

50-260

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER
INCIDENT REPORT

TO: N.C. MOSELEY

FROM: TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENN.
H.S. FOX

DATE OF DOCUMENT

3/9/77

DATE RECEIVED

4/6/77

☒ LETTER☐ NOTORIZED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

☒ ORIGINAL
☐ COPY☒ UNCLASSIFIED

1 signed

DESCRIPTION

LTR. TRANS THE FOLLOWING.....

(1P)

PLANT NAME: BROWNS FERRY #2

SAB

ENCLOSURE

LICENSEE EVENT REPORT FOR R.O. # 772 ON 12/
22/77 CONCERNING CPRRAT EXCEEDING THE LIMIT
OF 1.0

(1P)

ACKNOWLEDGED

DO NOT REMOVE

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:

W/3 CYS FOR ACTION

LIC. ASST.:

W/1 CYS

ACRS /6 CYS HOLDING/SENT

Schwencer

Sheppard

As CAT A

INTERNAL DISTRIBUTION

☒ REG FILE

NRC PDR

I & E (2)

MIPC

SCHROEDER/IPPOLITO

HOUSTON

NOVAK/CHECK

GRIMES

CASE

BUTLER

HANAUER

TEDESCO/MACCARY

EISENHUT

BAER

SHAO

VOLLMER/BUNCH

KREGER/J. COLLINS

EXTERNAL DISTRIBUTION

LPDR: Atkins, P1

TIC:

NSIC:

CONTROL NUMBER

770980248

100

100

100

100

100



TENNESSEE VALLEY AUTHORITY

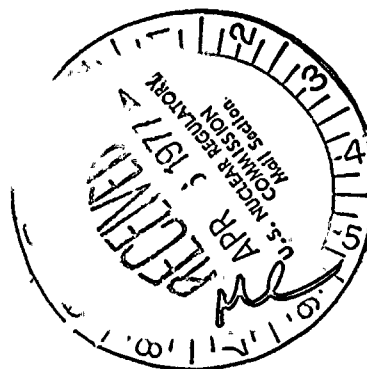
CHATTANOOGA, TENNESSEE 37401

Regulatory

File Cy.

March 9, 1977

Mr. Norman C. Moseley, Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
230 Peachtree Street, NW., Suite 1217
Atlanta, Georgia 30303



Dear Mr. Moseley:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 -
DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE
OCCURRENCE REPORT BFRO-50-260/772

The enclosed report is to provide details concerning CPRRAT that exceeded the limit of 1.0 on December 22, and December 27, 1976, following pre-conditioning ramp and process computer updating. In both cases, the CPRRAT was brought below the limit by rod insertion. This report is submitted in accordance with Browns Ferry Technical Specifications Section 6. This event occurred on Browns Ferry unit 2.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

for H. S. Fox

Director of Power Production

Enclosure (3)

CC (Enclosure):

Director (3)

Office of Management Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director (40)

Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE
NAME

LICENSE NUMBER

LICENSE
TYPE

EVENT
TYPE

[01] [A] [L] [B] [R] [F] [2]			[0] [0] [0] [0] [0] [0] [0] [0] [0] [0] [0] [0]										[4] [1] [1] [1] [1]				[0] [3]				
7	8	9	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32

[01] CONT		[57] [58]		[59] [60]		[0] [5] [0] [0] [2] [6] [0]										[1] [2] [2] [2] [7] [6]				[75] [76] [77] [78] [79] [80]					
7	8	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80

EVENT DESCRIPTION

[02]	Following a preconditioning ramp to 96% power, the process computer base distribution	80
[03]	was updated using OD-1. The limiting CPRRAT changed from .998 to 1.002 following the	80
[04]	OD-1. Rods were inserted to reduce the ratio to :999. This same event also occurred	80
[05]	on 12/27/76 where a CPRRAT of 1.006 after an OD-1 was corrected by rod insertion.	80
[06]	Conditions were corrected within the required two hours. BFR0-50-260/772	80

SYSTEM
CODE

CAUSE
CODE

COMPONENT CODE

PRIME
COMPONENT
SUPPLIER

COMPONENT
MANUFACTURER

VIOLATION

[57] [R] [C]		[58] [F]		[59] [F] [U] [E] [L] [X] [X]				[60] [N]		[61] [G] [0] [8] [0]				[62] [N]		
7	8	9	10	11	12	13	14	15	16	17	43	44	45	46	47	48

CAUSE DESCRIPTION

[08]	Significant power changes had occurred since the last OD-1 LPRM calibration. The OD-1	80
[09]	base distribution update accounted for uncertainties accumulated since the last update.	80
[10]		80

FACILITY
STATUS

% POWER

OTHER STATUS

METHOD OF
DISCOVERY

DISCOVERY DESCRIPTION

[11] [E]		[12] [0] [9] [6]		[13] NA				[14] [B]		[15] NA			
7	8	9	10	11	12	13	14	15	16	17	18	19	20

FORM OF
ACTIVITY
RELEASED

CONTENT
OF RELEASE

AMOUNT OF ACTIVITY

NA

LOCATION OF RELEASE

[12] [Z]		[13] [Z]		[14] NA				[15] NA					
7	8	9	10	11	12	13	14	15	16	17	18	19	20

PERSONNEL EXPOSURES

[13]	[0] [0] [0]	[12] [Z]	[13] NA	80
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PERSONNEL INJURIES

[14]	[0] [0] [0]	[12] NA	80
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OFFSITE CONSEQUENCES

[15]	NA	80
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LOSS OR DAMAGE TO FACILITY

[16]	[Z]	[10] NA	80
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PUBLICITY

[17]	NA	80
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ADDITIONAL FACTORS

[18]	The events were recognized reportable during a periodic review of the technical specifica-	80
[19]	tions by the reactor engineer. Prior to this time, the correction of the out-of-limit	80

(Over)

NAME:

PHONE:

Additional Factors (Continued)

condition within the interval specified in the Limiting Condition for
Operation was not considered reportable.