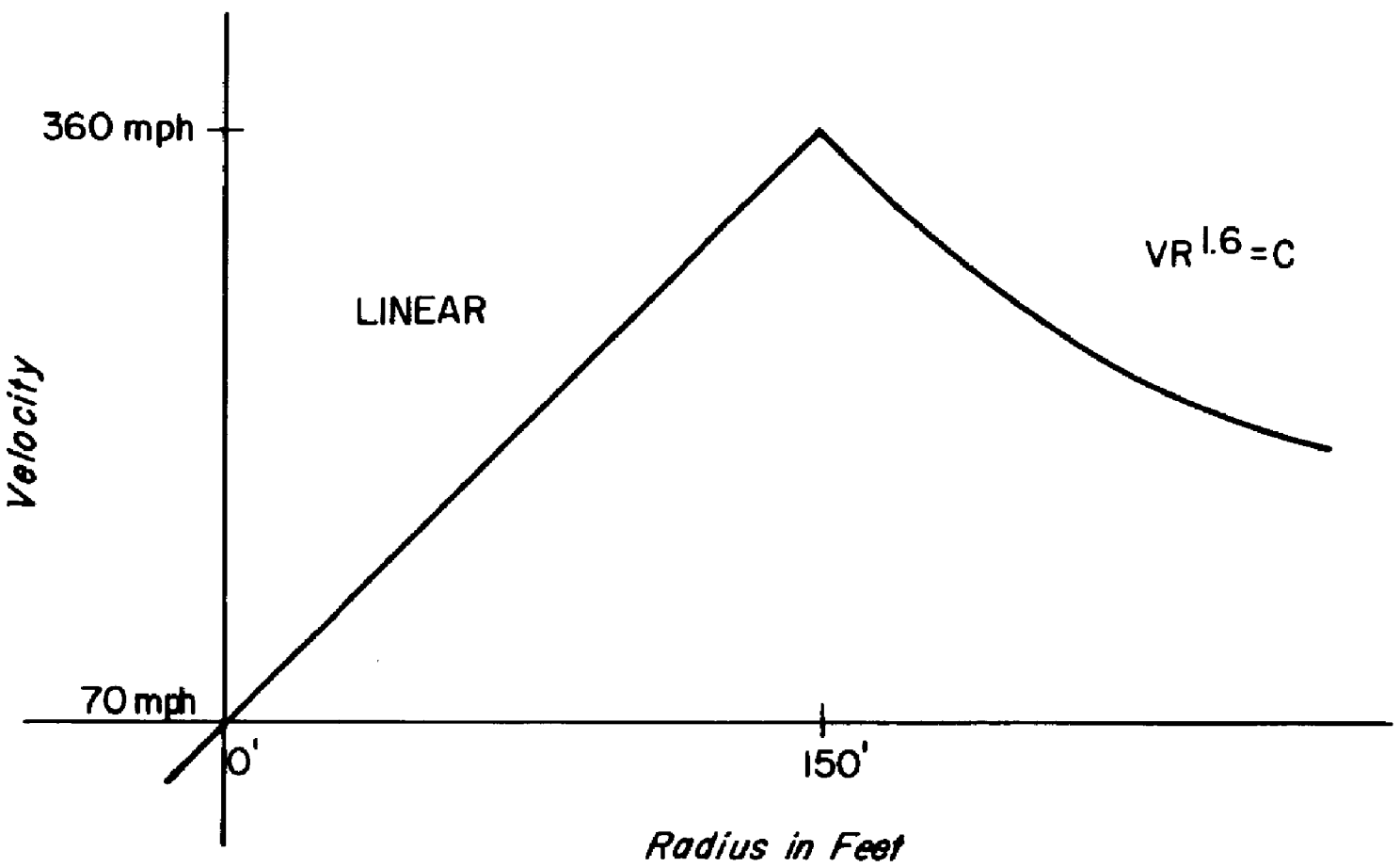


Figure 3-5. Turbine Missile Ejection Zones

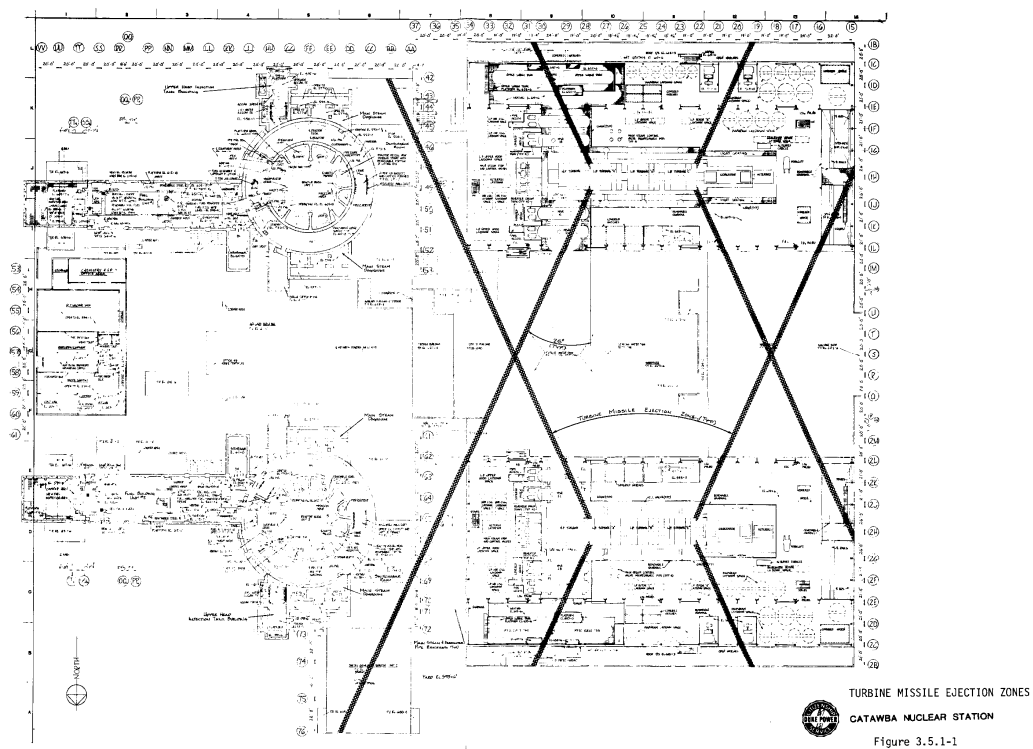
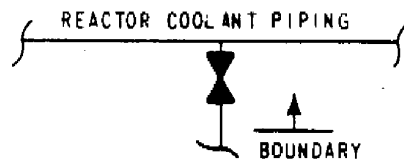


Figure 3-6. Loss Of Reactor Coolant Accident Boundary Limits

CASE I

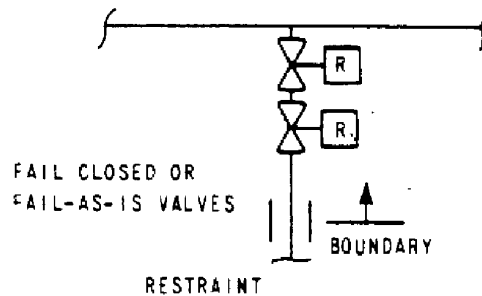
OUTGOING LINES WITH NORMALLY CLOSED VALVE



NOTE: PRESSURIZER SAFETY VALVES ARE INCLUDED UNDER THIS CASE.

CASE II

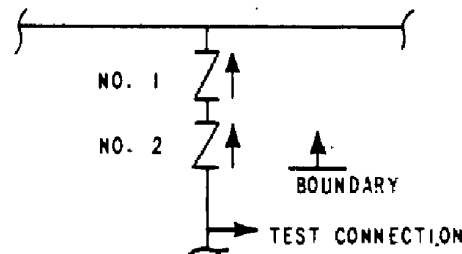
OUTGOING LINES WITH NORMALLY OPEN VALVES



NOTE: THE REACTOR COOLANT PUMP NO. 1 SEAL IS ASSUMED TO BE EQUIVALENT TO FIRST VALVE

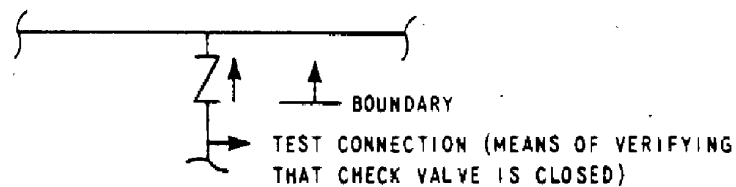
CASE III

INCOMING LINES NORMALLY WITH FLOW



CASE IV

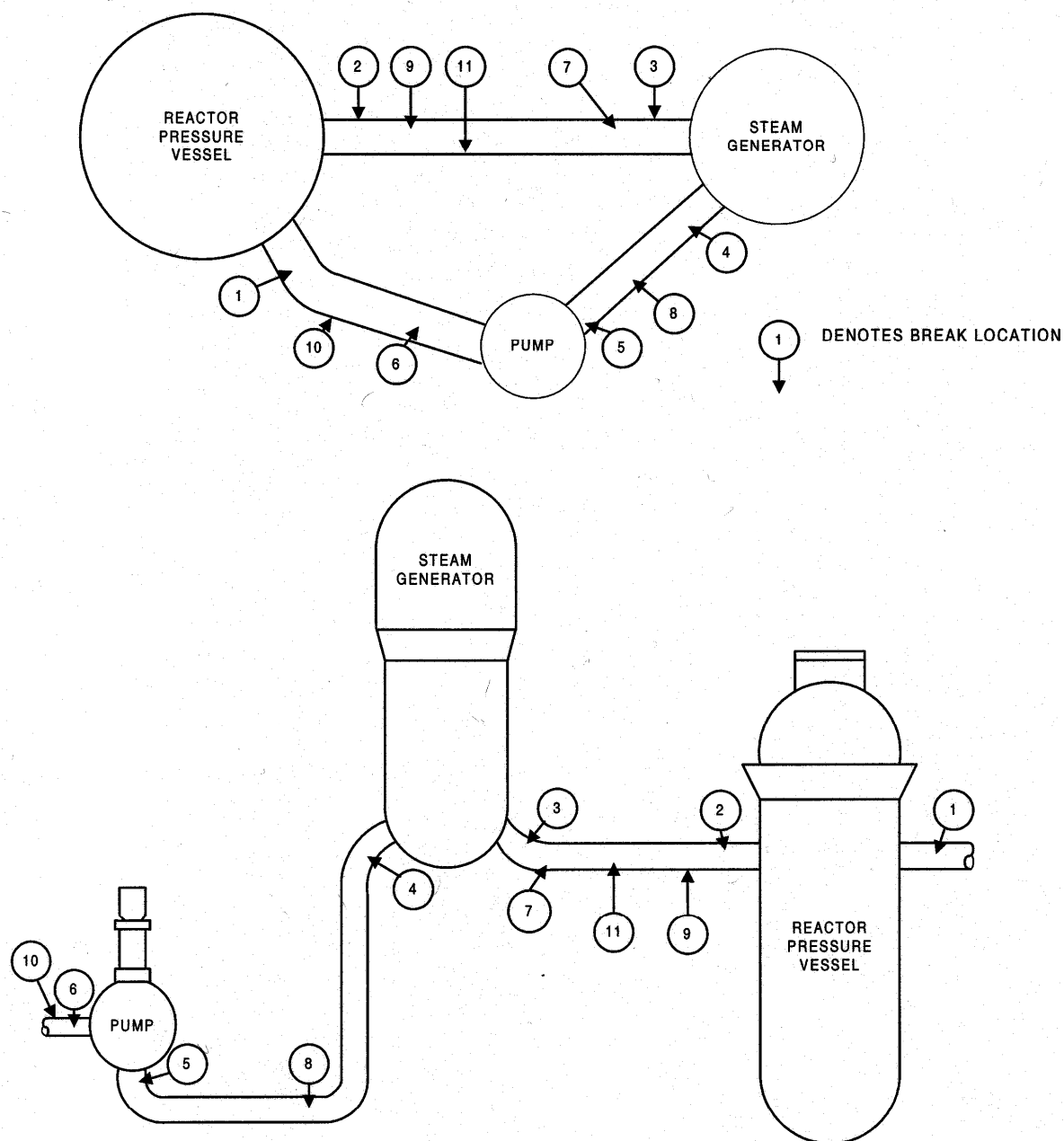
INCOMING LINES NORMALLY WITHOUT FLOW



CASE V

ALL INSTRUMENTATION TUBING AND INSTRUMENTS CONNECTED DIRECTLY TO THE REACTOR COOLANT SYSTEM IS CONSIDERED AS A BOUNDARY. HOWEVER, A BREAK WITHIN THIS BOUNDARY RESULTS IN A RELATIVELY SMALL FLOW WHICH CAN NORMALLY BE MADE UP WITH THE CHARGING SYSTEM.

Figure 3-7. Location Of Postulated Breaks In Main Reactor Coolant Loop





**Figure 3-8. Deleted Per 2001 Update**

(22 OCT 2001)

Figure 3-9. Reactor Coolant Loop Model

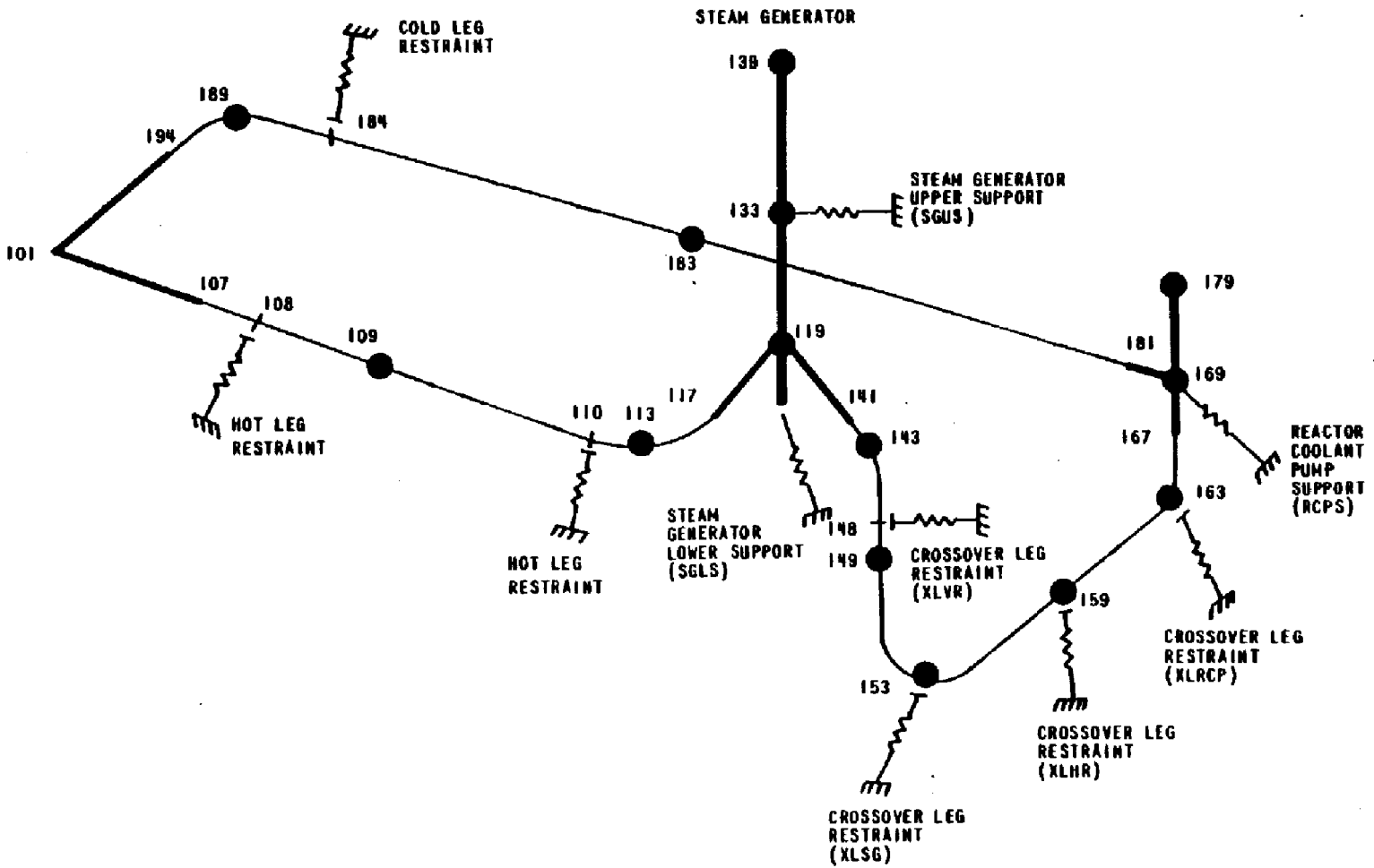


Figure 3-10. Time-History Dynamic Solution For Loka Loading

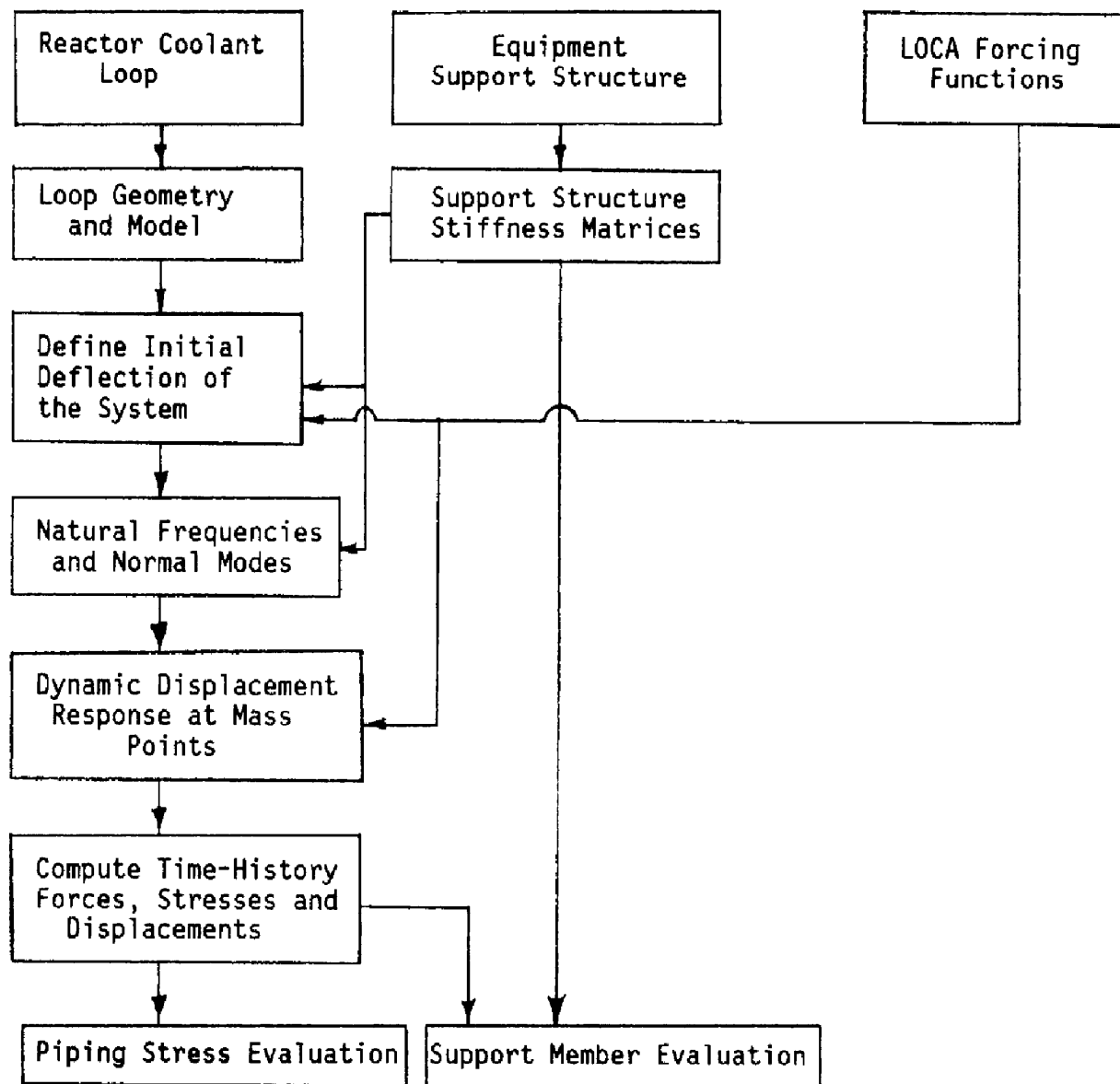
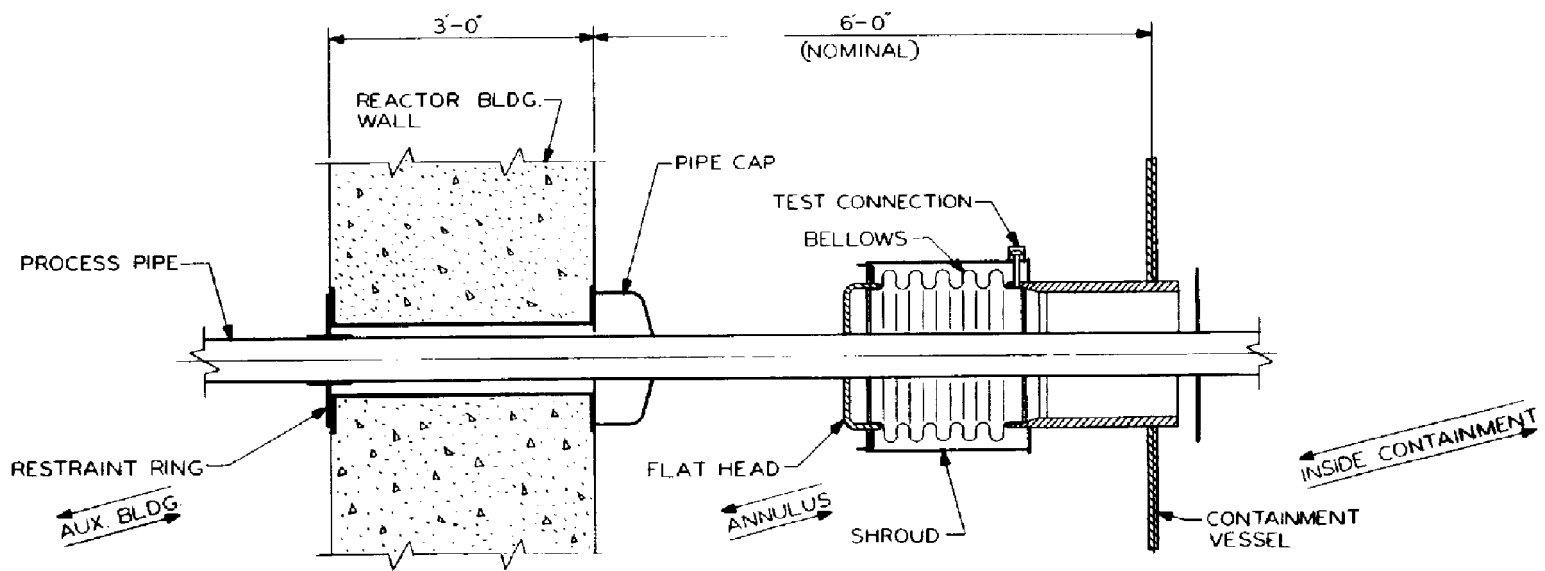
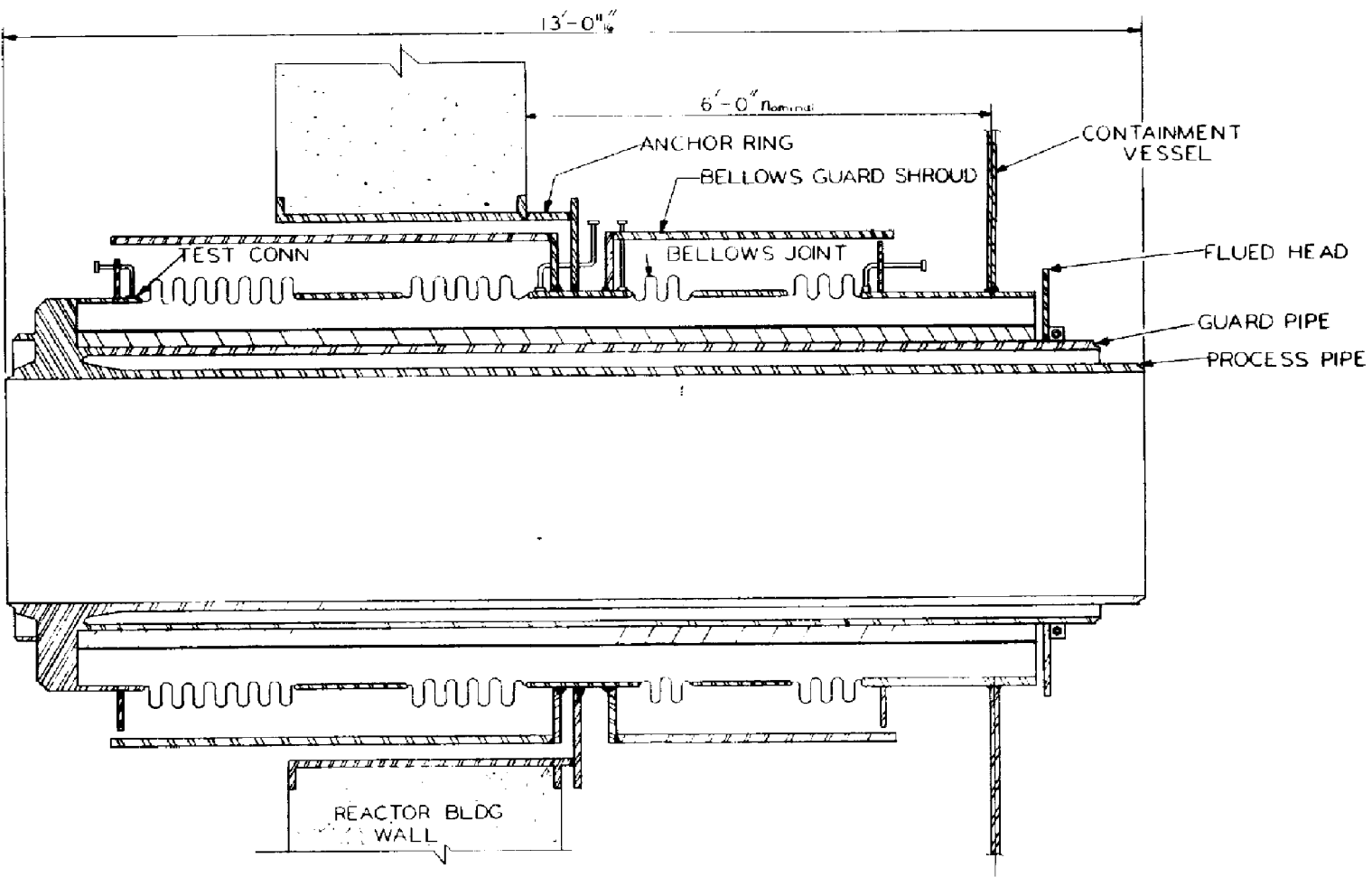
Time-History Dynamic Solution for LOCA Loading

Figure 3-11. Typical Cold Penetration



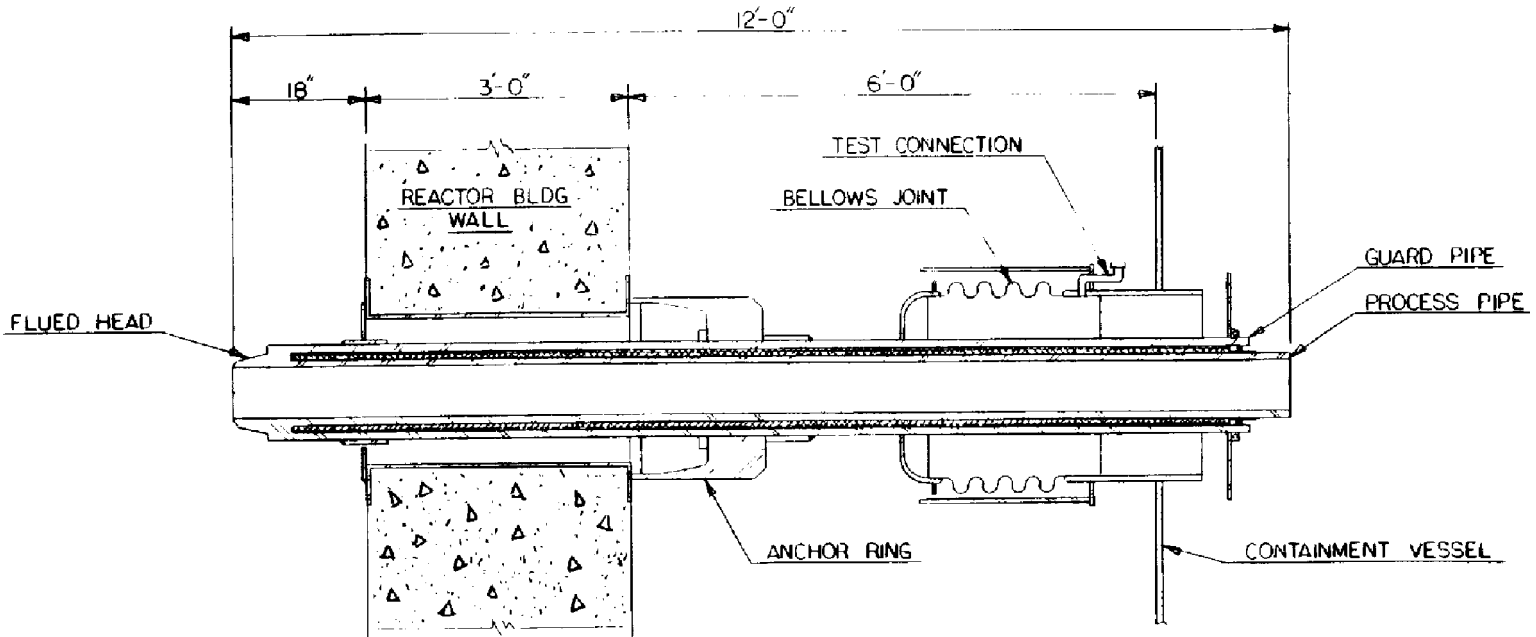
TYPICAL COLD PENETRATION

Figure 3-12. Typical Main Steam And Feedwater Penetration



TYPICAL MAIN STEAM AND  
FEEDWATER PENETRATION

Figure 3-13. Typical Hot Penetration



TYPICAL HOT PENETRATION

**Figure 3-14. Deleted Per 1990 Update**

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Figure 3-213. Design Response Spectrum, .15g

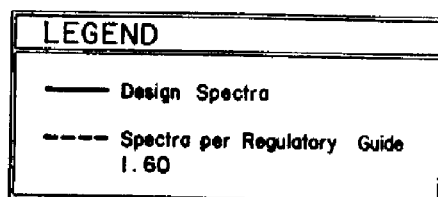
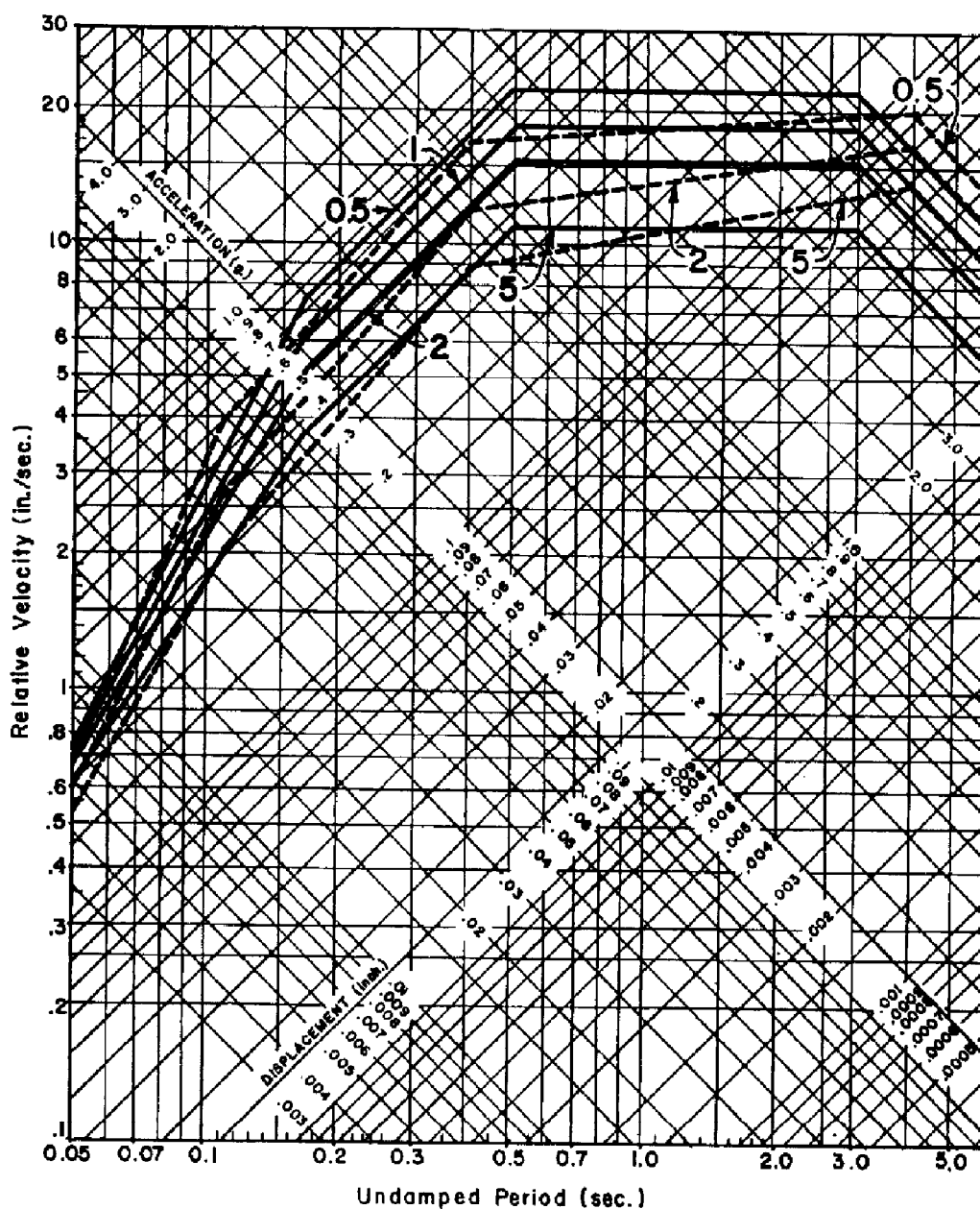


