

D 10/04/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-296

REC: OREILLY J P
NRC

ORG: FOX H S
TN VALLEY AUTH

DOCDATE: 09/29/78
DATE RCVD: 10/02/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

COPIES RECEIVED
LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-296/78-025) ON 09/17/78 CONCERNING
WHILE PERFORMING LEAK RATE TESTING, FOUR MSIV'S FCV--1-14, 1-26, 1-51 AND 1-52
EXCEEDED THE LEAKAGE LIMITS OF TECH SPEC 4.7.A.2.1...W/ATT.

PLANT NAME: BROWNS FERRY - UNIT 3

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: DL

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF ORB#3 BC**W/4 ENCL

INTERNAL:

REG FILE**W/ENCL

I & E**W/2 ENCL

I & C SYSTEMS BR**W/ENCL

NOVAK/CHECK**W/ENCL

AD FOR ENG**W/ENCL

HANAUER**W/ENCL

AD FOR SYS & PROJ**W/ENCL

ENGINEERING BR**W/ENCL

KREGER/J. COLLINS**W/ENCL

K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL

MIPC**W/3 ENCL

EMERGENCY PLAN BR**W/ENCL

EEB**W/ENCL

PLANT SYSTEMS BR**W/ENCL

AD FOR PLANT SYSTEMS**W/ENCL

REACTOR SAFETY BR**W/ENCL

VOLLMER/BUNCH**W/ENCL

POWER SYS BR**W/ENCL

EXTERNAL:

LPDR'S

ATHENS, AL**W/ENCL

NSIC**W/ENCL

ACRS CAT B**W/16 ENCL

DISTRIBUTION: LTR 44 ENCL 44
SIZE: 1P+1P+1P

CONTROL NBR: 780680162

THE END

A04

R

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

September 29, 1978

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET
NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE
REPORT BFRO-50-296/7825

The enclosed report provides details concerning four main steam isolation
valves which exceeded the leakage limits of technical specification
4.7.A.2.1 while performing local leak rate testing during refueling.
This report is submitted in accordance with Browns Ferry unit 3 technical
specification 6.7.2.a(9).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. S. Fox
Director of Power Production

REGULATORY DOCKET FILE COPY

Enclosure (3)
cc (Enclosure):
Director (3)
Office of Management Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director (40)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

U.S. NRC
COMMUNICATIONS
BRANCH

1978 OCT 2 AM 11 59

U.S. NRC
DISTRIBUTION
SERVICES UNIT

A002/s *
111

NOTED
JUL 2 1950
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JUL 2 1950

100-100000
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LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	A	L	B	R	F	3	2	0	0	-	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5
7	8	14						15	25										26	30				57	CAT	58		
		LICENSEE CODE							LICENSE NUMBER											LICENSE TYPE								

**REPORT
SOURCE**

REPORT SOURCE 01 L 6 0 5 0 0 0 2 9 6 7 0 9 1 7 7 8 8 0 9 2 9 7 8 9
7 8 60 61 68 69 74 75 80
DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

[02] During refueling, while performing local leak rate testing, four MSIV's, FCV-1-14, [03] 1-26, 1-51 and 1-52, exceeded the leakage limits of T. S. 4.7.A.2.i. FCV-1-14 leakage [04] was 1843 SCFH; FCV-1-26 leakage was 442,4096 SCFH; FCV-1-51 leakage was 50,7488 SCFH; [05] FCV-1-52 leakage was 1997.1238 SCFH. Redundant systems were not applicable since the [06] reactor was in the refueling mode. There was no hazard to the public health or [07] safety. Previous occurrences: BFR0-50-259/7723 and BFR0-50-260/789.

08 |-----| 80

09		SYSTEM CODE C D		CAUSE CODE E		CAUSE SUBCODE B		COMPONENT CODE V A L V E X						COMP. SUBCODE F		VALVE SUBCODE G																									
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	
LER/RO REPORT NUMBER		EVENT YEAR 7 8		SEQUENTIAL REPORT NO. 0 2 5		OCCURRENCE CODE 0 1						REPORT TYPE T		REVISION NO. 0		ACTION TAKEN Z		FUTURE ACTION B		EFFECT ON PLANT Z		SHUTDOWN METHOD Z		HOURS 0 0 0 0						ATTACHMENT SUBMITTED Y		NPRD-4 FORM SUB. N		PRIME COMP. SUPPLIER. N		COMPONENT MANUFACTURER A 5 8 5					

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The defective valves were 26 inch globe valves, pneumatic actuation, operating

1 1 pressure rating of 1250 psig at 575°F. The cause is not known at this time. The

1 2 valves will be repaired and retested.

1 3

1 4 | _____ 80

FACILITY STATUS (28) 1 5 H 7 8 9
% POWER (29) 0 0 0 10 11 12
OTHER STATUS (30) NA 13 44
METHOD OF DISCOVERY (31) B 45 46
DISCOVERY DESCRIPTION (32) Surveillance Test 47 80

ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE	
1	6	Z	33	NA	35	NA	36
7	8	9	10	11	44	45	80

PERSONNEL EXPOSURES									
NUMBER			TYPE		DESCRIPTION (39)				
1	7	0	0	0	(37)	Z	(38)	NA	

PERSONNEL INJURIES		
NUMBER	DESCRIPTION	(41)

1	8	0	0	0	(40)	NA													80
7	8	9	11	12													80		
LOSS OF OR DAMAGE TO FACILITY (43)																			
TYPE OF DAMAGE																			

TYPE		DESCRIPTION	
1	9	Z	42 NA

PUBLICITY
 ISSUED DESCRIPTION (45) NA
 (2) (0) (44)

NRC USE ONLY

PHONE:

Handwritten marks and scribbles in the top right corner.

LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 7825 Technical Specification Involved 4.7.A.2.i

Reported Under Technical Specification 6.7.2.a(9)

Date of Occurrence 9/17/78 Time of Occurrence 0900 Unit 3

Identification and Description of Occurrence:

Four MSIV's, FCV-1-14, 1-26, 1-51 and 1-52, exceeded the leakage allowed by T.S. 4.7.A.2.i during local leak rate testing. The following are the leak rates:

FCV-1-14	1843.4169 SCFH	FCV-1-51	50.7488 SCFH
FCV-1-26	442.4096 SCFH	FCV-1-52	1997.1238 SCFH

Conditions Prior to Occurrence:

Unit shutdown for refueling. The reactor head was off.

Action specified in the Technical Specification Surveillance Requirements met due to inoperable equipment. Describe.

Repair and retest to be done.

Apparent Cause of Occurrence:

Unknown at this time.

Analysis of Occurrence:

None

Corrective Action:

Valves will be repaired and satisfactorily retested prior to return to service of the unit.

Failure Data:

BFRO-50-259/7723
BFRO-50-260/789

Tennessee Valley Authority - Browns Ferry Nuclear Plant

