

830 Power Building
TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401

SEP 21 1977

Mr. James P. O'Reilly, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region II - Suite 1217
230 Peachtree Street, NW.
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

This is in response to F. J. Long's August 30, 1977, letter; RII:HAW 50-259/77-15, 50-260/77-15, 50-296/77-15, which transmitted for our review an IE Inspection Report (same number). We have reviewed that report and do not consider any part of it to be proprietary.

Very truly yours,



J. E. Gilleland
Assistant Manager of Power

830 Power Building
TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401

SEP 21 1977

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U.S. Nuclear Regulatory Commission
Region II - Suite 1217
230 Peachtree Street, NW.
Atlanta, Georgia 30303

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
230 PEACHTREE STREET, N.W. SUITE 1217
ATLANTA, GEORGIA 30303

AUG 30 1977

In Reply Refer To:

RII:HAW

50-259/77-15

50-260/77-15

50-296/77-15

Tennessee Valley Authority
Attn: Mr. Godwin Williams, Jr.
Manager of Power
830 Power Building
Chattanooga, Tennessee 37401

Gentlemen:

This refers to the inspection conducted by Mr. H. A. Wilber of this office on August 16-19, 1977, of activities authorized by NRC Operating License Nos. DPR-33, DPR-52 and DPR-68 for the Browns Ferry 1, 2 and 3 facilities, and to the discussion of our findings held with Mr. J. G. Dewease, Plant Superintendent at the conclusion of the inspection.

Areas examined during the inspection and our findings are discussed in the enclosed inspection report. Within these areas, the inspection consisted of selective examinations of procedures and representative records, interviews with personnel, and observations by the inspector.

Within the scope of this inspection, no items of noncompliance were disclosed.

In accordance with Section 2.790 of the NRC's "Rules of Practice", Part 2, Title 10, Code of Federal Regulations, a copy of this letter and the enclosed inspection report will be placed in the NRC's Public Document Room. If this report contains any information that you (or your contractor) believe to be proprietary, it is necessary that you make a written application within 20 days to this office to withhold such information from public disclosure. Any such application must include a full statement of the reasons on the basis of which it is claimed that the information is proprietary, and should be prepared so that proprietary information identified in the application is contained in a separate part of the document. If we do not hear from you in this regard within the specified period, the report will be placed in the Public Document Room.

J

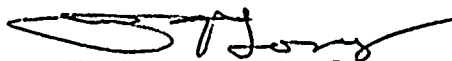
AUG 30 1977

Tennessee Valley Authority

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Should you have any questions concerning this letter, we will be glad to discuss them with you.

Very truly yours,



F. J. Long, Chief
Reactor Operations and Nuclear
Support Branch

Enclosure:

RII Inspection Report Nos.

50-259/77-15

50-260/77-15

50-296/77-15

cc: J. G. Dewease
Plant Superintendent
Box 2000
Decatur, Alabama 35602



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
230 PEACHTREE STREET, N.W. SUITE 1217
ATLANTA, GEORGIA 30303

Report Nos.: 50-259/77-15, 50-260/77-15 and 50-296/77-15

Docket Nos.: 50-259, 50-260 and 50-296

License Nos.: DPR-33, DPR-52 and DPR-68

Licensee: Tennessee Valley Authority
830 Power Building
Chattanooga, Tennessee 37401

Facility Name: Browns Ferry Units 1, 2 and 3

Inspection at: Browns Ferry site

Inspection conducted: August 16-19, 1977

Inspector: H. A. Wilber

Reviewed by:

H. C. Dance
H. C. Dance, Chief

Reactor Projects Section No. 1

Reactor Operations and Nuclear Support Branch

8/30/77
Date

Inspection Summary

Inspection on August 16-19, 1977 (Report Nos. 50-259/77-15, 50-260/77-15 and 50-296/77-15)

Areas Inspected: Routine, unannounced inspection of reportable occurrences, licensee action on IE Circulars, licensee adherence to safety limits, limiting safety system settings and limiting conditions of operation and plant tour. The inspection involved 29 total inspector-hours on site by one NRC inspector.