Figure 3-214. Response Acceleration Spectra

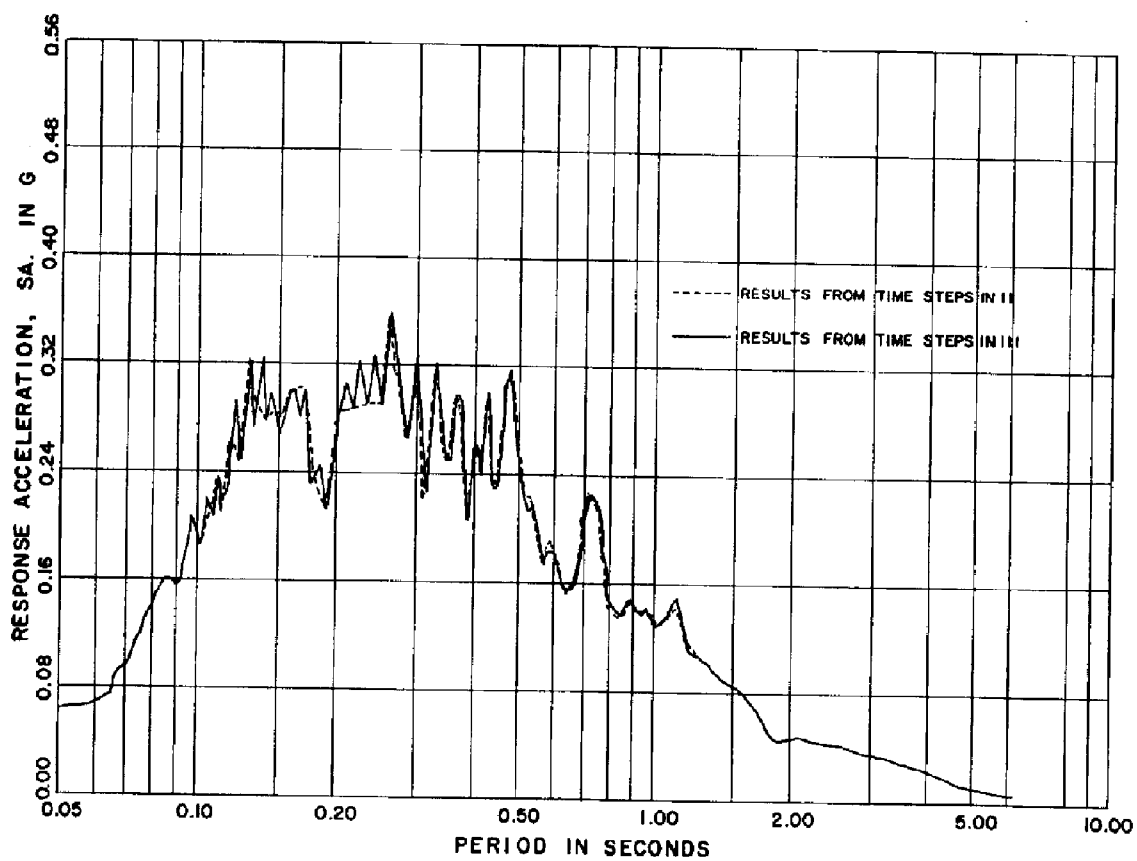
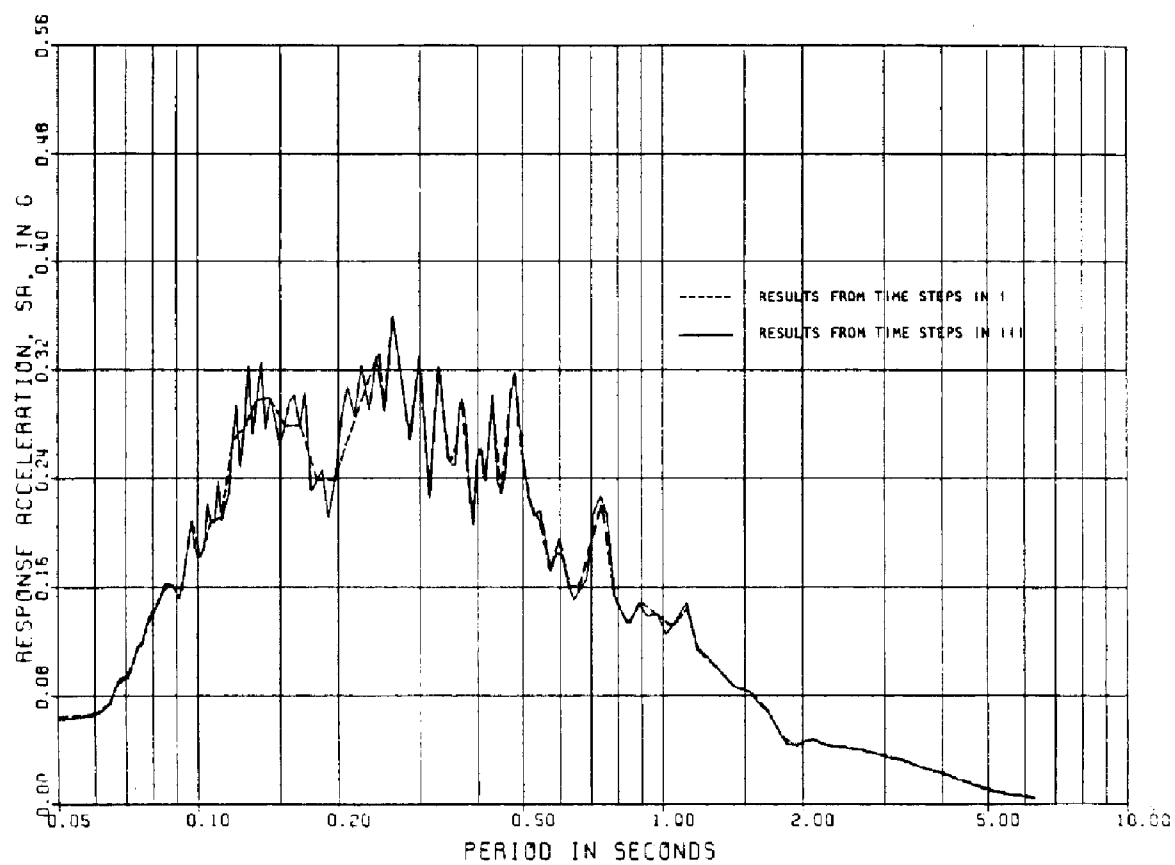
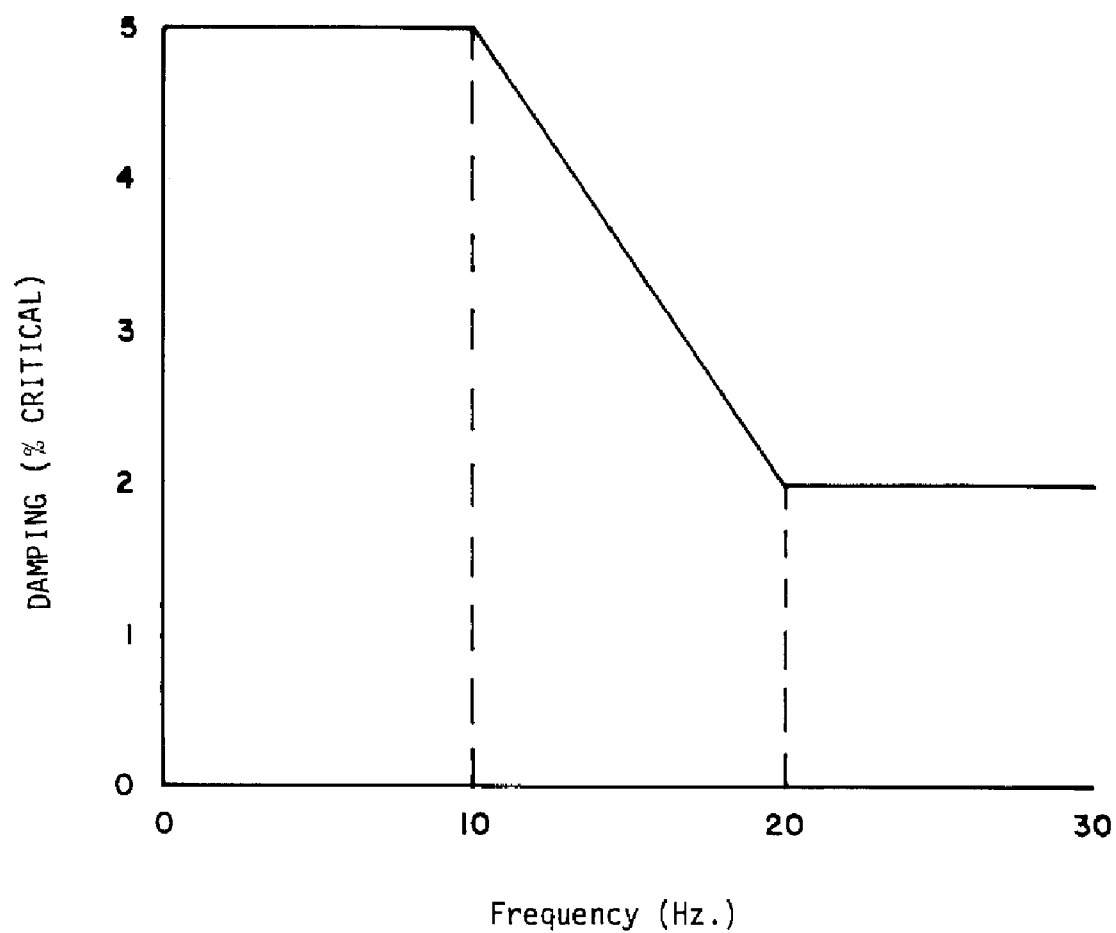


Figure 3-215. Response Acceleration Spectra



**Figure 3-216. Damping Valve For Seismic Analysis Of Piping (Applicable To Both OBE and SSE, Independent Of Pipe Diameter)**




**Figure 3-217. Containment Interior Structure Lumped Mass Model**



Security-Related, Withhold under 10 CFR 2.390

**Figure 3-218. Auxiliary Building Elevation**

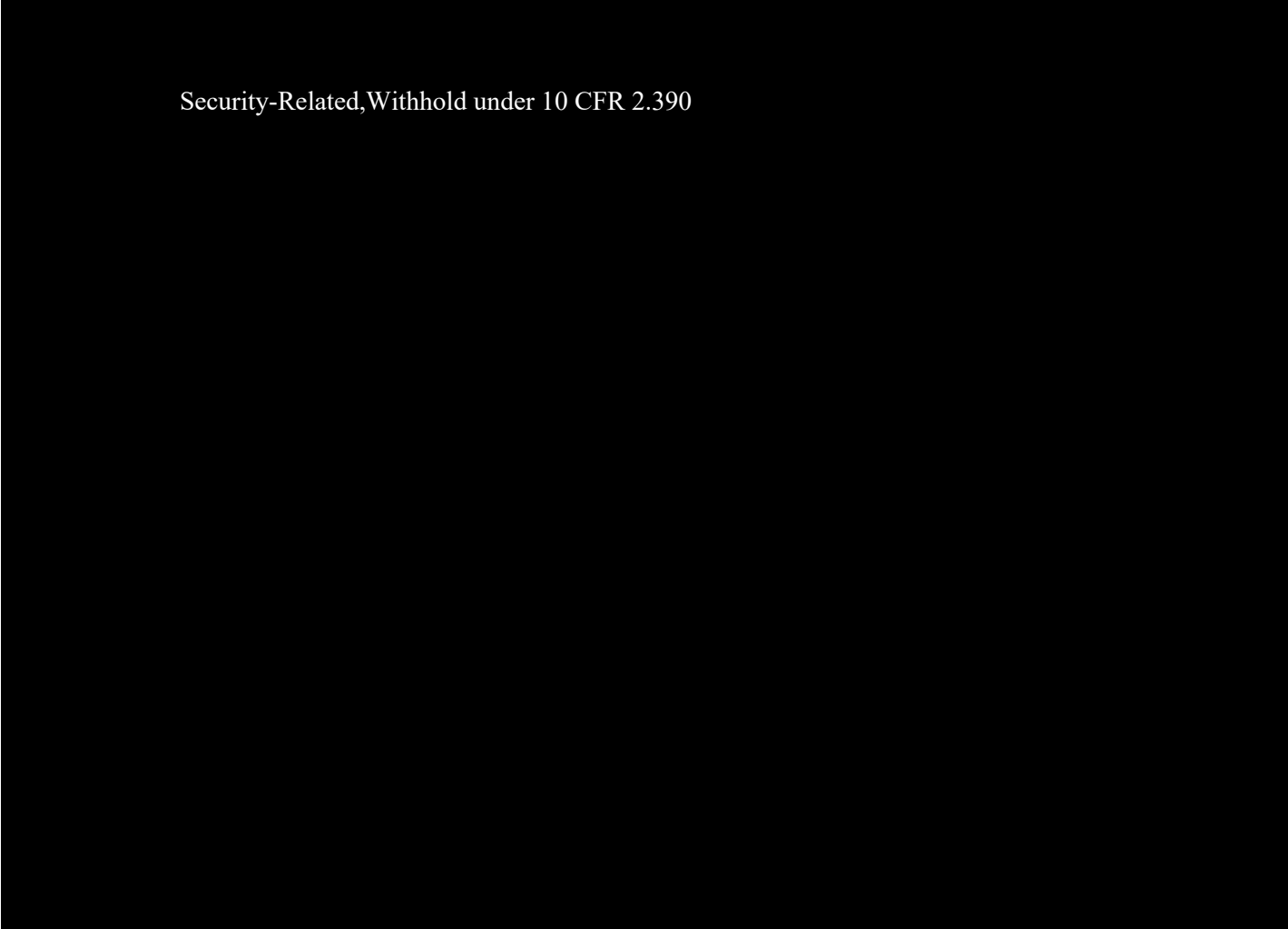
Security-Related, Withhold under 10 CFR 2.390





**Figure 3-219. Exterior Doghouse And UHI Elevation**

Security-Related, Withhold under 10 CFR 2.390



**Figure 3-220. Diesel Generator Building Elevation**

Security-Related, Withhold under 10 CFR 2.390




Figure 3-221. Nuclear Service Water Pump Structure Elevation

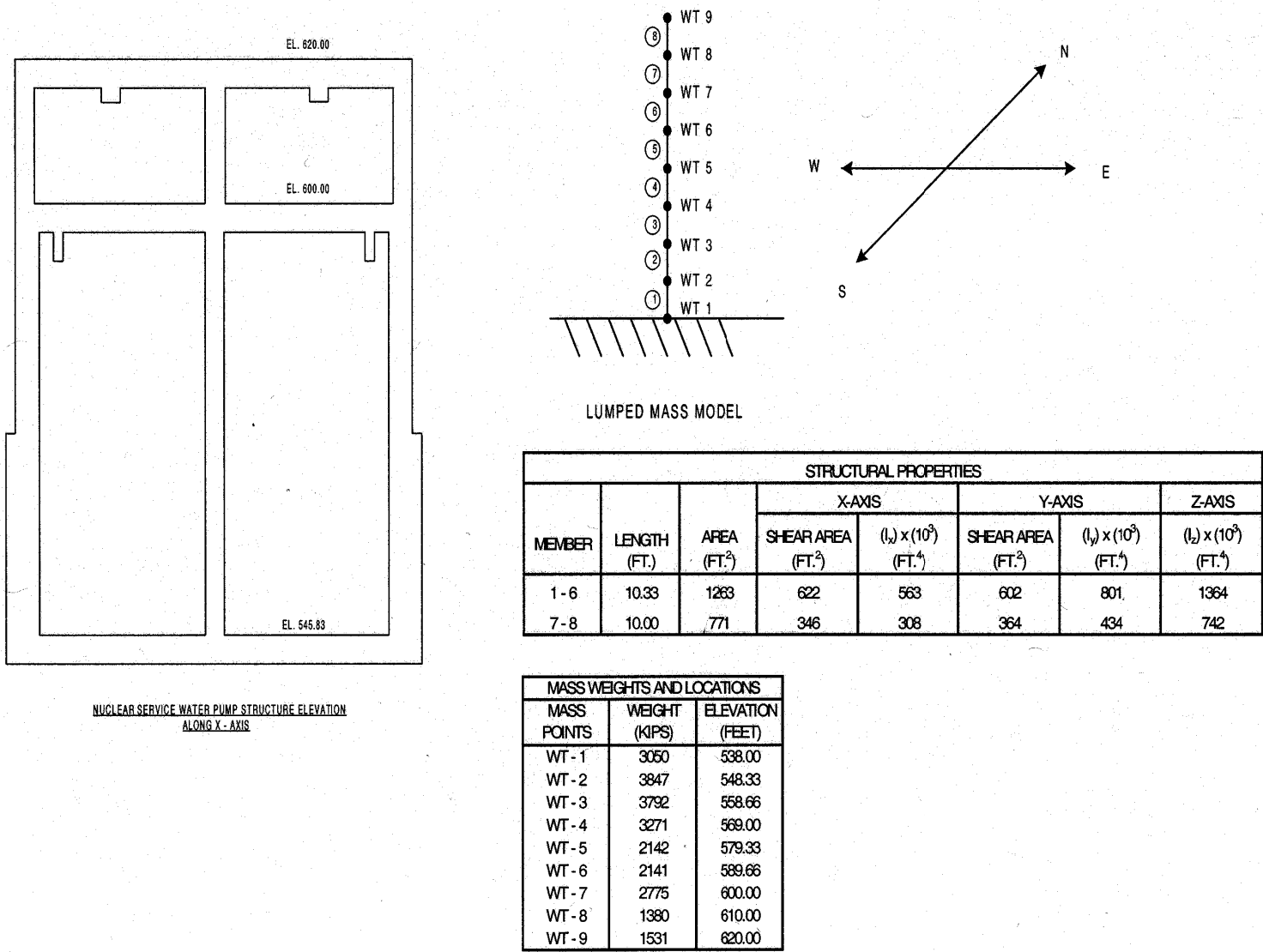


Figure 3-222. Reactor Building First And Second Horizontal Mode Shapes

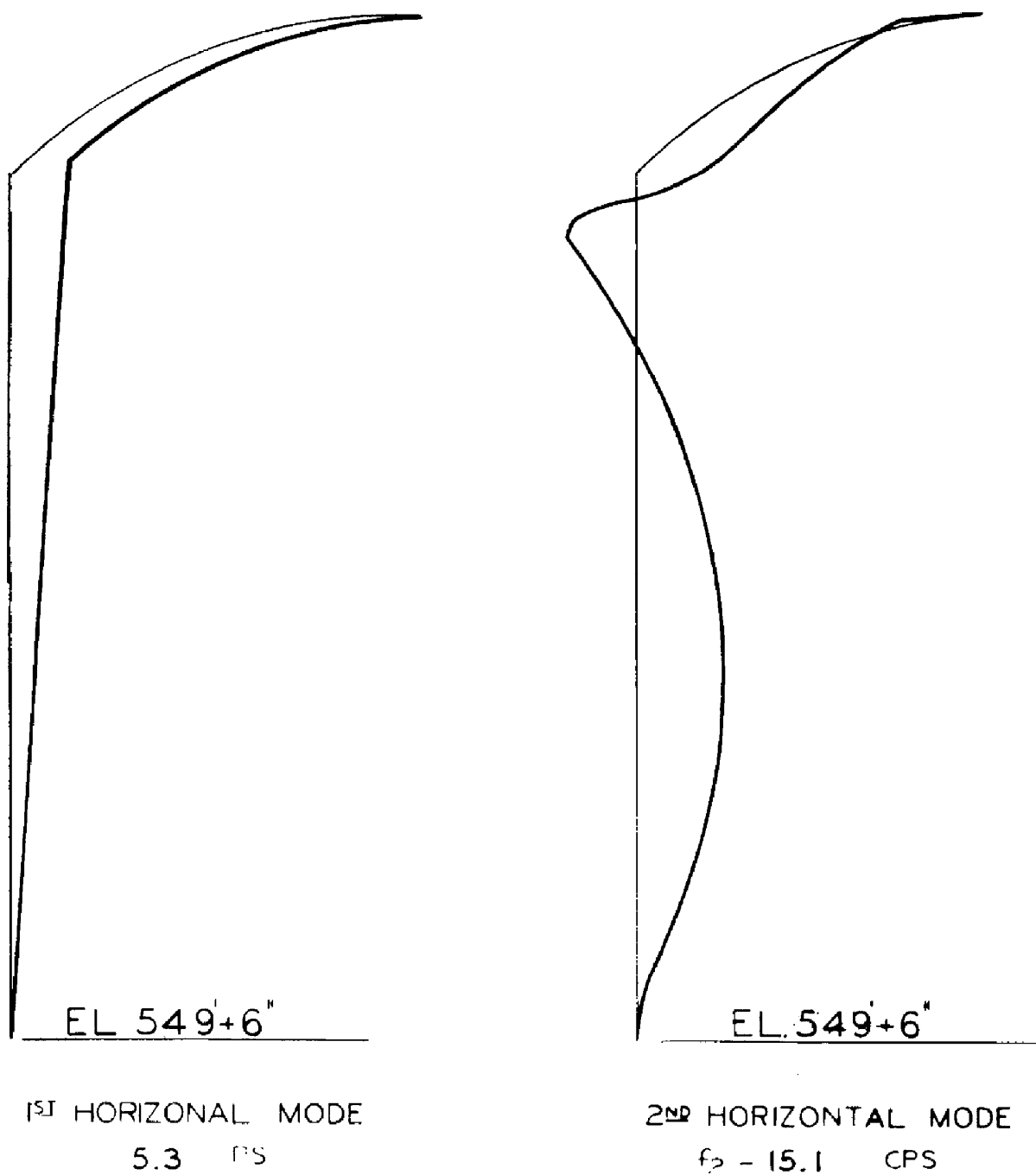


Figure 3-223. Reactor Building First And Second Vertical Mode Shapes

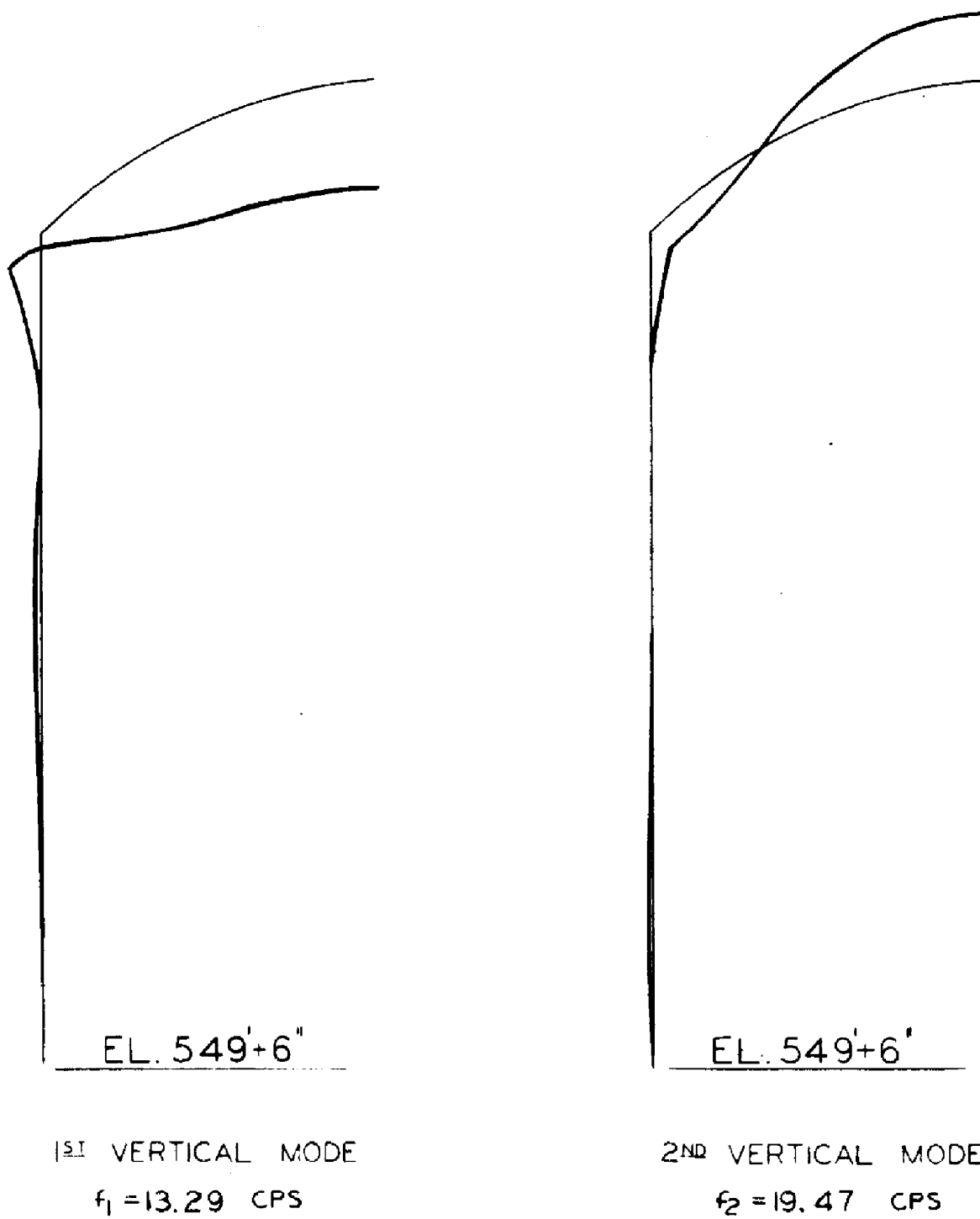


Figure 3-224. Reactor Building Shear Force, Q (lb/in), Due To SSE

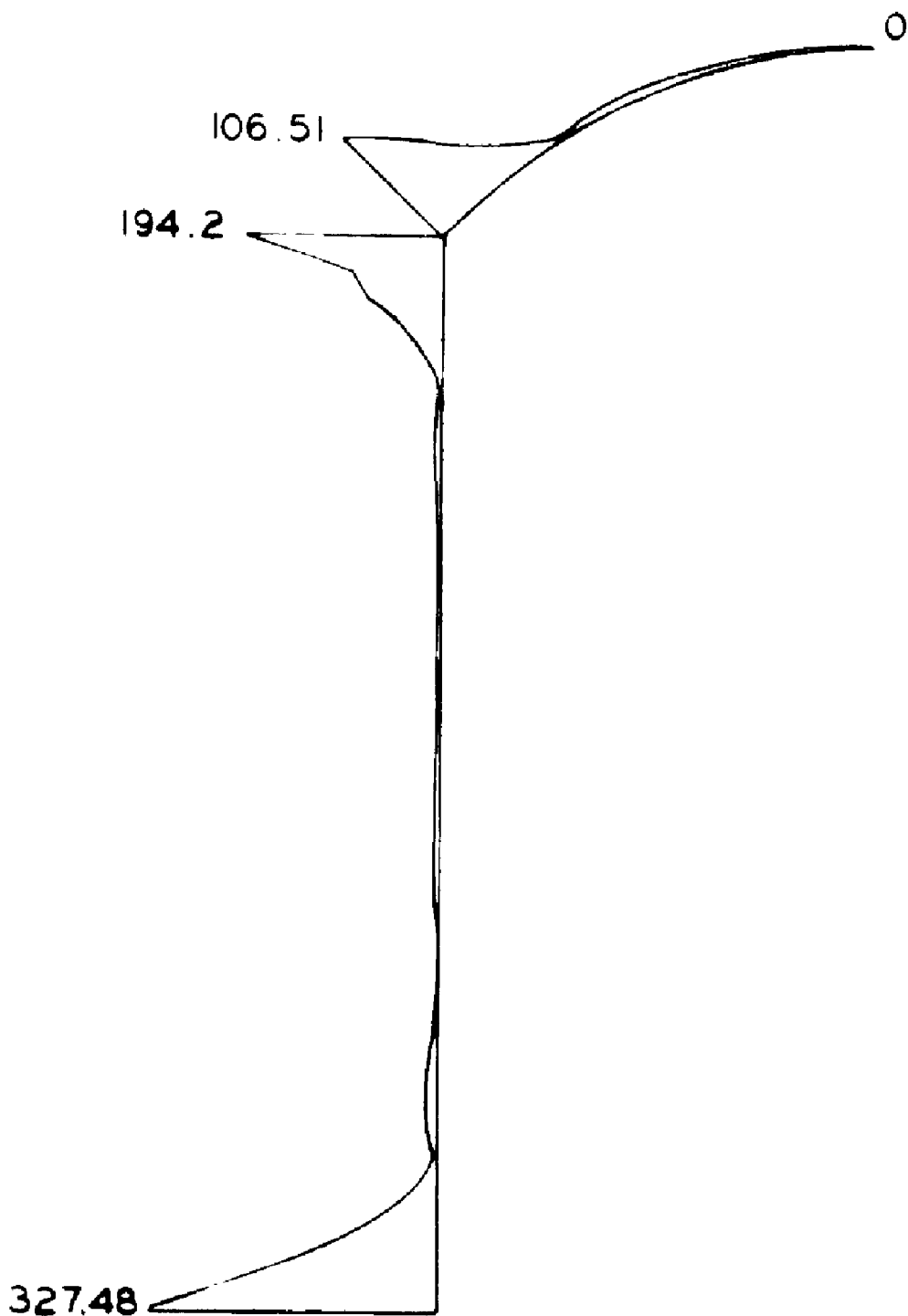


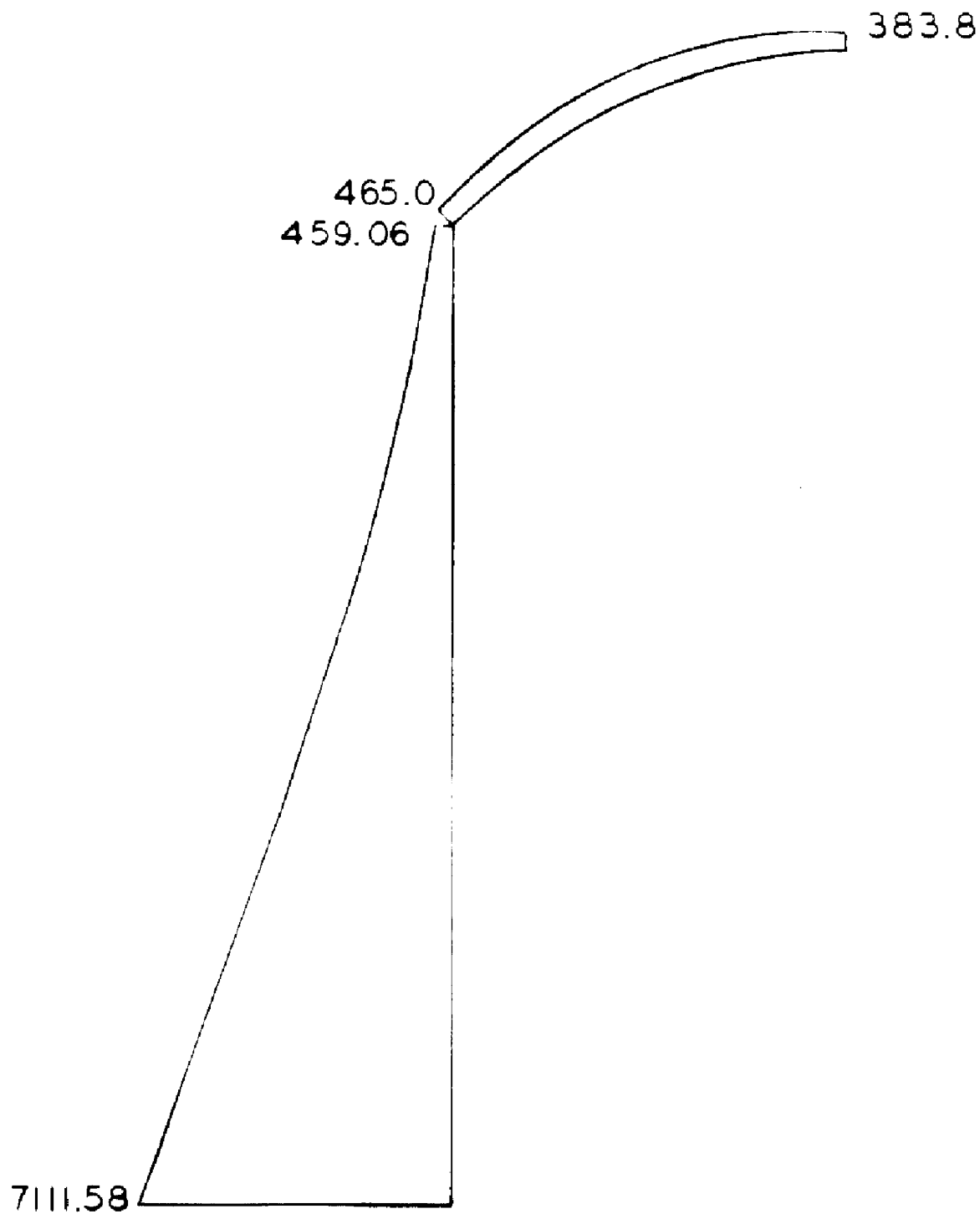
Figure 3-225. Reactor Building Meridional Force,  $N\phi$  (lb/in), Due To SSE

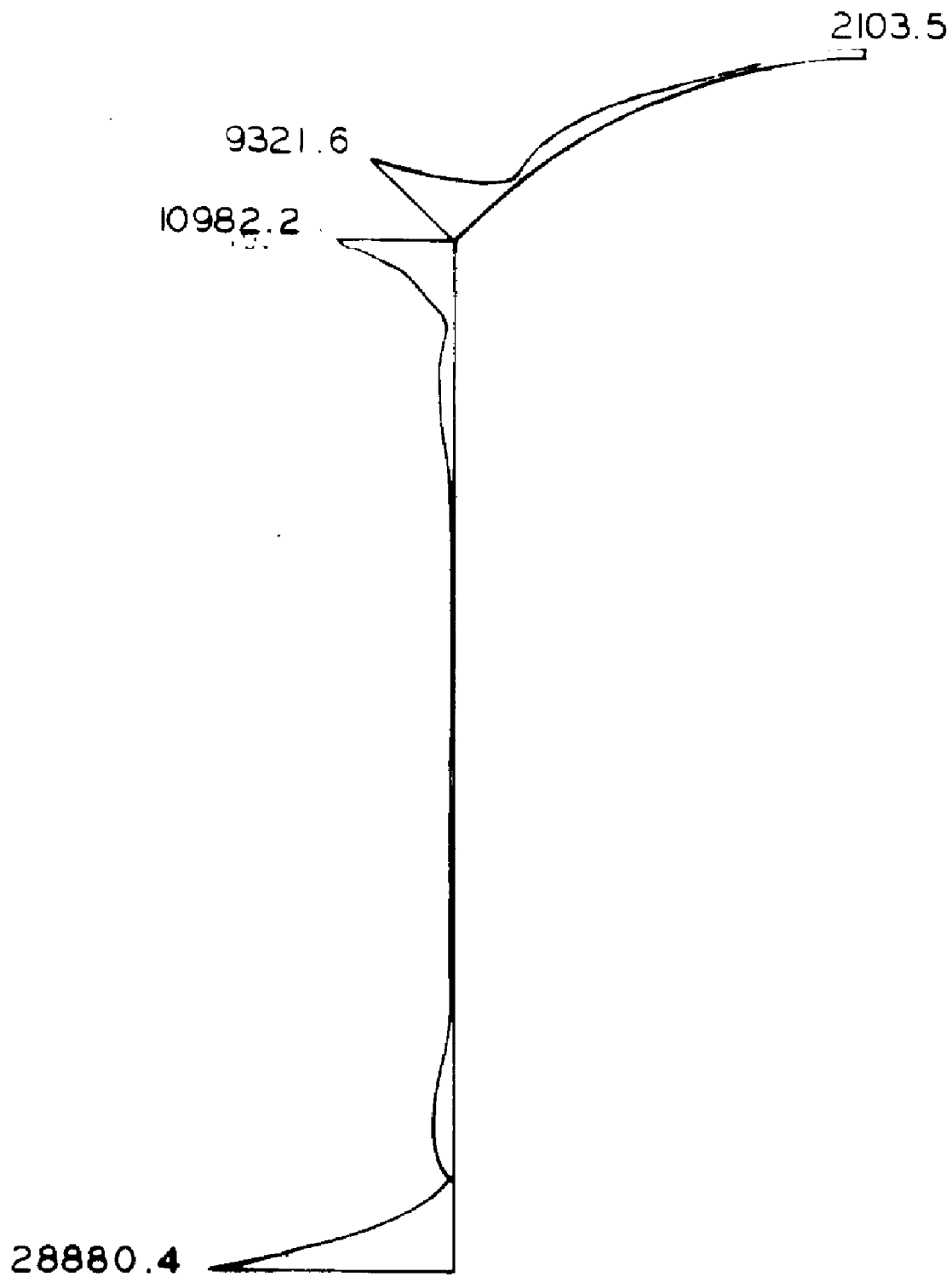
Figure 3-226. Reactor Building Meridional Moment,  $M\phi$  (in-lb/in.), Due To SSE



