



AMERICAN REVOLUTION BICENTENNIAL
1776-1976

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

July 2, 1976

Mr. Norman C. Moseley, Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
230 Peachtree Street, NW., 8th Floor
Atlanta, Georgia 30303

Dear Mr. Moseley:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 -
DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE
OCCURRENCE REPORT BFRO-50-260/765

The enclosed report is to provide details concerning HCU 30-31 that was found incapable of being scrambled due to valves 2-85-612 and 2-85-615 being in the closed position and is submitted in accordance with Appendix E to Regulatory Guide 1.16, Revision 4, August 1975. This event occurred on Browns Ferry Nuclear Plant unit 2.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



H. S. Fox

Director of Power Production

Enclosure (3)

CC (Enclosure):

Director (3)

Office of Management Information and Program Control

U.S. Nuclear Regulatory Commission

Washington, D.C. 20555

Director (40)

Office of Inspection and Enforcement

U.S. Nuclear Regulatory Commission

Washington, D.C. 20555

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ONS

LICENSEE EVENT REPORT

CONTROL BLOCK:

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME														LICENSE NUMBER														LICENSE TYPE					EVENT TYPE	
01	A	L	B	R	F	2	00-000000-00								4	1	1	1	1	0	1													
7	8	9				14	15											25	26	27	28	29	30	31	32									

CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER										EVENT DATE					REPORT DATE											
01	CONT	*	T	L	050-0260										0	6	1	9	7	6												
7	8		57	58	59	60	61											68	69							74	75					80

EVENT DESCRIPTION

02	HCU 30-31 was found incapable of being scrambled due to valves 2-85-612 and																												7	8	9																													80
03	2-85-615 being in the closed position. Interim Technical Specification 4.3.C																												7	8	9																													80
04	requires valves 2-85-612 and 2-85-615 for each HCU to be open at all times.																												7	8	9																													80
05	(BFRO-50-260/765)																												7	8	9																													80
06																													7	8	9																													80

SYSTEM CODE				CAUSE CODE		COMPONENT CODE											PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION	
07	R	B	A	C R D R V E											N	G	O	8	O	Y				
7	8	9	10	11	12										43	44			47	48				

CAUSE DESCRIPTION

08	Incorrect verification of required valve position caused by personnel																												7	8	9																													80
09	error.																												7	8	9																													80
10																													7	8	9																													80

FACILITY STATUS			% POWER			OTHER STATUS							METHOD OF DISCOVERY		DISCOVERY DESCRIPTION																		
11	H	0	0	0	NA							B	NA																				
7	8	9	10	11	12	13							44	45	46																		80

FORM OF ACTIVITY RELEASED			CONTENT OF RELEASE			AMOUNT OF ACTIVITY											LOCATION OF RELEASE															
12	Z	Z	NA							NA																						
7	8	9	10	11	12	13								44	45																	80

PERSONNEL EXPOSURES

NUMBER			TYPE		DESCRIPTION																												
13	0	0	0	Z	NA																												
7	8	9	11	12	13																												80

PERSONNEL INJURIES

NUMBER			DESCRIPTION																													
14	0	0	0	NA																												
7	8	9	11	12																												80

OFFSITE CONSEQUENCES

15	NA																												7	8	9																													80
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LOSS OR DAMAGE TO FACILITY

TYPE			DESCRIPTION																													
16	Z	NA																														
7	8	9	10																													80

PUBLICITY

17	NA																												7	8	9																													80
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ADDITIONAL FACTORS

18	NA																												7	8	9																													80
19																													7	8	9																													80

NAME: _____ PHONE: _____