Results: No items of noncompliance or deviations were disclosed in the four areas inspected.

DETAILS I

Prepared by:

H. C. Dance / pr
H. A. Wilber, Reactor Inspector
Reactor Projects Section No. 1
Reactor Operations and Nuclear
Support Branch

8/30/77
Date

Dates of Inspection: August 16-19, 1977

Reviewed by:

H. C. Dance
H. C. Dance, Chief
Reactor Projects Section No. 1
Reactor Operations and Nuclear
Support Branch

8/30/77
Date

1. Persons Contacted

*J. G. Dewease, Plant Superintendent
H. L. Abercrombie, Assistant Plant Superintendent
T. P. Bragg, QA Supervisor
*W. A. Roberts, Maintenance Supervisor
J. B. Studdard, Operations Supervisor
*C. R. Brimer, Shift-Coordinator
*J. P. Butler, QA Engineer
W. H. Kinsey, Results Engineer, Mechanical Section
C. J. Rozear, Mechanical Engineer
T. E. Scott, Instrument Engineer
R. W. McGee, QA Engineering Aide
G. Craft, Unit Operator
J. McCullough, Unit Operator
D. South, Unit Operator

*Denotes those present at the exit interview.

2. Licensee Action on Previous Inspection Findings

This area was not reviewed during this inspection.

3. Unresolved Items

No new unresolved items were identified during this inspection.

4. Exit Interview

The inspector met with the TVA representatives denoted in paragraph 1 at the conclusion of the inspection on August 19, 1977, to summarize the scope and findings of this inspection.

5. IE Circular

The inspector verified that IE Circular 77-10, "Vacuum Conditions Resulting in Damage to Liquid Process Tanks," had been received at the site. A review by the plant staff is now in progress to ascertain the applicability of the circular to systems at Browns Ferry.

6. Review of Reportable Occurrences

The inspector performed an in-office review of the Reportable Occurrences listed below to determine that NRC reporting requirements were met and that corrective action stated in the report was appropriate. The reports were further reviewed at the site by examination of the Plant Operating Review Committee Minutes, test records, and log books and discussion with various staff members.

BFRO 259/7719, June 30, 1977, Pressure peaks at value greater than previously analyzed for ATWS pump trip safety analyses.

BFRO 259/7720, July 7, 1977, RHRSW pump A1 was found inoperable during routine surveillance testing while RHRSW pumps B1 and B3 were out of service.

BFRO 260/7710, July 12, 1977, Drywell oxygen sensor became erratic in operation.

BFRO 296/778, June 21, 1977, Standby liquid control system relief valve opened at 1300 psig instead of 1400 psig.

BFRO 296/779, June 23, 1977, Smoke detector gave false alarm.

BFRO 296/7710, July 5, 1977, HPCI turbine steam supply valve failed to open during a test.

BFRO 296/7711, July 11, 1977, Smoke detector gave a false alarm.

The inspector identified no discrepancies in the licensee's response or actions on the events reviewed.

7. Safety Limits, Limiting Safety System Setpoints, and Limiting Conditions for Operations

The inspector reviewed plant records and observed panel instrumentation to verify that the reactor operations were in conformance with the conditions defined in the Technical Specifications.

Safety Limits

The inspector reviewed recorder charts of reactor pressure to verify that the Safety Limit of 1375 psig, given in Technical Specification 1.2.A, was not exceeded during the Unit No. 1 scrams on February 11, 1977, April 21, 1977, April 28, 1977 and July 29, 1977; during the Unit No. 2 scram on May 3, 1977 and during the Unit No. 3 scrams on March 18, 1977, May 10, 1977 and May 15, 1977. No discrepancies were identified.