Figure 3-227. Reactor Building Membrane Shear N (lb/in) Due To SSE

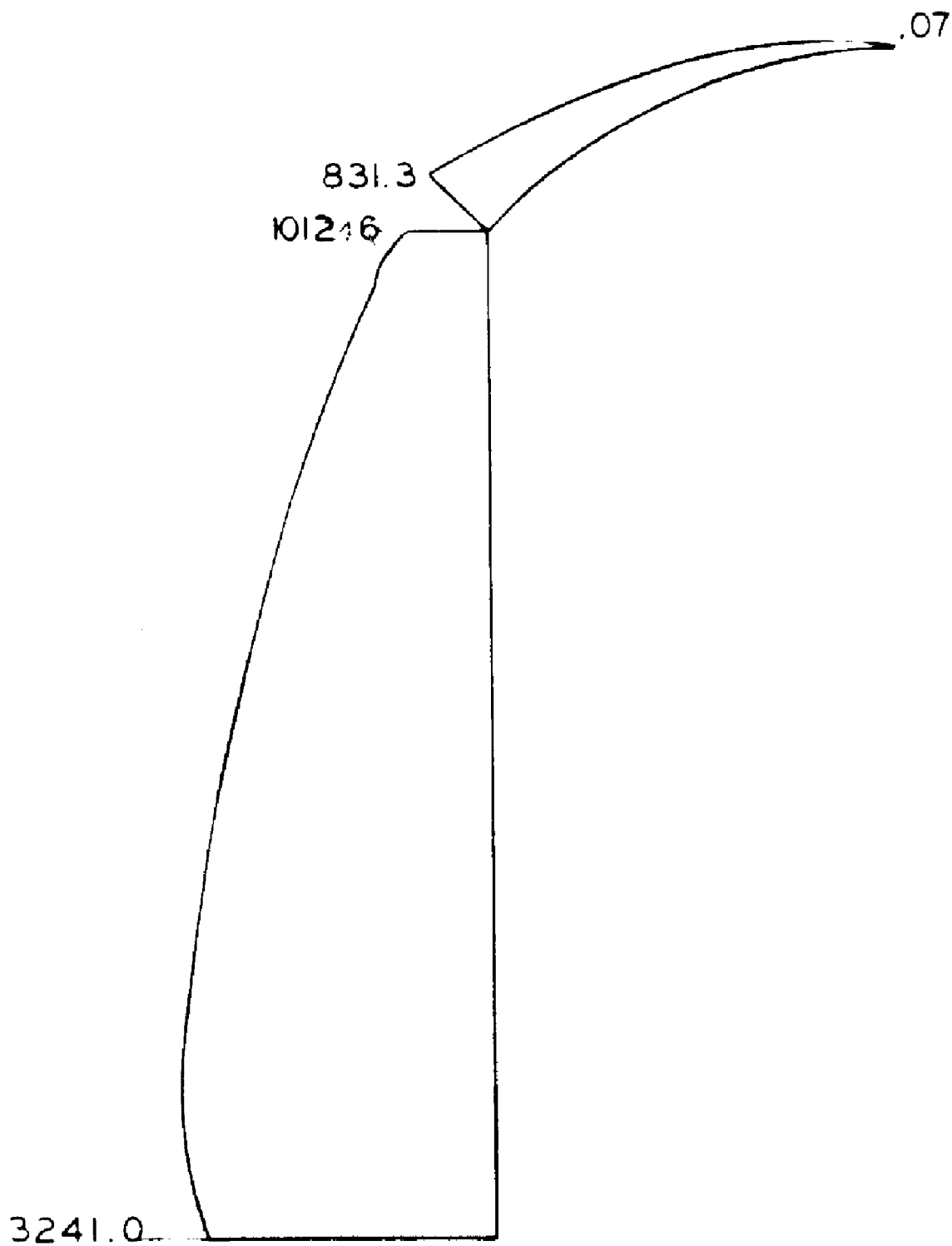


Figure 3-228. Reactor Building Hoop Force  $N_{\theta}$  (lb/in) Due To SSE

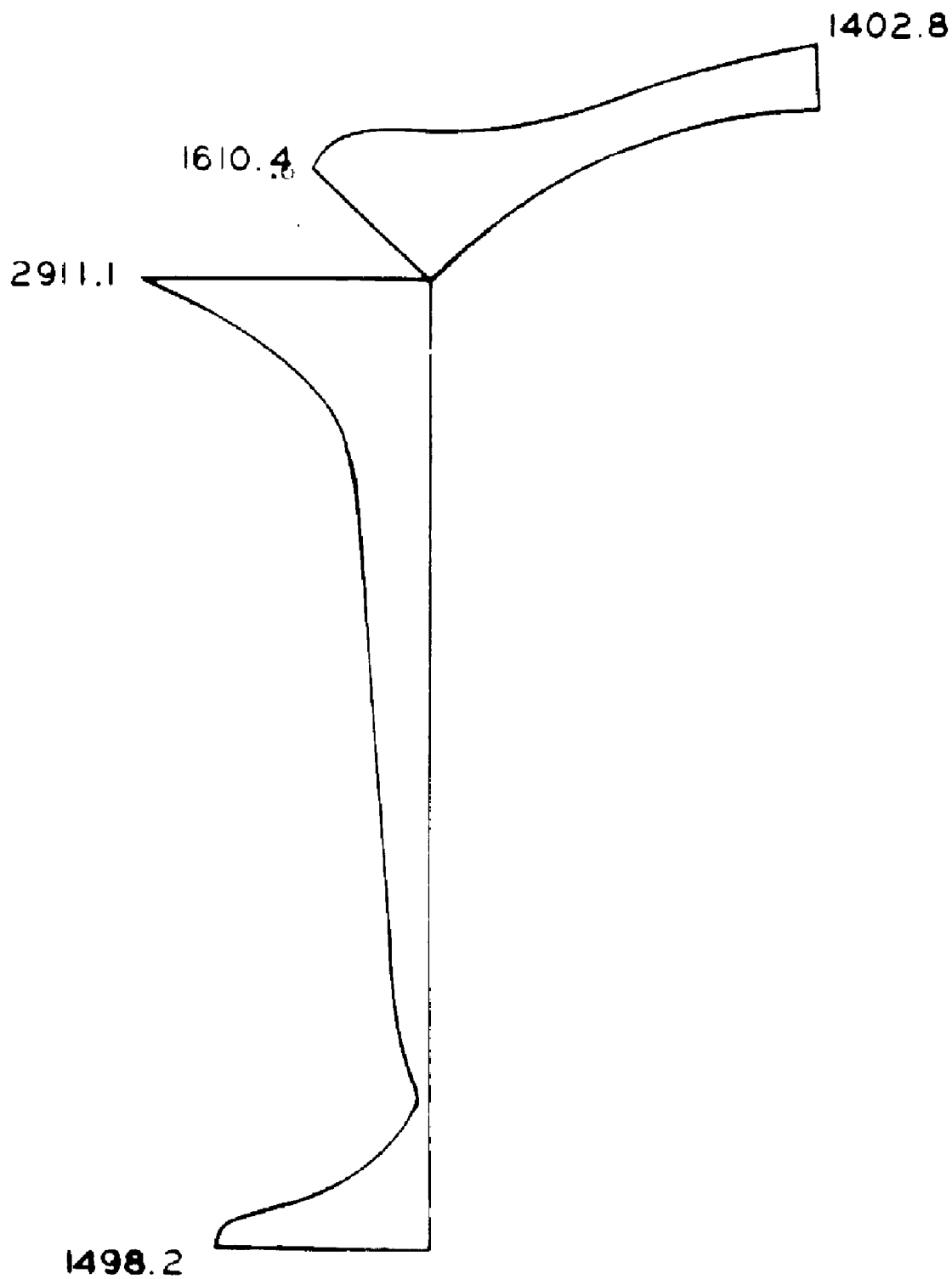


Figure 3-229. Reactor Building Hoop Moment  $M_{\theta}$  (in-lb/in) Due To SSE

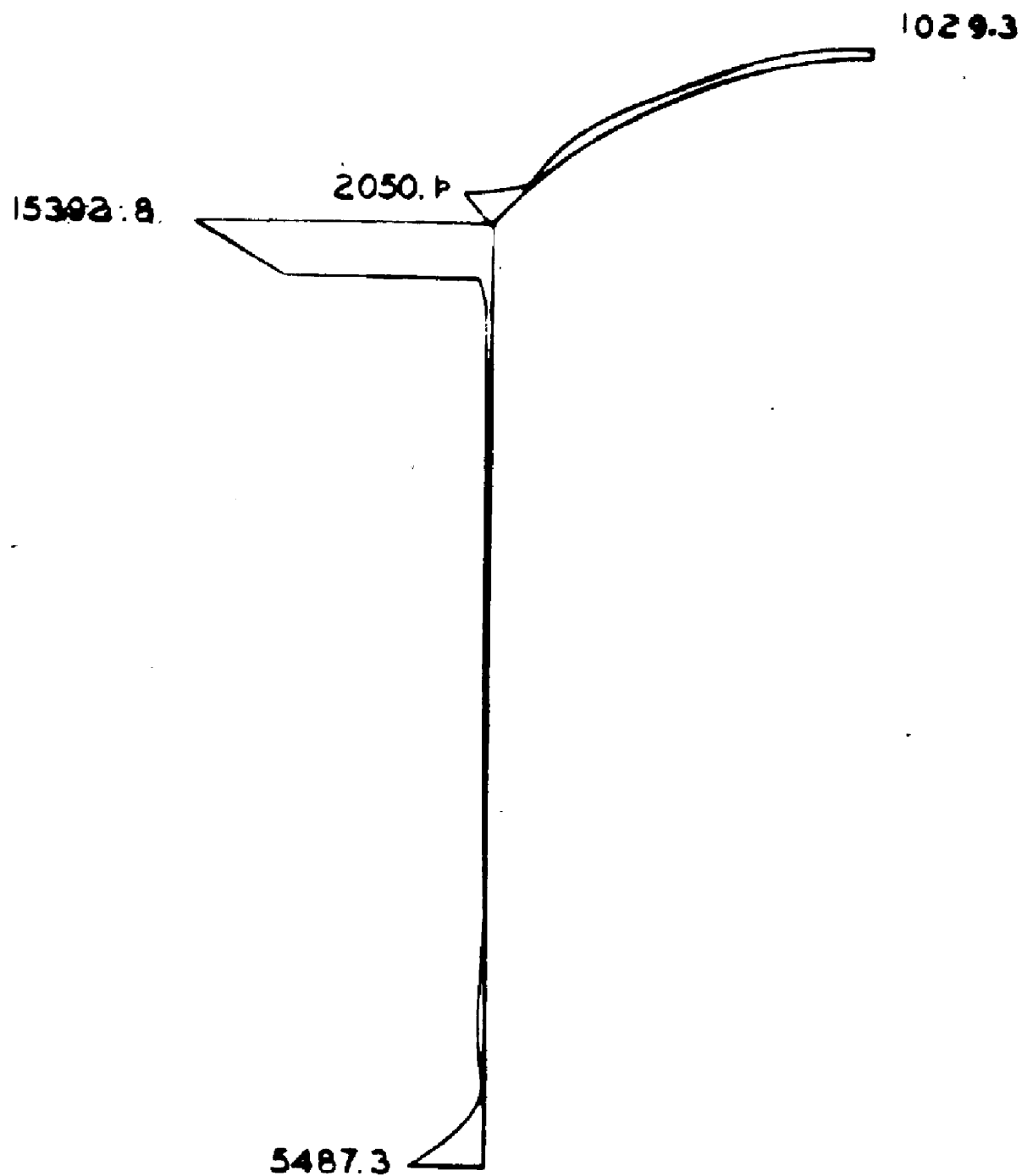
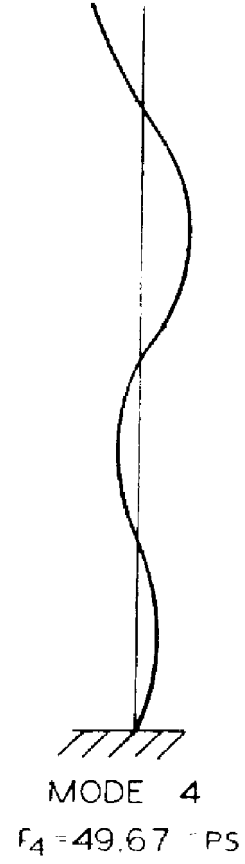
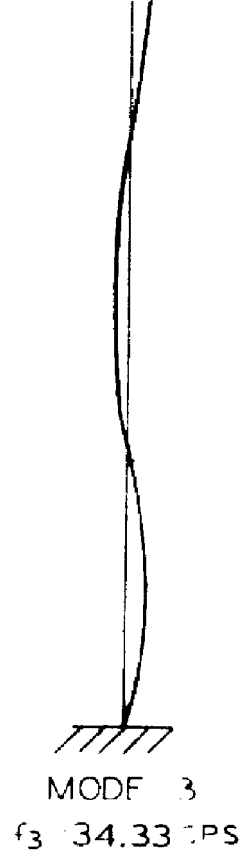
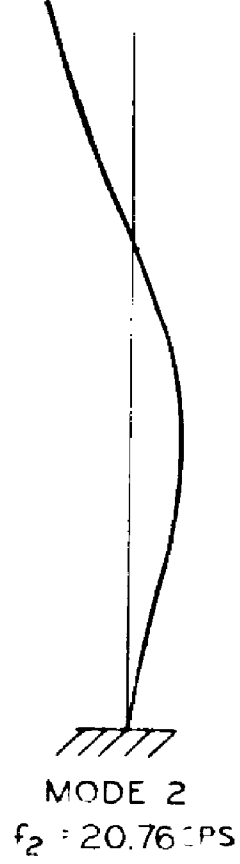
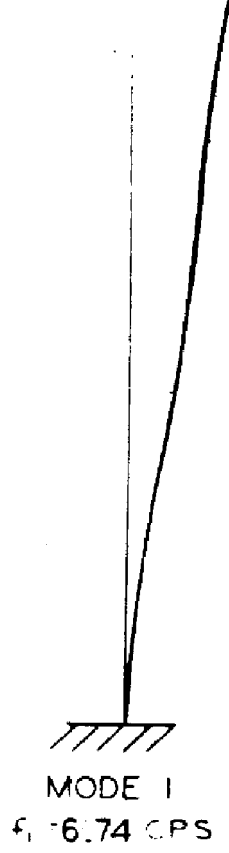


Figure 3-230. Containment Interior Structure First Four Horizontal Mode Shapes North-South Direction



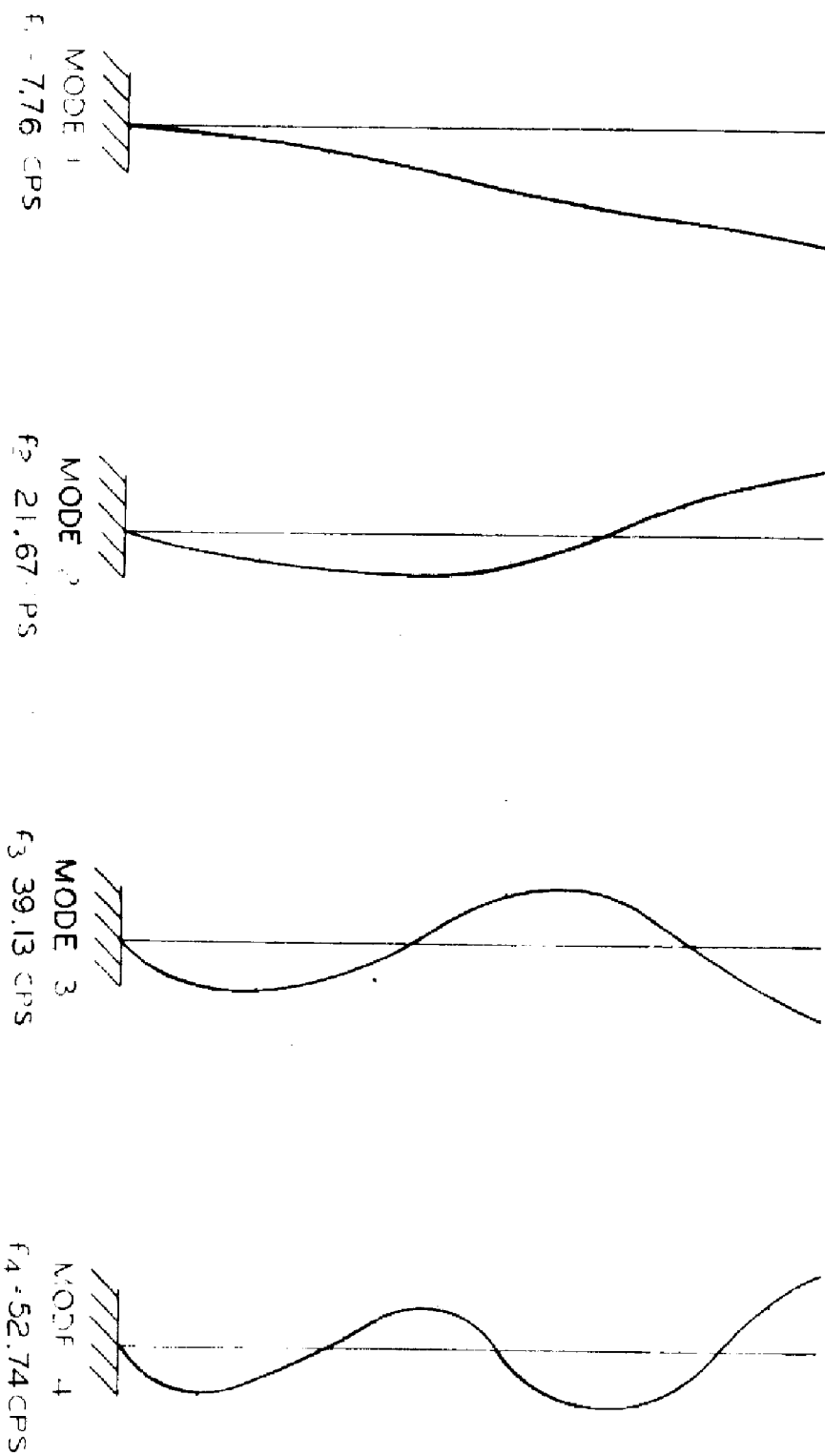
**Figure 3-231. Containment Interior Structure First Four Horizontal Mode Shapes East-West Direction**

Figure 3-232. Containment Interior Structure First Two Vertical Mode Shapes Vertical Direction

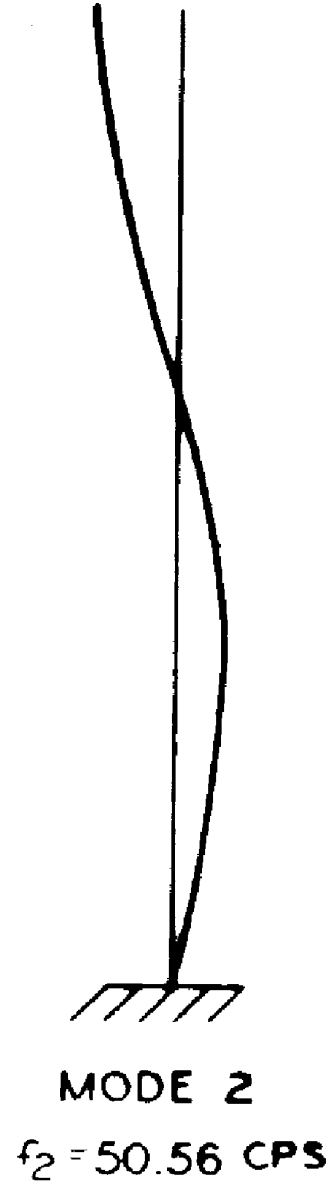
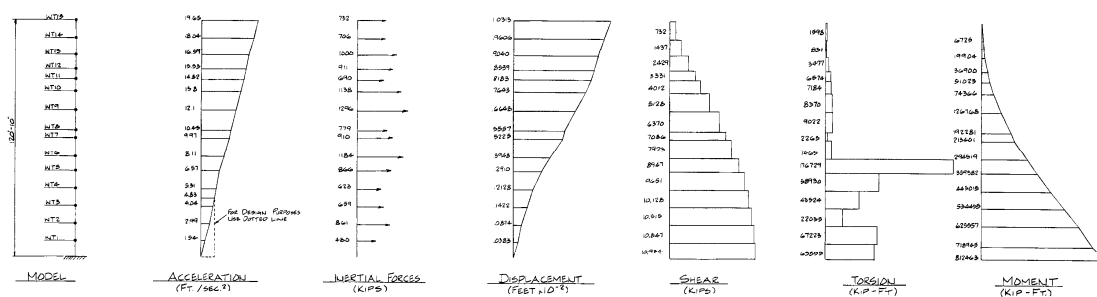
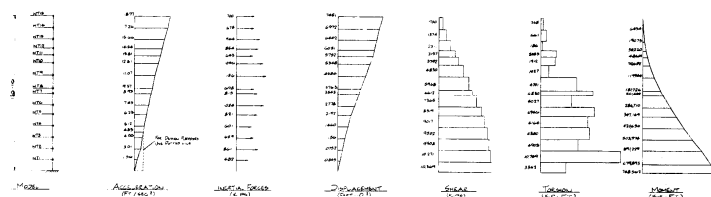


Figure 3-233. Reactor Building Interior Structure Response Loads North-South, SSE



REACTOR BUILDING INTERIOR  
STRUCTURE RESPONSE LOADS  
NORTH-SOUTH, SSE  
CATAWBA NUCLEAR STATION  
Figure 3.7.2-17

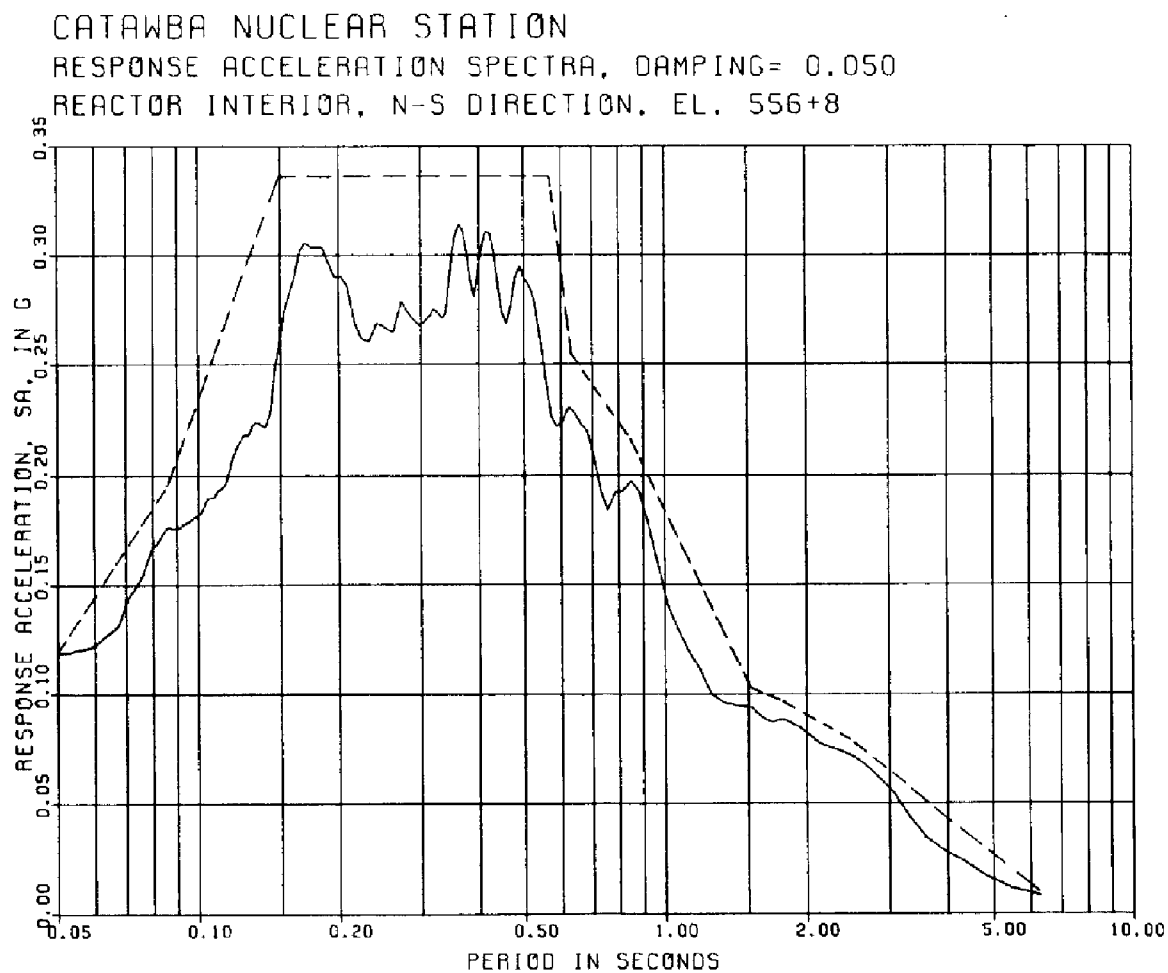
Figure 3-234. Reactor Building Interior Structure Response Loads East-West, SSE



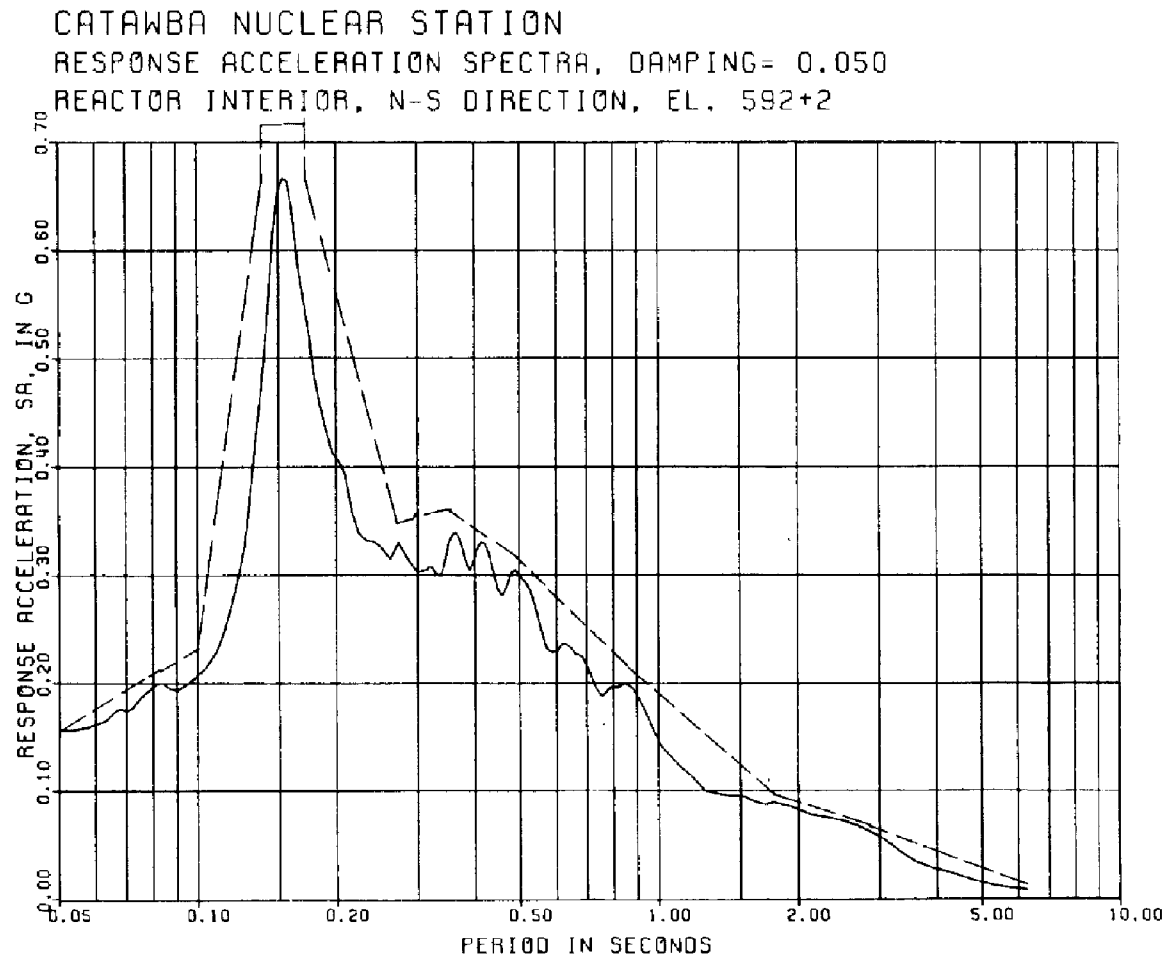
REACTOR BUILDING INTERIOR  
STRUCTURE RESPONSE LOADS  
EAST-WEST, SSE  
CATAWBA NUCLEAR STATION  
Figure 3.7.2-18



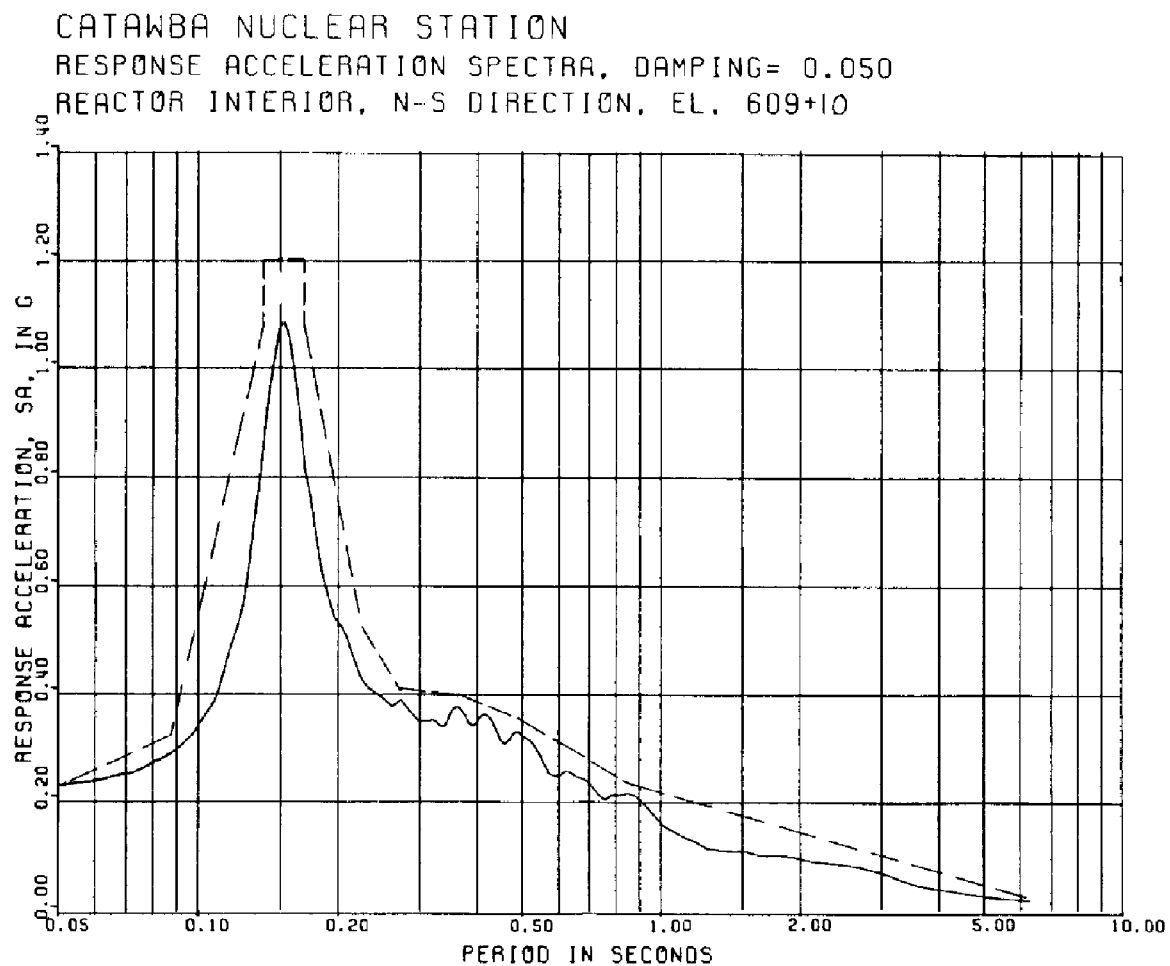
**Figure 3-235. Response Acceleration Spectrum, Damping = 0.05, Reactor Interior, North-South Direction, Elevation 556+8, (OBE)**



**Figure 3-236. Response Acceleration Spectrum, Damping = 0.05, Reactor Interior, North-South Direction, Elevation 592+2, (OBE)**

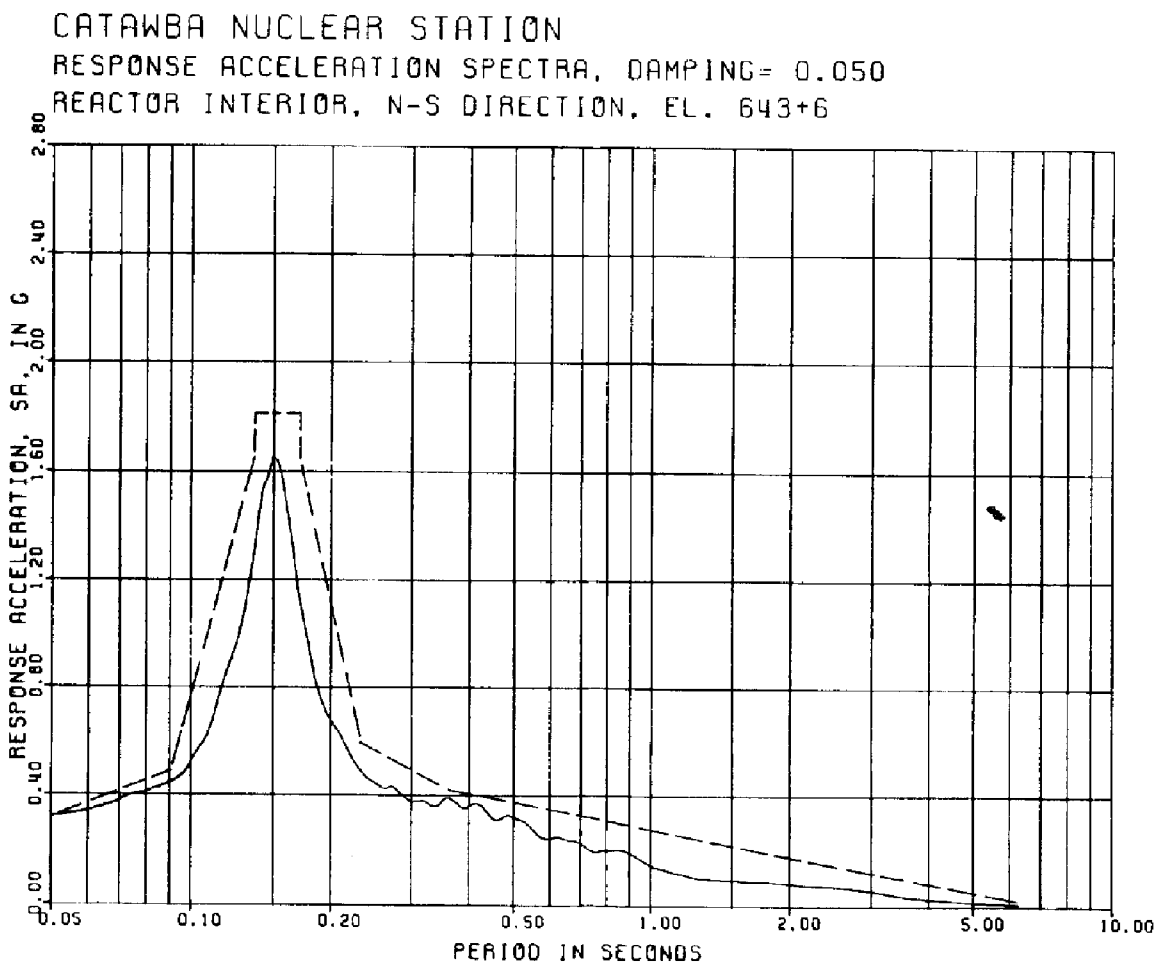


**Figure 3-237. Response Acceleration Spectrum, Damping = 0.05, Reactor Interior, North-South Direction, Elevation 609+10, (OBE)**



(22 OCT 2001)

Figure 3-238. Response Acceleration Spectrum, Damping = 0.05, Reactor Interior, North-South Direction, Elevation 643+6, (OBE)



(22 OCT 2001)

**Figure 3-239. Response Acceleration Spectrum, Damping = 0.05, Reactor Interior, North-South Direction, Elevation 668+10, (OBE)**

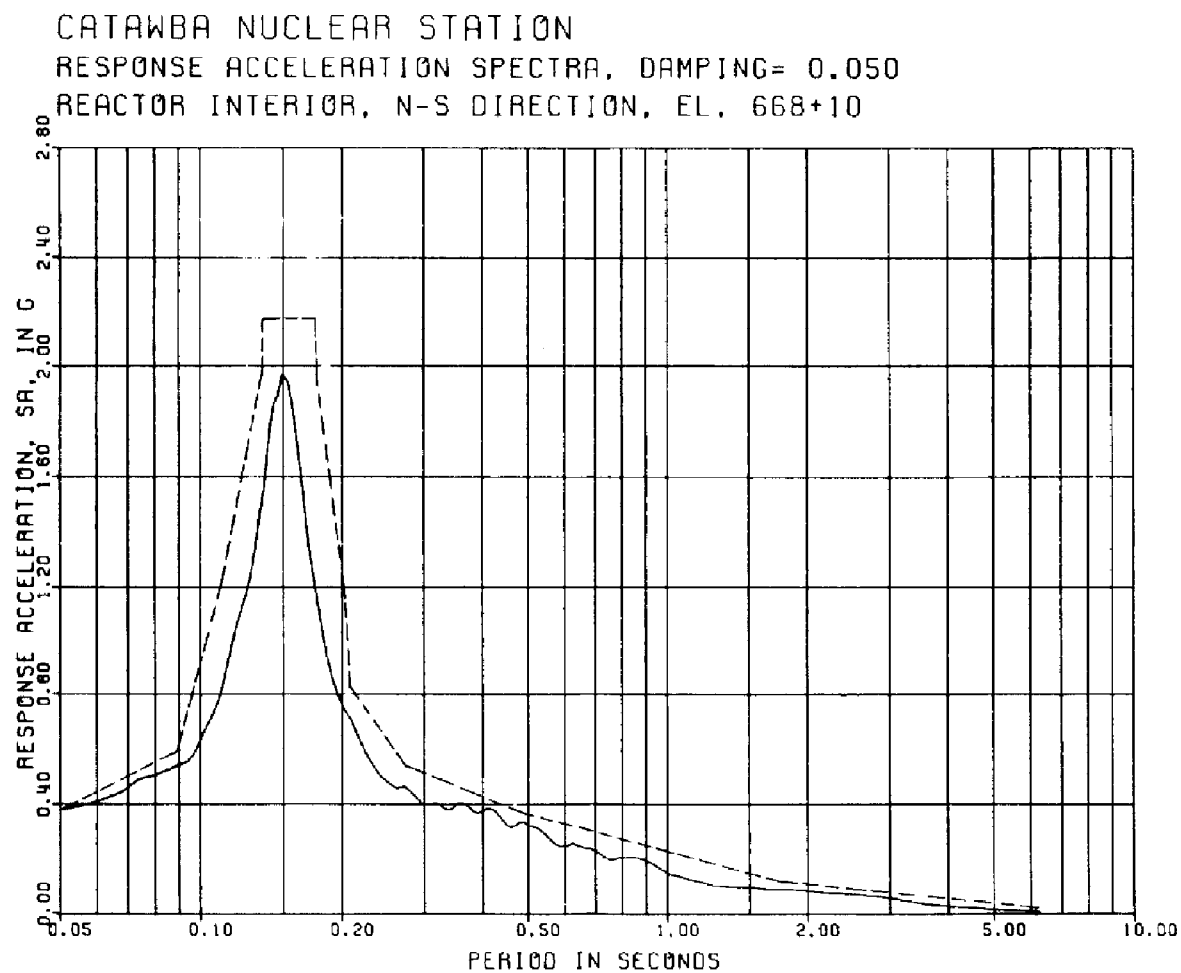
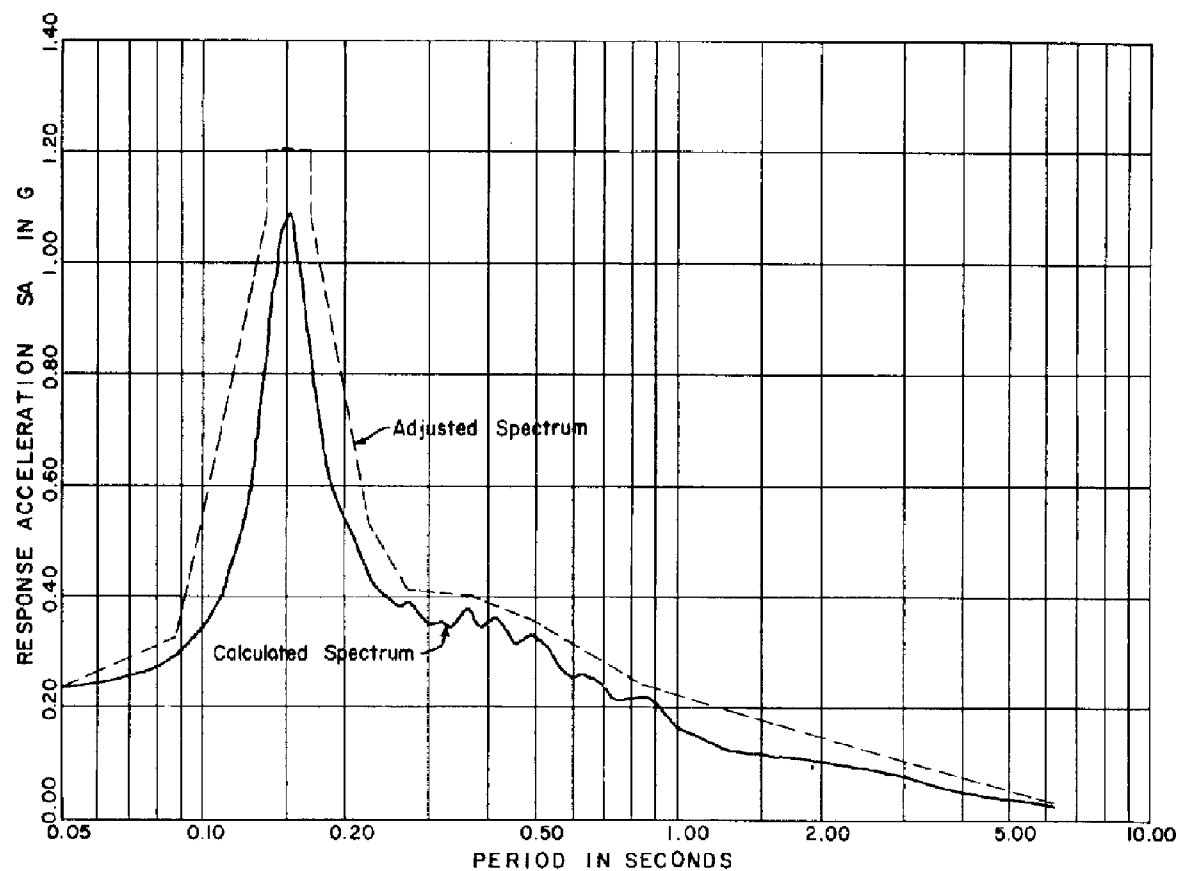


