

208/02/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-296

REC: OREILLY J P
NRC

ORG: FOX H S
TN VALLEY AUTH

DOCDATE: 07/25/78
DATE RCVD: 07/28.78

DOCTYPE: LETTER NOTARIZED: NO

COPIES RECEIVED

SUBJECT:

LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-296/78-019) ON 07/07/78 CONCERNING
EXCESSIVE DRYWELL FLOOR DRAIN LEAK RATE OBSERVED, CAUSED BY PARTIALLY BROKEN
WELD ON 1-INCH SOCKET WELD FITTING ON INSTRUMENT SENSING LINE ON A JET PUMP
RISER.

PLANT NAME: BROWNS FERRY - UNIT 3

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: DL

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF ORB#3 BC**W/4 ENCL

INTERNAL:

REG FILE**W/ENCL
I & E**W/2 ENCL
I & C SYSTEMS BR**W/ENCL
NOVAK/CHECK**W/ENCL
AD FOR ENG**W/ENCL
HANAUER**W/ENCL
AD FOR SYS & PROJ**W/ENCL
ENGINEERING BR**W/ENCL
KREGER/J. COLLINS**W/ENCL
K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL
MIPC**W/3 ENCL
EMERGENCY PLAN BR**W/ENCL
EEB**W/ENCL
PLANT SYSTEMS BR**W/ENCL
AD FOR PLANT SYSTEMS**W/ENCL
REACTOR SAFETY BR**W/ENCL
VOLLMER/BUNCH**W/ENCL
POWER SYS BR**W/ENCL

EXTERNAL:

LPDR'S
ATHENS, AL**W/ENCL
TIC, LIZ CARTER**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

