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Mr. E. Case	1 signed			SENT AEC PDR XX SENT LOCAL PDR XX

CLASS	UNCLASS	PROP INFO	INPUT	NO CYS REC'D	DOCKET NO:
	XXX			1	50-260

DESCRIPTION: Ltr trans the following:

ENCLOSURES: Abnormal Occurrence AO-50-260/753W on 1-19-75 re relief valve PCV-1-4 operated spuriously several times....

(1 cy encl rec'd)

ACKNOWLEDGED Do Not Remove

PLANT NAME: Browns Ferry Unit 2

FOR ACTION/INFORMATION DHL 2-3-75

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TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

Regulatory Docket File

January 29, 1975

Mr. Edson G. Case
Acting Director of Licensing
U.S. Nuclear Regulatory Commission
Washington, DC 20545

Dear Mr. Case:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - ABNORMAL OCCURRENCE REPORT BFAO-50-260/753W

The enclosed report is to provide details concerning relief valve PCV-1-4 which operated spuriously several times and is submitted in accordance with Appendix A to Regulatory Guide 1.16, Revision 1, October 1973. This event occurred on Browns Ferry Nuclear Plant unit 2 on January 19, 1975.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

E. F. Thomas

E. F. Thomas
Director of Power Production

Enclosure
CC (Enclosure):
Mr. Norman C. Moseley, Director
U.S. Nuclear Regulatory Commission
Regional Office
230 Peachtree Street, NW., Suite 818
Atlanta, Georgia 30303

1104

ABNORMAL OCCURRENCE REPORT

Report No.: BFAO-50-260/753W
Report Date: January 29, 1975
Occurrence Date: January 19, 1975
Facility: Browns Ferry Nuclear Plant unit 2

Identification of Occurrence

On January 19, 1975, beginning at approximately 0100 hours, relief valve PCV-1-4 operated spuriously several times.

Conditions Prior to Occurrence

The reactor was operating at approximately 340 MWe.

Description of Occurrence

During power ascension, the main steam relief valve PCV-1-4 operated several times while reactor pressure was normal.

Designation of Apparent Cause of Occurrence

Disassembly and inspection of the relief valve revealed both second-stage and pilot-stage leakage. The pilot-stage leakage resulted from an indentation in both the seat and disc. It appears that closure of the pilot on a foreign object caused the indentation. No foreign material was found in the valve. It is concluded that pilot leakage caused the spurious valve actuation.

Analysis of Occurrence

There were no adverse effects to the public health and safety or damage to systems, components, or structures as a result of this occurrence. An orderly unit shutdown was conducted. No extended blowdown occurred.

Corrective Action

The relief valve was replaced. Post-maintenance functional tests at approximately 250 psi and at rated pressure were satisfactorily performed. Repairs were made to the valve that was removed, and steam setpoint tests will be conducted. A torus inspection conducted subsequent to this event confirmed normal conditions.

Failure Data

Previous failures are reported on Abnormal Occurrence Reports BFAO-50-260/7430W; BFAO-50-260/7429W; BFAO-7426W, unit 1; BFAO-749W, unit 1; BFAO-7349W, unit 1; and BFAO-7314W, unit 1.

Valve Data

Manufacturer - Target Rock Corporation
Model - 67F
Design Pressure - 1,200 psi
Setpoint - 1,080 psig
Capacity - 800,000 lb/hr saturated steam
Serial No. - 57