Figure 3-240. Response Acceleration Spectrum, Damping = 0.05, Reactor Interior, North-South Direction, Elevation 609+10, (OBE)



(22 OCT 2001)

Figure 3-241. Soil Pressure Diagram For Substructure Walls Of Auxiliary Building

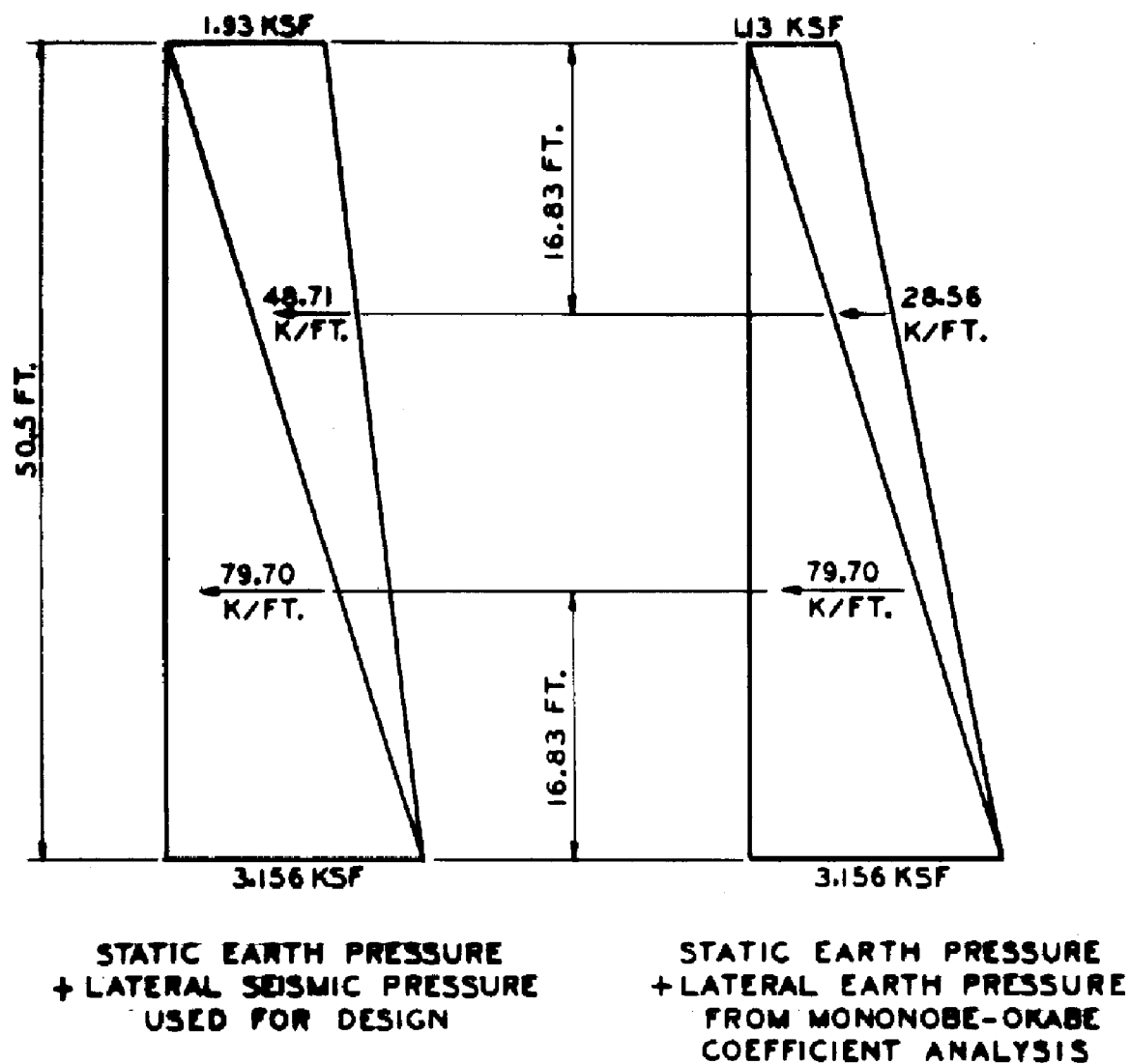
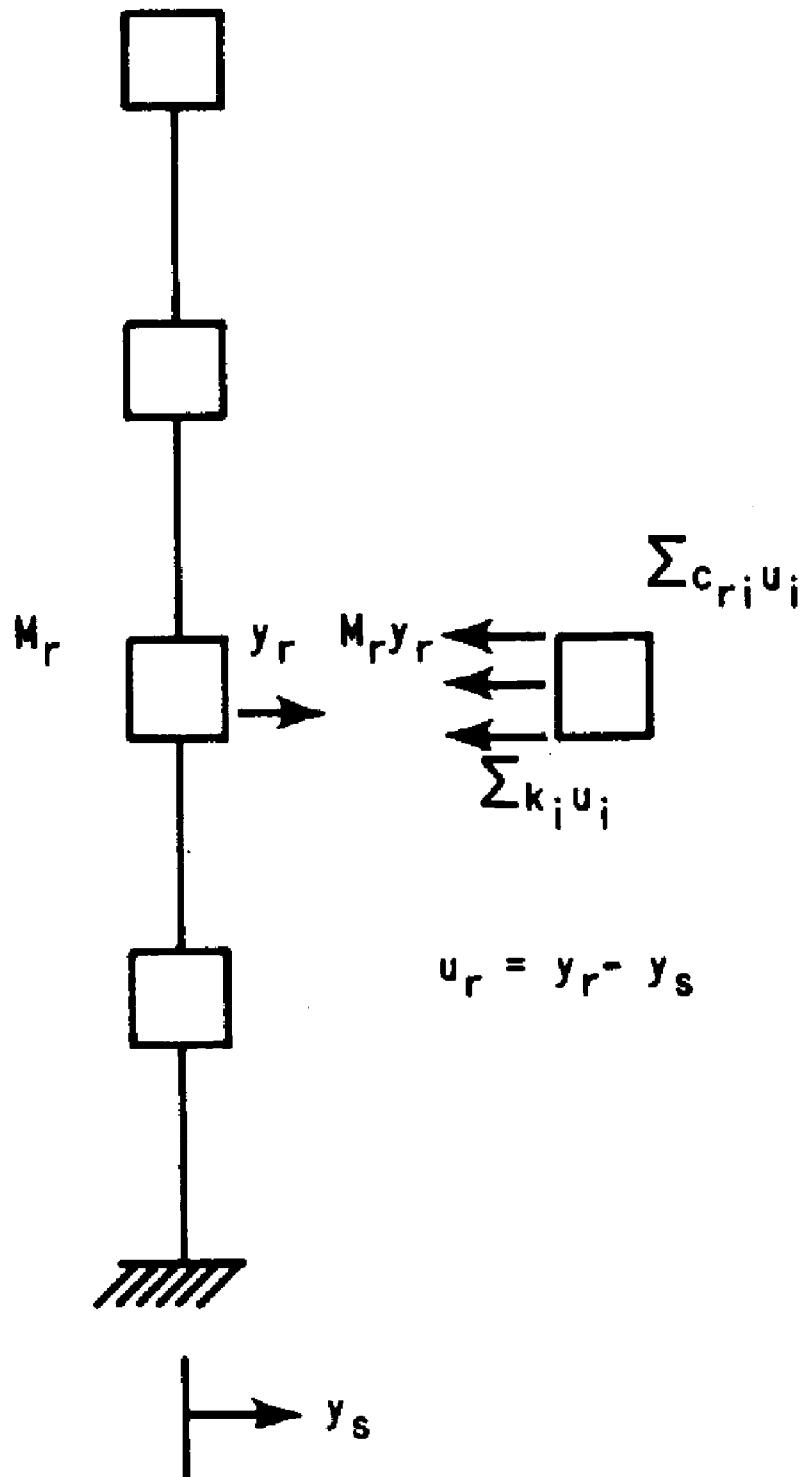


Figure 3-242. Multi-Degree Of Freedom System





**Figure 3-243. Reactor Building And Containment Structural Outline**

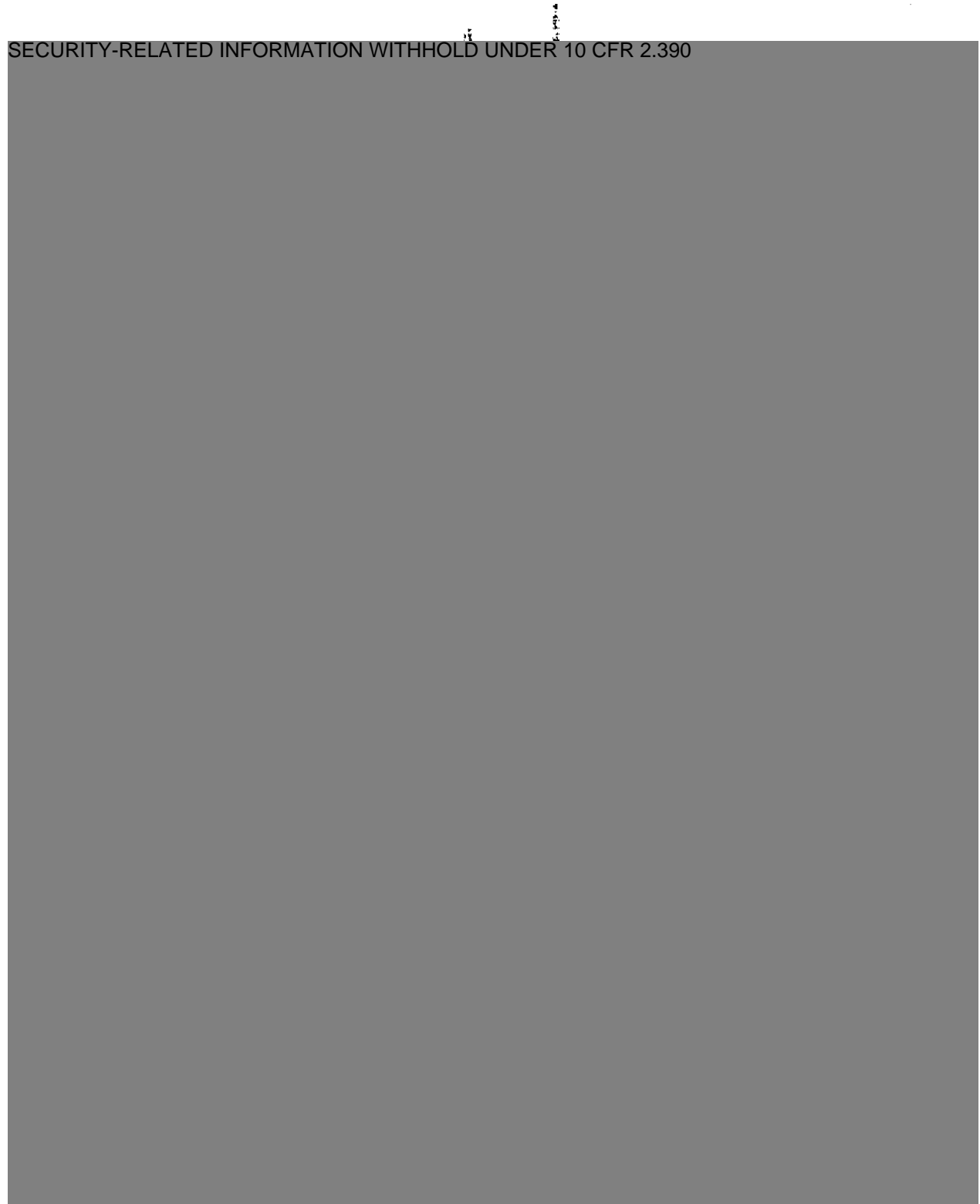


Diagram illustrating the shape of a dome for clarity. The diagram shows a circular cross-section with a central vertical line labeled "Center Line". A horizontal line is labeled "Center Line". A note in the center reads: "NOTE: This diagram shows the shape of the dome for clarity." The diagram is surrounded by a circular border with various labels and markings. A north arrow is located at the top left.

(22 OCT 2001)

Figure 3-245. Reactor Building Top Radial Bars

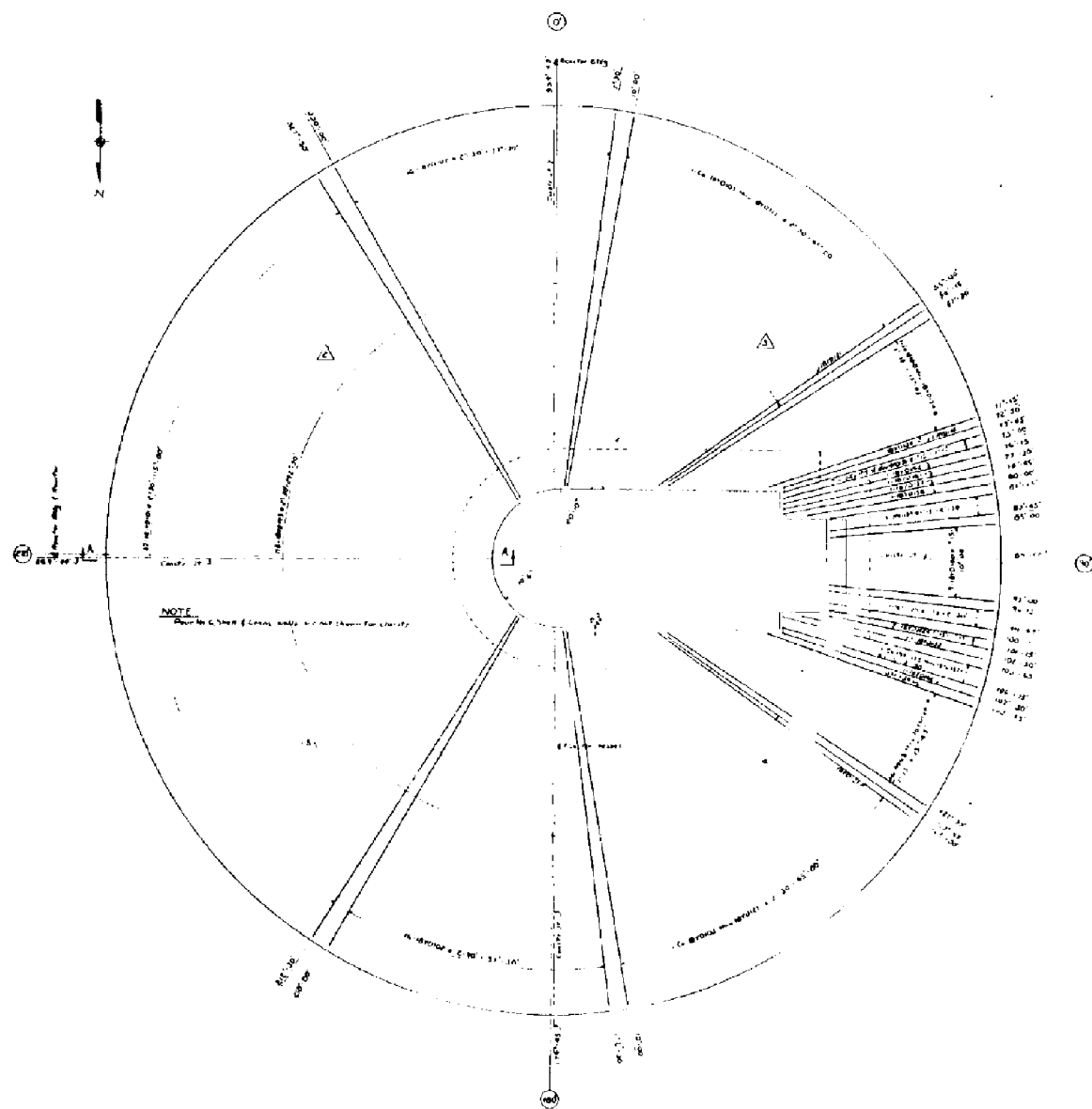
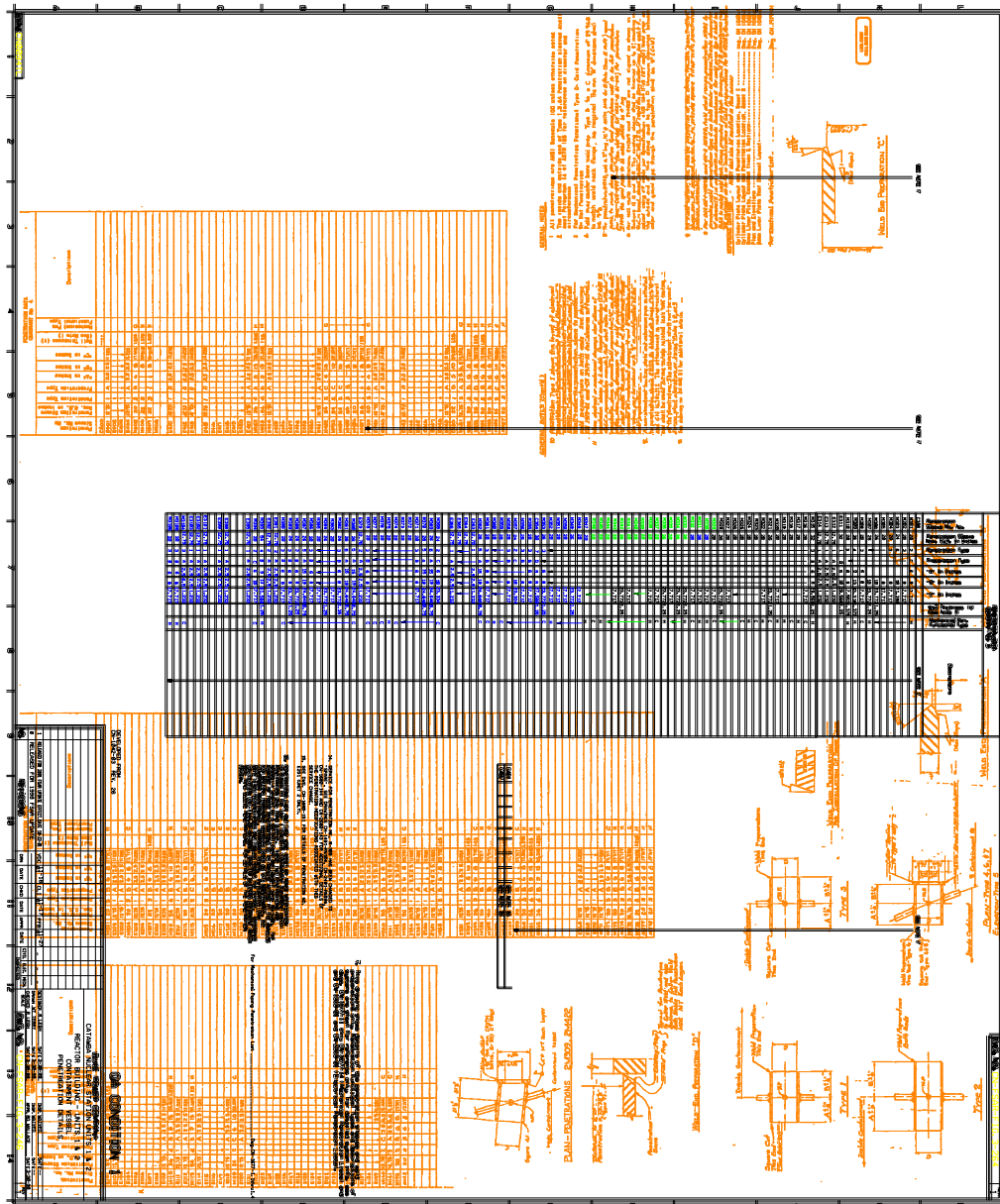


Figure 3-246. Reactor Building Circular Bars




(



Figure 3.8.1-5

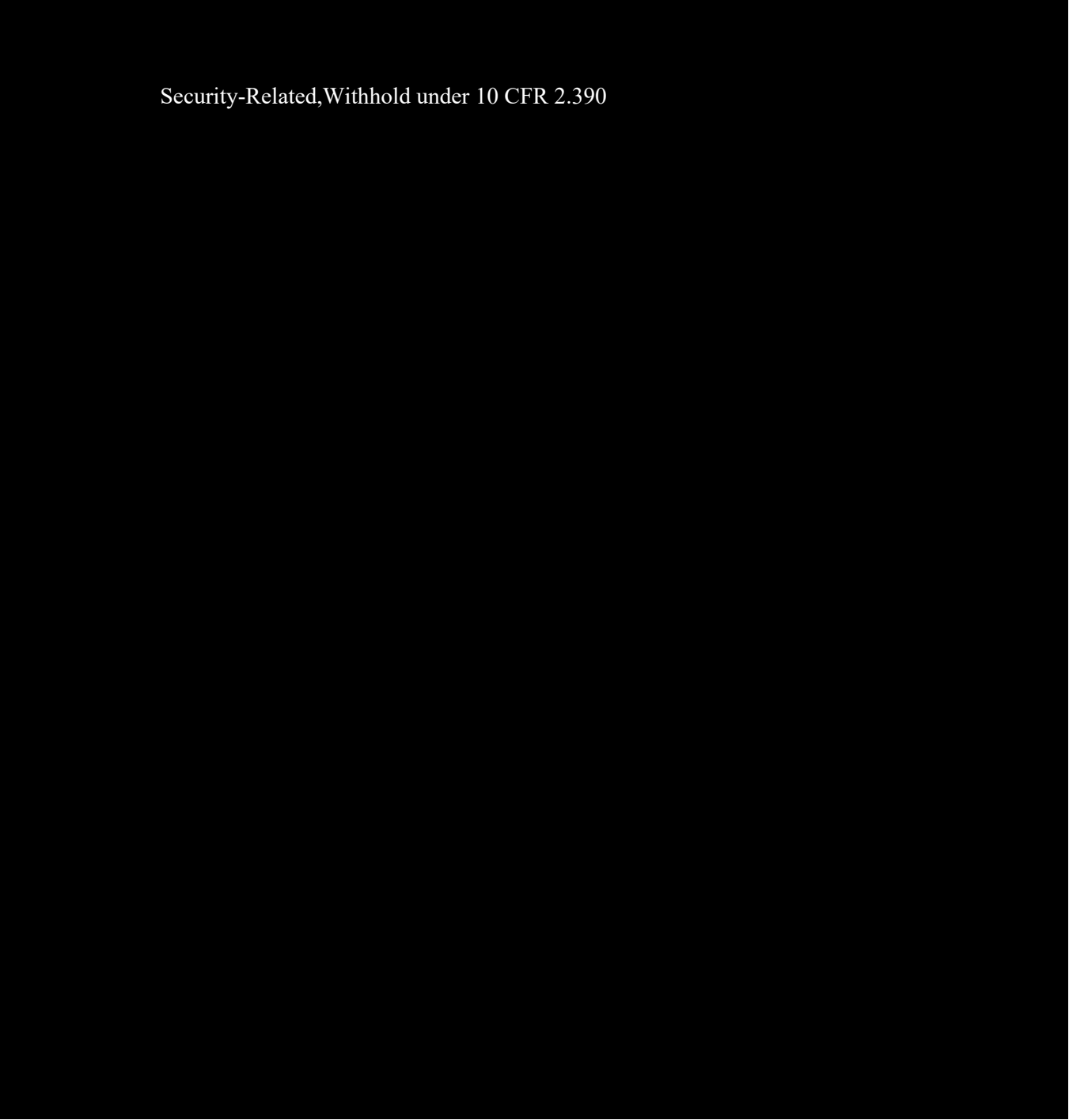
**Figure 3-248. Reactor Building Shell Wall Developed Elevation**

Security-Related, Withhold under 10 CFR 2.390




**Figure 3-249. Reactor Building Shell Wall Developed Elevation**

Security-Related, Withhold under 10 CFR 2.390



**Figure 3-250. Reactor Building Shell Wall Developed Elevation**


Security-Related, Withhold under 10 CFR 2.390





**Figure 3-251. Reactor Building Equipment Hatch Reinforcing**


Security-Related, Withhold under 10 CFR 2.390



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**Figure 3-252. Dome And Ring Girder Reinforcing**

Security-Related, Withhold under 10 CFR 2.390



**Figure 3-253. Dome And Ring Girder Reinforcing**

Security-Related, Withhold under 10 CFR 2.390



(22 OCT 2001)

Figure 3-254. Dome And Ring Girder Reinforcing

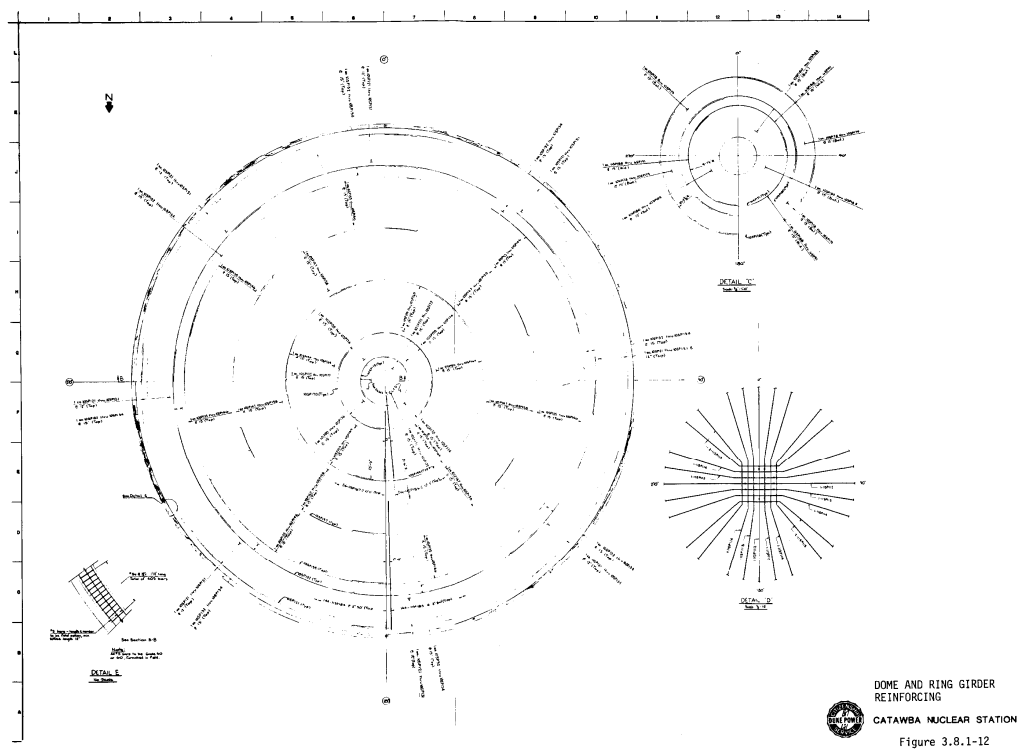


Figure 3-255. Equipment Hatch Model

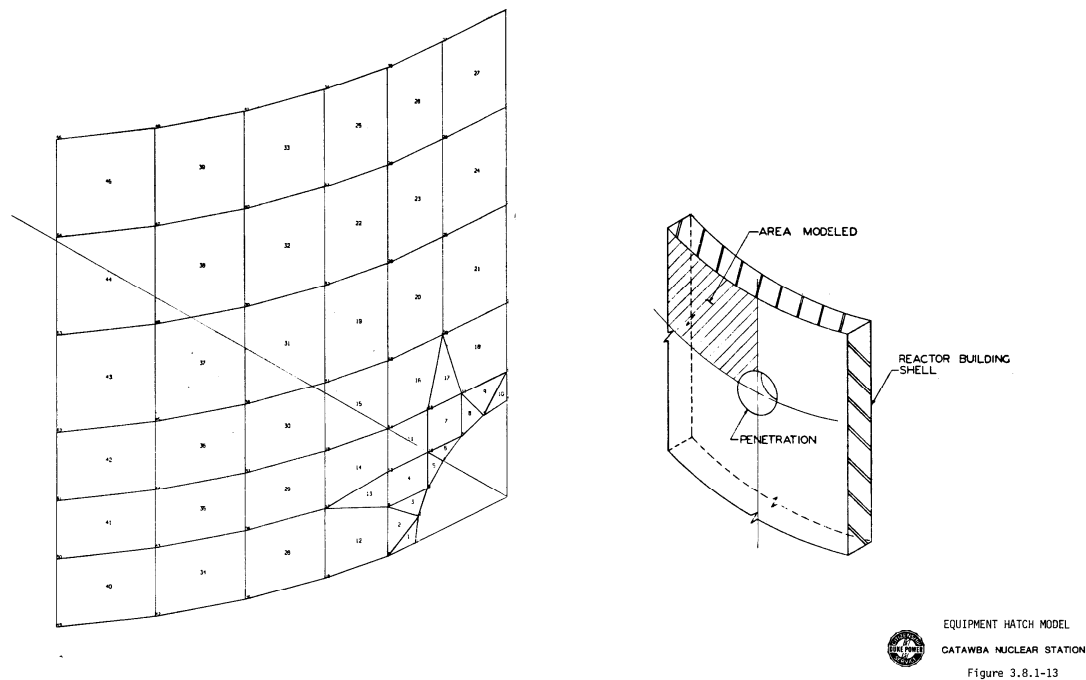


Figure 3-256. K-Shell Model For Reactor Building Analysis

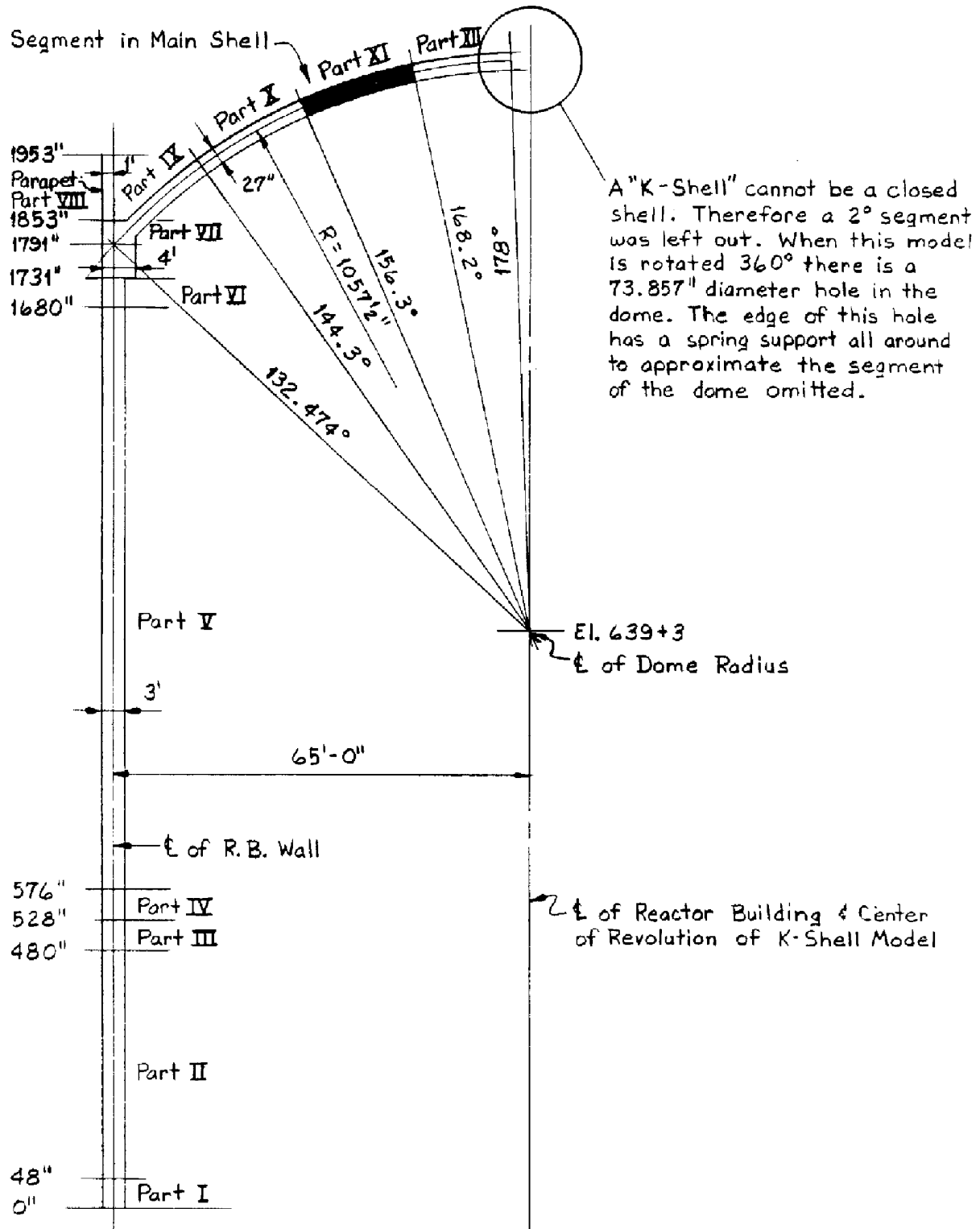
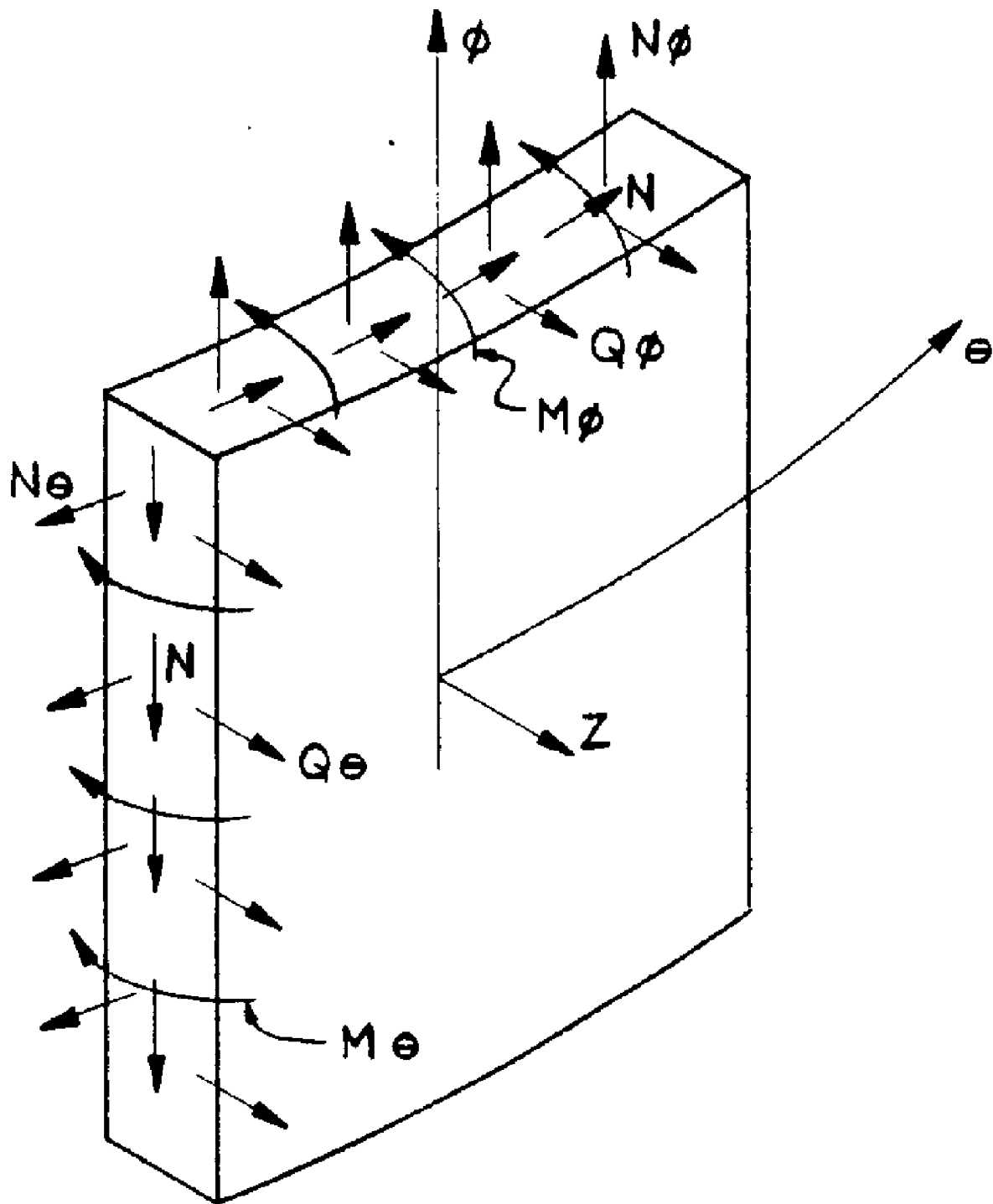