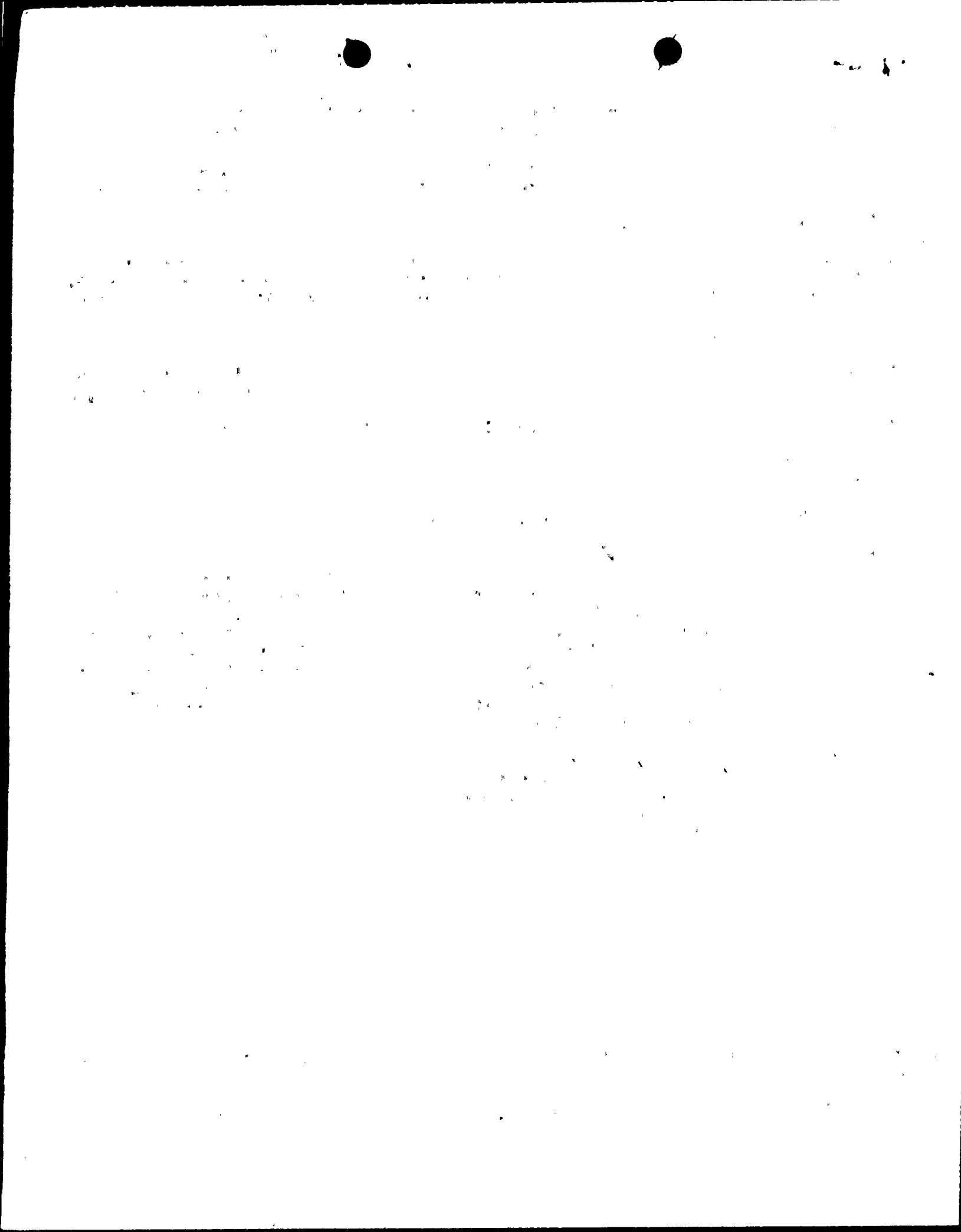
A104

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P

CONTROL NBR: 782140087

***** THE END *****

CP



TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

July 25, 1978

REGULATORY DOCKET FILE COPY

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET
NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE
REPORT BFR0-50-296/7819

The enclosed report provides details concerning an excessive drywell floor
drain leak rate which was observed by the operator during normal operation.
This report is submitted in accordance with Browns Ferry unit 3 technical
specification 6.7.2.a.(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. S. Fox
Director of Power Production

Enclosure (3)

cc (Enclosure):

Director (3)

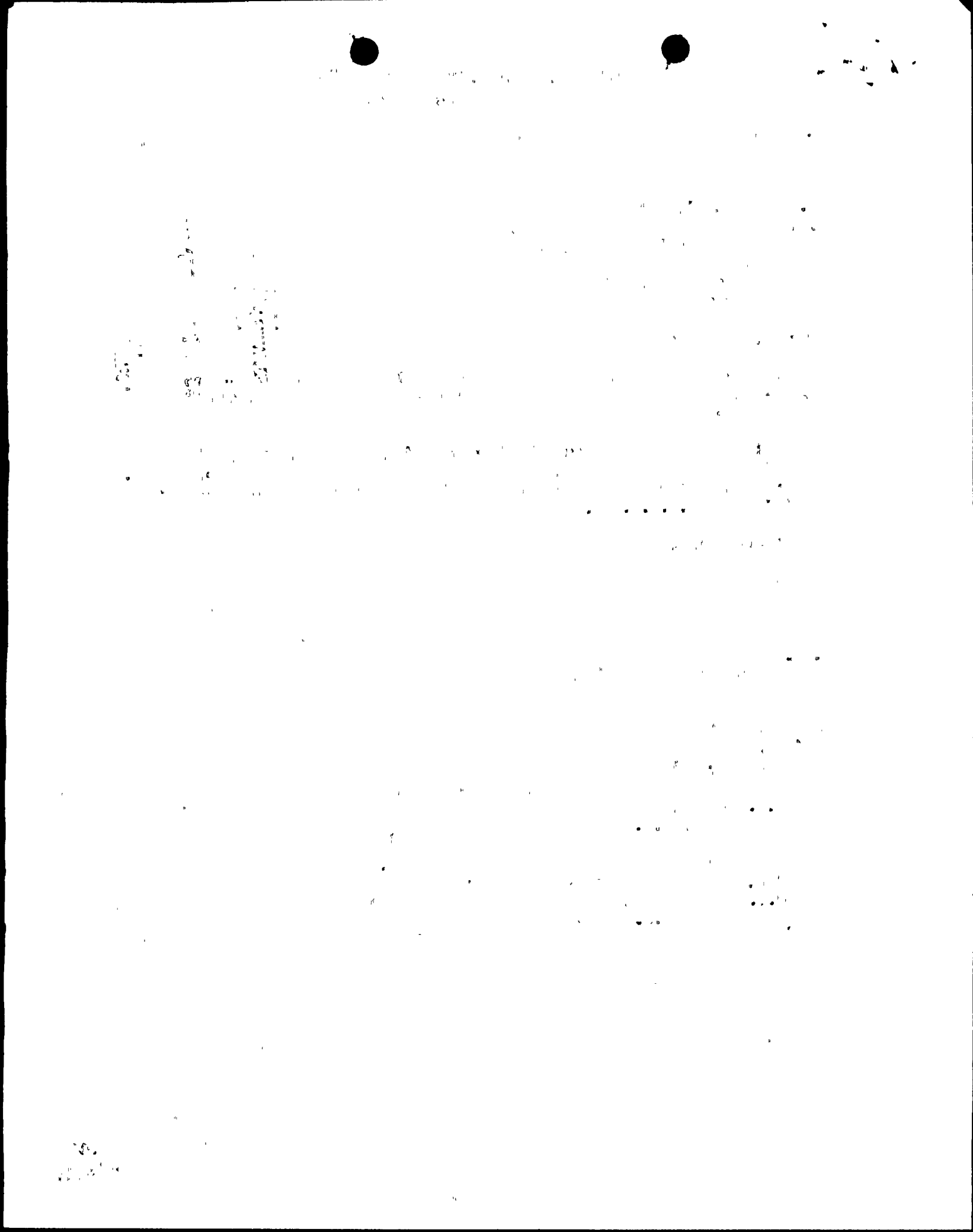
Office of Management Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director (40)

Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

782140087

A002
5/11



LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	A	L	B	R	F	3	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4	5		
7	8	LICENSEE CODE						14	15	LICENSE NUMBER										25	LICENSE TYPE					30	57	CAT	58

REPORT
SOURCE

0 1
 7 8
 REPORT SOURCE L 6 0 5 0 0 0 2 9 6 7 0 7 0 7 7 8 8 0 7 2 5 7 8 9
 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During normal operation, excessive drywell floor drain leak rate was observed by the
0 3 | operator (T.S.3.6.C.1). The reactor was manually placed in cold shutdown within the
0 4 | 24 hour T.S. time limit (T.S.3.6.C.3). There was no effect on the public health or
0 5 | safety. Redundant systems were available had the instrumentation become inoperable.
0 6 | The leakage did not affect connected instrumentation.
0 7 |

SYSTEM CODE C B (11)		CAUSE CODE E (12)		CAUSE SUBCODE B (13)		COMPONENT CODE P L P E X X (14)				COMP. SUBCODE A (15)		VALVE SUBCODE Z (16)	
LER/RO REPORT NUMBER 17 8 (17)		EVENT YEAR 7 8 (21-22)		SEQUENTIAL REPORT NO. 0 1 9 (24-26)		OCCURRENCE CODE 0 1 (28-29)		REPORT TYPE T (30)		REVISION NO. 0 (32)			
ACTION TAKEN B (18) X (19)		FUTURE ACTION		EFFECT ON PLANT A (20)		SHUTDOWN METHOD A (21)		HOURS 0 0 4 8 (22-24)		ATTACHMENT SUBMITTED Y (23)		NPRD-4 FORM SUB. N (24)	
PRIME COMP. SUPPLIER L (25)		COMPONENT MANUFACTURER X 9 9 9 (26-29)											

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 A partially broken weld on a 1-inch socket weld fitting on an instrument sensing line
1 1 on a jet pump riser was the cause of the leakage. The line was a class A stainless-
1 2 steel pipe pressurized to reactor pressure. The line was rewelded and the unit re-
1 3 turned to service. Other sensing lines of a similar configuration were inspected to
1 4 prevent recurrence. Design evaluation of the problem has been requested.

FACILITY STATUS		% POWER		OTHER STATUS (30)		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION (32)			
1	5	E	(28)	1	0	0	(29)	N/A	A	(31)	Operator Observation

ACTIVITY CONTENT
RELEASED OF RELEASE

1 6 Z (33) Z (34) N/A

7 8 9 10 11

AMOUNT OF ACTIVITY (35)

44 45 46 47 48 49 50

LOCATION OF RELEASE (36)

N/A

45 50

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	37	Z	38	N/A	39

PERSONNEL INJURIES		NUMBER		DESCRIPTION (41)	
1	8	0	0	0	(40) N/A

TYPE		DESCRIPTION
1	9	L (42) No damage - Temporary use of unit was lost.

PUBLICITY
ISSUED DESCRIPTION (45)
N/A

NRC USE ONLY

PHONE:

