

50-260

## NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

INCIDENT REPORT

DATE OF DOCUMENT

7/18/77

DATE RECEIVED

7/21/77

TO:

Mr. Norman G. Moseley

FROM:

Tennessee Valley Authority  
Chattanooga, Tennessee  
H. S. Fox☒ LETTER☐ NOTORIZED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

☒ ORIGINAL  
☐ COPY☒ UNCLASSIFIED

DESCRIPTION

ENCLOSURE

DO NOT REMOVE

Licensee Event Report (RO 50-260/779) on  
5/26/77 concerning notification by the  
NSSS vendor that the minimum critical power  
ratio limit, as determined by analysis  
of a turbine trip without bypass, should  
be 1.29 for power operation with a core  
average exposure exceeding 8000 MWD/T.....

PLANT NAME:

Browns Ferry Unit No. 2  
RJL 7/21/77

(1-P)

(1-P)

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED  
SEND DIRECTLY TO KREGER/J. COLLINS

## FOR ACTION/INFORMATION

BRANCH CHIEF:

W/ 3 CYS FOR ACTION

LIC ASST.:

Schwerner (4)

## INTERNAL DISTRIBUTION

REG FILE

NRC PDR

L &amp; E (2)

MIPC

SCHROEDER/IPPOLITO

HOUSTON

NOVAK/CHECK

GRIMES

KNIGHT

BUTLER

HANAUER

TEDESCO

EISENHUT

BAER

SHAO

VOLLMER/BUNCH

KREGER/ J. COLLINS

ROSA

## EXTERNAL DISTRIBUTION

LPDR: A. Hays, D.

TIC:

NSIC:

ACRS (16) SENT AS CAT. B

CONTROL 18ER

772020242





# TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

JUL 18 1977

Mr. Norman C. Moseley  
U.S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region II  
230 Peachtree Street, NW., Suite 1217  
Atlanta, Georgia 30303

Regulatory Docket File



Dear Mr. Moseley:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 -  
DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE  
OCCURRENCE REPORT WFO-50-260/779

The enclosed report supersedes my letter to you dated June 9, 1977.  
Reevaluation of the initial reportable occurrence concerning notifi-  
cation by the NSSS vendor that the minimum critical power ratio limit,  
as determined by analysis of a turbine trip (or load rejection) without  
bypass, should be 1.29 for power operation with a core average exposure  
exceeding 8000 MWD/T indicated no violation had occurred.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. S. Fox  
Director of Power Production

Enclosure (3)

cc (Enclosure):

Director (3)

Office of Management Information and Program Control  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Director (40)

Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

772020242



THE UNIVERSITY OF CHICAGO

1954

THE UNIVERSITY OF CHICAGO  
DEPARTMENT OF CHEMISTRY  
JANUARY 1954

TO THE HONORABLE CHAIRMAN OF THE BOARD OF TRUSTEES  
OF THE UNIVERSITY OF CHICAGO

SIR:

I have the honor to acknowledge the receipt of your letter of the 12th inst. and in reply to inform you that the same has been forwarded to the appropriate authorities for their consideration.

I am, Sir, very respectfully,  
Yours truly,  
[Signature]

1954

CONTROL BLOCK:

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE  
NAME

LICENSE NUMBER

LICENSE  
TYPEEVENT  
TYPE

1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
	R	R	F	2		0	0	-	0	0	0	0	0	-	0	0		4	1	1	1	1								0	1

CATEGORY

REPORT  
TYPEREPORT  
SOURCE

DOCKET NUMBER

EVENT DATE

REPORT DATE

57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80
		P	L		0	5	0	-	0	2	6	0		0	5	2	6	7	7				

## EVENT DESCRIPTION

During steady state operation with a critical power ratio of 1.26 and a core average

exposure of 5854 MWD/T, the NSSS vendor gave notification that the minimum critical

power ratio limit, as determined by analysis of a turbine trip (or load rejection)

without bypass, should be 1.29 for power operation with a core average exposure exceeding

5000 MWD/T. The critical power ratio limit currently in use is 1.25. (BFR0-50-260/779)

SYSTEM  
CODECAUSE  
CODE

COMPONENT CODE

PRIME  
COMPONENT  
SUPPLIERCOMPONENT  
MANUFACTURER

VIOLATION

10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
E	C		C		F	U	E	L	X	X		N		G	O	8	0			N		

## CAUSE DESCRIPTION

A non-conservative error in the steam line volume used in the transient/safety analysis

was discovered.

FACILITY  
STATUS

% POWER

OTHER STATUS

METHOD OF  
DISCOVERY

DISCOVERY DESCRIPTION

9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
E		0	9	7		NA																	

FORM OF  
ACTIVITY  
RELEASEDCONTENT  
OF RELEASE

AMOUNT OF ACTIVITY

LOCATION OF RELEASE

9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
Z						NA																	

## PERSONNEL EXPOSURES

NUMBER

TYPE

DESCRIPTION

11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
	0	0																			

## PERSONNEL INJURIES

NUMBER

DESCRIPTION

11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
	0	0																			

## OFFSITE CONSEQUENCES

NA

## LOSS OR DAMAGE TO FACILITY

TYPE

DESCRIPTION

9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
Z																							

## PUBLICITY

NA

## ADDITIONAL FACTORS

NA

BFR0-50-260/779

RECEIVED DOCUMENT  
PROCESSING UNIT

1977 JUL 21 AM 11 26