Figure 3-257. Axis Orientation For Reactor Building Analysis



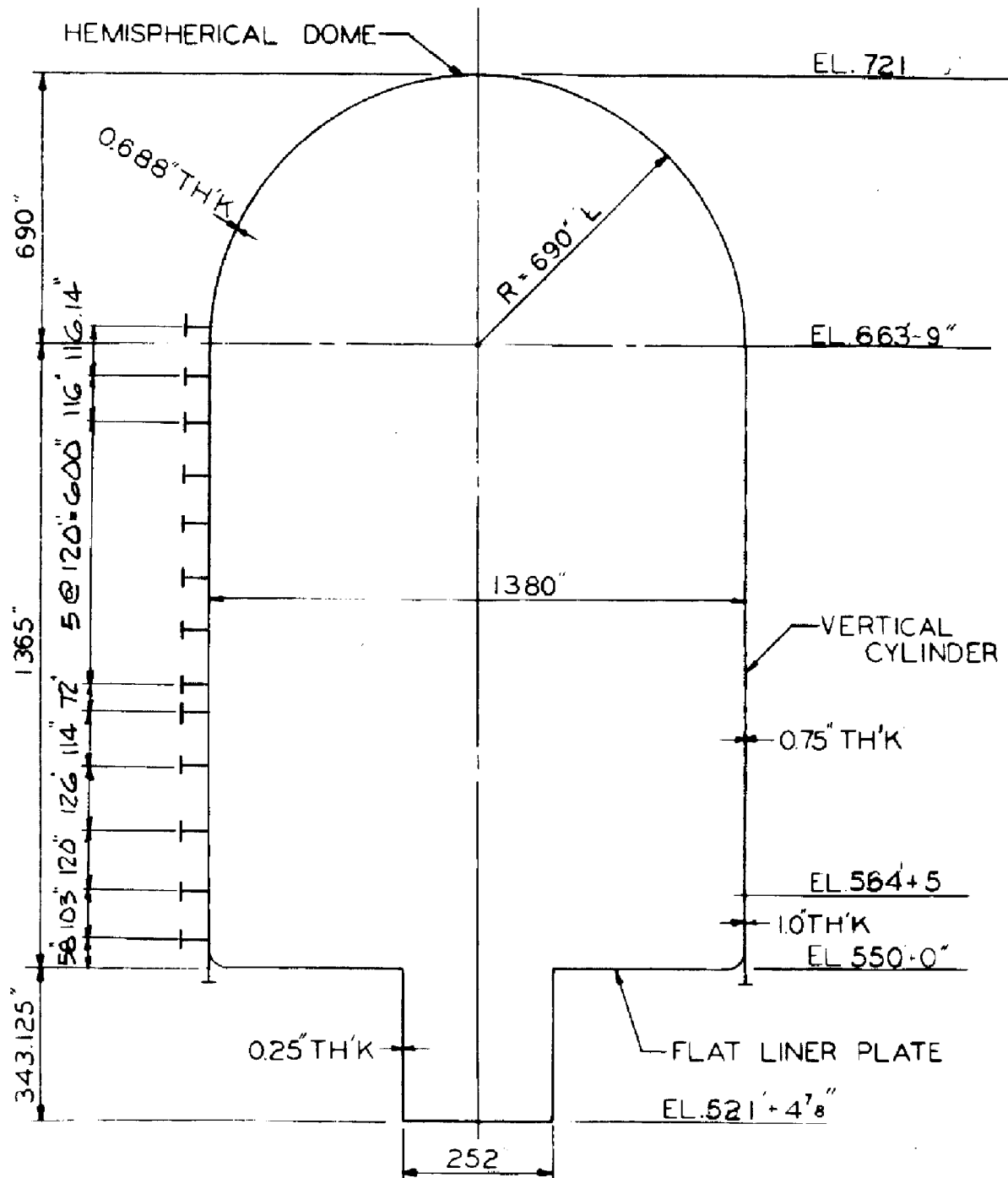
**Figure 3-258. Reactor Building Shell And Crane Wall (Reinforcement Details)**

SECURITY-RELATED INFORMATION WITHHOLD UNDER 10 CFR 2.390






Figure 3-259. Containment Geometry



CONTAINMENT GEOMETRY

**Figure 3-260. Containment Personnel And Equipment Hatch Typical Details**

Security-Related, Withhold under 10 CFR 2.390



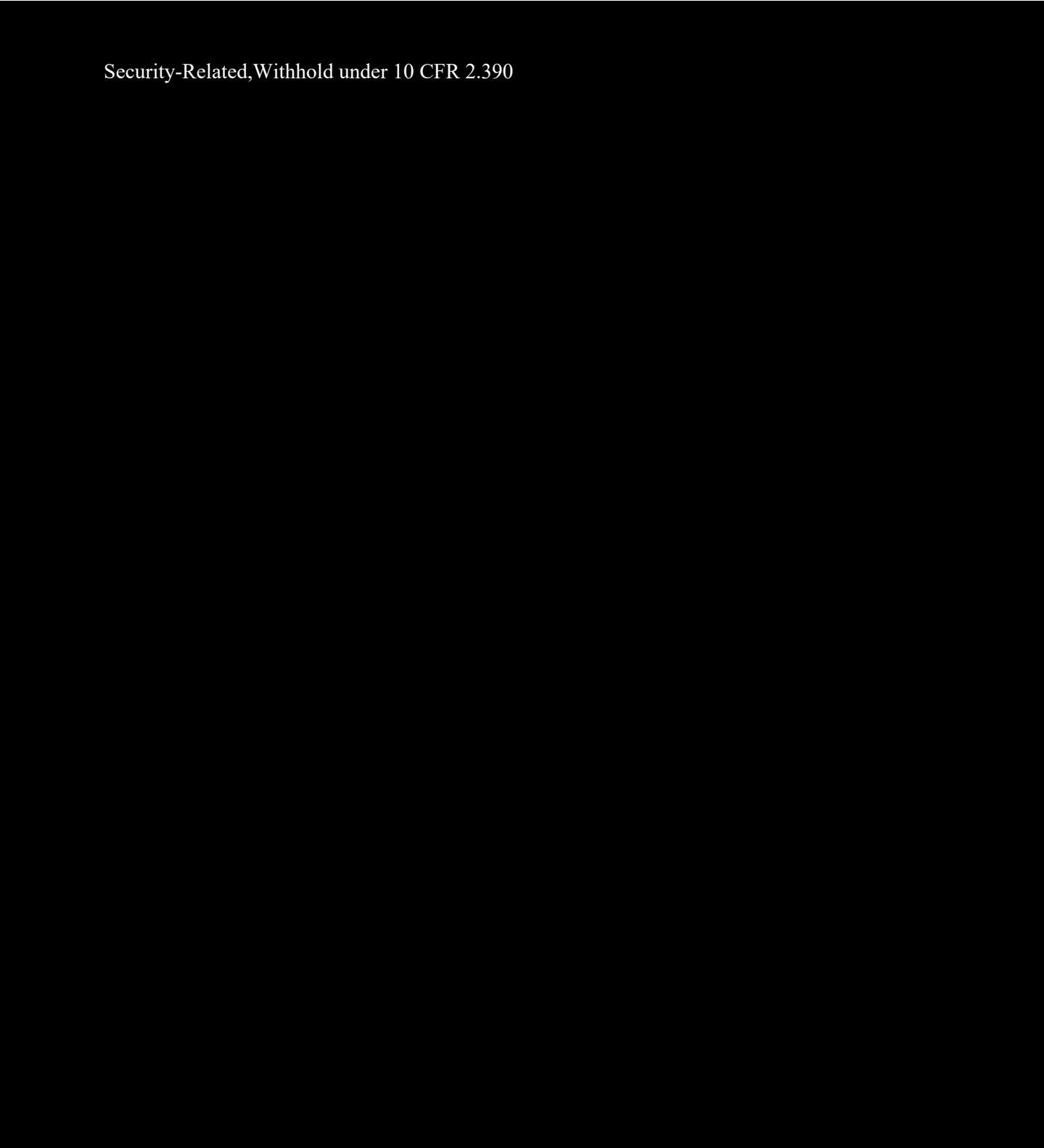
[illegible]CONTAINMENT MECHANICAL AND  
ELECTRICAL PENETRATIONS TYPICAL  
DETAILS

CATAWBA NUCLEAR STATION

Figure 3.8.2-3


**Figure 3-262. Containment Vessel And Cylinder Plate Layout And Penetration Location**

Security-Related, Withhold under 10 CFR 2.390



**Figure 3-263. Containment Vessel And Cylinder Plate Layout And Penetration Location**

Security-Related, Withhold under 10 CFR 2.390




**Figure 3-264. Containment Vessel Penetration Details**

SECURITY-RELATED INFORMATION WITHHOLD UNDER 10 CFR 2.390



**Figure 3-265. Containment Vessel Base Linear Plate Embedded Items**

Security-Related, Withhold under 10 CFR 2.390



**Figure 3-266. Containment Vessel Base Linear Plate Test Channel Layout**

Security-Related, Withhold under 10 CFR 2.390


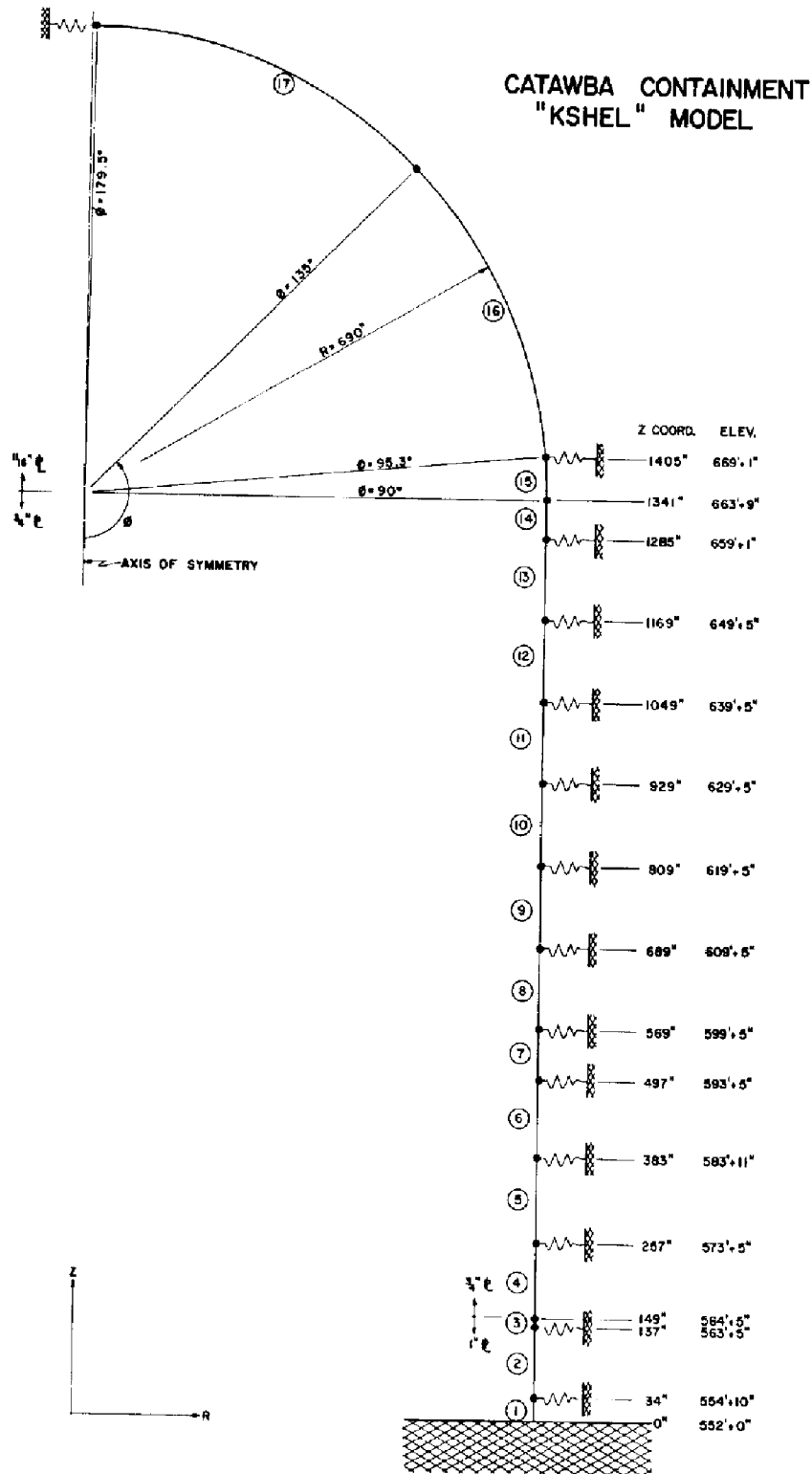


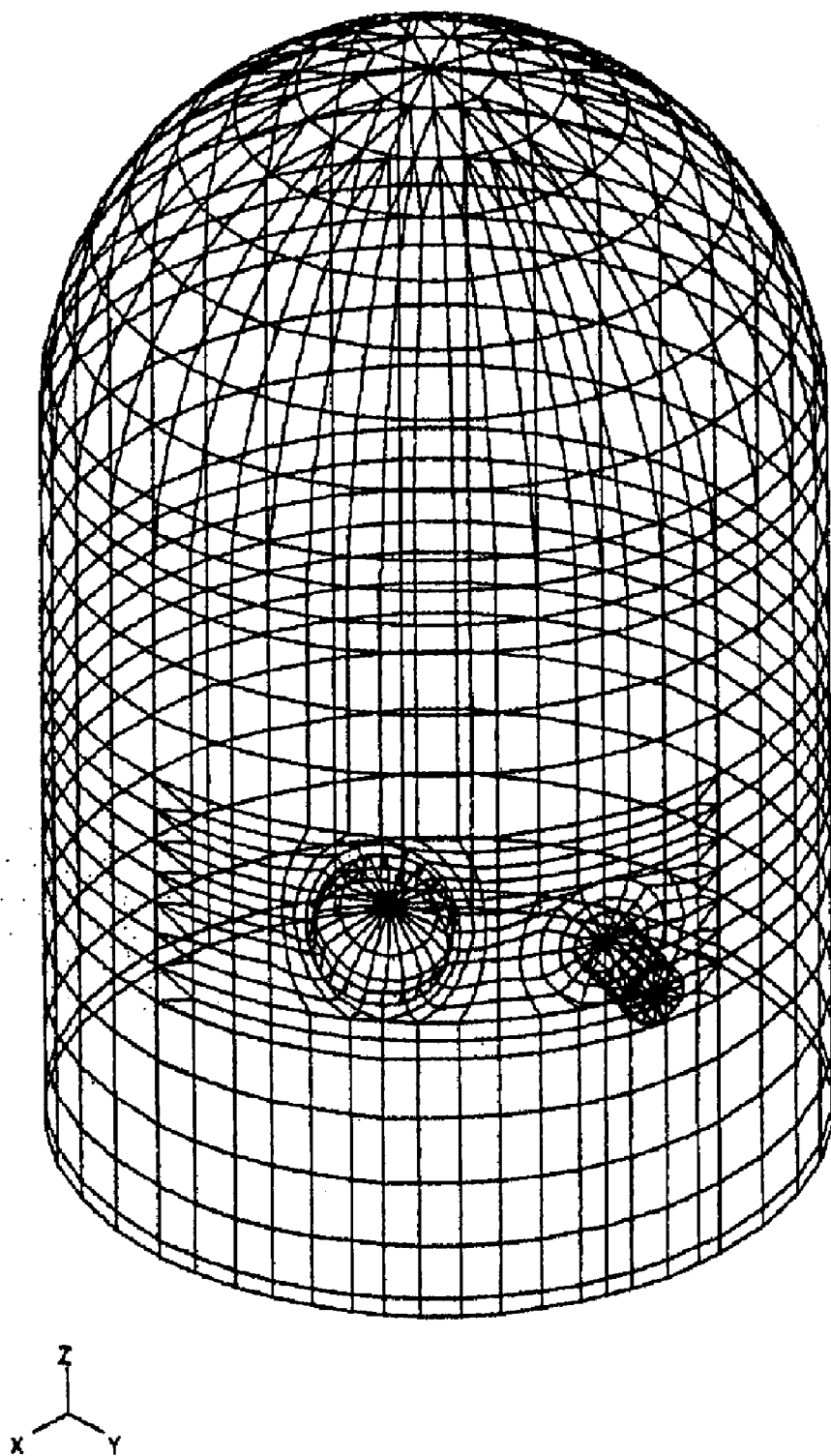


Figure 3-267. Containment Vessel Model For "KSHEL" Analysis



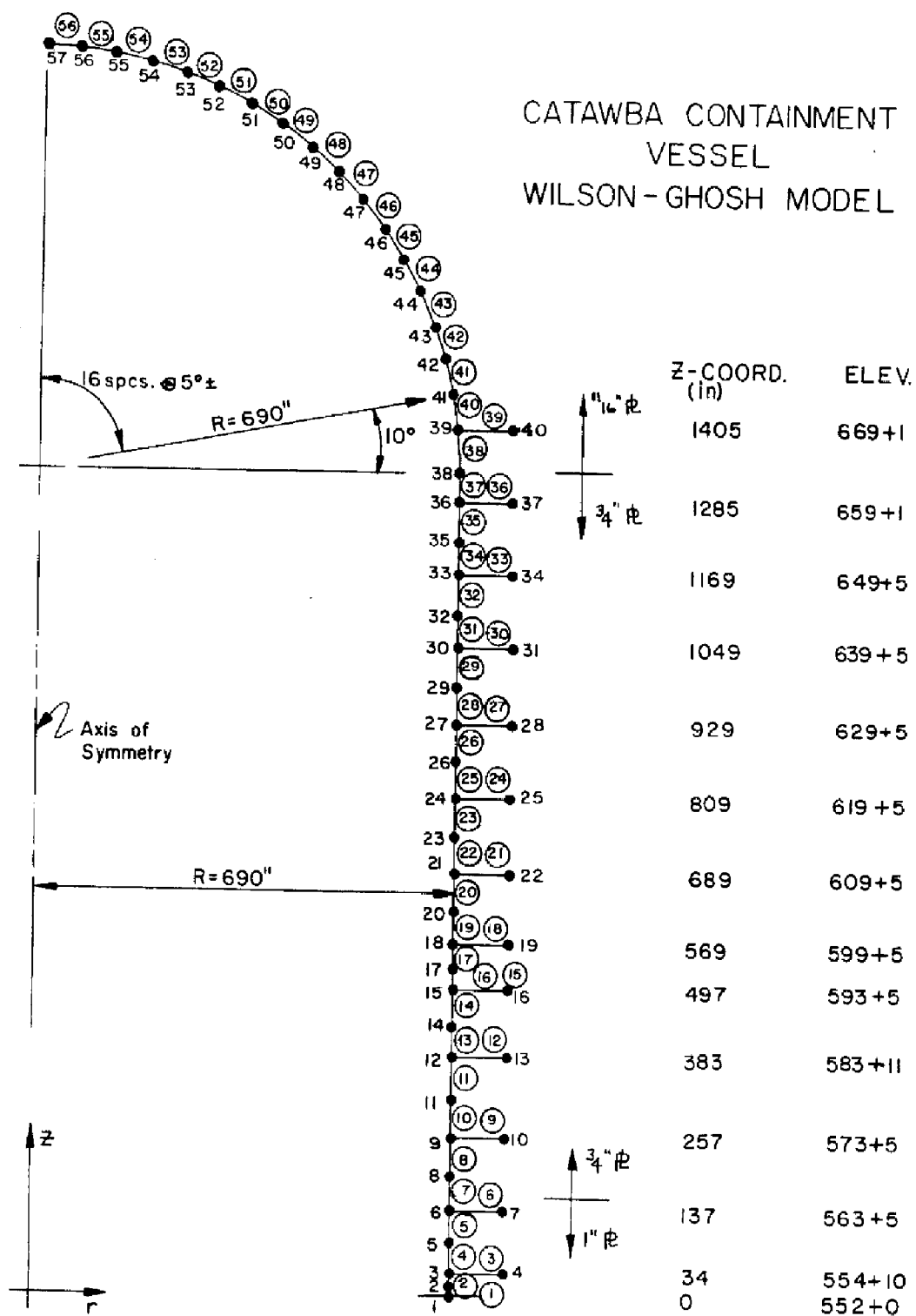
(22 OCT 2001)

Figure 3-268. Containment Vessel Model For “Stardyne” Localized Analysis



(22 OCT 2001)

Figure 3-269. Containment Vessel Model For "Wilson-Ghosh" Analysis



(22 OCT 2001)

Figure 3-270. TMD Compartment Pressure Transients - Ice Condenser Region

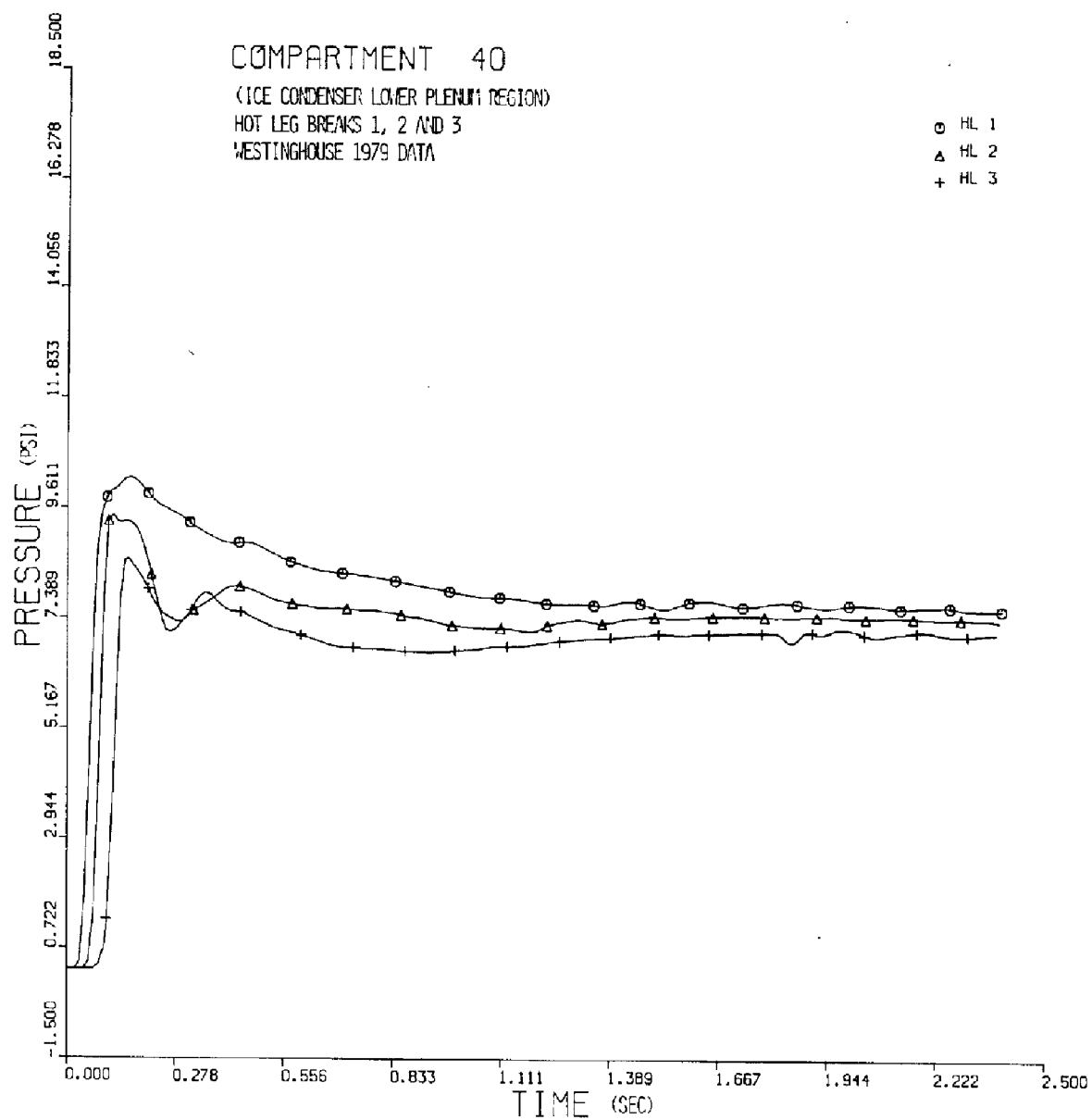


Figure 3-271. TMD Compartment Pressure Transients - Ice Condenser Region

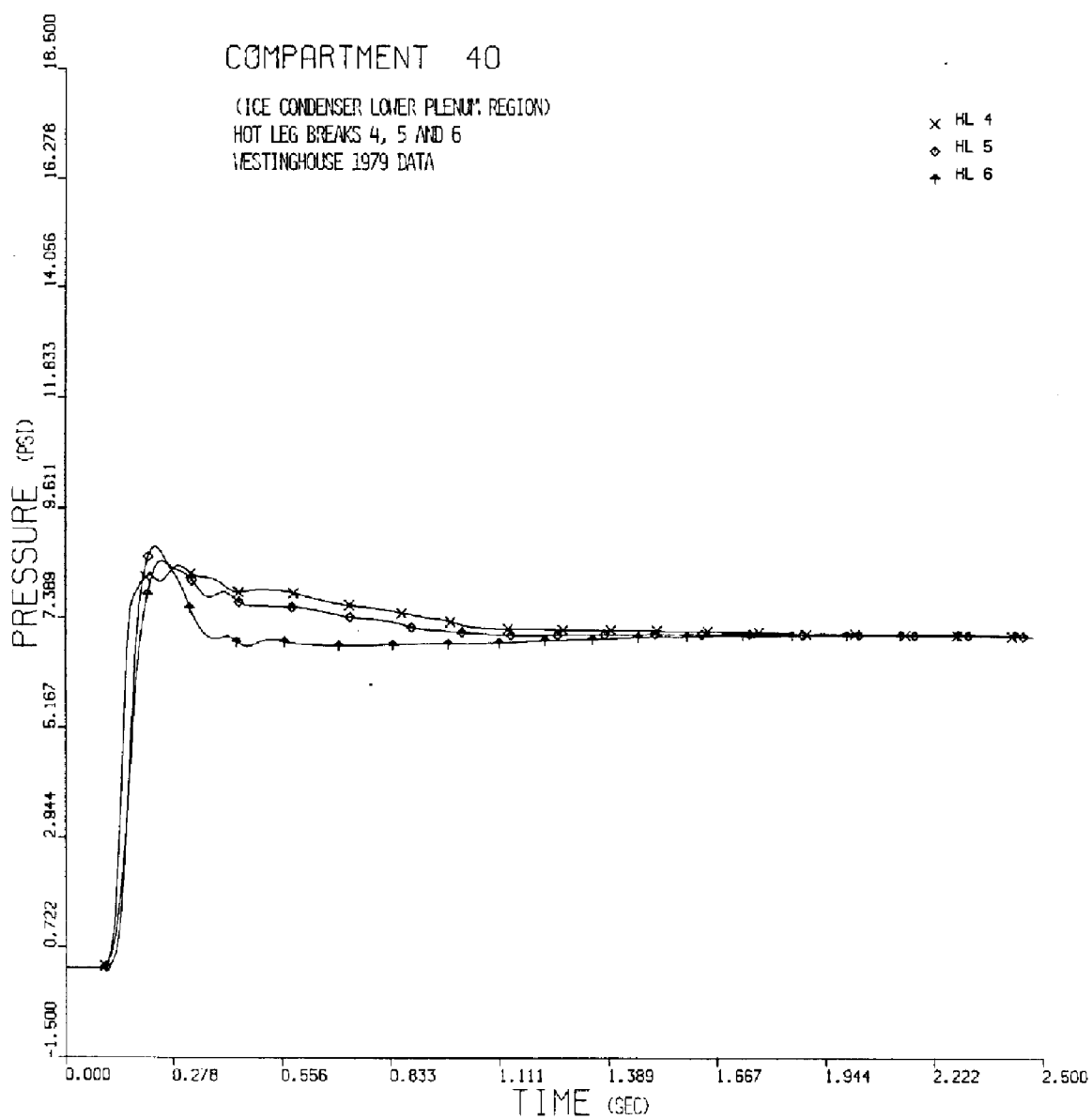
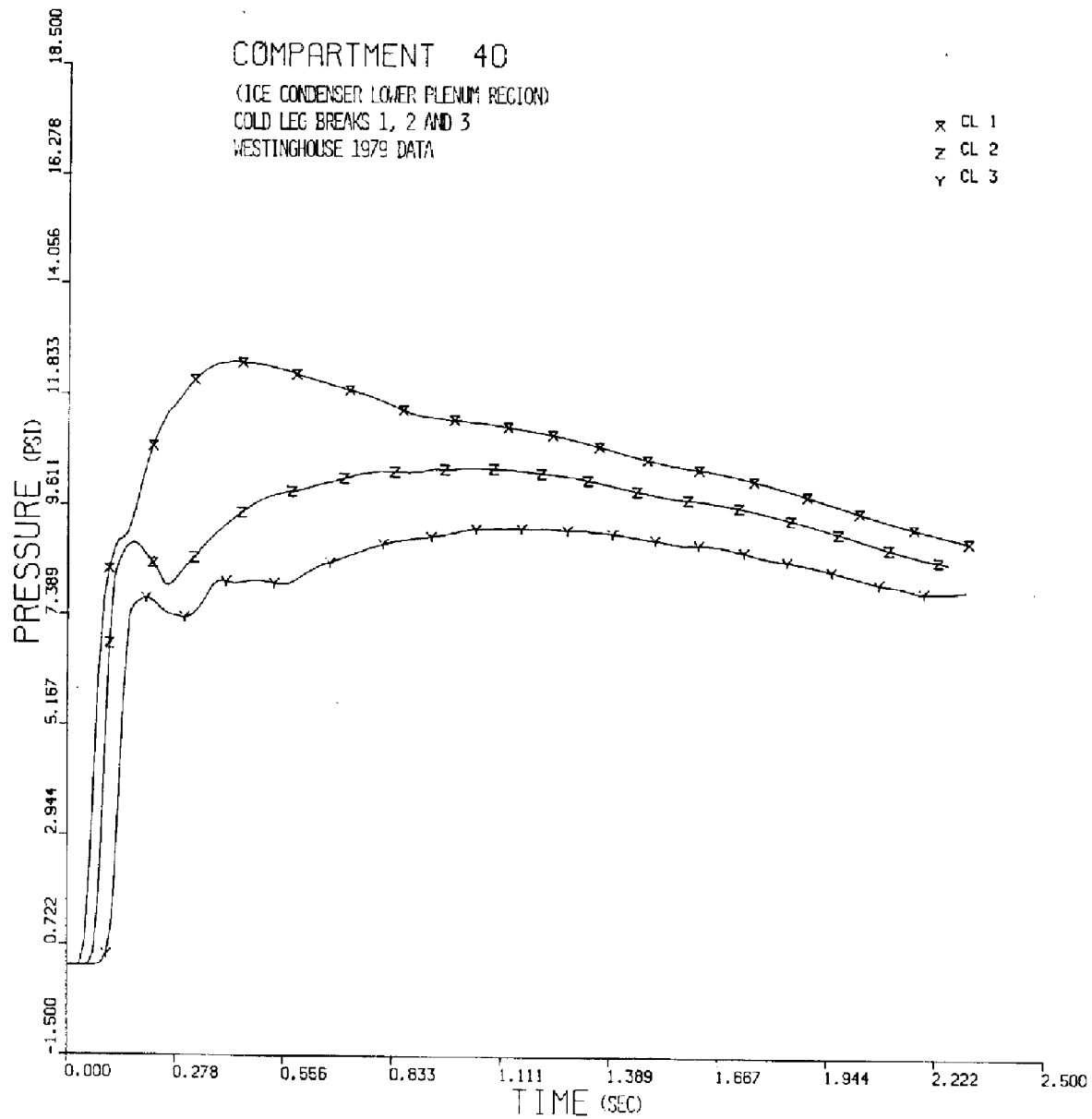


Figure 3-272. TMD Compartment Pressure Transients - Ice Condenser Region



(22 OCT 2001)