Limiting Safety System Settings

The inspector reviewed the certification records for 10 relief valves and one safety valve for Unit No. 1, 11 relief valves for Unit No. 2 and 10 relief valves for Unit No. 3. All valve settings met the requirements of Technical Specifications 2.2.A, 2.2.B and 2.2.C.

The inspector reviewed Intermediate Range Monitor calibration records to verify compliance with Technical Specification 2.1.A.3. The inspector identified no discrepancies.

Limiting Conditions of Operation (LCO)

The inspector reviewed the measured scram times of selected rods for each of the three units for conformance to Technical Specification 3.3.C. Tests of January 11, 1977, April 29, 1977 and August 2, 1977, were reviewed for Unit No. 1. Tests of December 8, 1976, March 8, 1977 and June 24, 1977, were reviewed for Unit No. 2. Test of December 5, 1976, March 19, 1977 and June 20, 1977, were reviewed for Unit No. 3. The inspector identified no discrepancies in the data reviewed.

The inspector reviewed completed surveillance test records to verify the trip points for drywell pressure and scram discharge volumes met the requirements defined in Table 3.1.A of the technical specifications. The dates of tests were July 27, 1977 (Unit 1), August 2, 1977 (Unit 2), and July 22, 1977 (Unit 3) for the drywell pressures and July 5, 1977 (Unit 1), July 10, 1977 (Unit 2), and July 26, 1977 (Unit 3) for the scram discharge volumes. The inspector identified no discrepancies in the data reviewed.

The inspector observed panel instruments readings for recirculating pump's speeds and logs flows and verified that the jet pump operability criteria of Technical Specification 3.6.E were met on each of the three units.

The inspector observed panel instrument readings from LI-64-54 and TIS 64-55B for the torus water levels and temperatures on each unit and verified that the conditions conformed to the requirements of Technical Specification 3.7.A.

The inspector reviewed operator records for the months of January 1977 through July 1977 to verify conformance to the diesel fuel oil requirements of Technical Specification 3.9.A.7. No discrepancies were identified in the areas reviewed.

The inspector verified on each unit that the controls for the Reactor Core Isolation Cooling System (RCIC) and the High Pressure Coolant Injection System (HPCI) were in "automatic" and that the controller tapes were set to deliver the flows required by Technical Specifications 3.5.E and 3.5.F.

The inspector observed panel instruments PI 75-20, PI 75-48, PI 4-51 and PI 74-65 and verified conformance to the pressure requirements of Technical Specification 3.5.H.

8. Plant Tour

The inspector toured the control rooms of all three units to observe operation activities on the day shift and midnight shift. The inspector noted that several members of plant management, including the Plant Superintendent, were in the control room observing the status of the units. The inspector toured various levels in all three reactor buildings to observe housekeeping, fire extinguisher test records, and maintenance activities. The inspector verified that a "Cutting, Welding, Open Flame Work Permit" was properly executed and posted at the work site for the work being done on the MG set floor drains on Unit No. 2. All the fire extinguishers observed by the inspector had current inspection tags attached.

The inspector identified no discrepancies in the areas observed.

The inspector observed 52 multiconductor cables that were tied to the side of cable tray JAV on the 2nd elevation of Unit 3 reactor building and to other trays on both the first and 2nd elevations of Unit 3 reactor building; these cables did not have "Flamemastic" applied. These cables were identified as temporary signal cables that were used during the power ascension test program for Unit 3. The licensee has committed to remove these cables by the next Unit 3 refueling outage. These cables were excluded from the requirements of the flamemastic program by internal correspondence; however, the licensee did initiate a survey of the three reactor

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50-260/77-15 and 50-296/77-15 I-5

buildings to verify conformance with the requirements of Mechanical Maintenance Instruction (MMI No. 75) "Installation and Repair of Penetrations and Fire Stops."

10-10-10



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TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

830 Power Building

September 15, 1977

Central File

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Dear Mr. O'Reilly:

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Very truly yours,

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for J. E. Gilleland
Assistant Manager of Power

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