Figure 3-273. TMD Compartment Pressure Transients - Ice Condenser Region

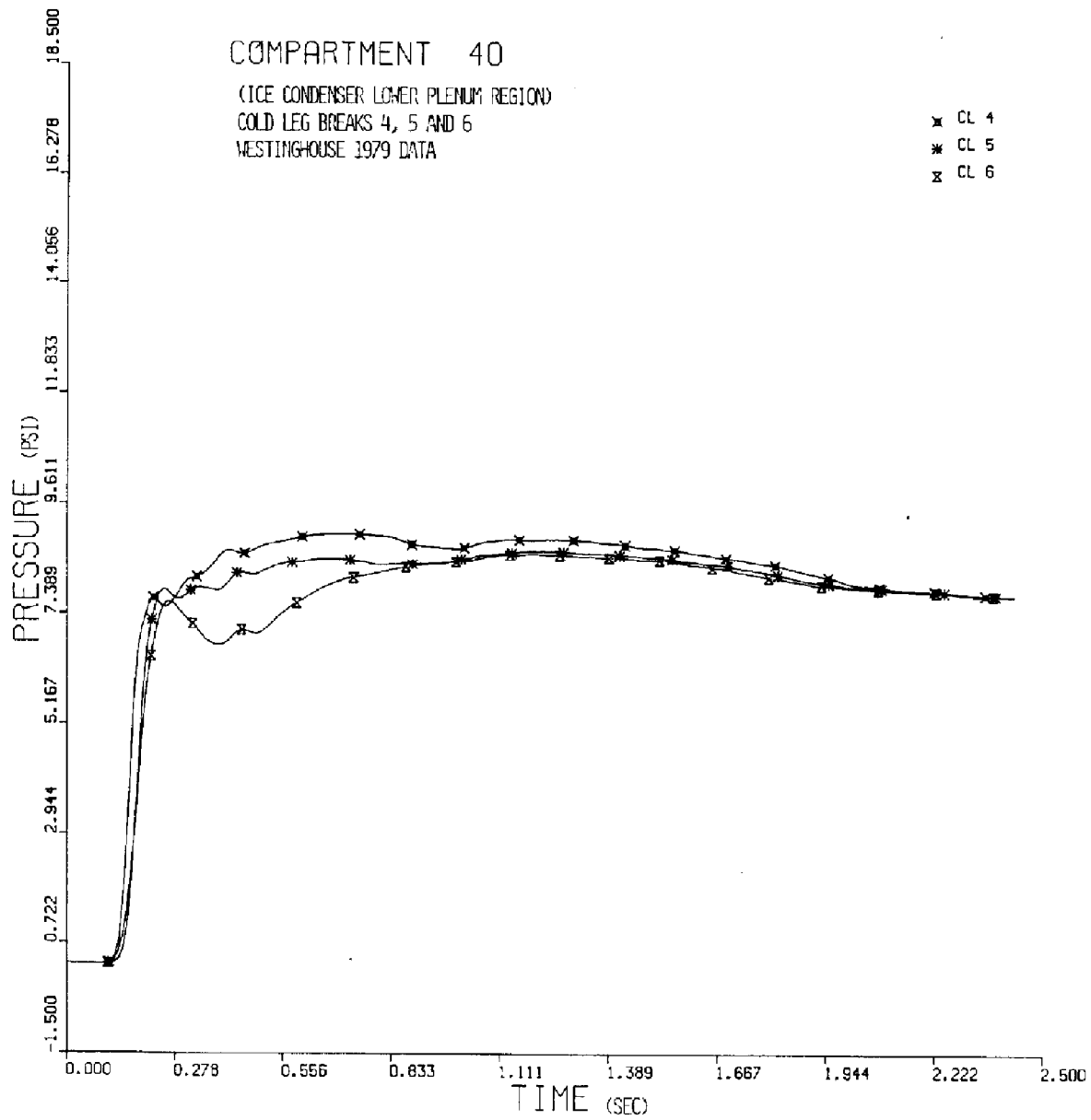
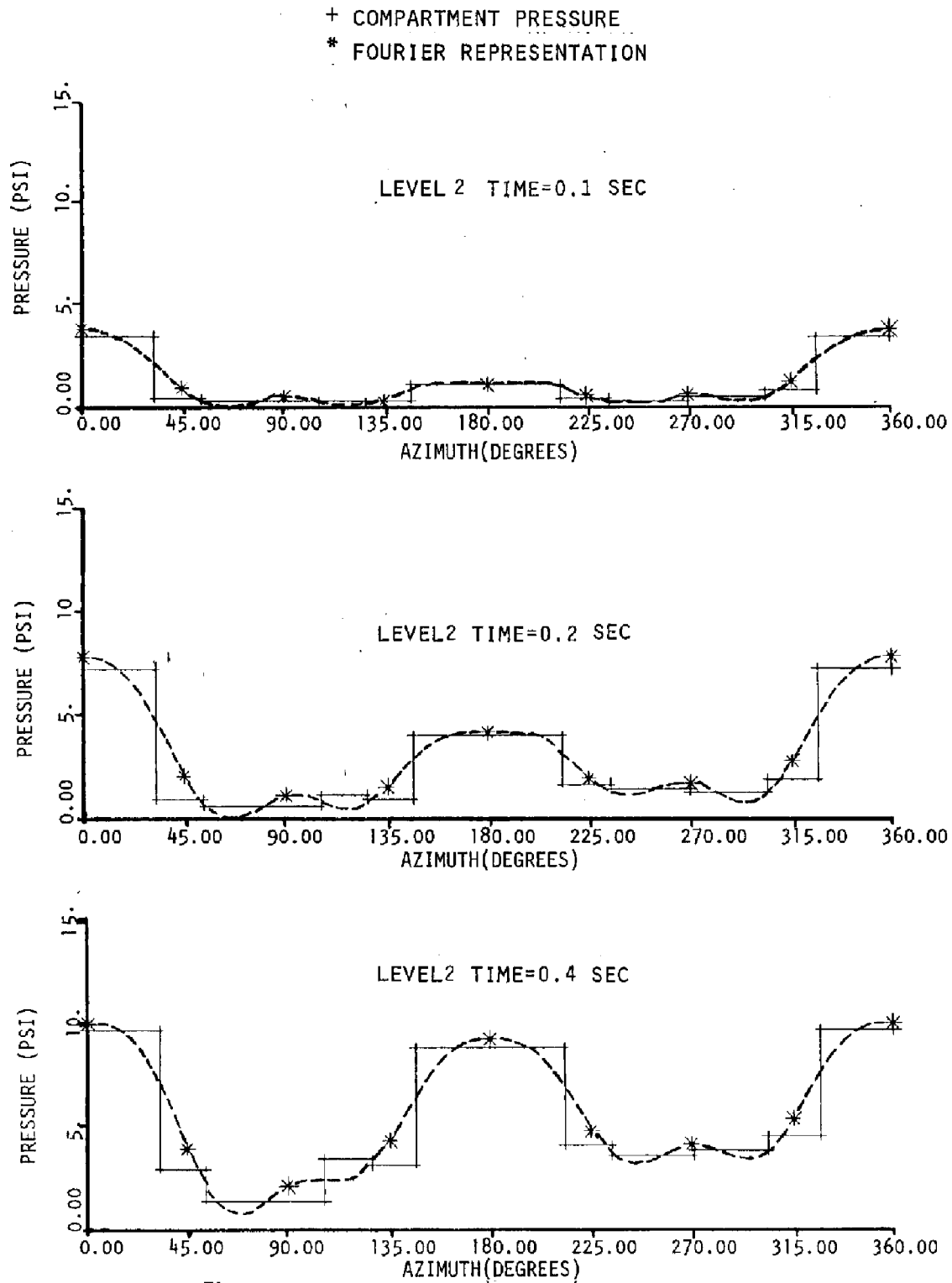


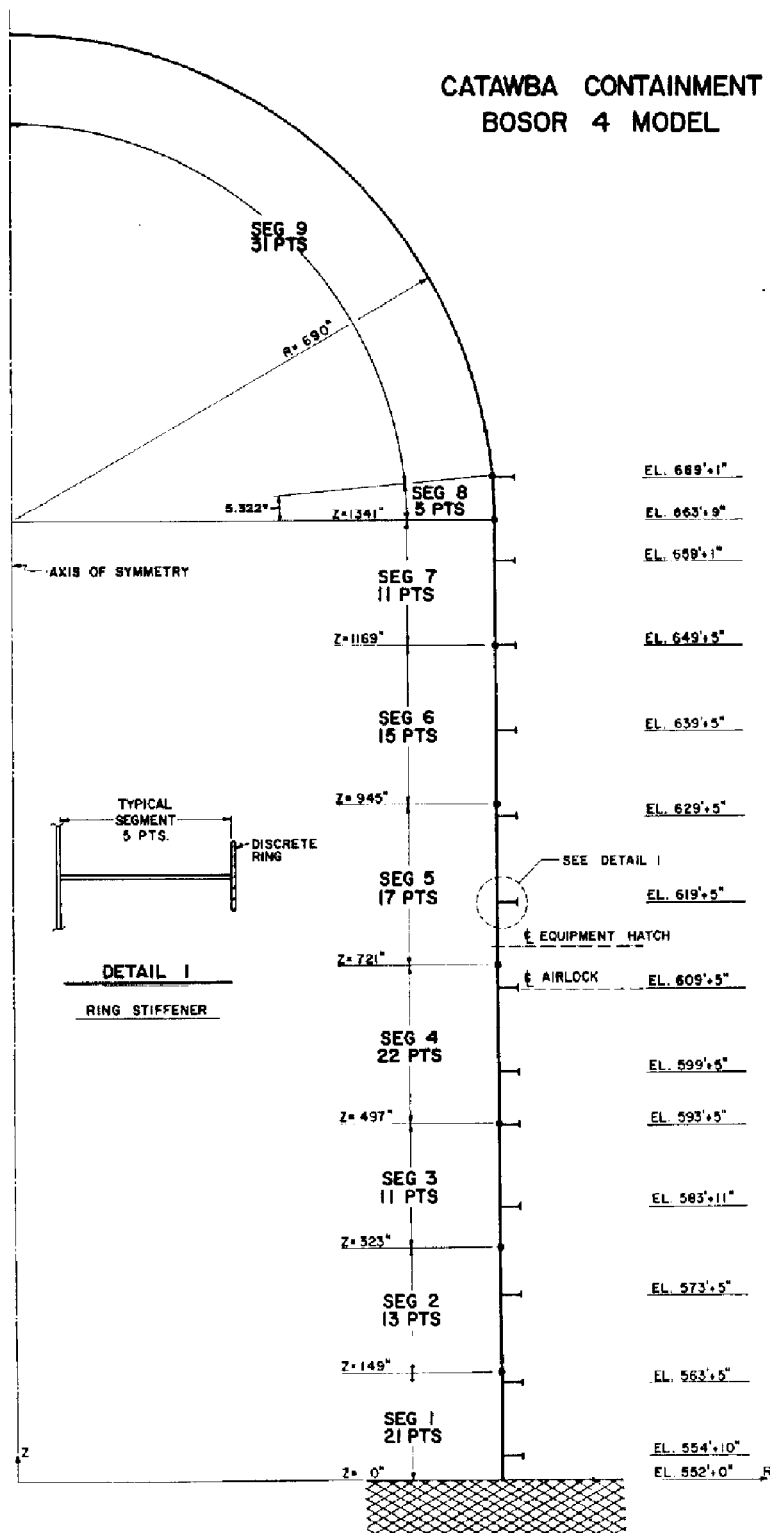
Figure 3-274. Fourier Series Representation Of Transient Pressures



(22 OCT 2001)



Figure 3-275. Containment Vessel Model For "BOSOR4" Stability Analysis



(22 OCT 2001)

Figure 3-276. Localized "Nastran" Model For Stability Analysis

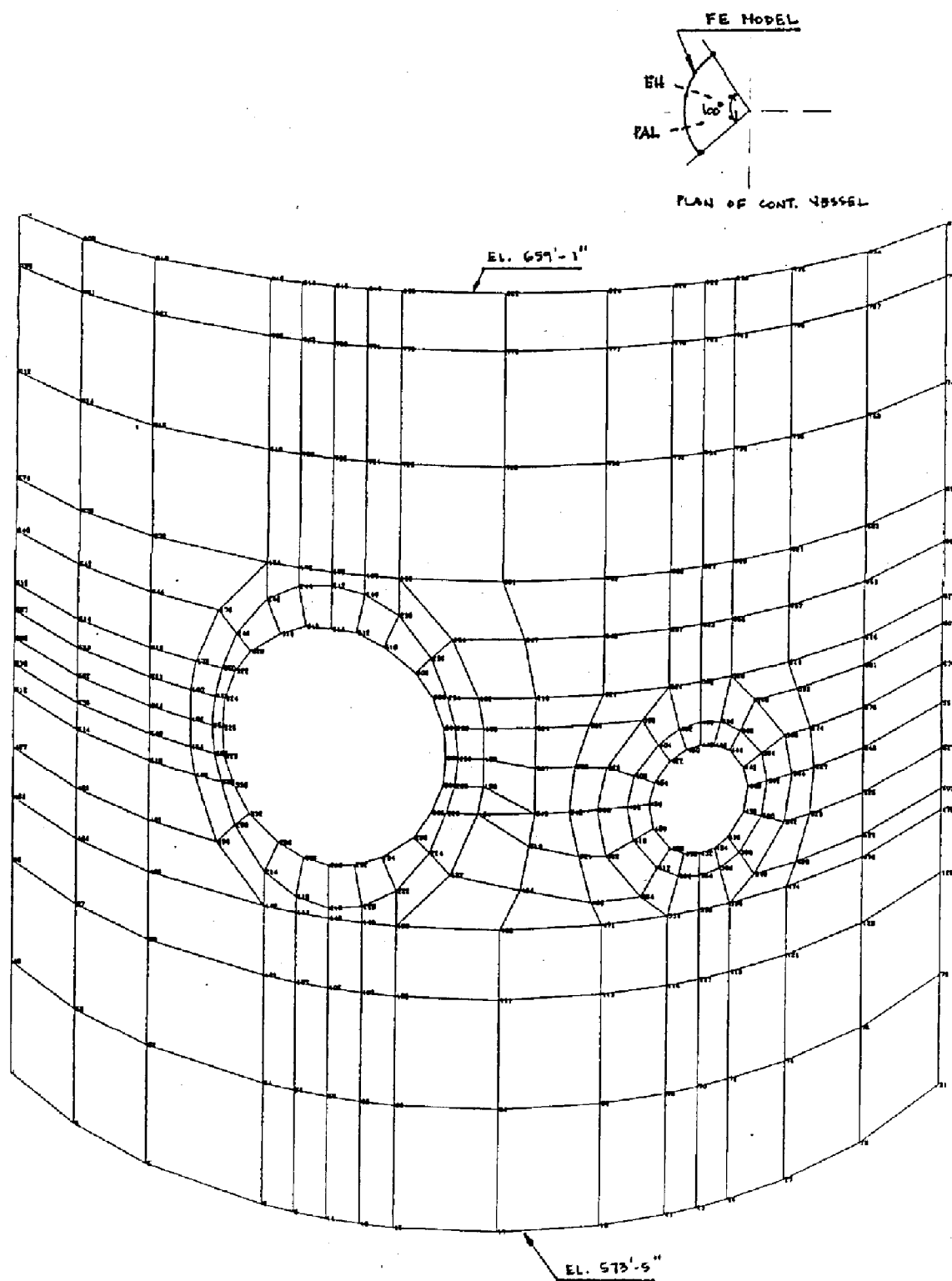
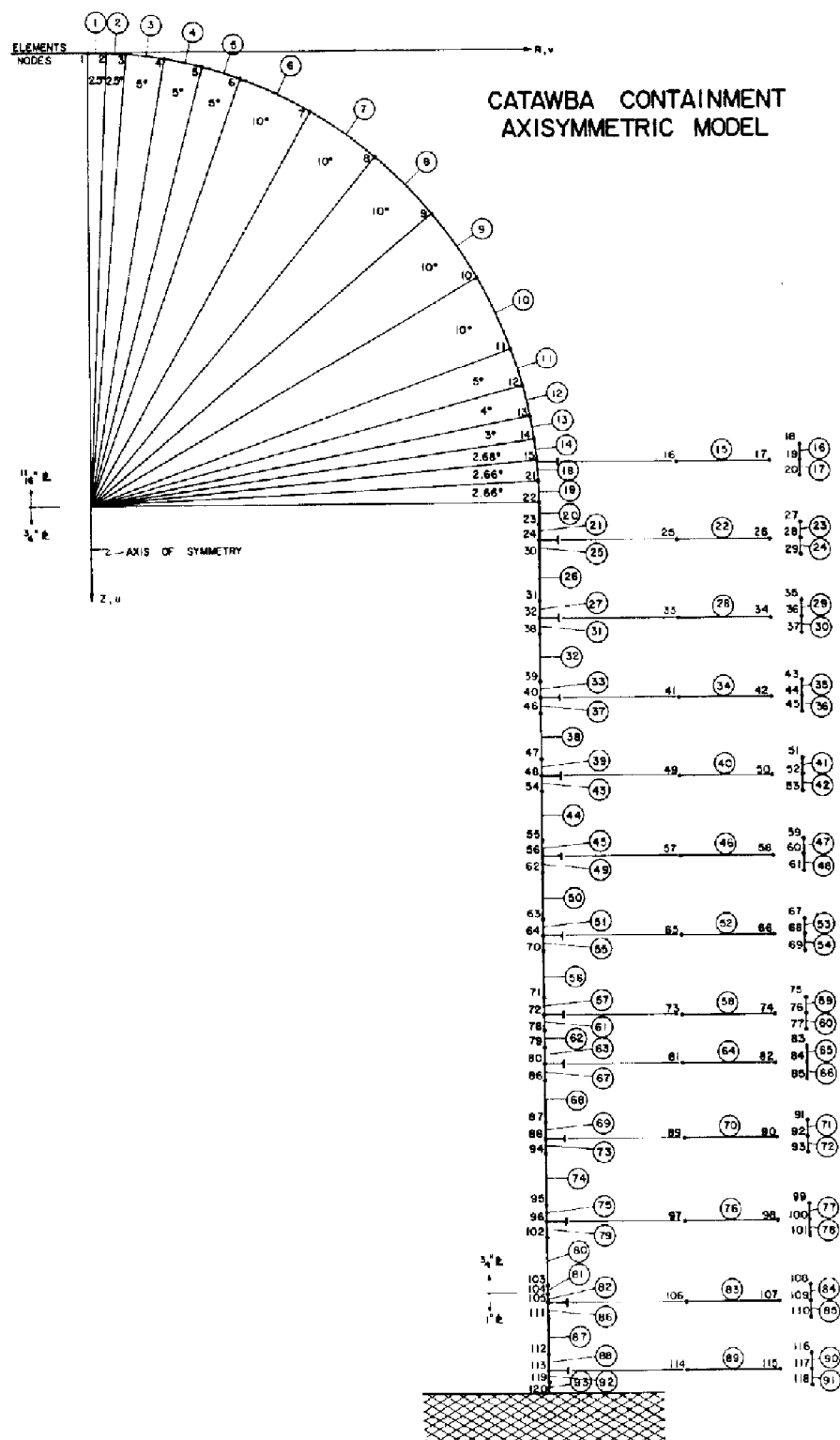
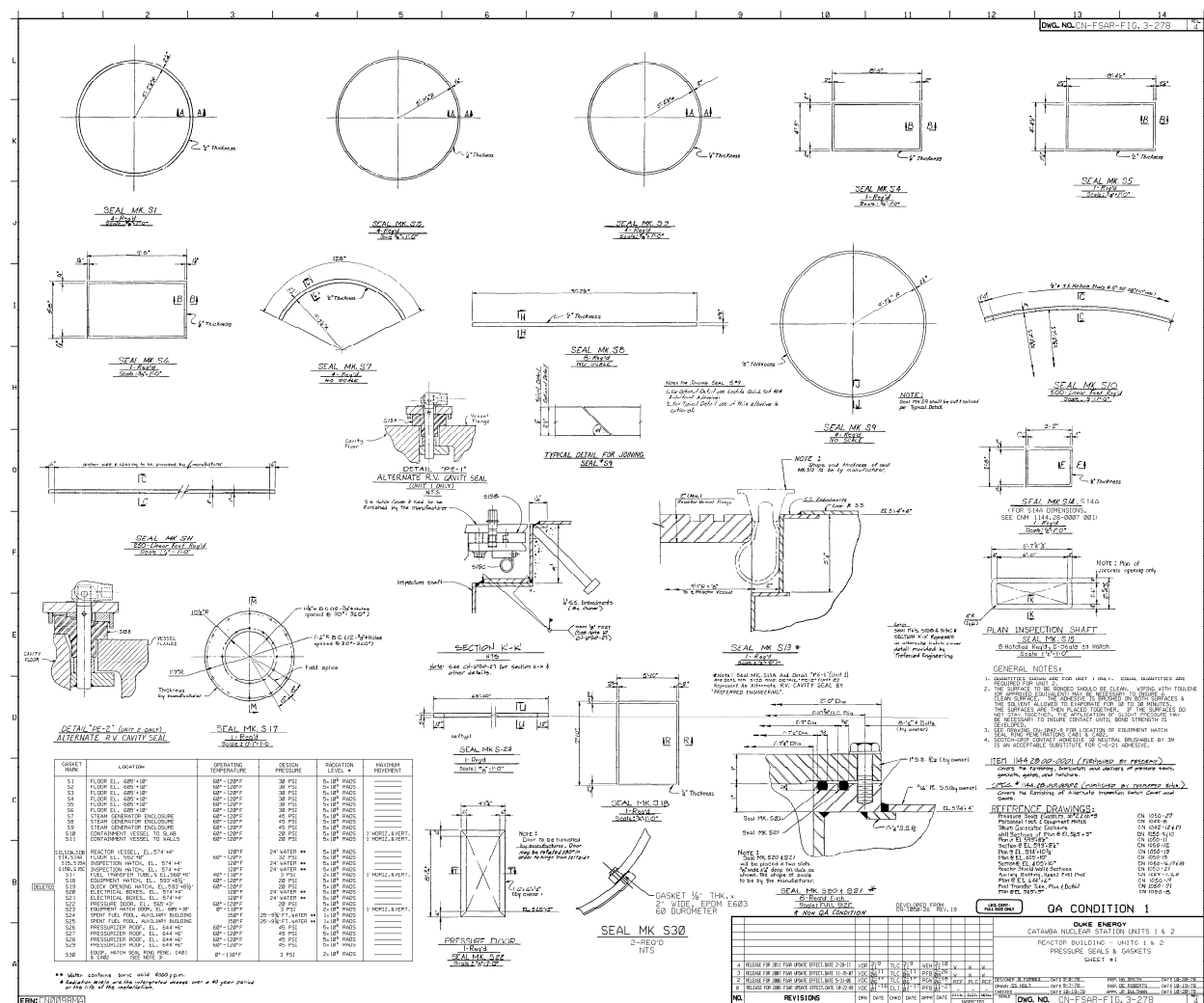


Figure 3-277. Containment Vessel Model For "Marc" Ultimate Capacity Analysis



**(21 OCT 2010)**



**(15 NOV 2007)**

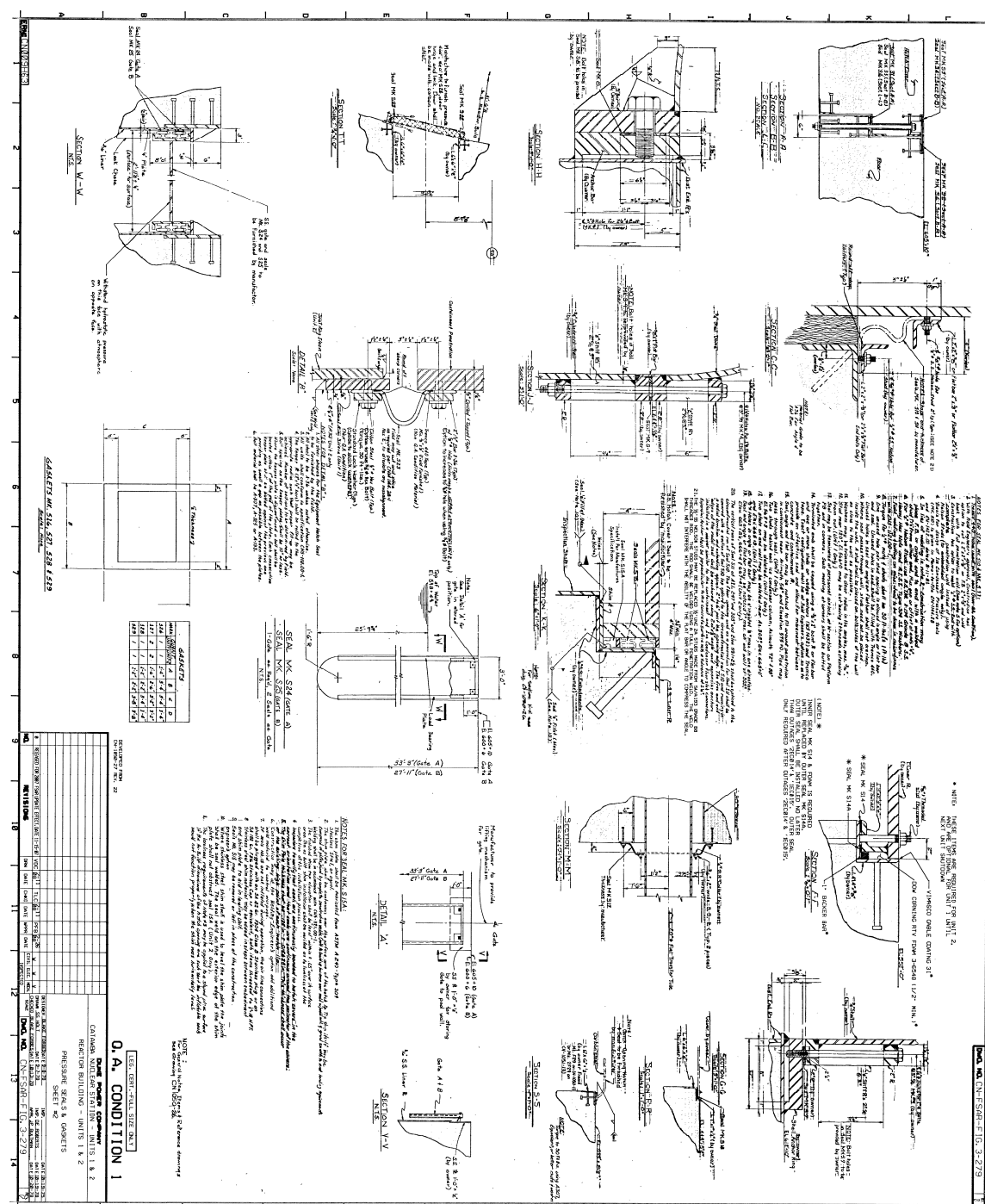


Figure 3-280. Catawba Operating Floor Elastic Model

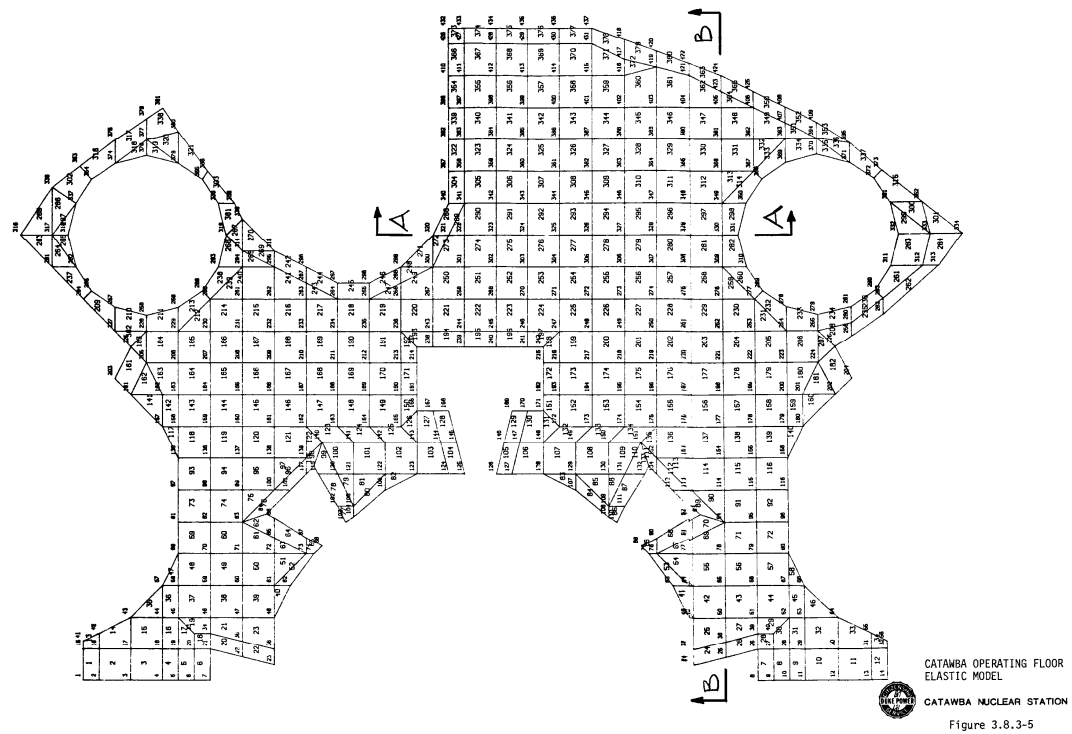


Figure 3-281. Catawba Operating Floor Element Model

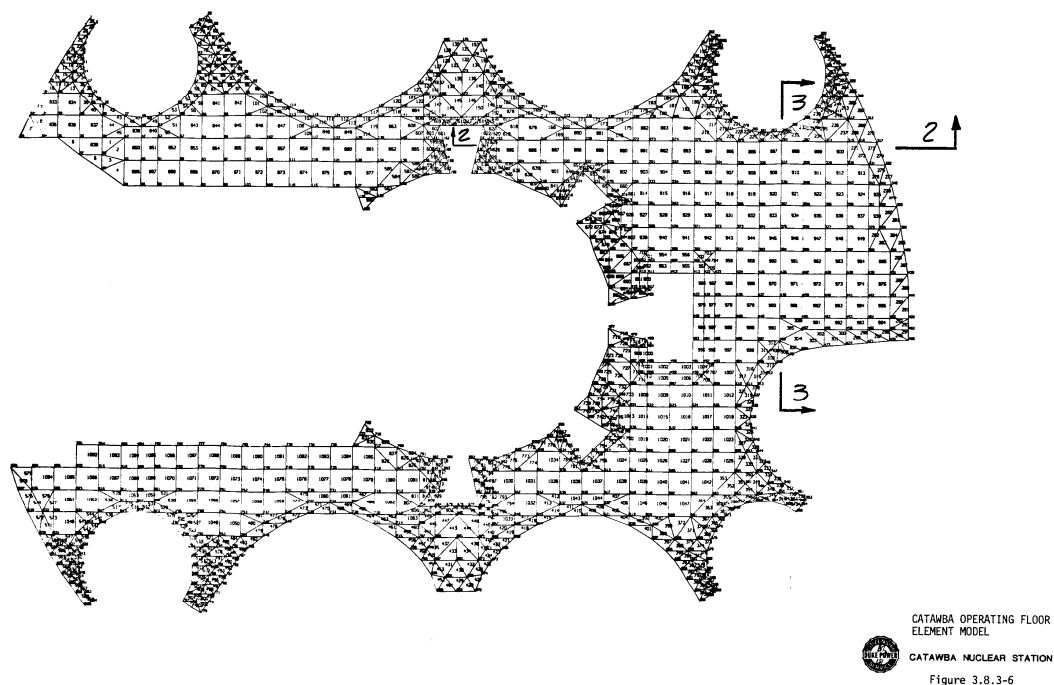
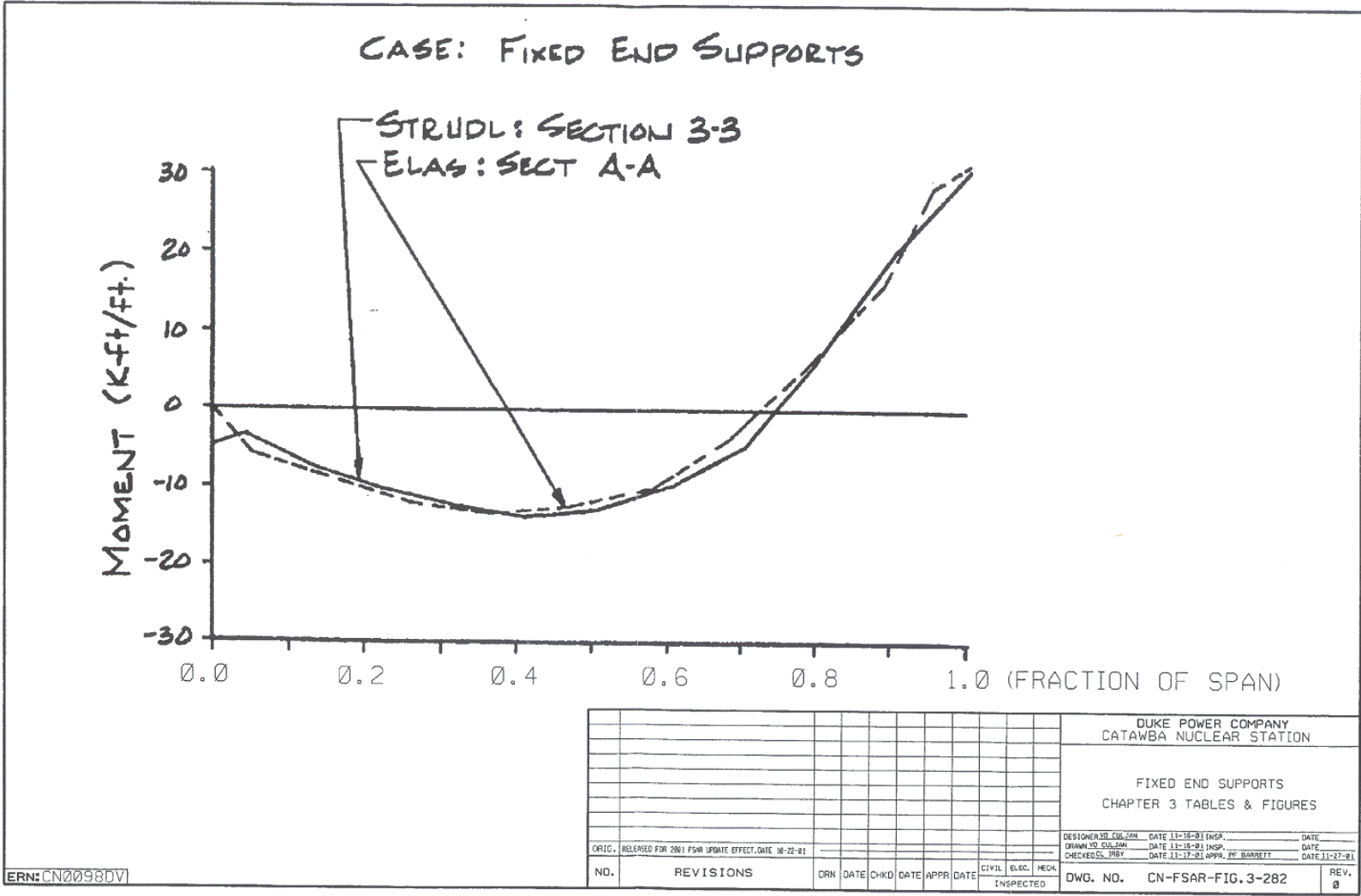


Figure 3-282. Fixed End Supports





UFSAR Figure 3-283 (Page 1 of 1)



Figure 3-284. Fixed End Supports

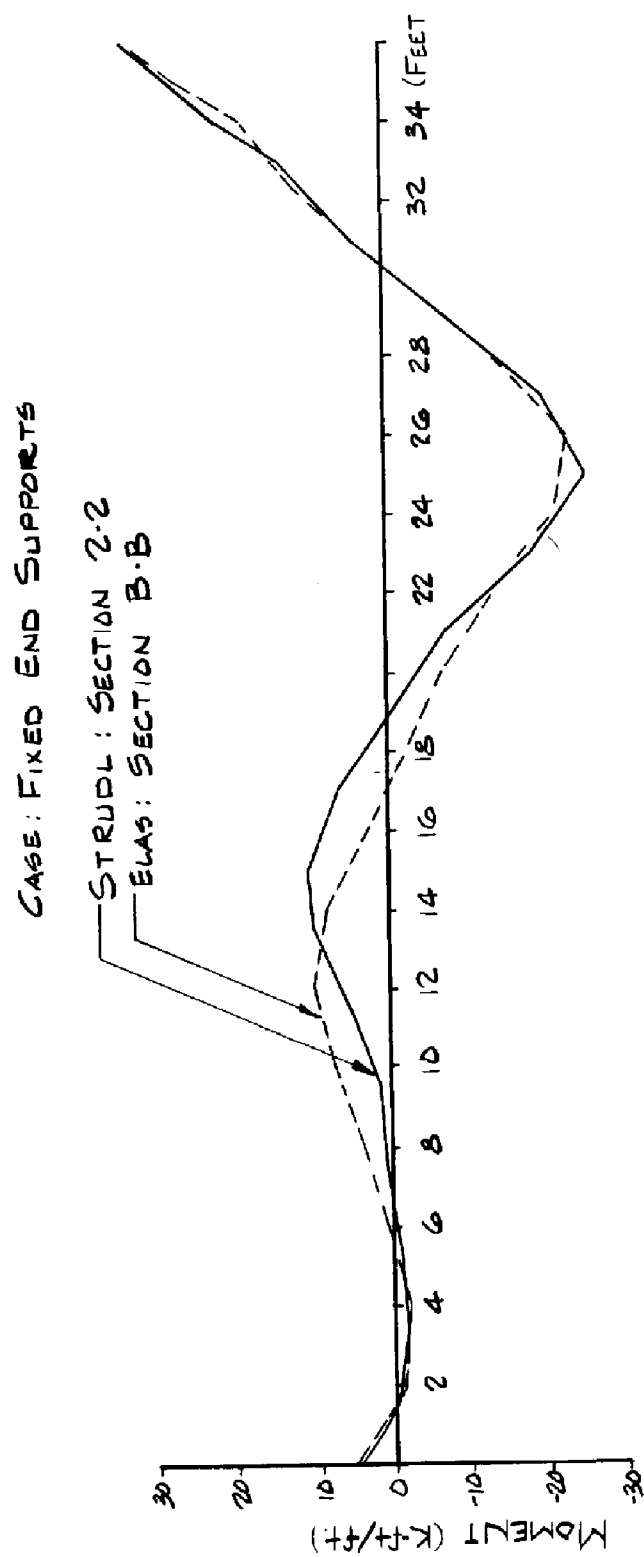
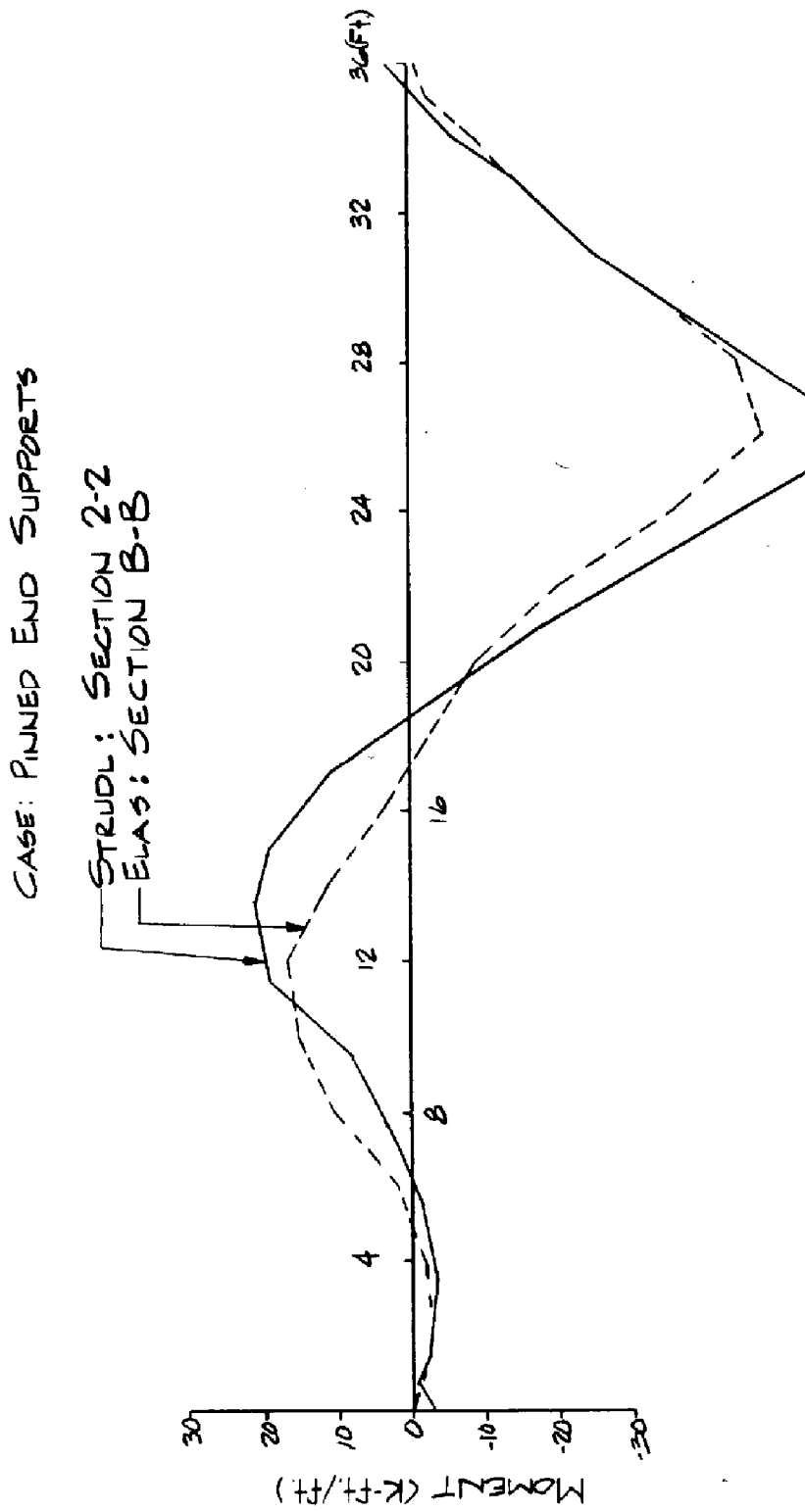
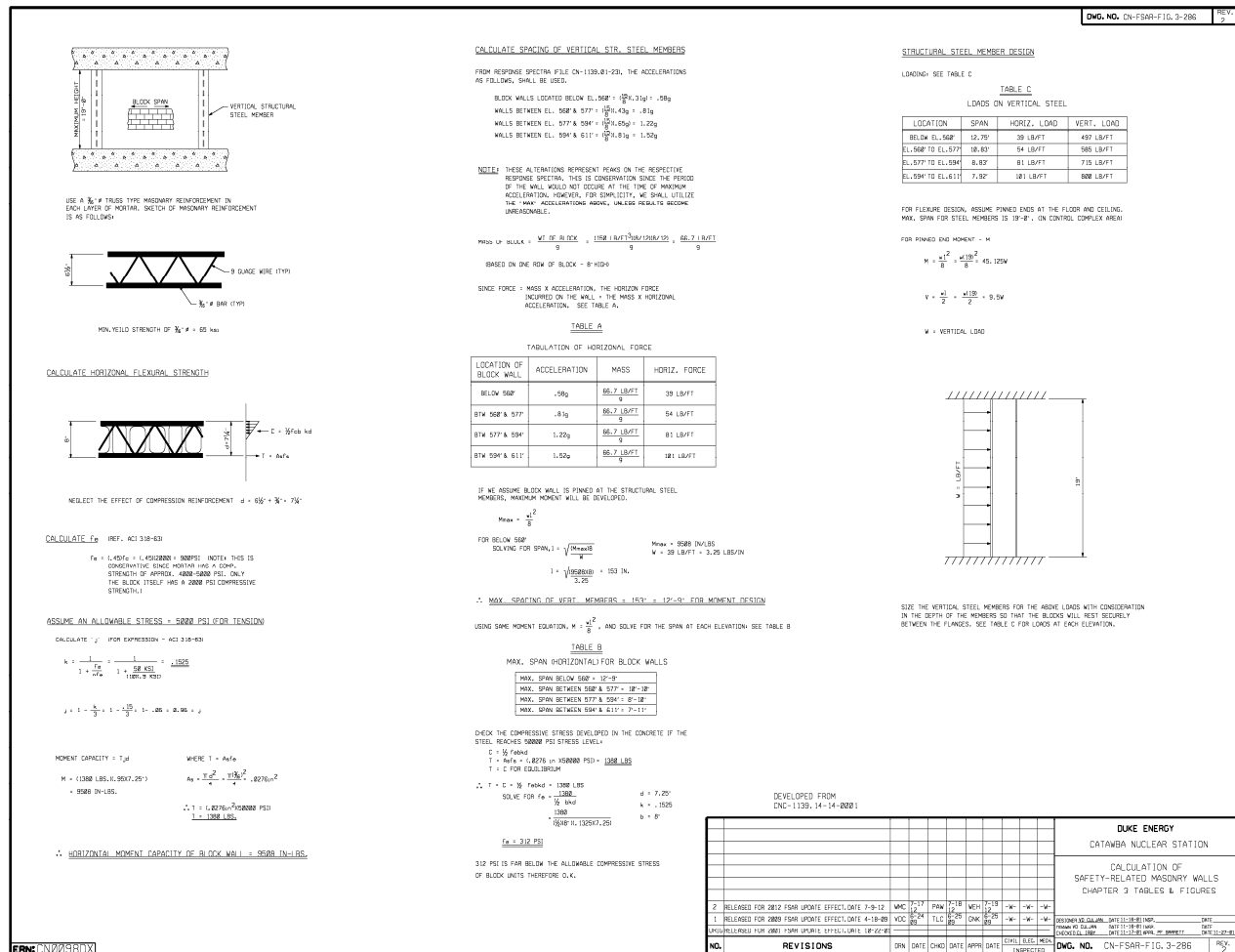


Figure 3-285. Pinned End Supports



(22 OCT 2001)

Figure 3-286. Calculation Of Safety-Related Masonry Walls



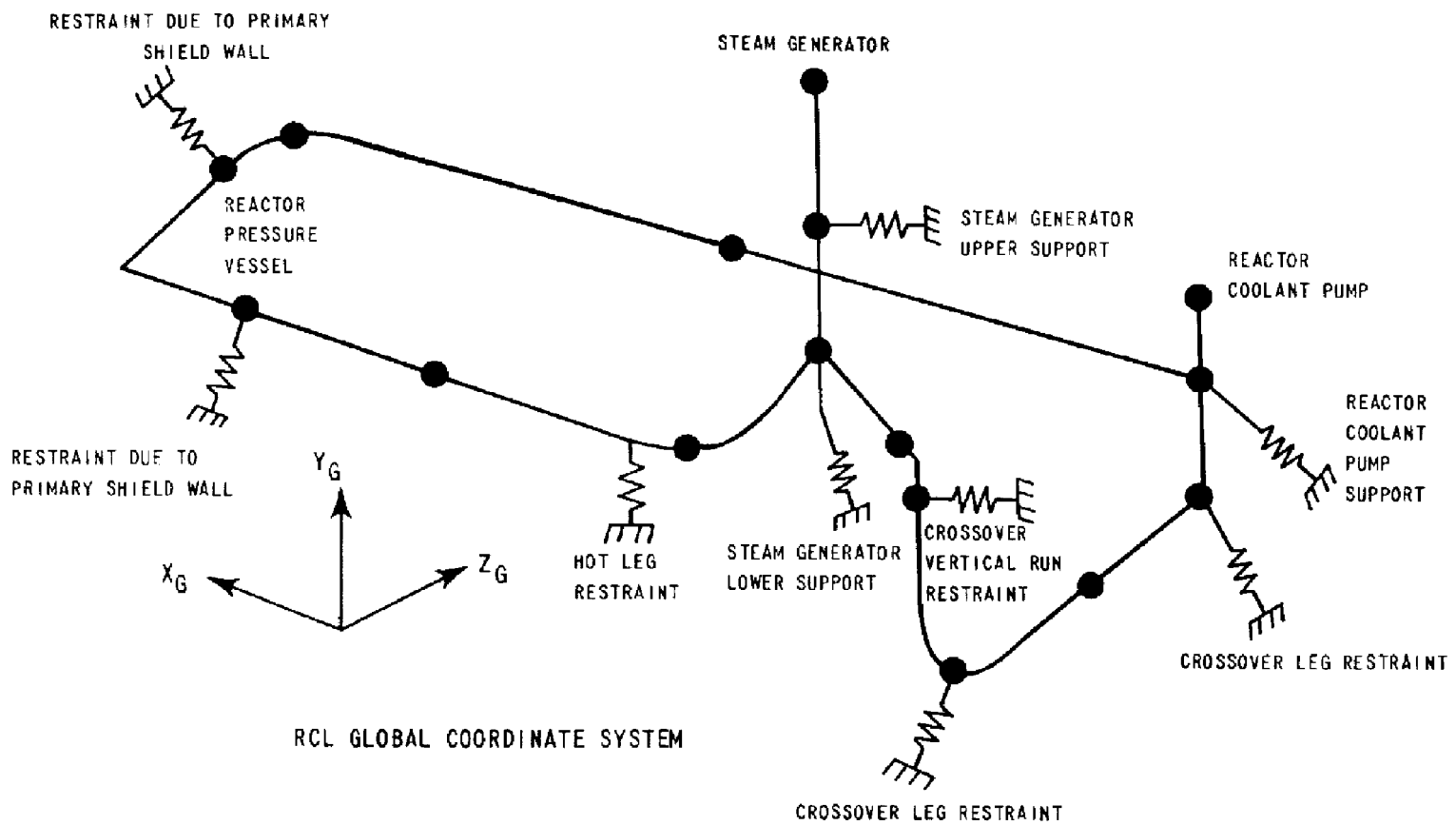


Figure 3-287. Reactor Coolant Loop Supports System Dynamic Structural Model

Figure 3-288. Through-Wall Thermal Gradients

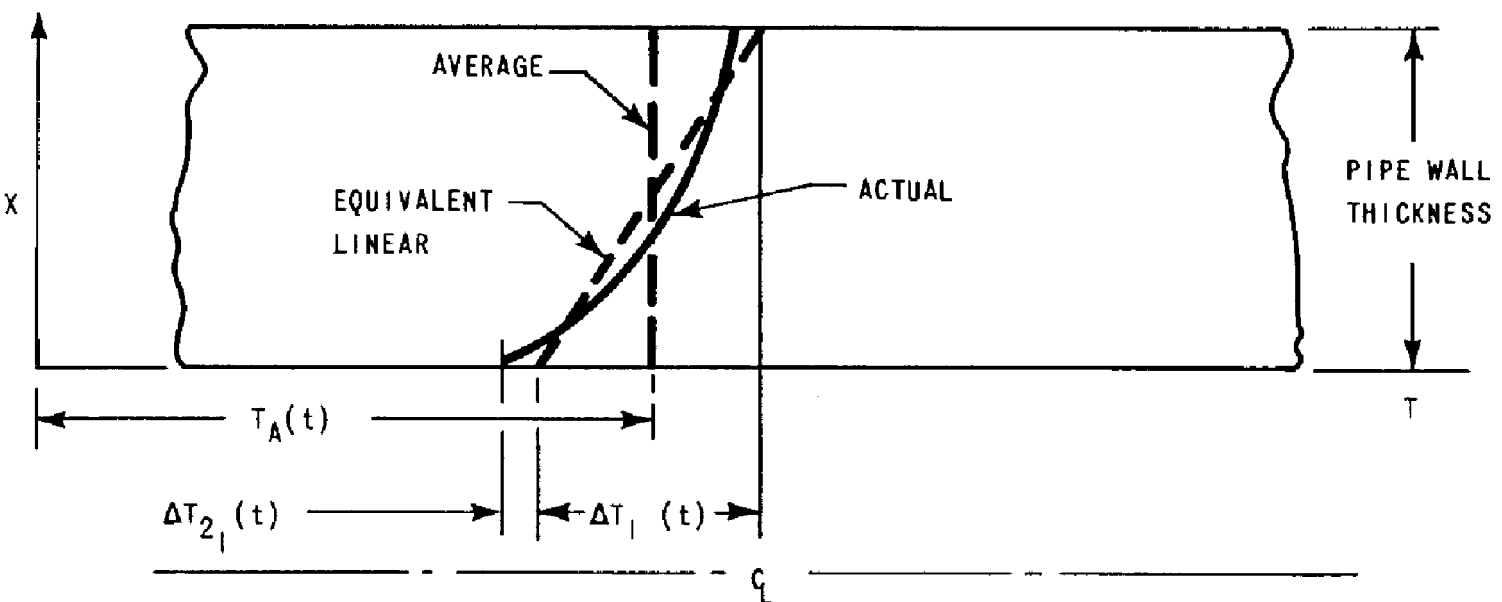
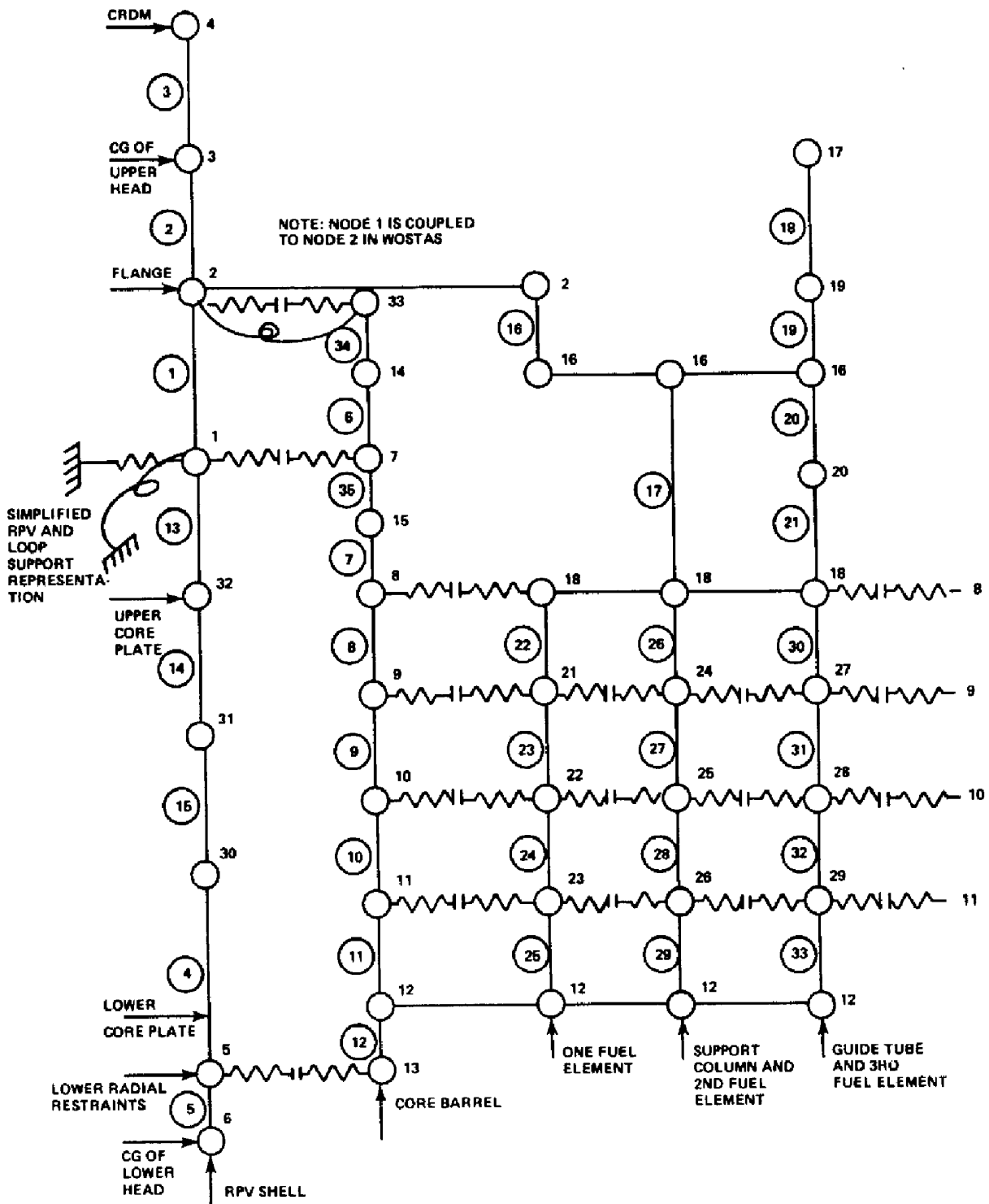


Figure 3-289. Reactor Internals Mathematical Model For DARI Variables



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Figure 3-290. Reactor Internals Mathematical Model For WOSTAS Variables

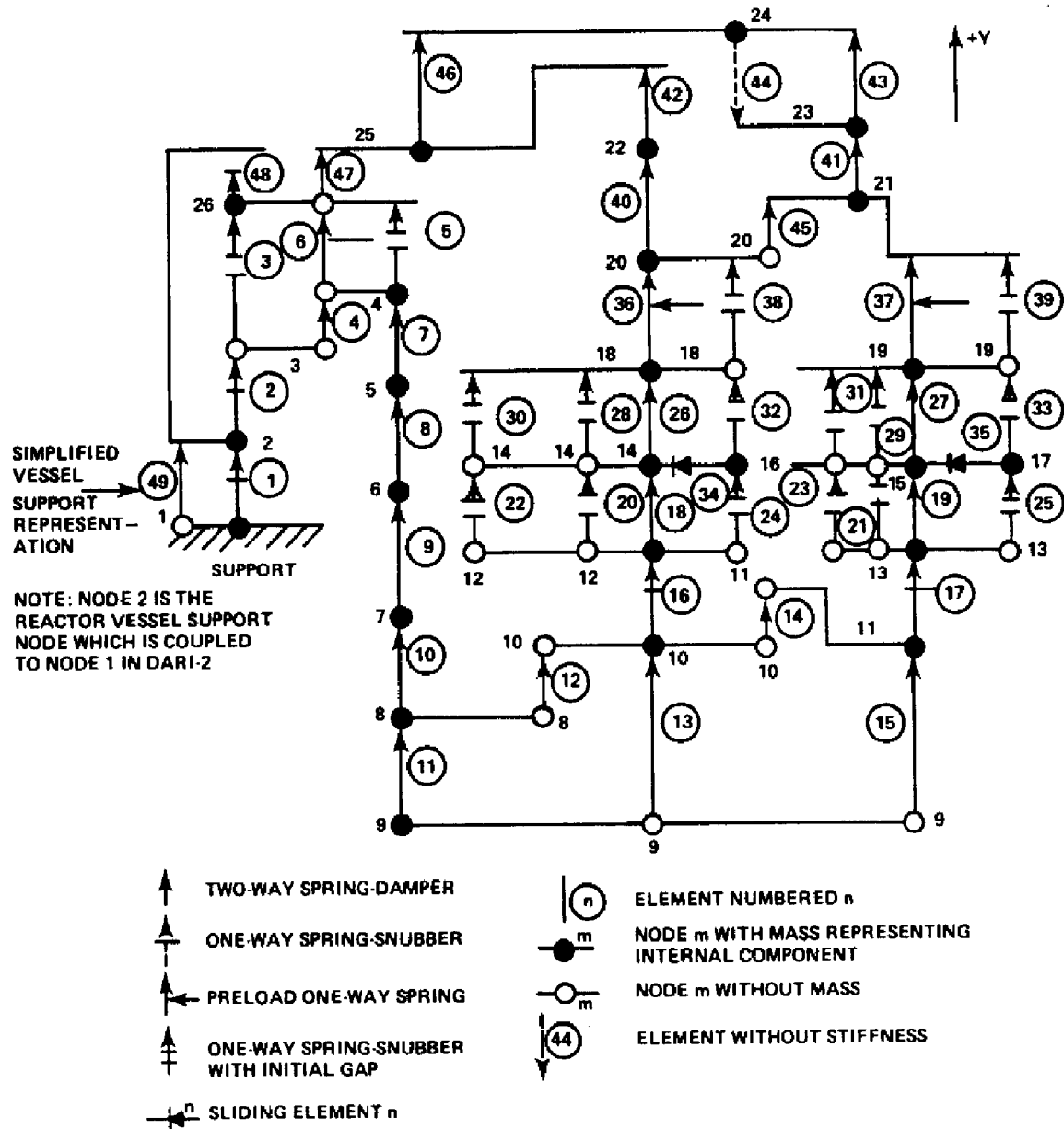




Figure 3-291. Pre And Post Hot Functional Examination Points For Internals And Integrity

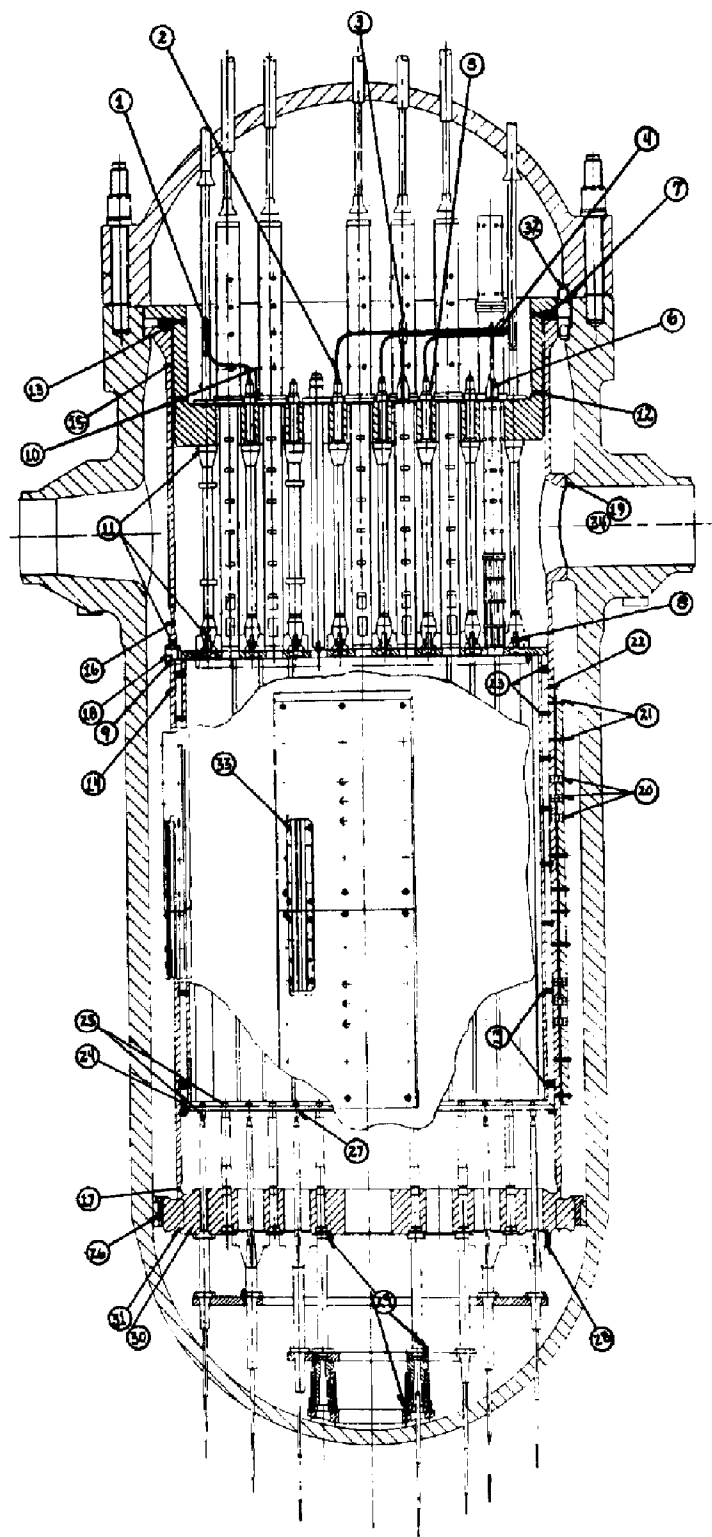


Figure 3-292. Auxiliary Building Auxiliary Feedwater System

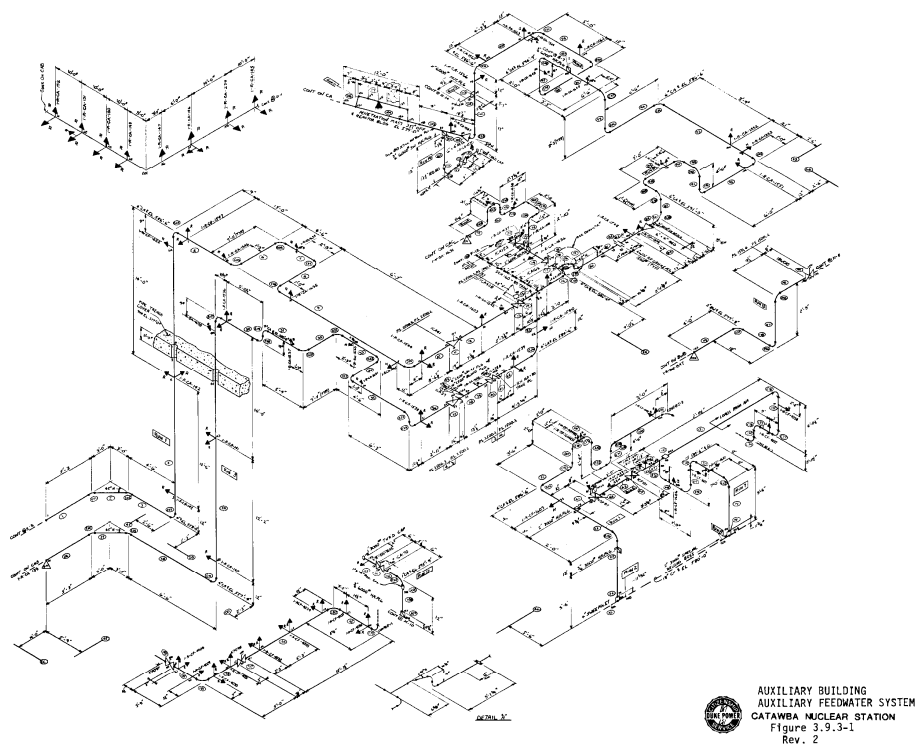


Figure 3-293. Full Length Control Rod Drive Mechanism

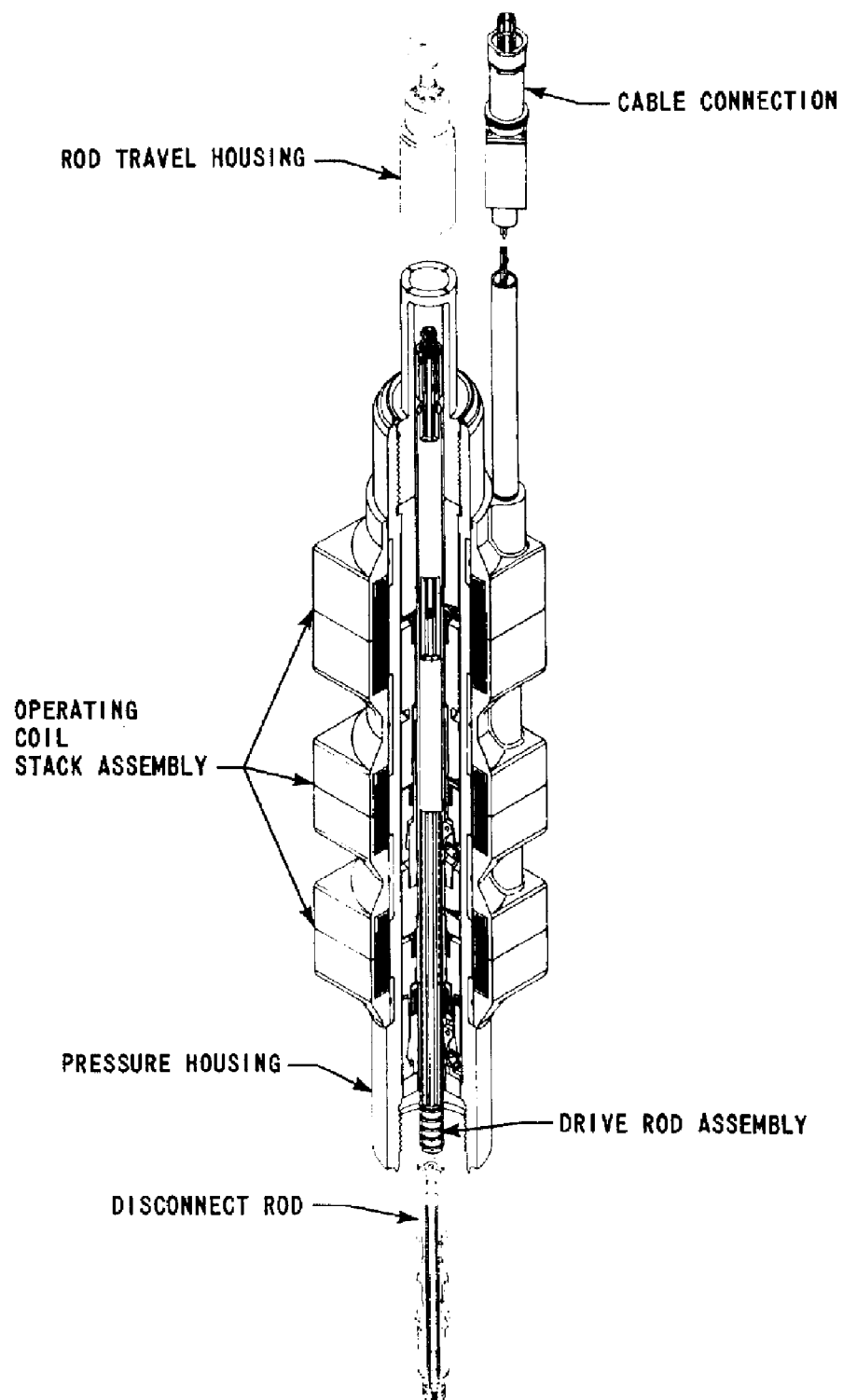


Figure 3-294. Full Length Control Rod Drive Mechanism Schematic

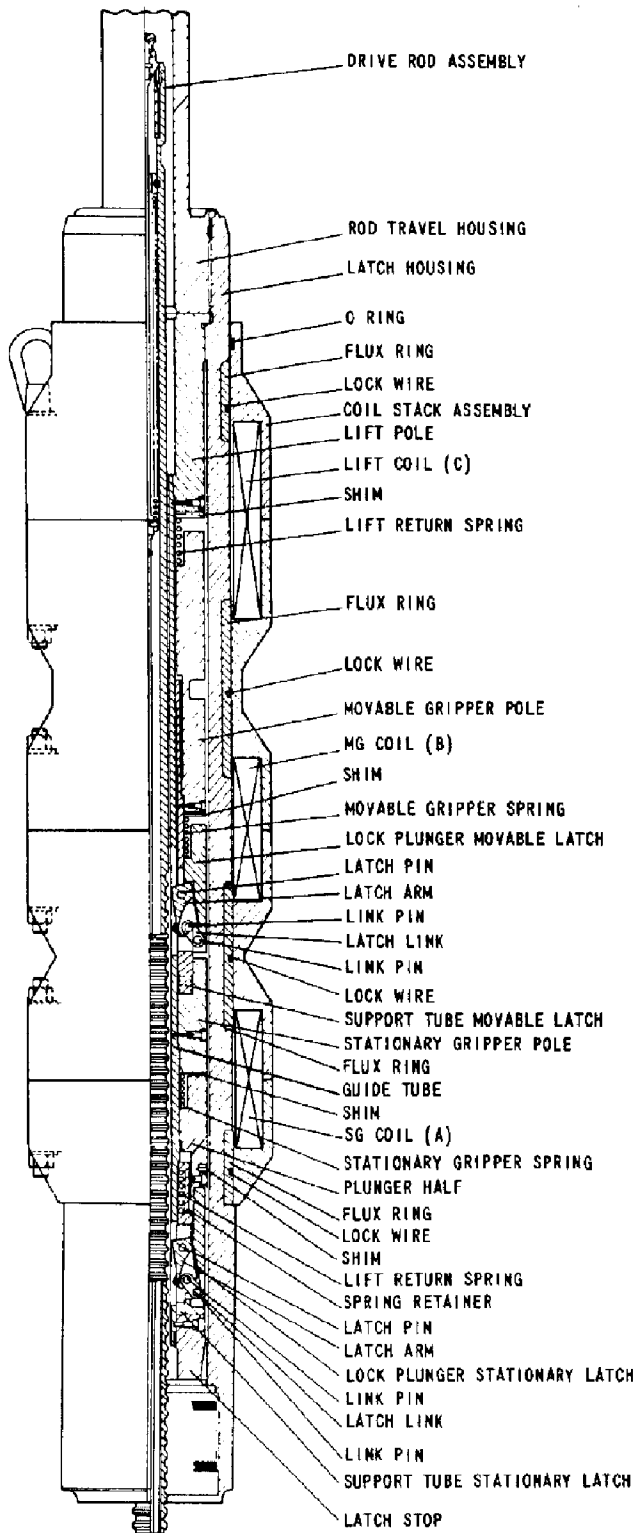


Figure 3-295. Nominal Latch Clearance At Minimum And Maximum Temperature

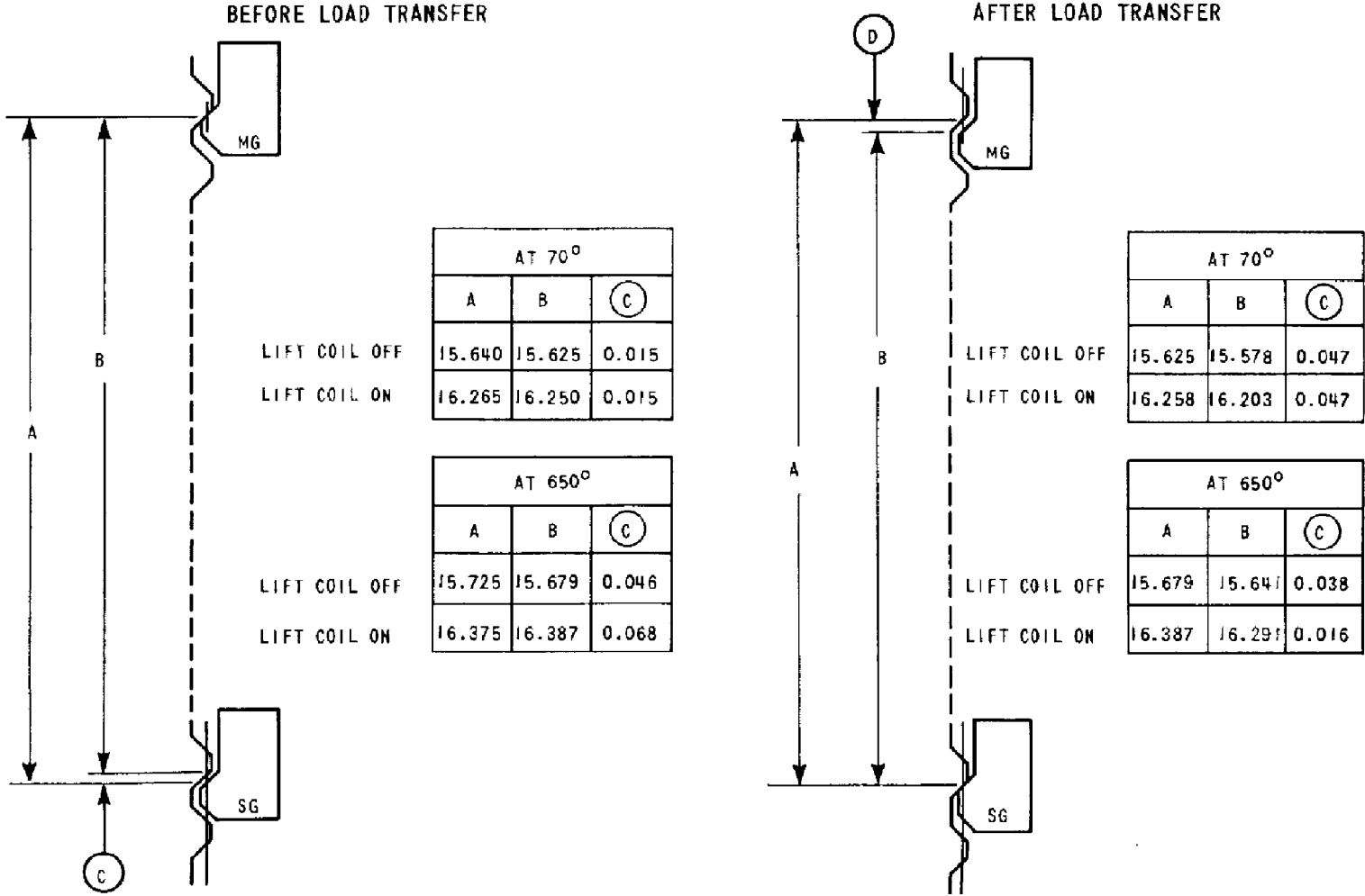
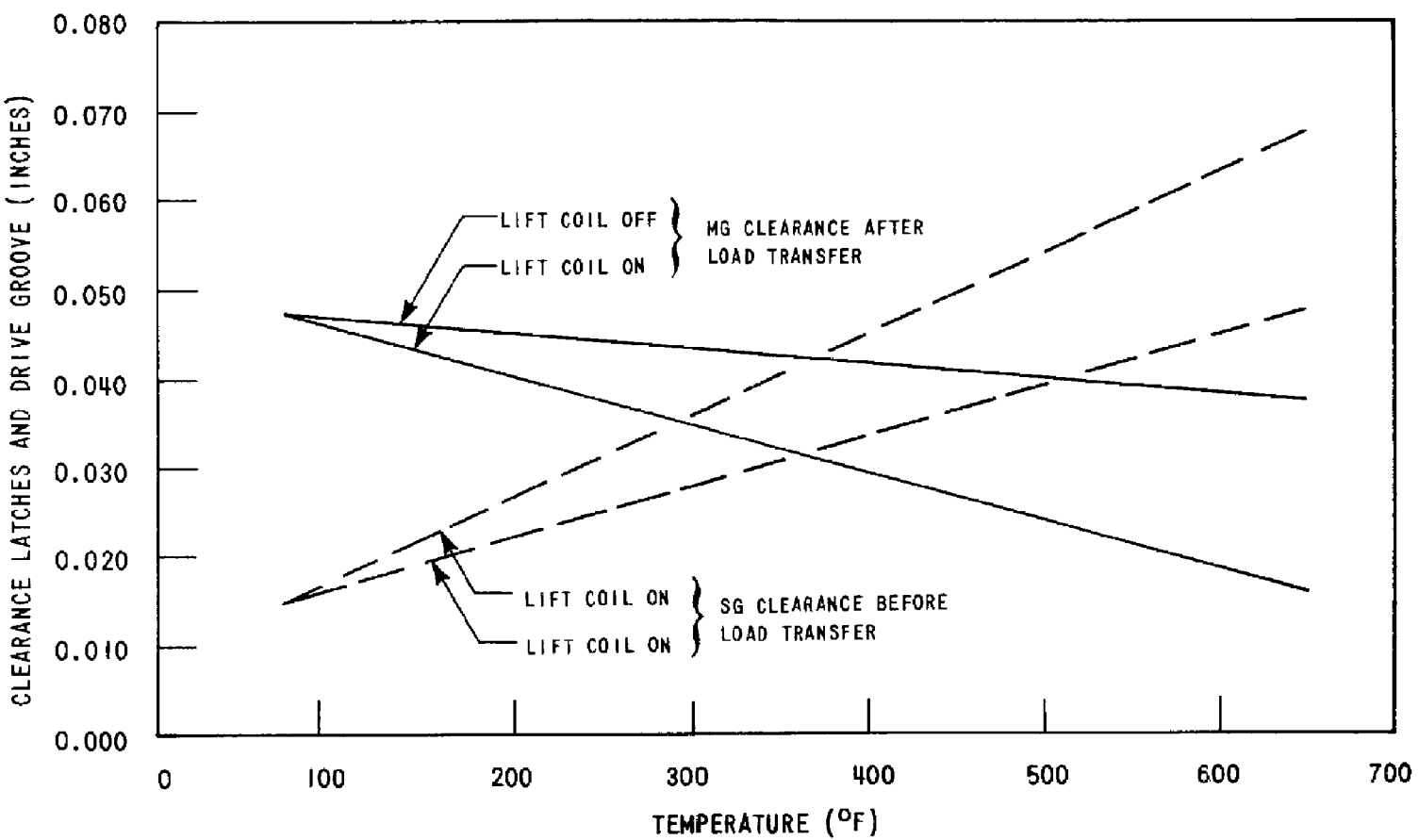


Figure 3-296. Control Rod Drive Mechanism Latch Clearance Thermal Effect



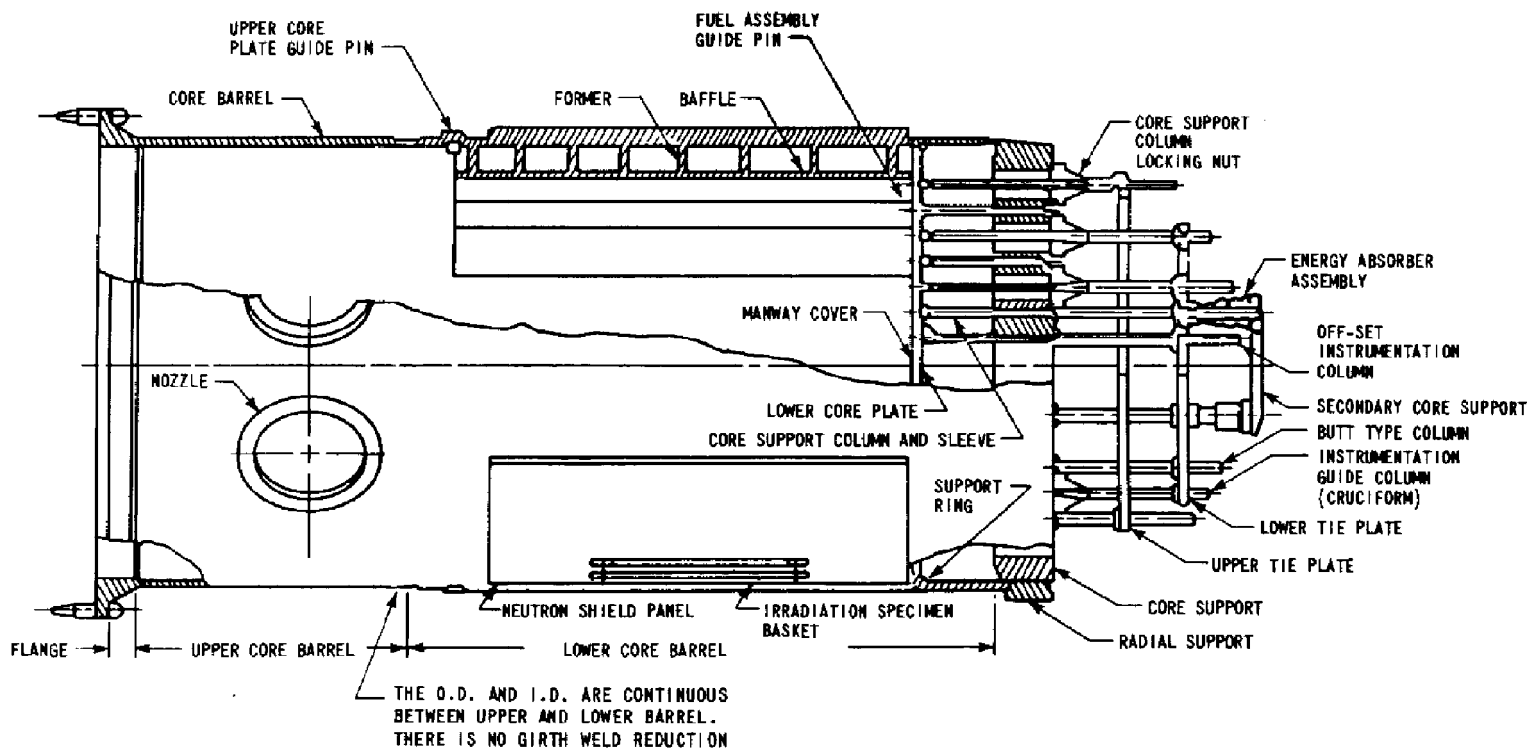
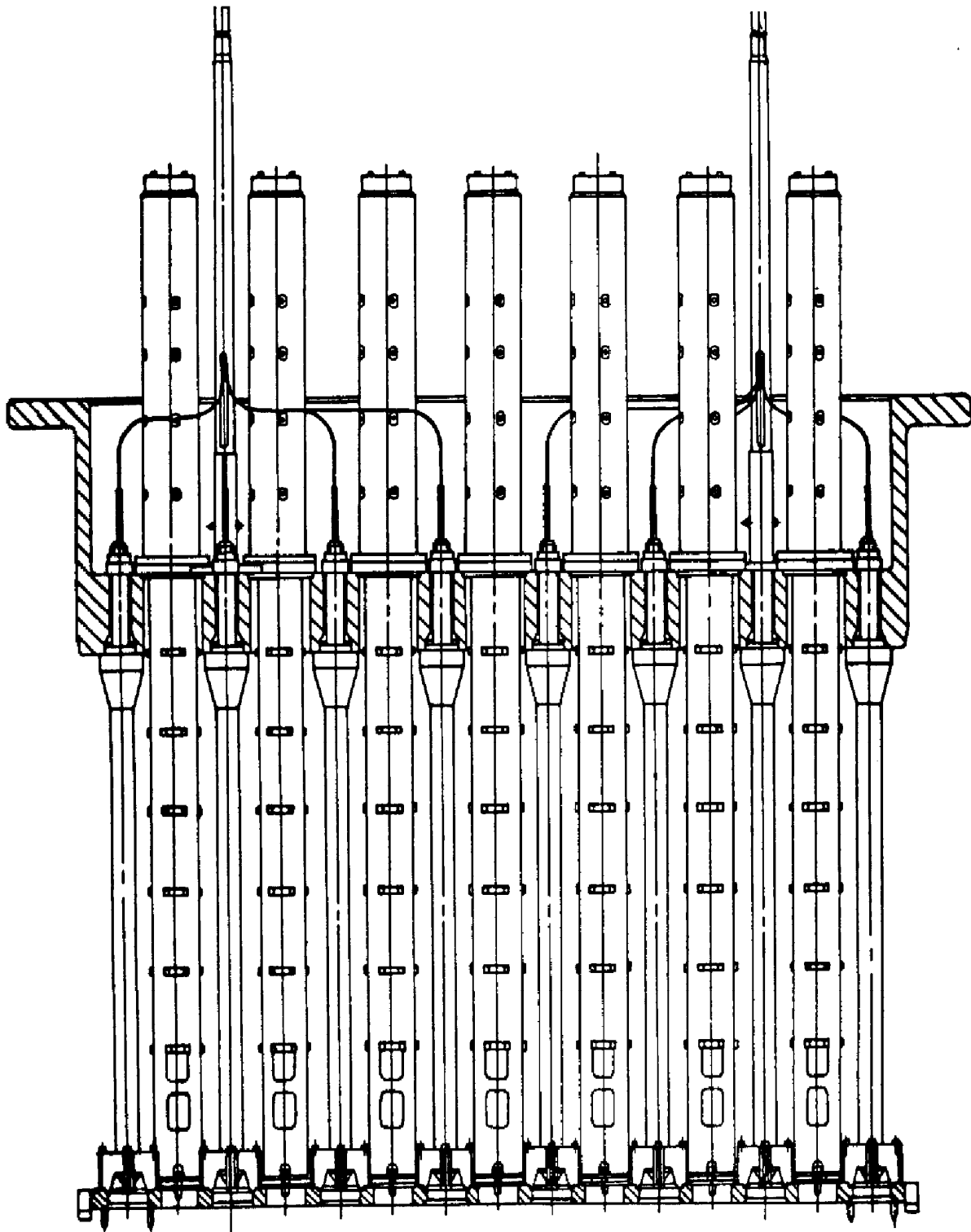


Figure 3-297. Lower Core Support Assembly (Core Barrel Assembly)

Figure 3-298. Upper Core Support Structure



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Figure 3-299. Plan View Of Upper Core Support Structure

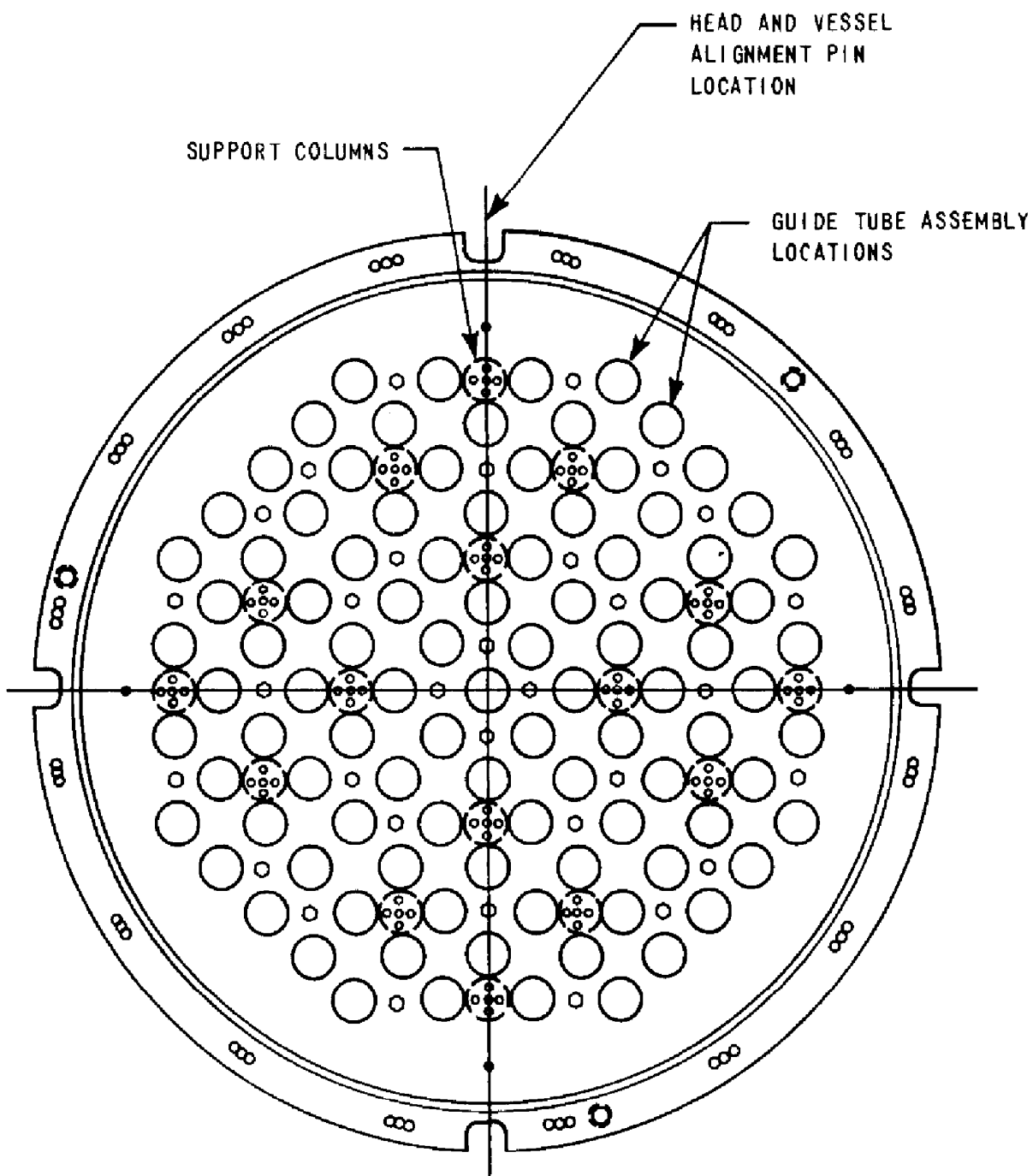


Figure 3-300. Direct Generation Interactive Scheme Flow Chart

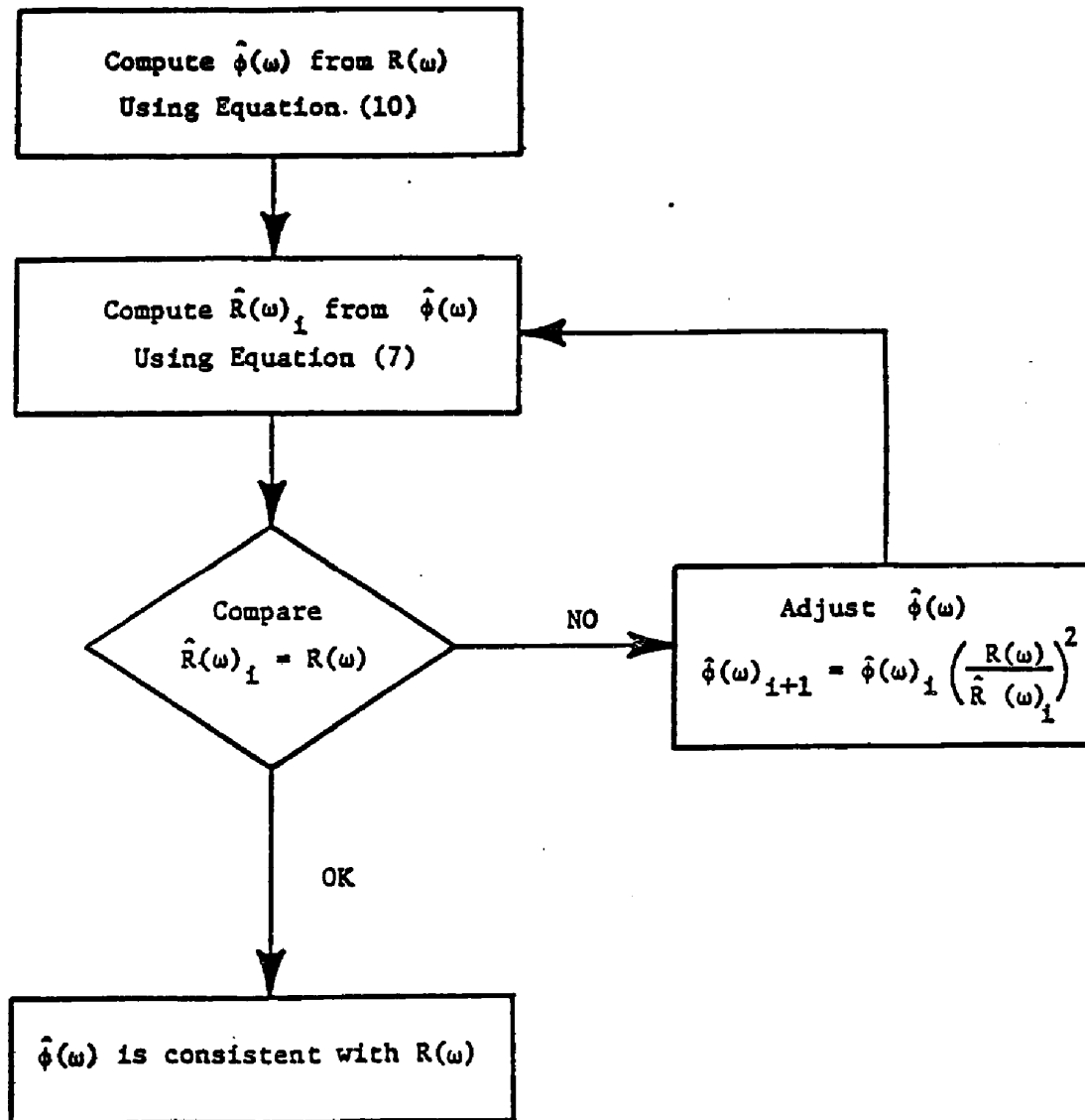
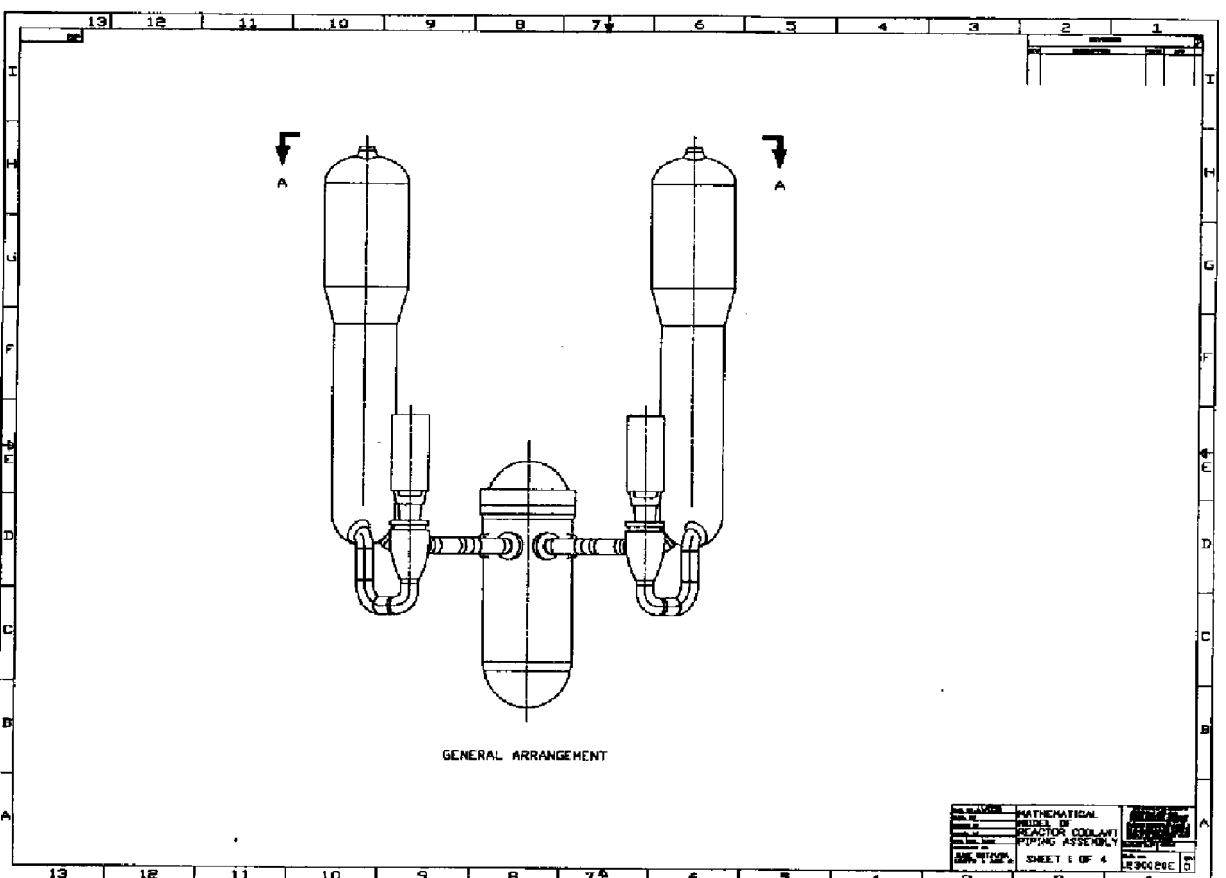
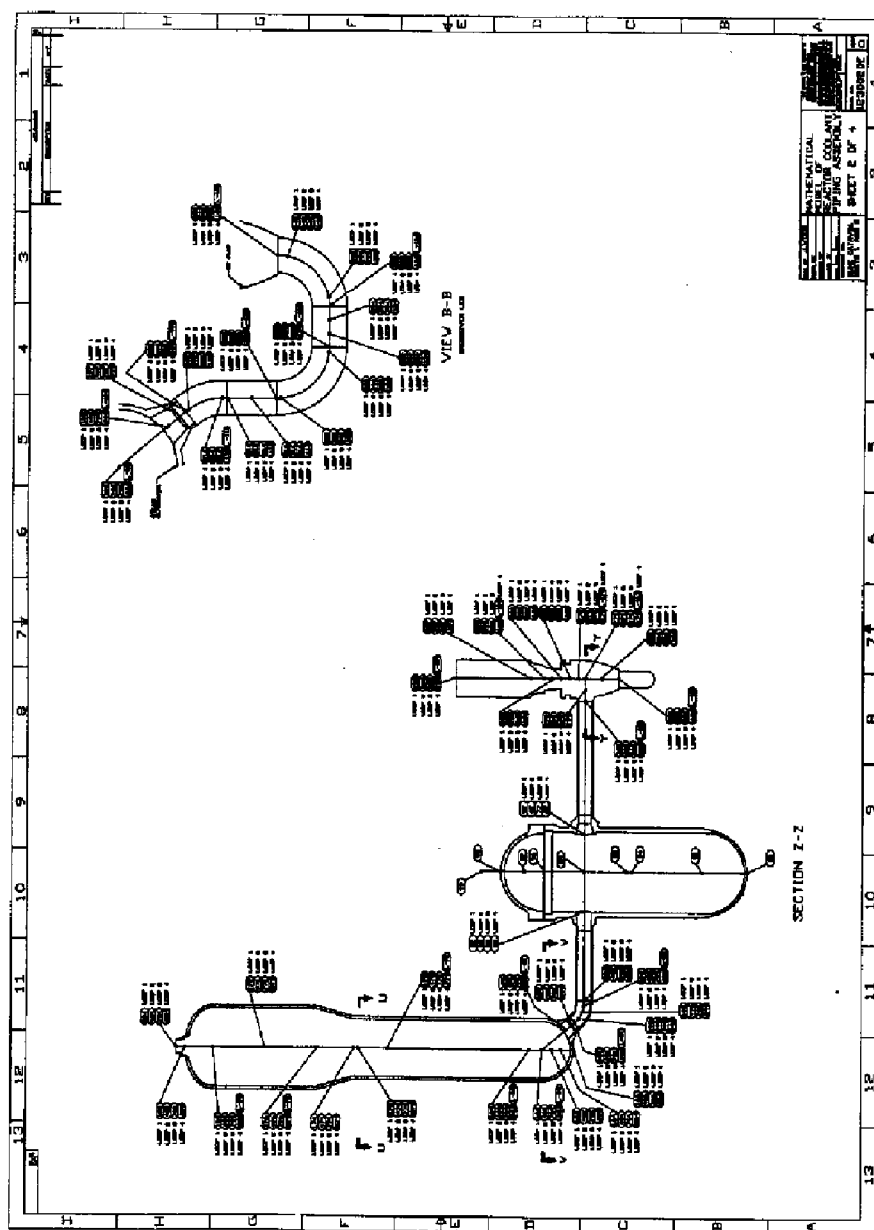


Figure 3-301. Reactor Coolant Loop Model for Steam Generator Replacement



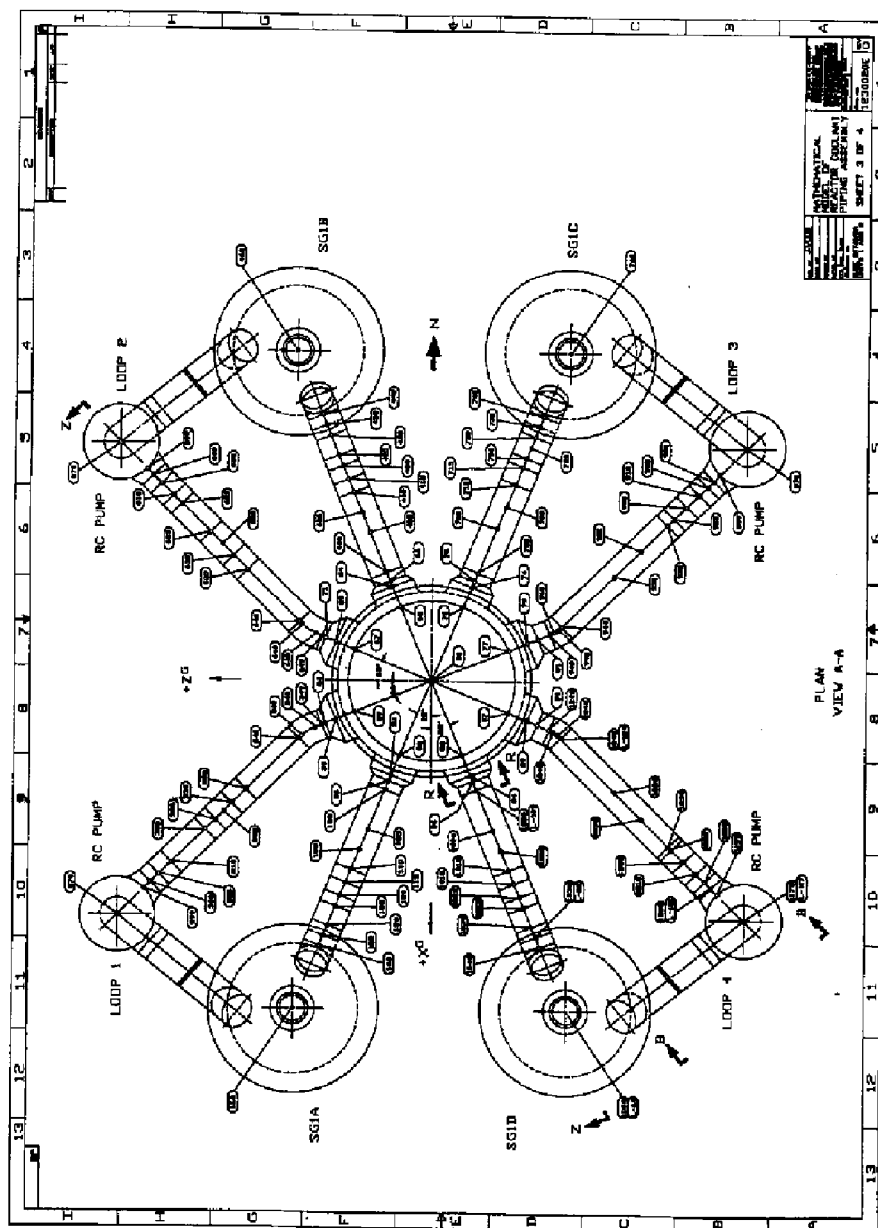
(22 OCT 2001)

Figure 3-302. Reactor Coolant Loop Model for Steam Generator Replacement



(22 OCT 2001)

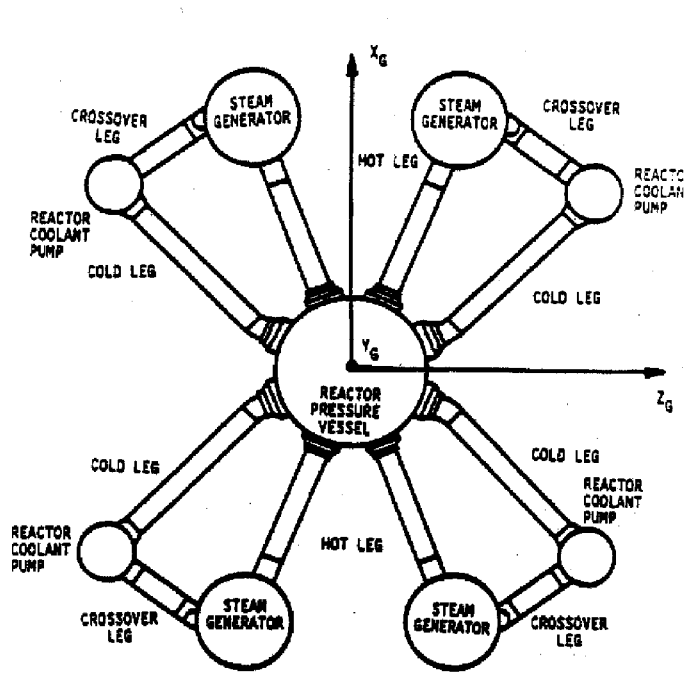
Figure 3-303. Reactor Coolant Loop Model for Steam Generator Replacement



(22 OCT 2001)



Figure 3-305. Loop Layout and Global Coordinates



**Figure 3-306. RPV Shell Submodel**

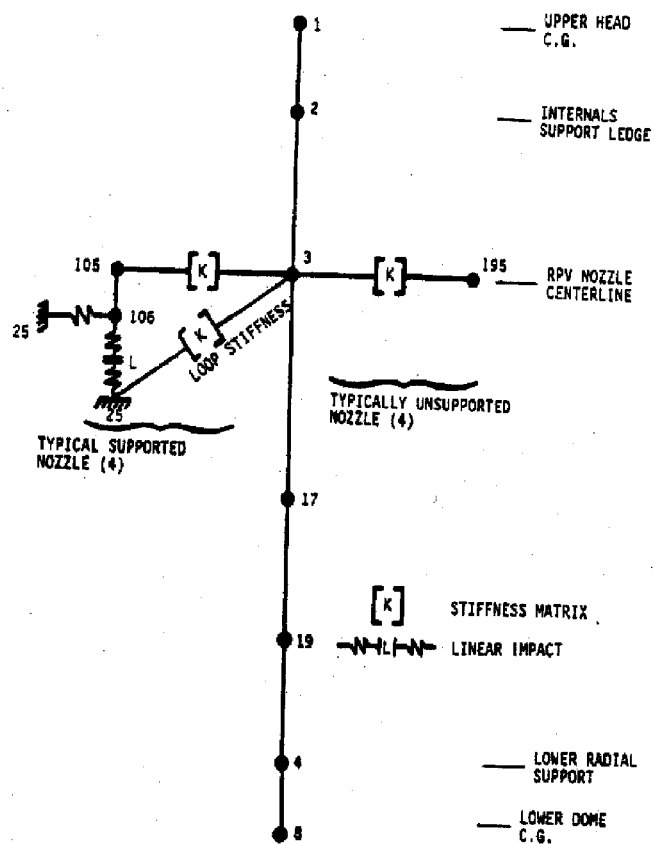




Figure 3-307. Core Barrel Submodel

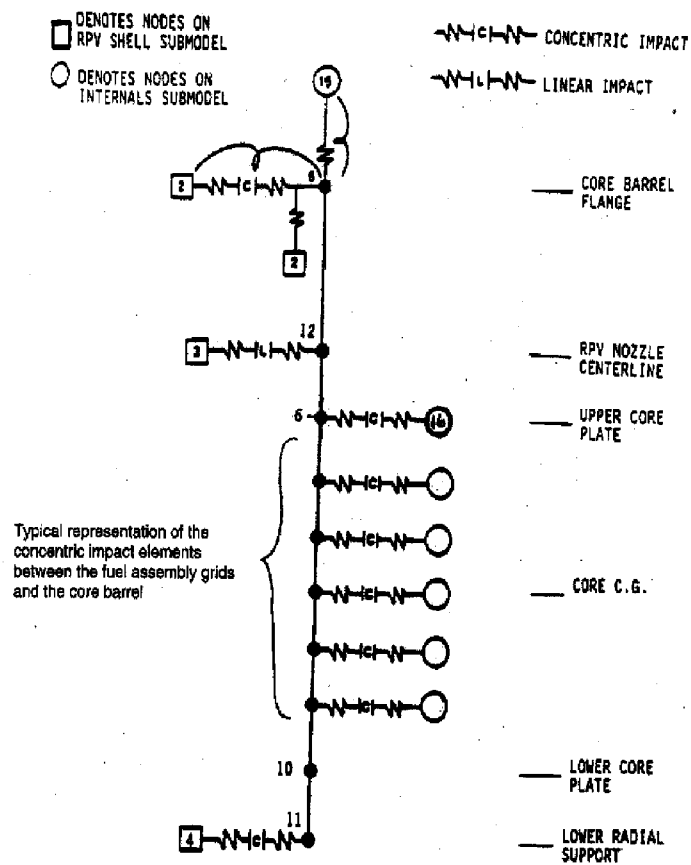


Figure 3-308. Reactor Internals Submodel

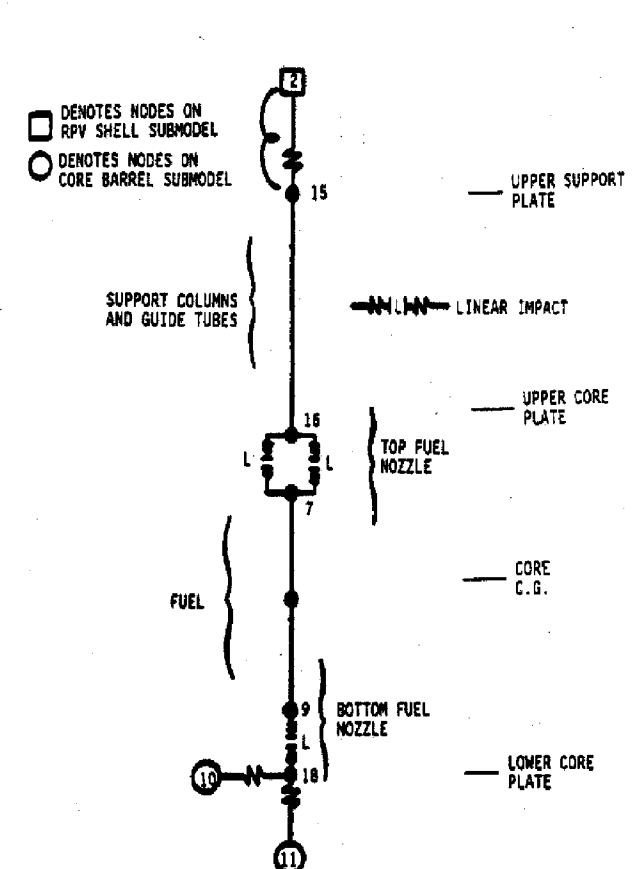


Figure 3-309. Hydrodynamic Masses in Vessel/Barrel Downcomer Annulus

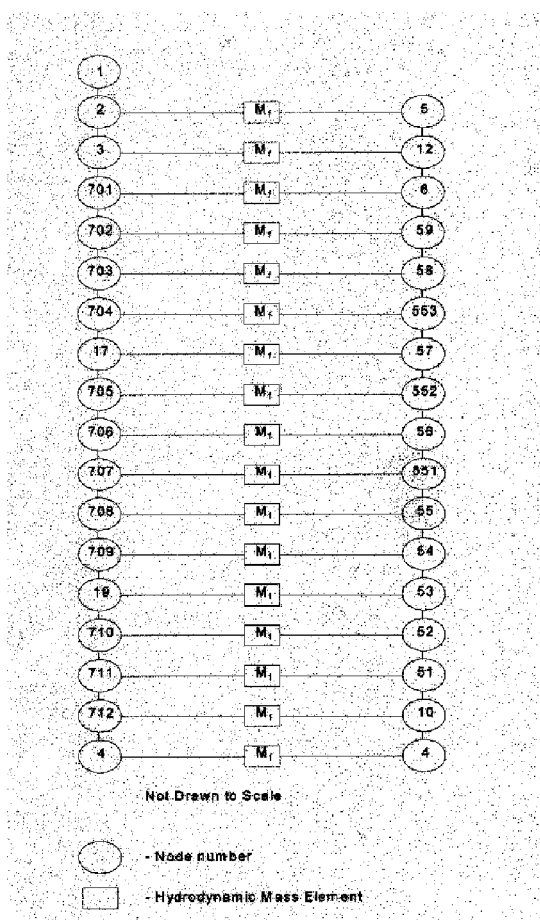


Figure 3-310. Response Acceleration Spectra

